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Radioactive Air Emissions Notice of Construction for Plutonium Finishing Plant Project W-460, "Plutonium Stabilization and Handling"

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May 2000

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Assistant Secretary for Environmental Management

Project Hanford Management Contractor for the
U.S. Department of Energy under Contract DE-AC06-96RL13200



**United States
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1 **RADIOACTIVE AIR EMISSIONS NOTICE OF CONSTRUCTION FOR PLUTONIUM**
2 **FINISHING PLANT PROJECT W-460,**
3 **-- "PLUTONIUM STABILIZATION AND HANDLING"**

4
5
6

1.0 INTRODUCTION

7 The following description and any attachments and references are provided to the Washington State
8 Department of Health (WDOH), Division of Radiation Protection, Air Emissions & Defense Waste
9 Section as a notice of construction (NOC) in accordance with Washington Administrative Code
10 (WAC) 246-247, Radiation Protection – Air Emissions. The WAC 246-247-060, "Applications,
11 registration, and licensing", states "This section describes the information requirements for approval to
12 construct, modify, and operate an emission unit. Any NOC requires the submittal of information listed in
13 Appendix A."

14
15 Additionally, the following description, attachments, and references are provided to the
16 U.S. Environmental Protection Agency (EPA) as an NOC, in accordance with Title 40 Code of Federal
17 Regulations (CFR), Part 61, "National Emission Standards for Hazardous Air Pollutants". The
18 information required for submittal to the EPA is specified in 40 CFR 61.07. The potential emissions from
19 this activity are estimated to provide greater than 0.1 millirem year total effective dose equivalent (TEDE)
20 to the hypothetical offsite maximally exposed individual (MEI) and commencement is needed within a
21 short time. Therefore, this application also is intended to provide notification of the anticipated date of
22 initial startup in accordance with the requirement listed in 40 CFR 61.09(a)(1), and it is requested that
23 approval of this application also constitutes EPA acceptance of this initial startup notification. Written
24 notification of the actual date of initial startup, in accordance with the requirement listed in
25 40 CFR 61.09(a)(2), will be provided later.

26
27 This NOC covers the activities associated with the construction and operation activities involving
28 stabilization and/or repackaging of plutonium in the 2736-ZB Building. A new exhaust stack will be built
29 and operated at the 2736-ZB Building to handle the effluents associated with the operation of the
30 stabilization and repackaging process. Figures provided are based on preliminary design.

31
32 **For the activities covered under this NOC, the unabated and abated TEDE to the hypothetical MEI**
33 **is 1.67 E+03 and 8.34 E-01 millirem per year, respectively.**

1 **2.0 FACILITY LOCATION (REQUIREMENT 1)**

2 U. S. Department of Energy, Richland Operations Office
3 825 Jadwin Avenue
4 P.O. Box 550
5 Richland, Washington 99352-3562

6
7 The coordinates for the proposed new stack (296-Z-7) are as follows:

8
9 2736-ZB Building, 200 West Area
10 Latitude: 46° 33' 00"
11 Longitude: 119° 37' 60"

12
13
14 **3.0 RESPONSIBLE MANAGER (REQUIREMENT 2)**

15 The responsible manager for the activities described under this NOC is as follows:
16
17 Mr. L. D. Romine, Director
18 Materials Disposition Division
19 U.S. Department of Energy,
20 P.O. Box 550
21 Richland, Washington 99352
22 (509) 376-4747

23
24
25 **4.0 TYPE OF PROPOSED ACTION (REQUIREMENT 3)**

26 The proposed action results in the construction of a new major emission unit.
27
28

29 **5.0 STATE ENVIRONMENTAL POLICY ACT (REQUIREMENT 4)**

30 The proposed action is categorically exempt from the requirements of the *State Environmental Policy Act*
31 under WAC 197-11-845.

32
33
34 **6.0 PROCESS DESCRIPTION (REQUIREMENT 5)**

35 Project W-460 will provide the equipment and modifications necessary for the Plutonium Finishing Plant
36 (PFP) to stabilize and/or repackaging plutonium and uranium, oxide and metals, for long-term storage.
37 Within the 2736-Z Building, existing vault storage cubicles will be modified to accommodate larger,
38 long-term storage canisters.

39

1 Project W-460 consists of distinct modules. A brief description of the Project W-460 modules and
2 processes are provided as follows.

- 4 • **#1-Stabilization Module.** The Stabilization Module consists of the material preparation area, furnace
5 area and the product fill area. In the material preparation area, canned items containing plutonium-
6 bearing materials are received, measured for accountability, and placed into a furnace tray (or boat)
7 for insertion into a furnace in the furnace area. The module also provides a waste pathway to dispose
8 of the waste cans and plastic. In the furnaces, the material in the boats will be heated to greater than
9 950°C for at least 2 hours, as specified in DOE Standard 3013 (DOE-STD-3013, *Criteria for Safe*
10 *Storage of Plutonium Metals and Oxides*). The material will be cooled and placed in a convenience
11 can, sampled to verify dryness and inserted into the Bagless Transfer System (BTS) Module.
- 12 • **#2-BTS Module.** In the BTS Module, the filled convenience cans will be received from the
13 Stabilization Module and placed into an inner can. The inner can head space will be backfilled with
14 helium. A plug will be welded to the inner wall of the container, and the middle of the weld would be
15 cut (maintaining glovebox confinement at all times).
- 16 • **#3-Inner Can Leak Test Module.** The Inner Can Leak Test Module will receive an inner welded
17 container (BTC). Operations in this module will verify the BTC meets or exceeds the leak tightness
18 requirements of DOE Standard 3013.
- 19 • **#4-Outer Can Weld Module.** The Outer Can Weld Module will receive a leak-checked BTC. The
20 BTC will be placed in an outer container. The outer can head space would be backfilled with helium,
21 and an outer container lid would be welded onto the container in accordance with the requirements of
22 DOE Standard 3013.
- 23 • **#5-Outer Can Leak Test Module.** The Outer Can Leak Test Module will receive an outer welded
24 container (3013 package) and operations in this module will verify the package meets or exceeds the
25 leak tightness requirements of 3013.
- 26 • **#6-NDA Laboratory Modification Module.** The Non-Destructive Analysis (NDA) Laboratory will
27 receive the 3013 package and analyze the 3013 package for isotopic distribution, heat load and
28 container baseline.
- 29 • **#7-Vault Modification Module.** The secure vault storage locations in the 2736-Z Building will be
30 modified to accommodate the 3013 packages. These packages will be sealed, offering no additional
31 potential-to-emit (PTE). No modifications to the existing 2736-Z Building ventilation system,
32 exhausting through minor stack 296-Z-6, will be made.
- 33 • **#8-Infrastructure Modification Module.** Project W-460 will modify existing infrastructure support
34 systems. Capacities of ventilation systems will be verified and enhanced if necessary. Configuration
35 of the systems will be modified, if necessary, to provide appropriate separation of PFP and process
36 enclosure ventilation. New systems will be installed if no system currently exists. Addition of a new
37 major exhaust stack and associated compliant monitoring equipment is planned. Equipment pads for
38 a nitrogen system and a gas bottle storage area will be installed.

47 Additional details regarding Project W-460 can be found in HNF-SD-W460-CDR-001, Rev. 1,
48 *Conceptual Design Report – Plutonium Stabilization and Handling, Project W-460*, and
49 HNF-SD-W460-FDC-001, Rev. 1, *Functional Design Criteria - Plutonium Stabilization and Handling*
50 (*PuSH*) *Project W-460*.

1 Plutonium and uranium that will be processed in the stabilization and repackaging process under
2 Project W-460 will be in the form of oxides and pure metal (Figure 1). Americium, plutonium, and
3 uranium oxides will be stabilized by heating the material in an oven to a temperature of greater than
4 950°C for a minimum of 2 hours.

5
6 The gloveboxes have two exhaust systems, normal and emergency. The normal exhaust exit points have
7 a roughing filter and fire screen inside the glovebox. Immediately outside each normal exhaust
8 connection is a testable high-efficiency particulate air (HEPA) filter. The design temperature inside the
9 glovebox is approximately 40°C. This is the normal glovebox exhaust temperature. The emergency
10 exhaust exits do not have any filters or screens inside the glovebox or immediately outside the glovebox.
11 The normal and emergency exhaust lines are combined and routed to the process exhaust HEPA filter
12 system, where the temperature will be below the 40°C glovebox temperature.

13
14 The offgas temperature from the furnaces will be a maximum 4 cubic foot per minute (cfm) at 1,000°C.
15 Each furnace will have an exhaust outlet line inside the glovebox that will segregate the furnace offgas
16 from the nitrogen atmosphere within the glovebox. The four exhaust lines (one per furnace) will combine
17 inside the glovebox and pass through a ceramic HEPA-grade filter. The temperature of the offgas at that
18 point should be about 150°C. The ceramic filter is designed for 980°C. After the filter, the offgas would
19 leave the glovebox and enter the glovebox exhaust header. In the header, the offgas will mix with the
20 approximately 300 cfm of glovebox exhaust (maximum temperature of 40°C) before reaching the two
21 testable stages of HEPA filters. The 4 cfm of furnace exhaust will not significantly raise the temperature
22 of the exhaust stream entering the process exhaust HEPA filter system.

23
24 In-line monitoring equipment will be provided for determining the moisture/volatile content of the
25 material processed. An alternative method exists to use thermogravimetric mass spectrometer analysis.
26 The material will be considered thermally stabilized when there is less than 0.5 percent loss on ignition
27 (LOI). Representative samples of 100 percent of the packages will be subjected to LOI testing (glovebox
28 GB-642D). The LOI process will verify dryness and suitability for packaging. The LOI process will heat
29 a sample taken from each material batch. The sample will be weighed and placed into a muffle furnace
30 where the sample will undergo a heating cycle similar to the stabilization process. After the heating cycle
31 is completed, the sample will be weighed again and the beginning and ending weights compared. If the
32 beginning and ending weights differ by less than 0.5 percent, the material will go to final packaging in the
33 BTS Module. If the difference in weights is greater than 0.5 percent, the material will be sent back
34 through the stabilization process until the LOI result is less than 0.5 percent. The furnaces will be heated
35 electrically. The purge gas in the LOI furnace will be 1 cfm of air. The temperature of the offgas will be
36 approximately 1,150°C to 1,200°C. The offgas will be discharged directly to the glovebox where the
37 offgas will mix with the approximately 13°C nitrogen atmosphere in the LOI glovebox.

38
39 Another alternative method (to LOI) for determining the moisture content of the processed material has
40 been approved by DOE and may be implemented in the future. The method, supercritical fluid extraction
41 (SFE), involves placing representative samples of stabilized material from a batch into porous sample
42 cells in a glovebox. Supercritical carbon dioxide (CO₂, approximately 100°C and 3,000 psi) is passed
43 through the sample, solubilizing water in the sample. The solubilized water is carried in the fluid stream
44 to a spectrometer for water detection and quantification. The measured fluid stream, composed of CO₂
45 and water, is released into the glovebox atmosphere. In normal operation, it is expected the SFE effluent
46 would be only CO₂.

47
48 All stabilized plutonium-bearing materials will be containerized in an inner-welded and outer-welded
49 container (i.e., the aforementioned 3013 package) and placed in secure vault storage (2736-Z Building)
50 pending final disposition. No modification to the existing 2736-Z Building ventilation system, which
51 exhausts through the 296-Z-6 stack, will be made.

1 **7.0 ANNUAL POSSESSION QUANTITY AND PHYSICAL FORM**
2 (**REQUIREMENTS 8, 10, AND 11**)

3 The following discusses two annual possession quantities. One quantity is for those activities associated
4 with modifications during construction affecting the existing 2736-ZB Building ventilation system, and
5 subsequent continued emissions through the existing 296-Z-5 exhaust stack. The other quantity is for
6 those activities that will result in emissions through the proposed new stack (296-Z-7).

7 The annual possession quantity for modifications during construction activities affecting the existing
8 2736-ZB Building ventilation system (exhausting through the existing 296-Z-5 exhaust stack) was
9 estimated based on emissions from calendar year 1998. It was conservatively estimated that during
10 modifications, residual contamination on the ductwork might be knocked loose and be re-introduced to
11 the existing ventilation system. The total measured release from the 296-Z-5 exhaust stack for calendar
12 year 1998, as documented in DOE/RL-99-41 (*Radionuclide Air Emissions Report for the Hanford Site,*
13 *Calendar Year 1998*), was used as a basis for estimating the annual possession quantity. These data
14 indicate that total alpha was not detected and total beta was 1.2×10^7 curies. Therefore, for conservatism,
15 100 times the 1998 emissions from the 296-Z-5 stack (i.e., 1.2×10^5 curies) is the maximum annual
16 possession quantity anticipated during construction activities associated with Project W-460. The
17 existing ventilation and monitoring systems for the 2736-ZB Building will remain operational during
18 modification/construction activities. Alarms will be activated in the event of off-normal emissions. Work
19 would cease until the source/extent of the contamination could be assessed. Work control procedures will
20 be modified and implemented to ensure personnel and public safety before continuation of activities.

21 The annual possession quantity for operations (i.e., stabilization and packaging activities) resulting in
22 emissions through the new stack (296-Z-7) is based on a conservative estimate for the maximum amount
23 of material that could be stabilized and repackaged in a year. The annual possession quantity for
24 Project W-460 assumed an annual throughput of 1.6 metric tonnes plutonium (100 percent
25 plutonium-239), 1.1 metric tonnes uranium (100 percent uranium-233), and 0.01 metric tonnes americium
26 (100 percent americium-241).

27 The physical form of all radionuclides encountered during construction, stabilization, and packaging
28 activities would be expected to be dry particulates. The physical form of all radionuclides emitted is
29 expected to be particulate.

30 Potential radionuclides expected to be encountered during construction, stabilization, and packaging
31 activities include: uranium-235, uranium-238, plutonium-238, plutonium-239, plutonium-240,
32 plutonium-241, plutonium-242, americium-241, and americium-243.

33
34 **8.0 ABATEMENT TECHNOLOGY AND CONCEPTUAL DRAWING(S)**
35 (**REQUIREMENTS 6 AND 7**)

36 Excavation activities, as necessary to support Project W-460 (e.g., installation of the nitrogen system and
37 gas bottle storage pads and new stack) will be conducted in a similar fashion as those activities performed
38 for the 200 West Area Regional Drainfield (Project L-281, September 1998). For example (as discussed
39 in the "Notice of Construction for Guzzler Excavation and Backfilling Activities in Support of the L-281:
40 200 West Area Regional Drainfield," approved by WDOH, September 8, 1998), during excavation in
41 potentially contaminated areas, the soils will be surveyed over every linear and vertical foot before
42 excavation. Any hot specks detected will be removed and containerized before the excavation is allowed
43 to continue, unless located at the bottom of the excavation (these specks will be covered with clean fill).

1 For the purpose of this NOC the term hot speck and speck are used interchangeably. A hot speck/speck is
2 defined as a very small amount (i.e., a pebble, metal turning, etc.) of contamination reading up to
3 500,000 counts-per minute (cpm) beta-gamma and/or >50 cpm alpha. The contaminated soil will be (in
4 piles) from the "clean" soil during excavation. Contaminated soil will be controlled using water, fixatives
5 or covers.

6
7 If the radiological contamination is determined to be more extensive, the spread of contamination will be
8 controlled both during the excavation and during the backfilling of the soil. A minimal amount of water
9 could be applied using a hand-held sprayer to control the generation of dust (and contamination).

10
11 After the backfilling of the excavation where radiologically contaminated soil in the excavation has been
12 removed and used as backfill, the area will be surveyed to verify no radiological contamination is present
13 on the soil surface. If contamination is present on the surface, the soil either will be removed and
14 containerized or covered with clean fill material or fixatives to prevent the spread of the contamination.
15 The area will be radiologically posted, both during and after completion of the project.

16
17 Figure 2 shows the overall ventilation schematic emission sources, identifying safety systems, structures
18 and components. This figure depicts the general flow patterns for the various offgas systems.
19 Figures 3 through 7 contain general schematics of the proposed ventilation system modifications for the
20 2736-ZB Building.

21
22 For clarity, additional preliminary engineering drawings are provided in Attachment A. These drawings
23 detail the following:

24

- 25 • Location of filters on exhaust systems of gloveboxes and furnaces: Drawing # H-2-829445
- 26 • Location of high-temperature equipment: Drawings # H-2-829969 and H-2-829971; two furnace
27 sketches supporting Drawing H-2-829969
- 28 • Direction and path of airflow: Drawing # H-2-829442
- 29 • Location of sampling lines on new stack (296-Z-7): Drawing # H-2-829485
- 30 • Location of additional controls on ventilation system: Drawing # H-2-829443.

31
32 Emissions resulting from work performed within the 2736-ZB Building will be exhausted out the existing
33 296-Z-5 stack, which contains two stages of HEPA filtration with a minimum efficiency of 99.95 percent
34 for particles with a median diameter of 0.3 micron. The average flow rate in 1998 was reported to be
35 4.9 cubic meters per second (10,000 cubic feet per minute) (DOE/RL-99-41).

36
37 Project W-460 stabilization and packaging activities will be conducted predominantly in Rooms 642 and
38 641 (Figures 8 and 9). The resulting emissions will be exhausted through the new stack (296-Z-7) that
39 will contain two stages of HEPA filtration (credit taken for only one stage) with a minimum efficiency of
40 99.95 percent for particles with a median diameter of 0.3 micron. The maximum flowrate from the new
41 stack is projected to be 0.8 cubic meter per second (1,800 cubic feet per minute).

1

9.0 MONITORING SYSTEM (REQUIREMENT 9)

2 The existing 296-Z-5 stack exhausts filtered air from the 2736-ZB Building. Emission sampling consists
3 of a record sampler for particulate radionuclides. This stack is registered with WDOH, with emissions
4 estimated or verified using methods approved by the EPA and WDOH. Most recent data are reported in
5 DOE/RL-99-41.

6
7 The existing 296-Z-6 stack exhausts filtered air from the 2736-Z Building. Emission sampling consists of
8 a record sampler for particulate radionuclides. This stack is registered with WDOH, with emissions
9 estimated or verified using methods approved by the EPA and WDOH. Most recent data are reported in
10 DOE/RL-99-41.

11
12 The new stack (296-Z-7) will exhaust filtered air from stabilization and packaging activities conducted in
13 the 2736-ZB Building. The stack/emission sampling will consist of a continuous air monitor record
14 sampler for particulate radionuclides and flow monitor (Attachment A, Drawing # H-2-829485).

15

16

17 10.0 RELEASE RATES (REQUIREMENTS 12 AND 13)

18 As discussed in Section 7.0, the annual possession quantity for construction activities modifying the
19 2736-ZB Building resulting in continued emissions from the existing ventilation system was estimated.
20 Activities associated with modifications to the existing 2736-ZB Building ventilation system will not
21 introduce new material. It was conservatively estimated that during modifications, residual contamination
22 on the ductwork might be knocked loose and re-introduced to the existing ventilation system. The total
23 measured release from the 296-Z-5 exhaust stack for calendar year 1998 was provided as a basis for a
24 conservative estimate of this annual possession quantity. For conservatism, it is assumed that the annual
25 possession quantity for this activity will not exceed 100 times the annual emissions from the existing
26 2736-ZB 296-Z-5 exhaust stack, as documented in DOE/RL-99-41. That is, total alpha was not detected,
27 and total beta was 1.2×10^{-7} curies; therefore, the annual possession quantity is estimated to be 1.2×10^{-5}
28 curies. The 296-Z-5 stack exhaust will be operated in a continuous mode.

29
30 The annual possession quantity for stabilization and packaging activities (with potential emissions
31 through the new 296-Z-7 stack) was multiplied by the conservative release factor of 1.0 E-03 for
32 particulates and solutions (40 CFR 61, Appendix D). Although the furnace(s) will operate at
33 temperatures near 1,000°C, the boiling point temperature of plutonium, americium, and uranium oxides
34 and metals is well above 1,000°C. At high temperatures, the oxides will undergo transformation to a
35 pure metal state. For americium, plutonium, and uranium oxides, the transformation begins to occur at a
36 temperature of approximately 1,150°C, 1,500°C, and 2,800°C, respectively. The boiling point of metallic
37 americium, plutonium and uranium occurs at 2,607°C, 3,232°C, and 3,818°C, respectively. The release
38 rate conservatively assumes that all material is stabilized and repackaged. Pure metals only will be
39 repackaged. The new stack will be operated in a continuous mode.

40

41

42 11.0 OFFSITE IMPACT (REQUIREMENTS 14 AND 15)

43 Summary and synopsis outputs from CAP88-PC are provided in Attachment B.

44

45

1 **12.0 COST FACTORS AND FACILITY LIFETIME (REQUIREMENTS 16 AND 17)**

2 Requirement 16 is not applicable because a best available radionuclide control technology (BARCT)
3 demonstration is provided (Attachment C).

4
5 The maximum design life of the project is approximately 11 years (completion on or before
6 October 1, 2010).

7
8 **13.0 TECHNOLOGY STANDARDS (REQUIREMENT 18)**

9
10 The 296-Z-5 and 296-Z-6 stacks are registered emissions units with WDOH. The design and operation of
11 these stacks will not be modified to support Project W-460.

12
13 The new stack (296-Z-7) will be a registered emissions unit with WDOH. The new stack will meet
14 control technology standards listed in WAC 246-247-110(18). Drawing H-2-829485 (Attachment A)
15 pertains to the installation design of the new stack monitoring system. The stack monitoring system will
16 use two shrouded probes located in the exhaust stream within the stack at an elevation of approximately
17 25 feet abovegrade. The location of the probes was selected to ensure good mixing before sampling, to
18 provide accessibility for maintenance, and to stay away from the top of the stack to minimize wind
19 effects. Each probe will have a separate sample line to deliver a sample stream to stack monitoring
20 equipment located at the base of the stack. One sample line will be dedicated to continuous alpha
21 monitoring and the other sample line will be dedicated to a record filter. The sample flow will be
22 maintained proportional to the stack flow. A stack mass flow sensor will be located near the location of
23 the shrouded probes. Inspection and test ports will be provided.

24
25 Fail and radiation high alarms will be provided on the alpha monitor. Alarm signals will be tied into an
26 existing annunciator panel and will be used to notify operators of any off-normal conditions requiring
27 immediate corrective action. Sample pumps located downstream of the alpha monitor and record sampler
28 in the pump skid will draw representative samples from the stack stream. Exhaust from the pumps will be
29 returned to the stack above the sample location.

30
31 Qualification of the stack monitoring system will follow the standard ANSI/HPS N13.1. Acceptance
32 criteria are summarized in Table 4 of that standard. Testing will be conducted on a scale model of the
33 stack to characterize the mixing and flow patterns of the stack configuration. Additional testing on the
34 completed stack will be conducted to validate the system to ensure representative sampling at the location
35 of the shrouded probes. Depositional losses in the sample lines will be evaluated using the Deposition 4.0
36 computer code as described in Paragraph 6.4.1 of ANSI/HPS N13.1. Results of qualification testing and
37 analysis will be documented and issued in a final report.

1

14.0 REFERENCES

2 AIR 92-107, letter, A.W. Conklin, Washington State Department of Health, to J.D. Bauer,
3 U.S. Department of Energy, Richland Operations Office, no subject, October 5, 1992.
4

5 ANSI/HPS N13.1-1999, *Sampling and Monitoring Releases of Airborne Radioactive Substances from the*
6 *Stacks and Ducts of Nuclear Facilities*, American National Standards Institute, Inc.
7

8 DOE-STD-3013-96 and/or latest revision, *Criteria for Safe Storage of Plutonium Metals and Oxides*.
9

10 DOE/RL-99-41, *Radionuclide Air Emissions Report for the Hanford Site Calendar Year 1998*, June 1999,
11 U.S. Department of Energy, Richland Operations Office, Richland, Washington.
12

13 HNF-SD-CP-SAR-021, Rev 1, *Plutonium Finishing Plant Final Safety Analysis Report*, Fluor Daniel
14 Hanford, Richland, Washington..
15

16 HNF-SD-W460-CDR-001, Rev. 1, *Conceptual Design Report – Plutonium Stabilization and Handling*,
17 *Project W-460*, Fluor Daniel Hanford, Richland, Washington.
18

19 HNF-SD-W460-FDC-001, Rev. 1, *Functional Design Criteria - Plutonium Stabilization and Handling*
20 *(PuSH) Project W-460*, Fluor Daniel Hanford, Richland, Washington.
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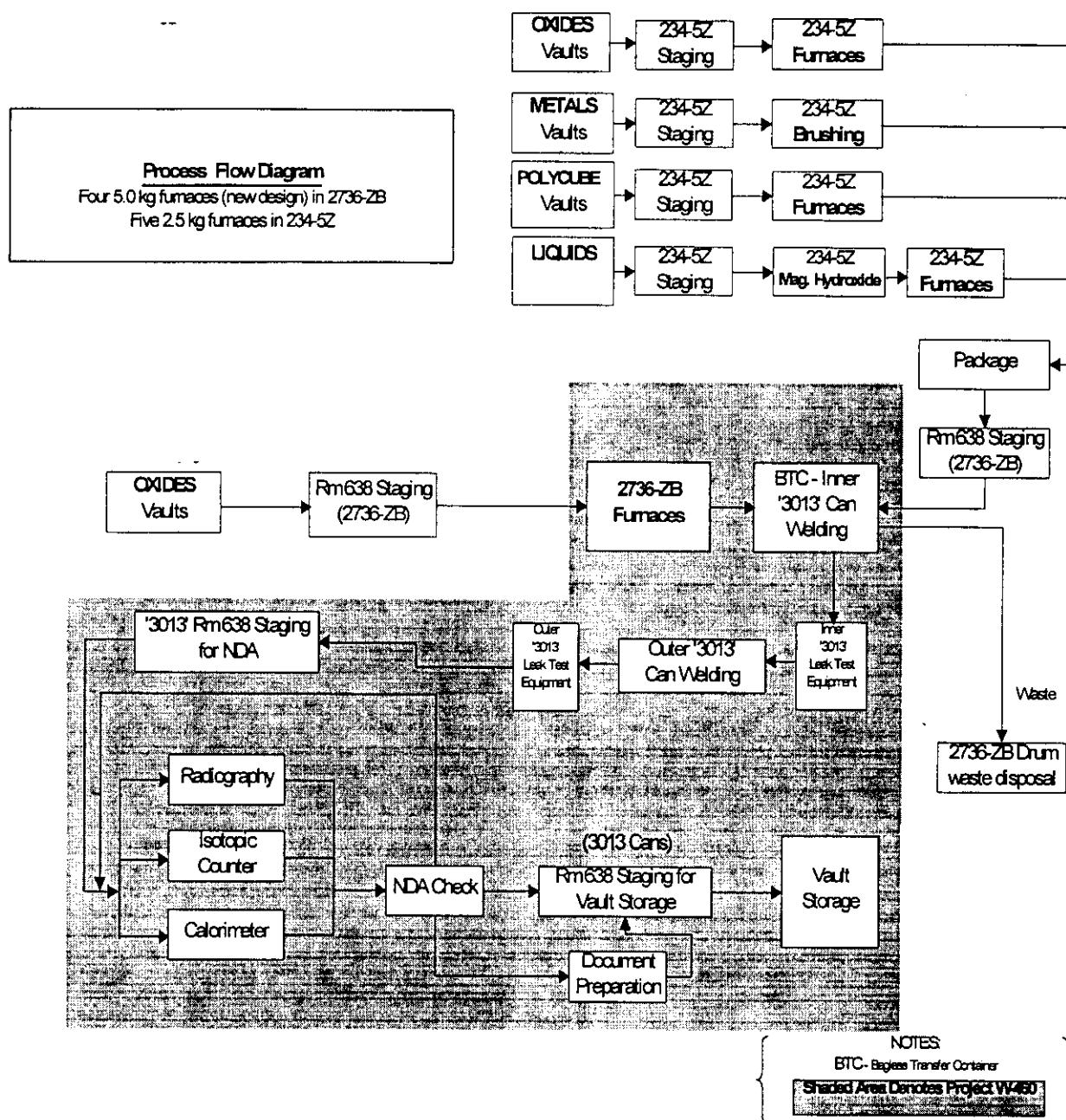


Figure 1. Process Flow Diagram.

Ventilation Schematic and Identification of Safety Systems, Structures, and Components.

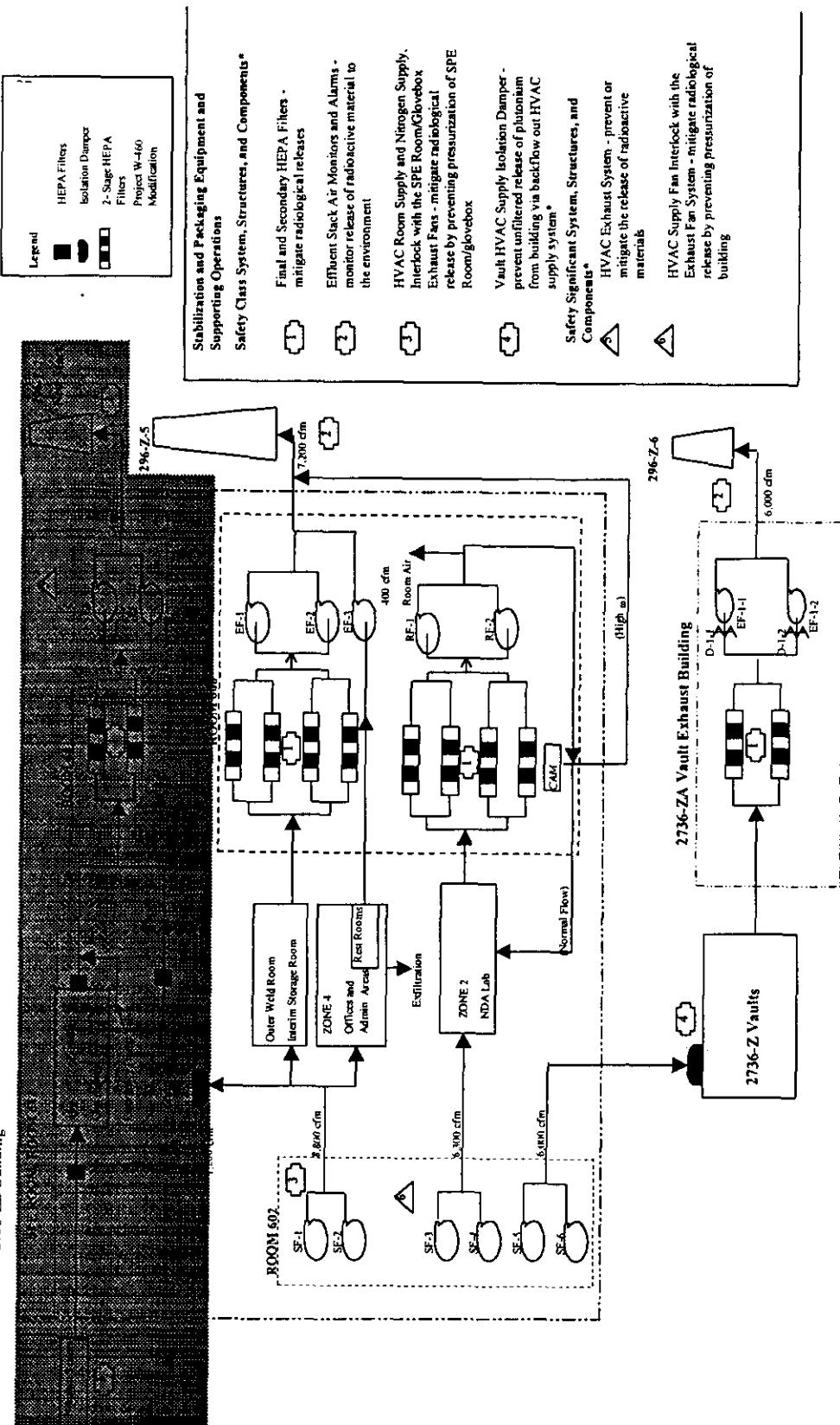


Figure 2. Ventilation Schematic for Project W-460.

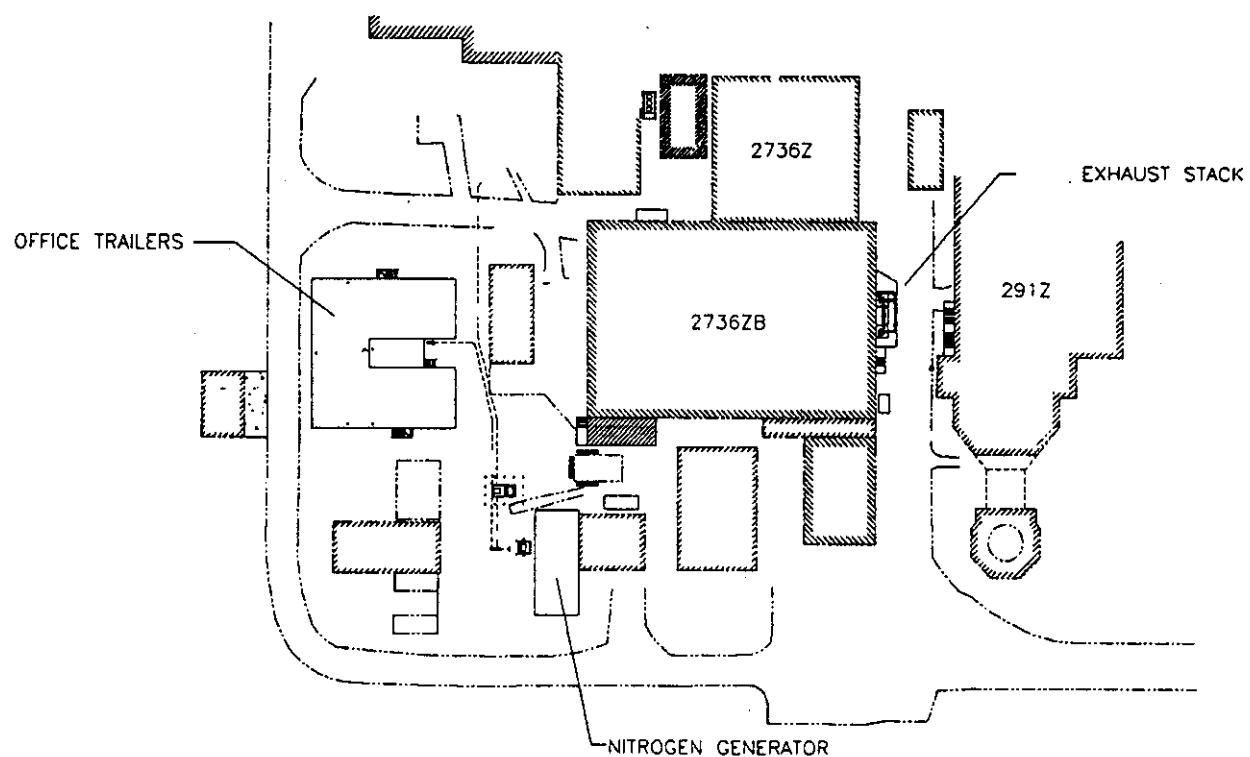


Figure 3. Site Plan.

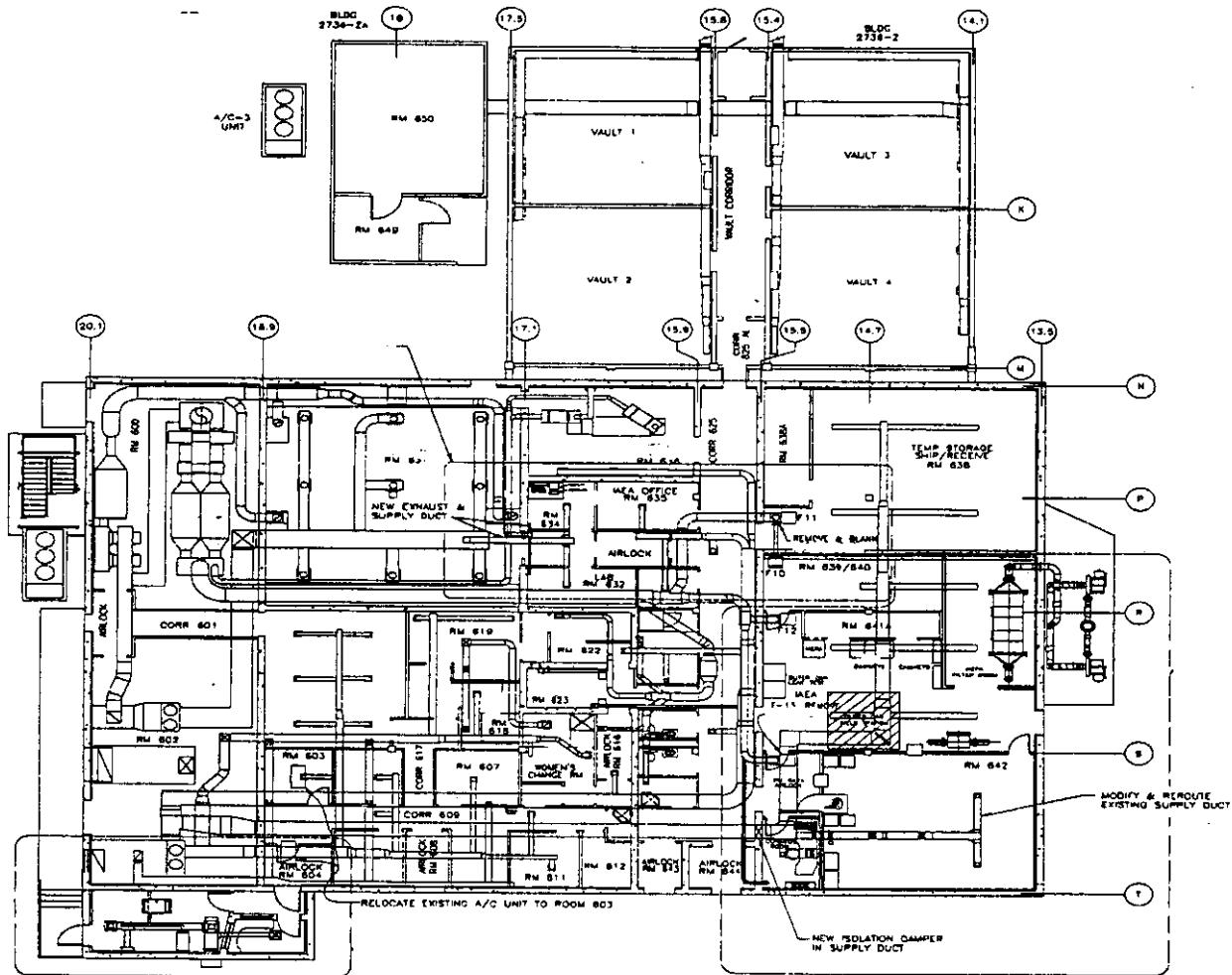


Figure 4. 2736-ZB Floor Plan.

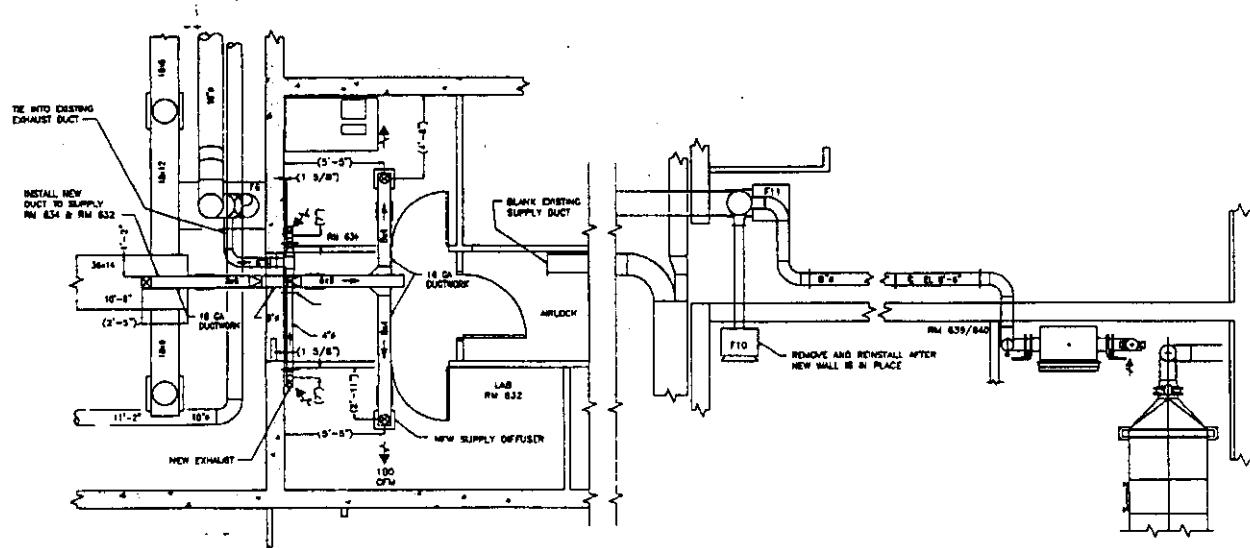


Figure 5. 2736-ZB Building Existing Ventilation System Modifications.

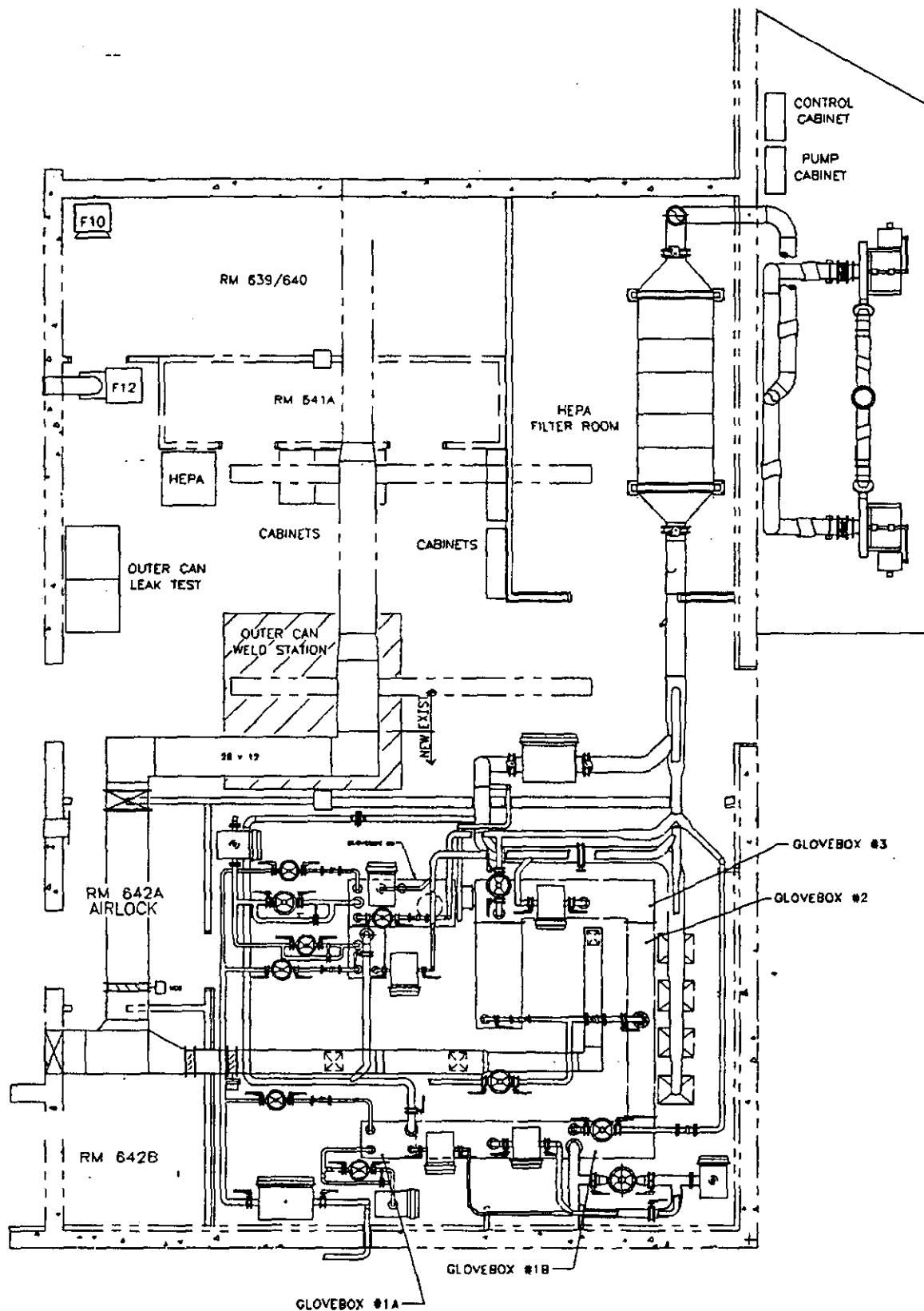


Figure 6. 2736-ZB Building New Process Ventilation System.

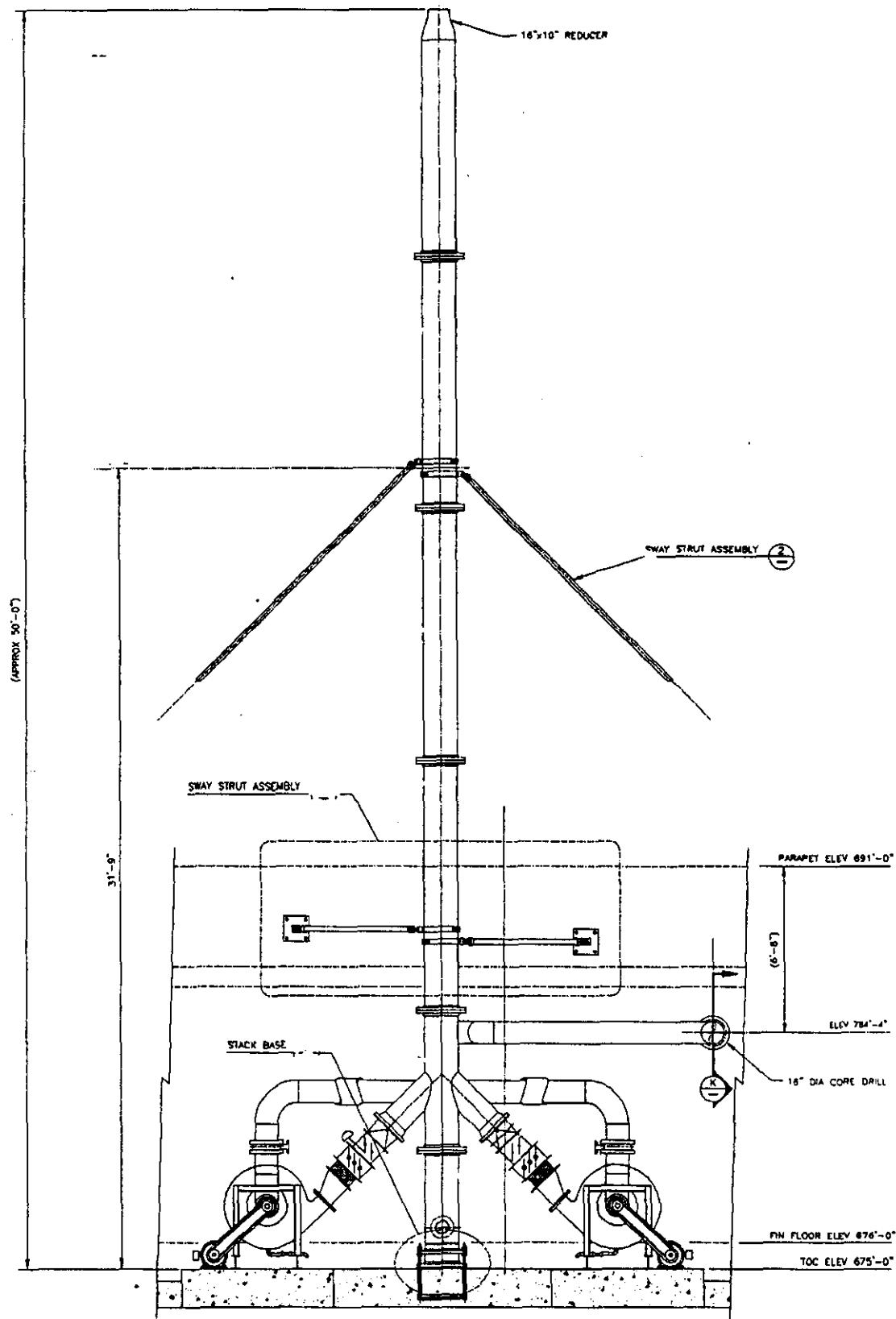


Figure 7. 2736-ZB Building New Process Exhaust System Stack.

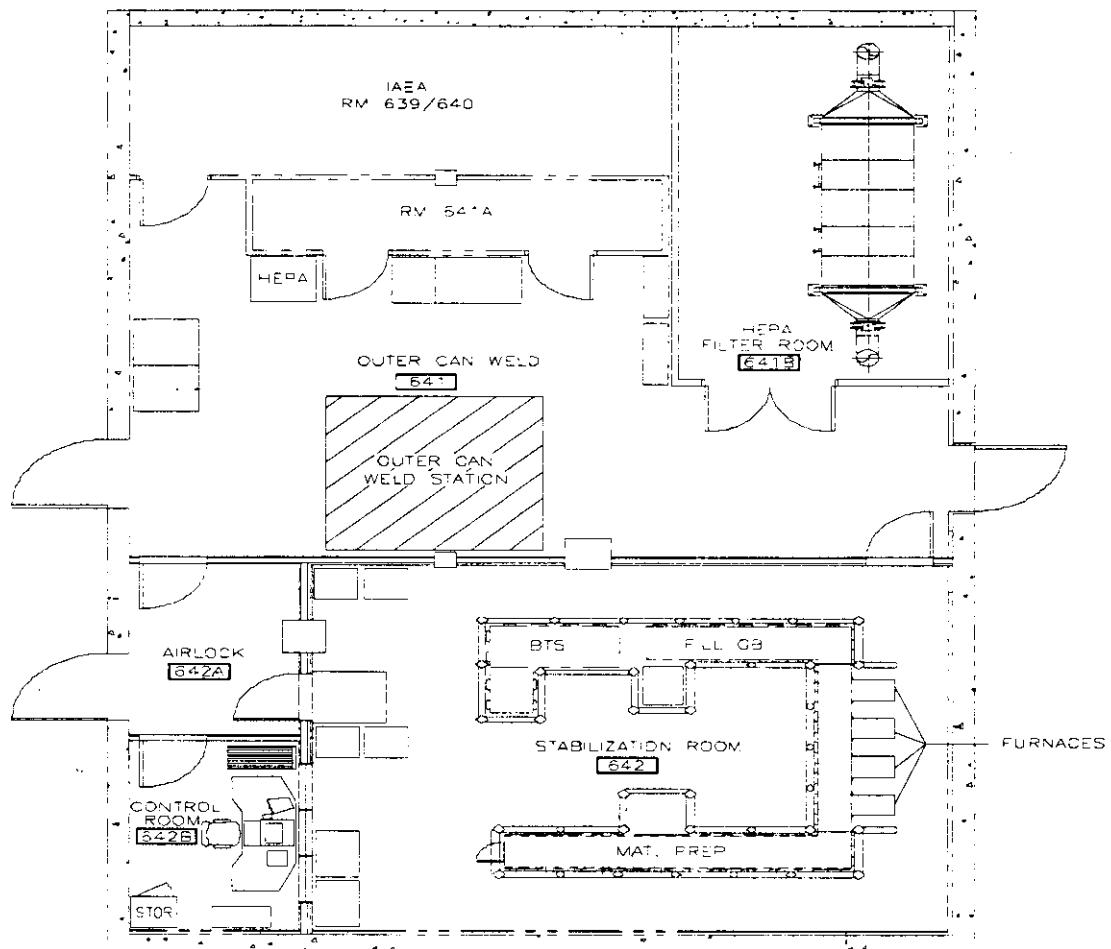


Figure 8. 2736-ZB Building, Rooms 641/642 Layout.

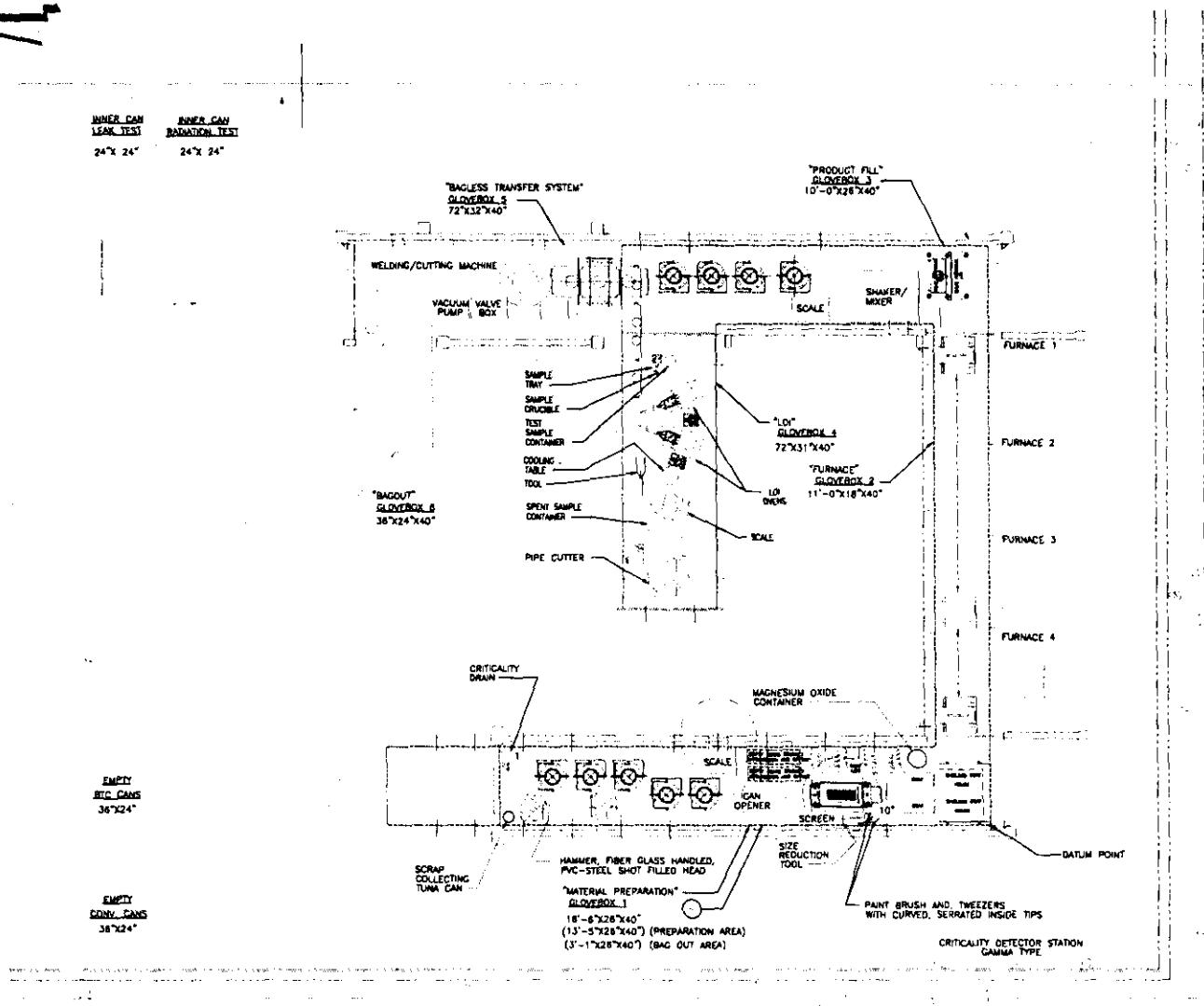


Figure 9. 2736-ZB Building, Room 642 Glovebox Layout.

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4 **ATTACHMENT A**
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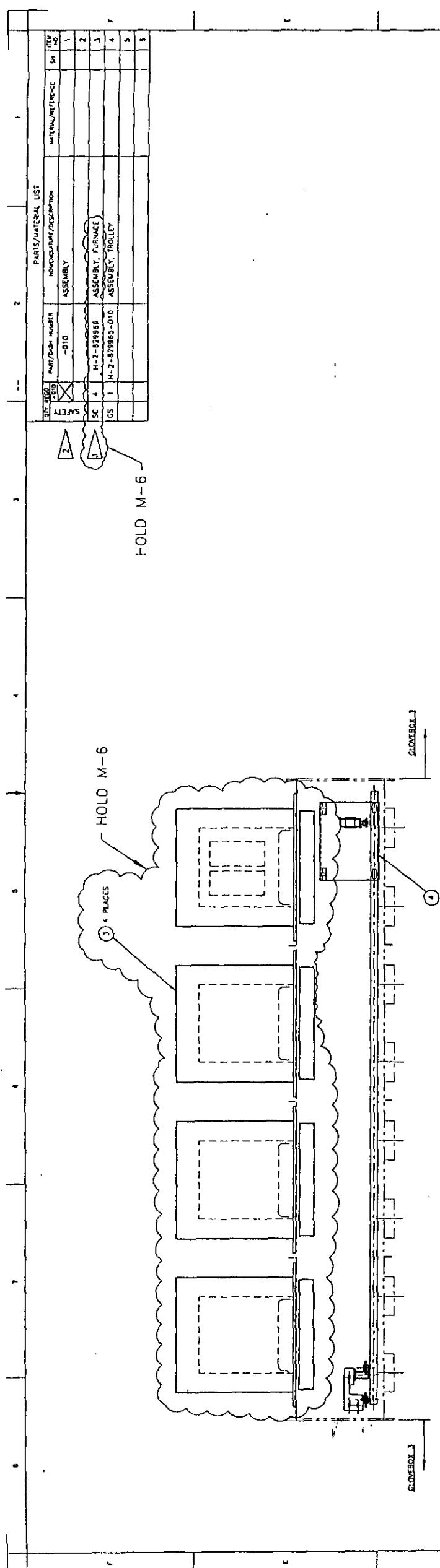
7 **PRELIMINARY ENGINEERING DRAWINGS**
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- Location of filters on exhaust systems of gloveboxes and furnaces: Drawing # H-2-829445 (sheet 1 of 5).
- Location of high-temperature equipment: Drawings # H-2-829969 (sheet 1 of 1) and H-2-829971 (sheet 1 of 1); 2 furnace sketches supporting Drawing H-2-829969.
- Direction and path of airflow: Drawing # H-2-829442 (sheet 1 of 1).
- Location of sampling lines on new stack (296-Z-7): Drawing # H-2-829485 (sheets 1 and 2 of 2).
- Location of additional controls on ventilation system: Drawing # H-2-829443 (sheets 1 – 9 of 9).

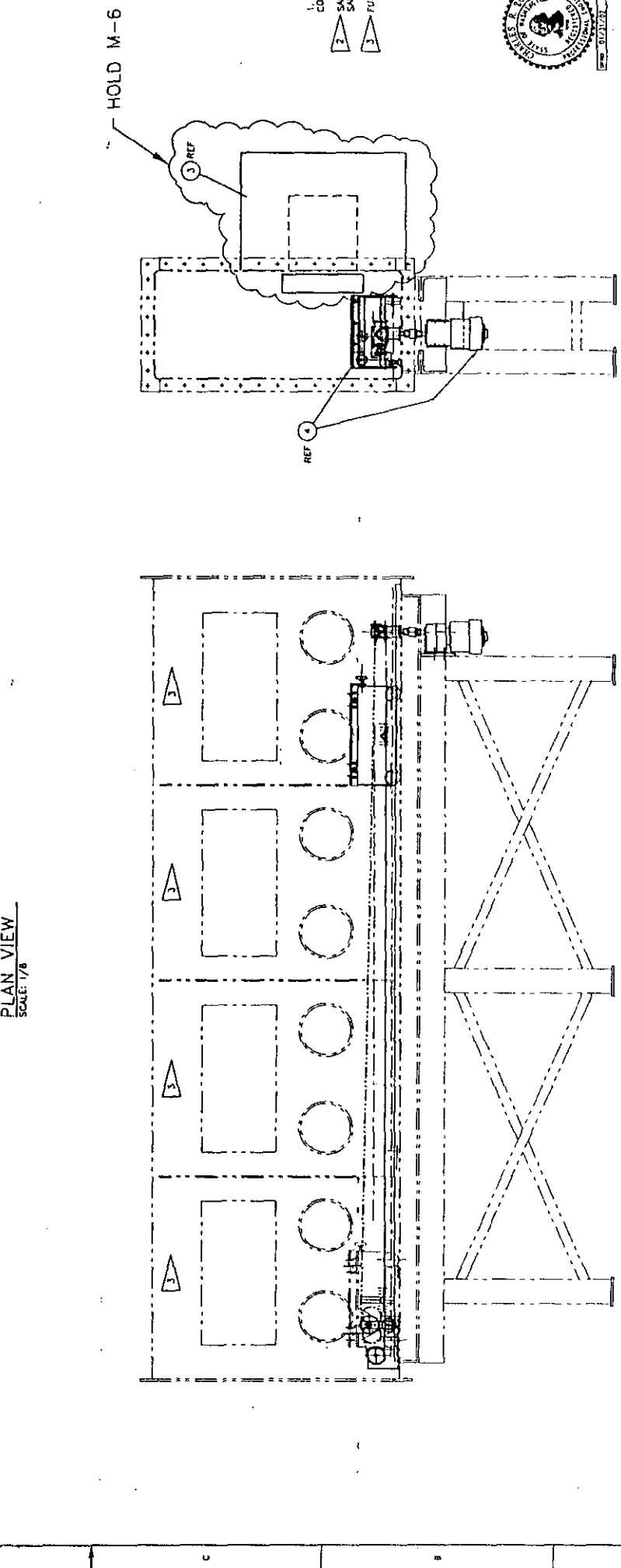
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PLAN VIEW
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REF ID: H-2-829969

FLUOR FEDERAL SERVICES

PROJECT: W-460, PLUTONIUM STABILIZATION AND HANDLING

U.S. DEPARTMENT OF ENERGY

MECHANICAL EQUIPMENT ASSEMBLY

GLOVEBOX GB-642B

EQUIPMENT ASSEMBLY

NAME	U.S. DEPARTMENT OF ENERGY
ADDRESS	MECHANICAL EQUIPMENT ASSEMBLY
PHONE	GLOVEBOX GB-642B
FAX	EQUIPMENT ASSEMBLY
EMAIL	

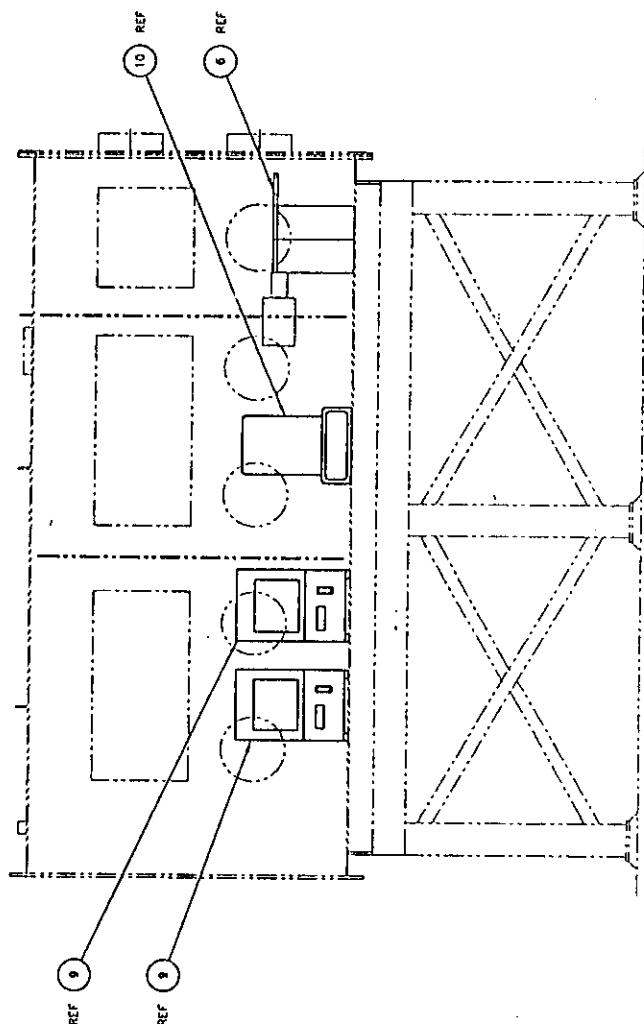
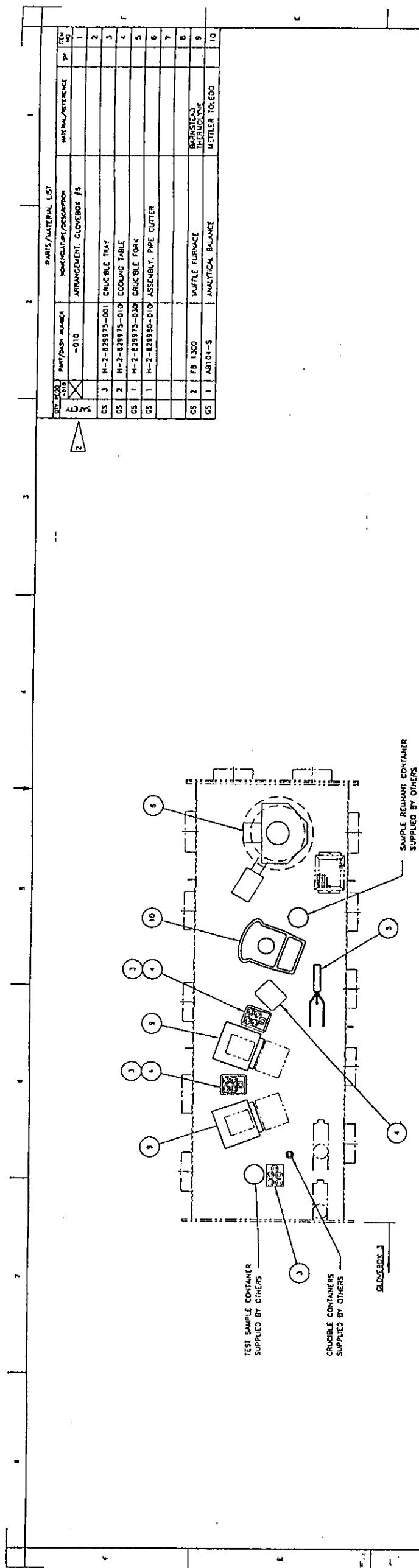
1 ASSEMBLY
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REF 1 PLACES

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DOC NO. TITLE
DRAWING TRACEABILITY LIST
REVISIONS
SCALE: 1/8

ATT A-2



PRELIMINARY

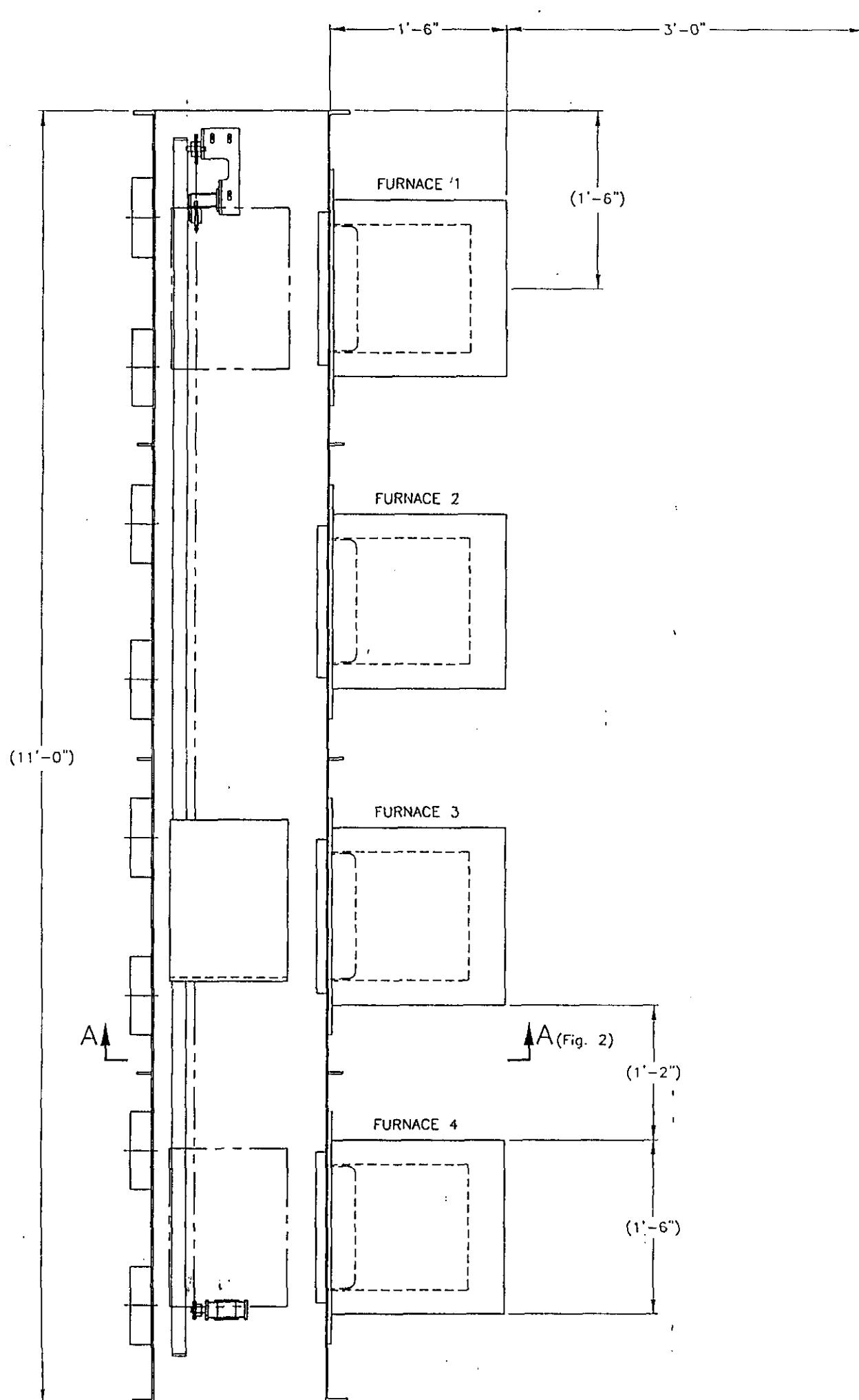
GENERAL NOTES

1. EQUIPMENT INSTALLATION SHALL BE IN ACCORDANCE WITH CONSTRUCTION SPECIFICATION W-400-C2, SECTION 13035.
2. SAFETY CLASSIFIED ITEMS ARE IDENTIFIED AS FOLLOWS: SAFETY CLASS (SC), SAFETY SIGNIFICANT (SS), OR GENERAL SERVICE (GS).

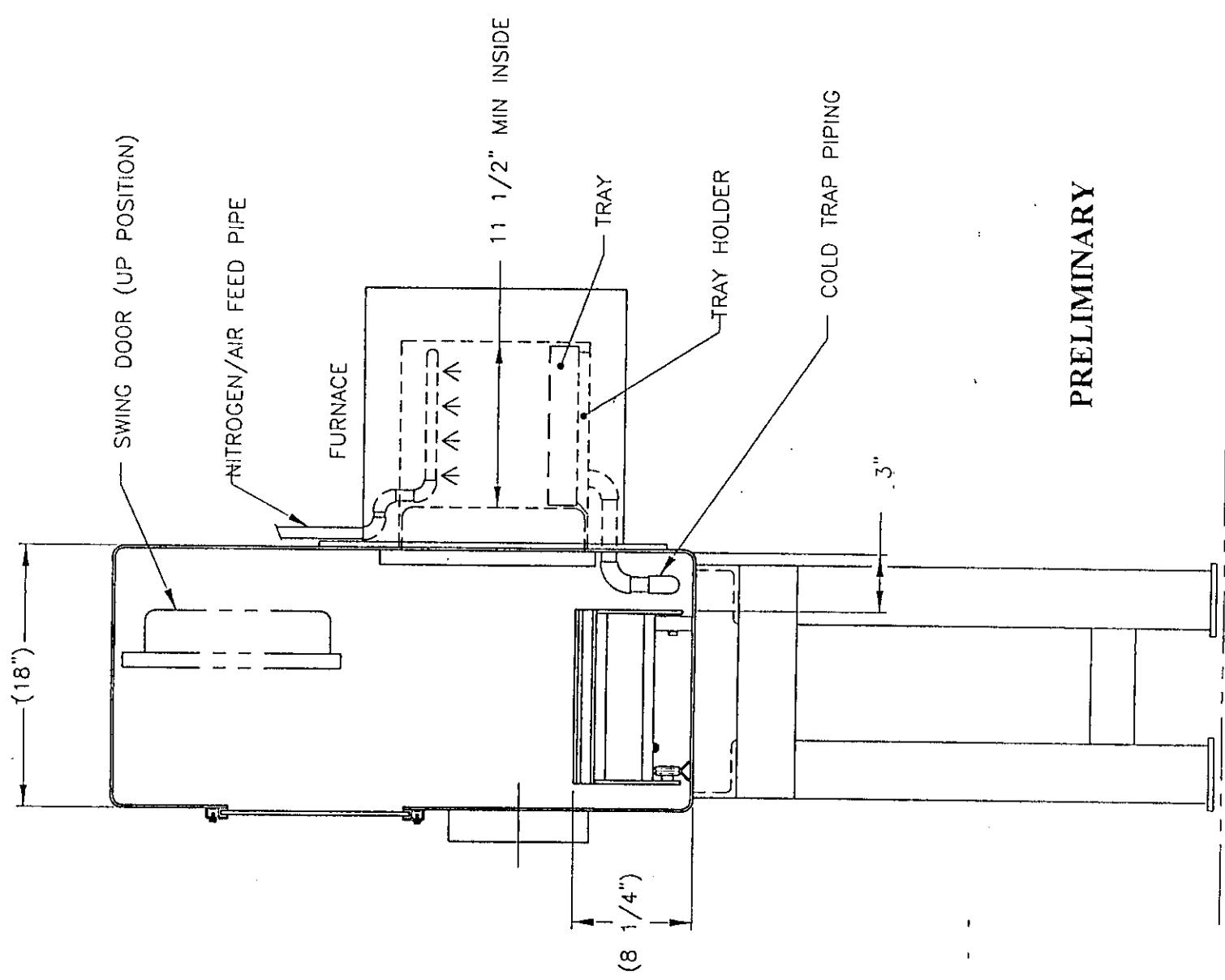
1 GLOVEBOX ASSEMBLY
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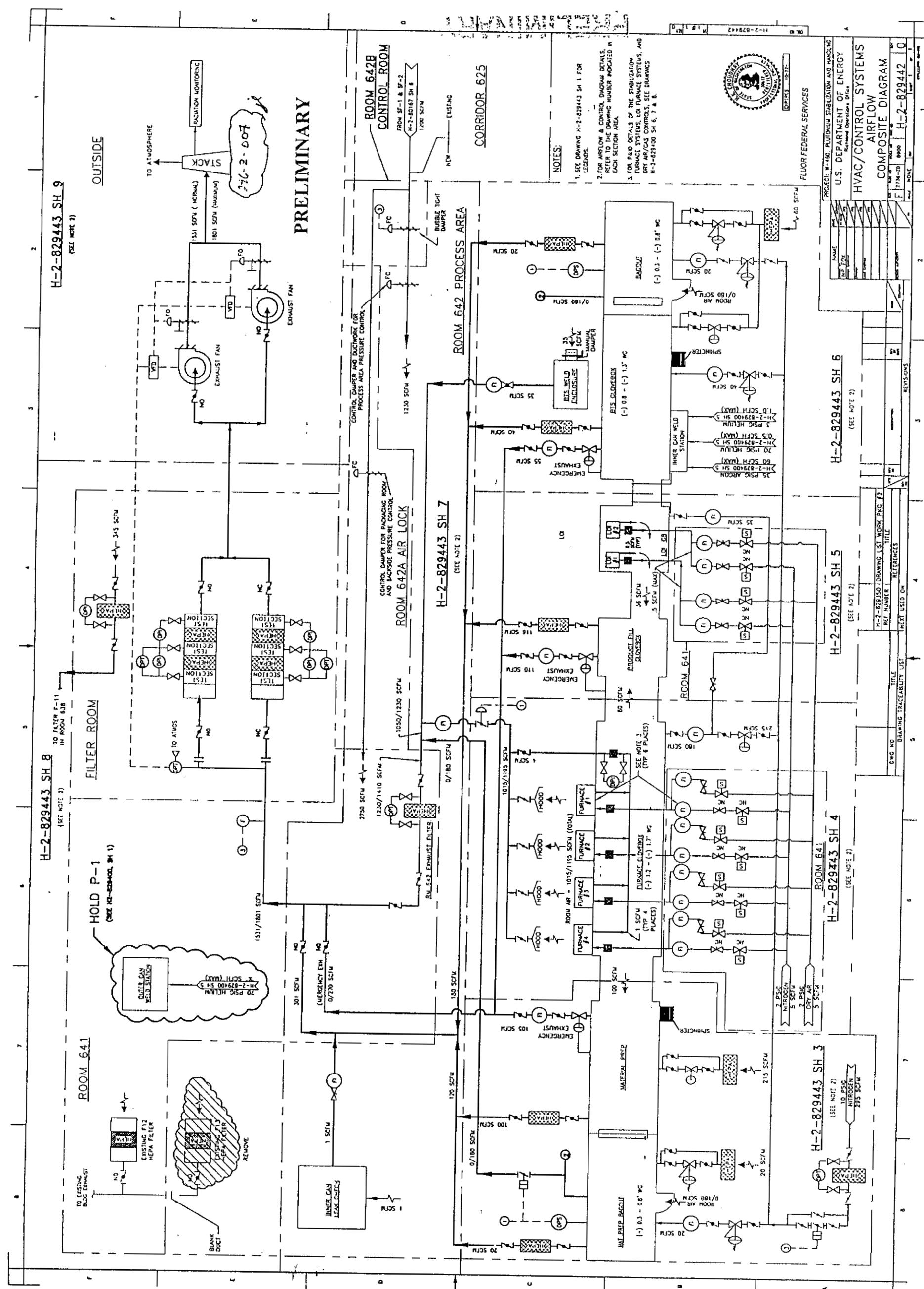


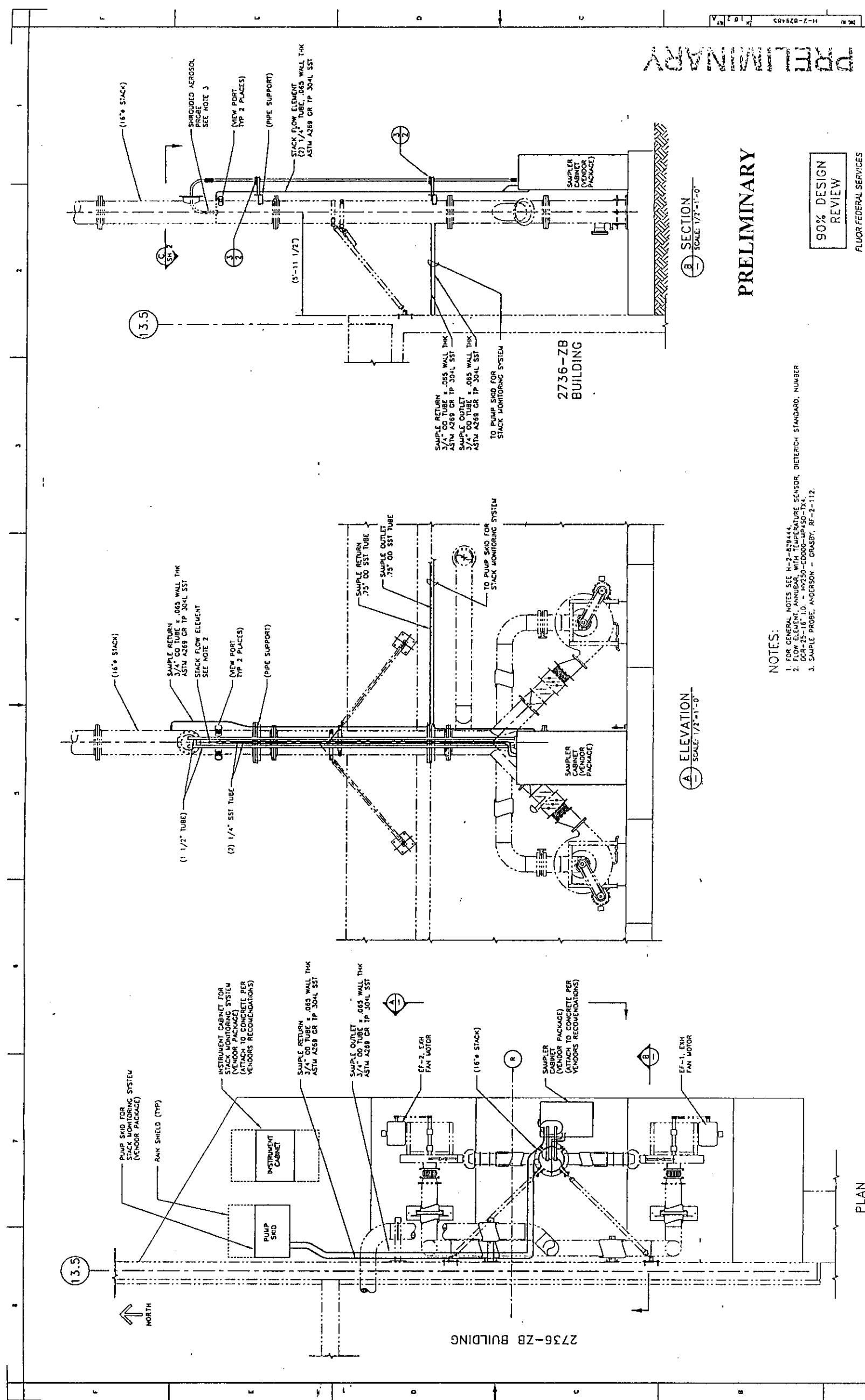
Furnace sketch (1 of 2) supporting Drawing H-2-829969.



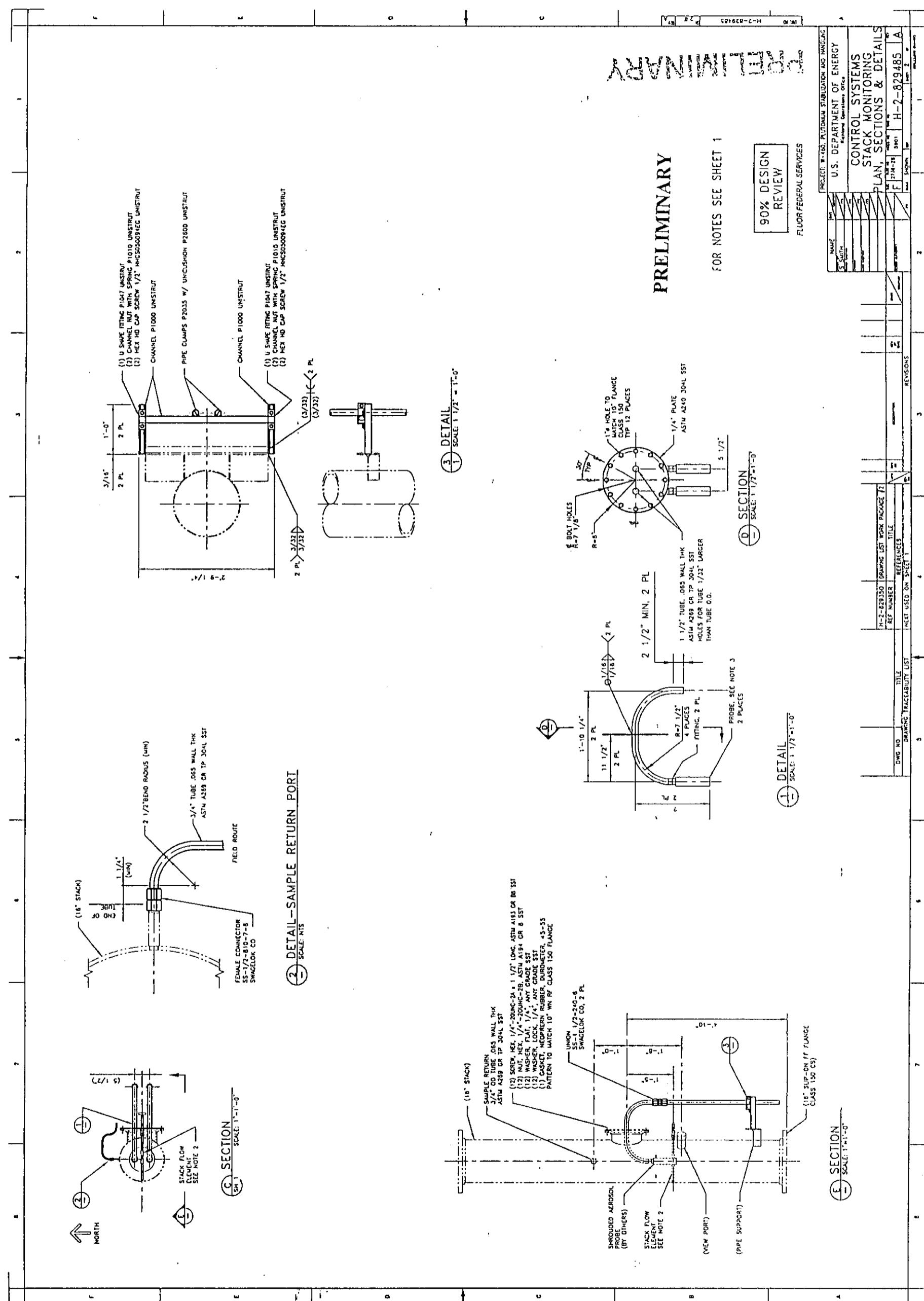
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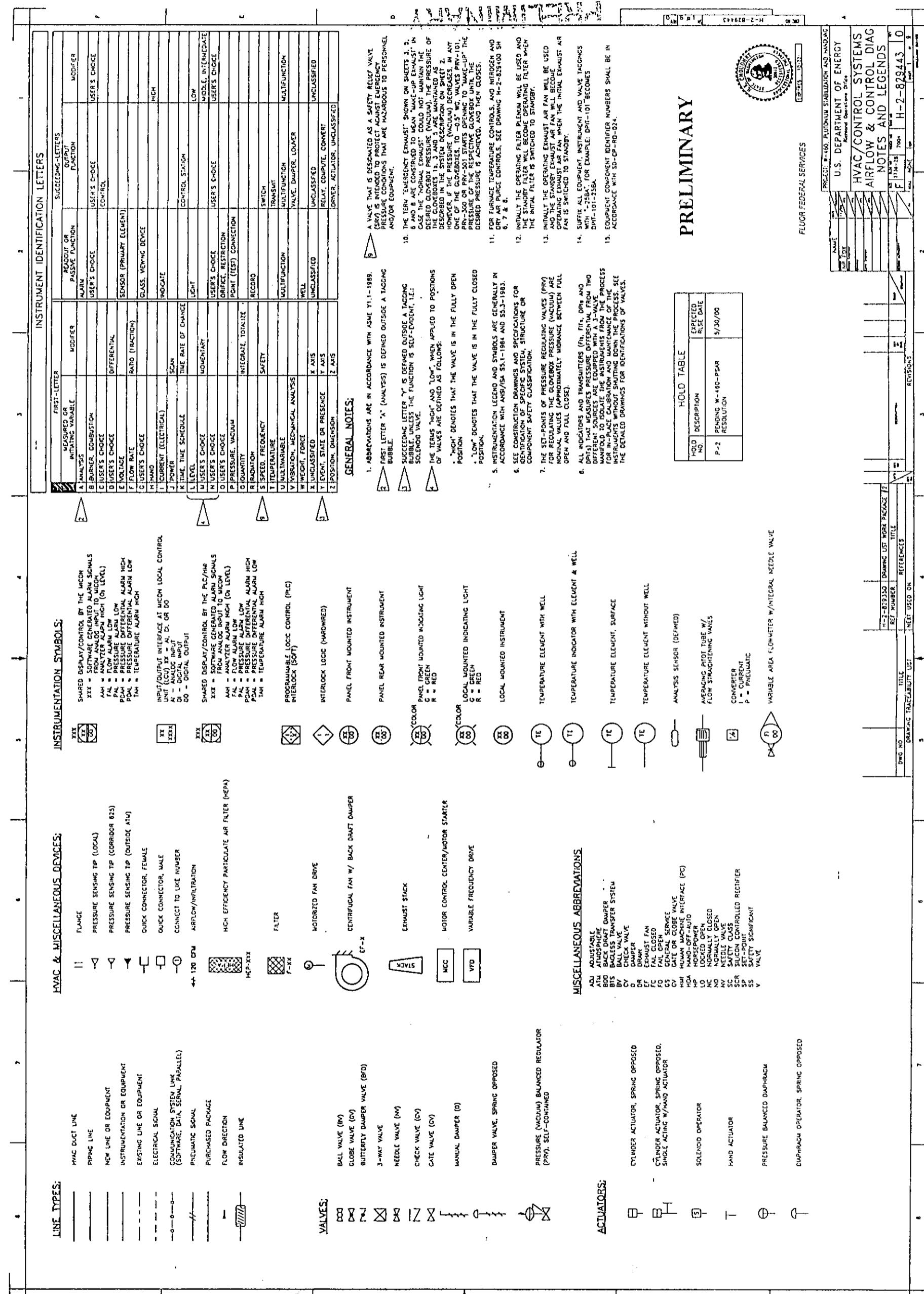
PRELIMINARY





PROJECT: W-160, PLUTONIUM STABILIZATION AND WASHING	
NAME	U.S. DEPARTMENT OF ENERGY
Instrument Control Systems	
SECTION	Stack Monitoring
PLAN & SECTIONS	





SYSTEM DESCRIPTION

111

COMPONENT/EQUIPMENT NO TABLE	
NUMBER	DESCRIPTION
1	GLOVEBOX CB-612A
2	GLOVEBOX CB-612B
3	GLOVEBOX CB-612C
4	GLOVEBOX CB-612D
5	GLOVEBOX CB-612E
6	GLOVEBOX CB-612F
7	BITS WELD ENCLOSURE INNER CAN LEAK CHECK
8	ROD 841 & 842
9	4000 TURB & THER PLENUM
10	EXHAUST FAN EF-1
11	EXHAUST FAN EF-2
12	EXHAUST FAN EF-3
13	EXHAUST FAN EF-4

DECREASES TO $\sim 0.5^\circ$ KC.
WHEN BOTH OF THE EXHAUST FANS ARE STOPPED OR WHEN ROOM 642 WHICH CONTAINS SAFETY FAN #1-11 IS BLOWN, AUTOMATIC INTERLOCK FOR LOW-LOW EXHAUST FLOW WITH A SETPOINT OF 1.05 IS PROVIDED AT THE EXHAUST FLOW METER. EP-1-25K IS AUTOMATICALLY SHUT OFF THE HYDROGEN SUPPLY TO CLOSERATES AND PAVS. HYDROGEN PLUTONIUM CONTAINMENT CLOSERATES 4420 (B4207), 4424 (W411 PREP), 6412 & 6415 ARE PROVIDED WITH VACUUM SAFETY RELIEF VALVES SPR-100, SPR-101, SPR-500, AND SPR-600 WHICH OPEN AT THE WATSONS NEGATIVE PRESSURE OF 1.5 WC. THE VACUUM SAFETY RELIEF VALVES HAVE SPRAY VALVES THAT CAN BE MANUALLY OPENED. THE EPASS WILL PRODUCE ROOM AIR TO THE CLOSERATES WHEN THE HYDROGEN GAS SYSTEM IS NO LONGER REQUIRED DURING SHUTDOWN. MAINTENANCE THE SPRAY VALVES ARE MONITORED WITH A LIMIT

ZONE HB [FIGURE 64-2] THE SUPPLY AIR TO ZONE 64-2 IS CONTROLLED BY A MODULATING DAMPER BY 900. FV-1000 IS HAD WIRED DIRECTLY TO THE EXHAUST FLOW SWITCHES FV-1000/PS-1000 AND FV-1000/PS-1000 WHICH MONITORS THE AIR FLOW IN THE PROCESS EXHAUST LINE. FV-1000 OPENS WHEN THERE IS A DEMAND, PROCESS FLOW IN THE EXHAUST LINE, AND THE SUPPLY AIR IS MADE TO REACH FV-1000 CLOSURE. AND THE SUPPLY AIR TO ROOM 64-2 IS TERMINATED WHEN FV-1000 CLOSURE. THE EXHAUST LINE, THE EXHAUST FLOW SWITCHES FV-1000/PS-1000, AND THE EXHAUST FLOW DIFFERENTIAL TRANSMITTER, DPT-1000, LOCATED IN ROOM 64-2, HAS A MEASURING SIGNAL TO THE LOCAL DPC-800 (IN ROOM 602) AND TO THE DPC-800 OUTPUT. ROOM 64-2 PRESSURE DIFFERENTIAL RELATIVE TO AIRSPACE IS MAINTAINED BETWEEN -0.20" WC AND -0.30" WC.

ZONE II
THE SUPPLY AIR TO ROOM 641 IS CONTROLLED BY A MODULATING DAMPER BY 801. THIS MODULATING DAMPER PIV. 901 IS CONTROLLED BY AN EXISTING CONTROLLER CPC-2. THE ROOM MANIFOLD PRESSURE DIFFERENTIAL RELAY TO ALARMER SET 501 IS CONNECTED TO THE ROOM MANIFOLD PRESSURE DIFFERENTIAL RELAY TO

INTERLOCK SCHEDULE

INTERLOCK	EVENT	ACTION	REF DWG/SH NO
	1 ↓	OPEN BLOWBOX 642A OR 642F DIFFERENTIAL PRESSURE DECREASES TO <0.1" WC	OPEN BLOWBOX EXHAUST DAMPER PCV-100, & PARTIALLY CLOSE FURNACE HOOD EXHAUST DAMPER PCV-200.
	2 ↓	HIGH DUTCH (LOSS OF NITROGEN) IN GLOVE BOXES 642A, 642C OR 642E	SHUT OFF ELECTRICAL POWER TO GLOVEBOXES IN ROOM 642, AFTER 5 WIGGLES TIME DELAY.
	3 ↓ (SS)	LOW-LOW EXHAUST FLOW (SS) (FLOW LESS THAN 250 SCFM)	CLOSE BUBBLE TIGHT DAMPER TV-1000 (SS) AND CLOSE NITROGEN SUPPLY VALVE PV-1001 (SS)
	4 ↓	HS-110 IS IN AUTO POSITION, HS-11A IS IN EF-1 POSITION, AND HS-1F1 IS IN RUN POSITION, - OR - EF-2 Shuts Down	START PRIMARY EXHAUST FAN EF-1, SHUT DOWN EXHAUST FAN EF-2, AND CLOSE DAMPER MV-1210
	5 ↓	HS-1210 IS IN AUTO POSITION, HS-1L1 IS IN EF-2 POSITION, AND HS-1F2 IS IN RUN POSITION, - OR - EF-1 Shuts Down	START STANDBY EXHAUST FAN EF-2, OPEN DAMPER MV-1210, SHUT DOWN EXHAUST FAN EF-1, AND CLOSE DAMPER MV-1110
	6 ↓	HS-EXH IN WICON POSITION - OR - LOSS OF WICON POWER	WICON CONTROLLER POC-EXH ENABLED, AND LOCAL CONTROLLER DPC-1101 DISABLED
	7 ↓	HS-EXH IN LOCAL POSITION - OR - LOSS OF WICON POWER	WICON CONTROLLER POC-EXH DISABLED, AND LOCAL CONTROLLER DPC-1101 ENABLED
	8 ↓	HS-642 IN WICON POSITION - OR - LOSS OF WICON POWER	WICON CONTROLLER POC-642 ENABLED, AND LOCAL CONTROLLER DPC-900 DISABLED
	9 ↓	HS-642 IN LOCAL POSITION - OR - LOSS OF WICON POWER	WICON CONTROLLER POC-642 DISABLED, AND LOCAL CONTROLLER DPC-900 ENABLED
	10 ↓	ROCK 642 FIRE/SECURITY ALARM AV-200	CLOSE ROOM FILTER EXHAUST ISOLATION VALVE

OFF-PAGE CONNECTION IDENTIFICATION

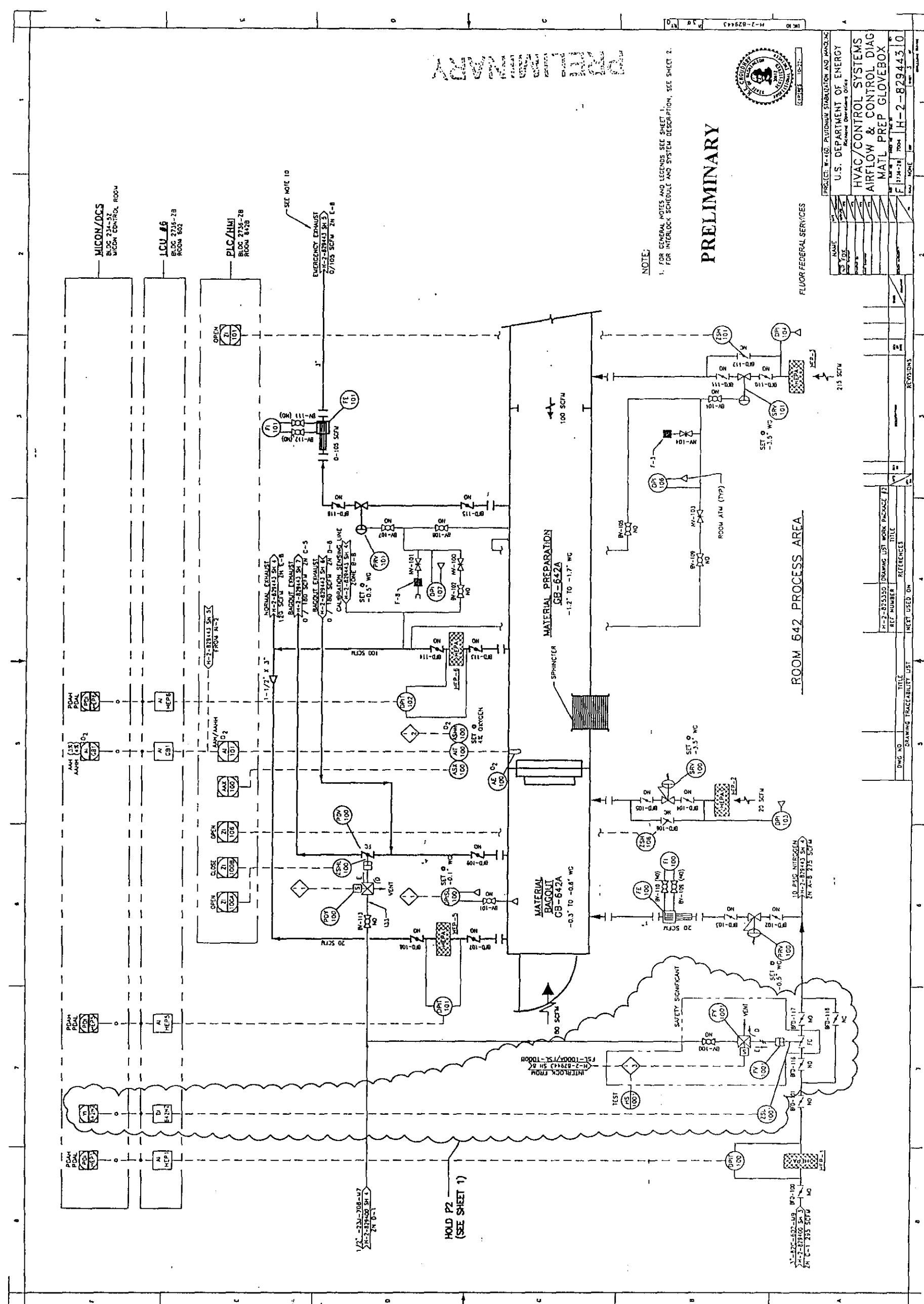
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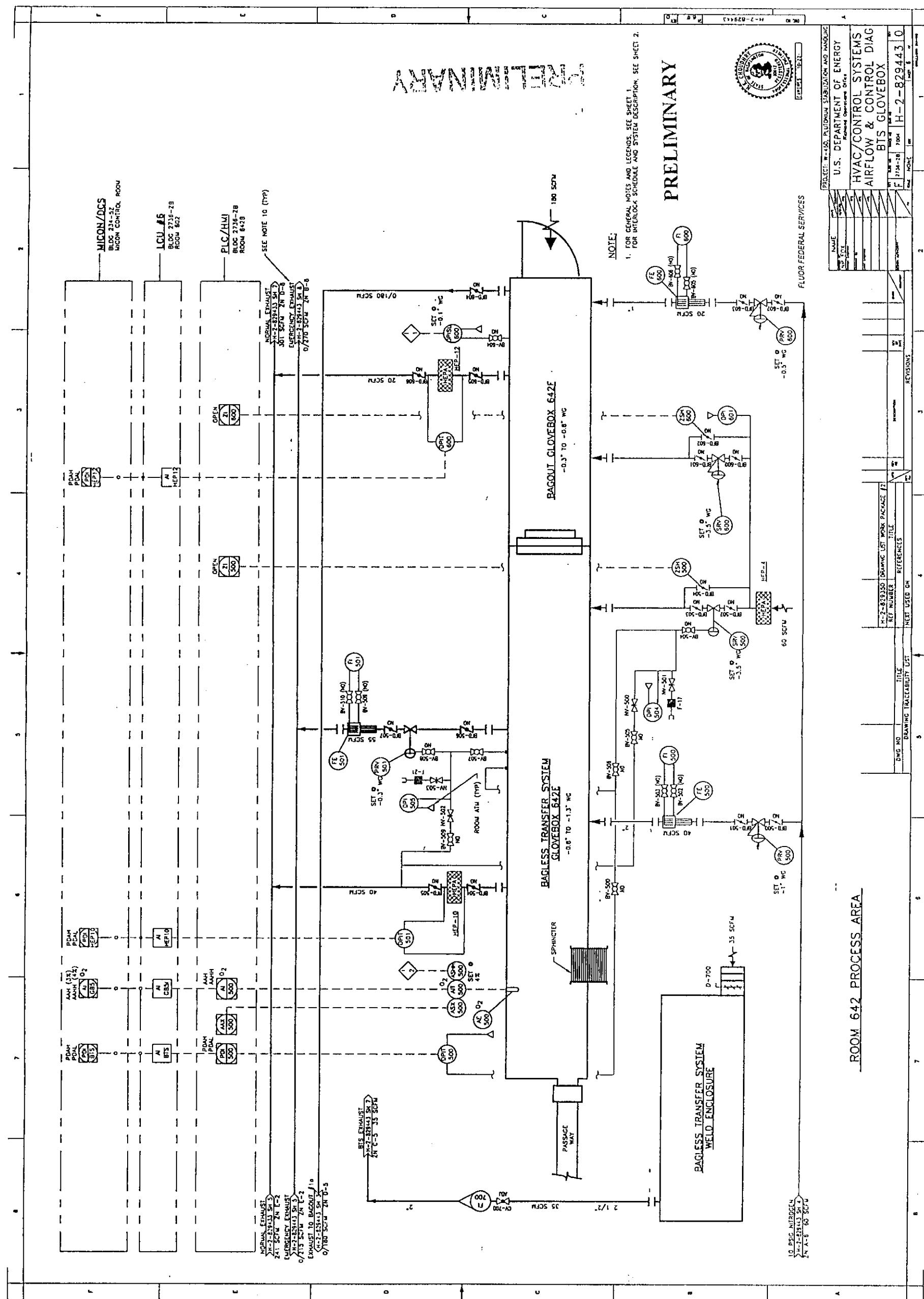
1. FOR GENERAL NOTES AND LEGENDS. SEE SHEET 1.

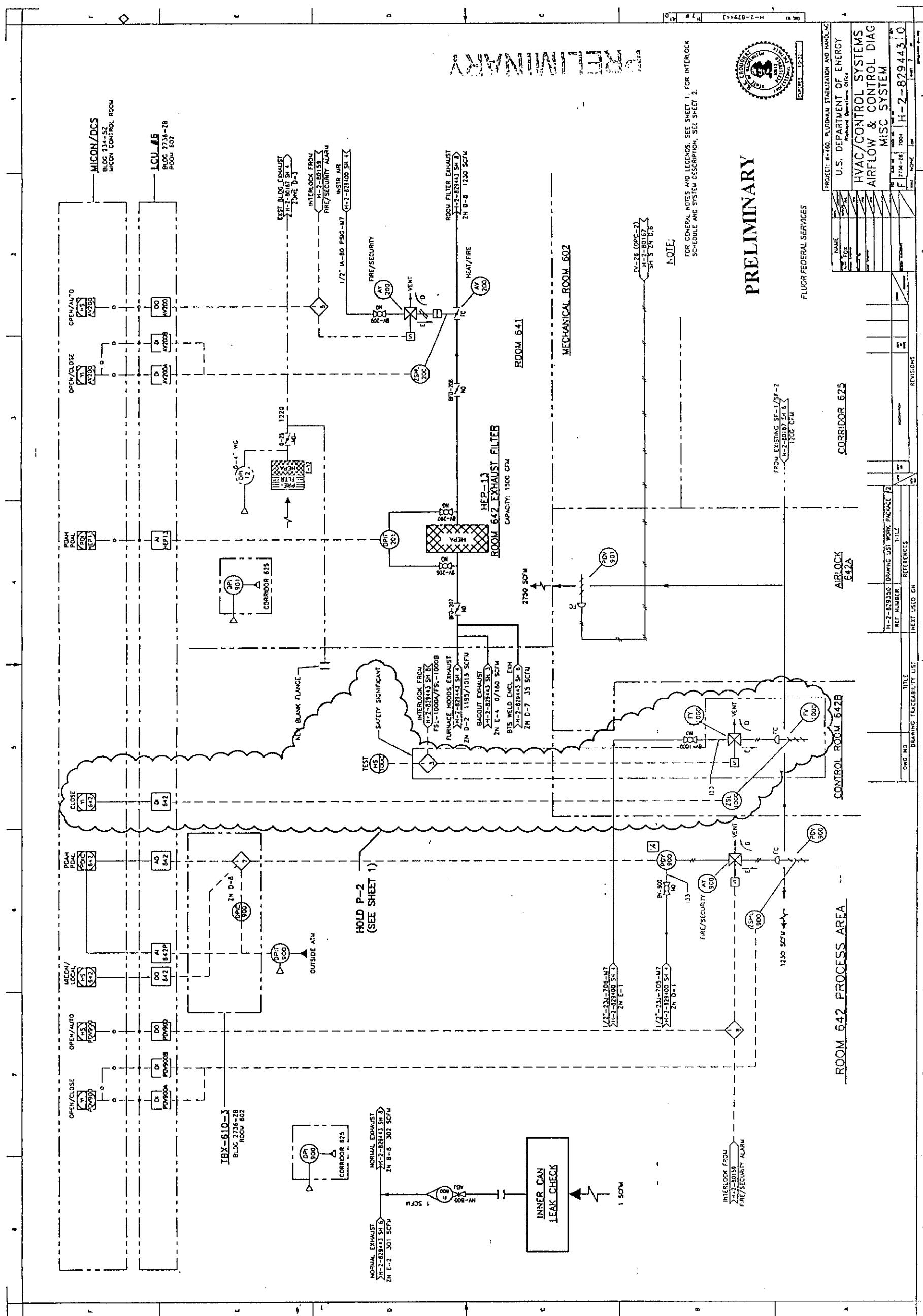


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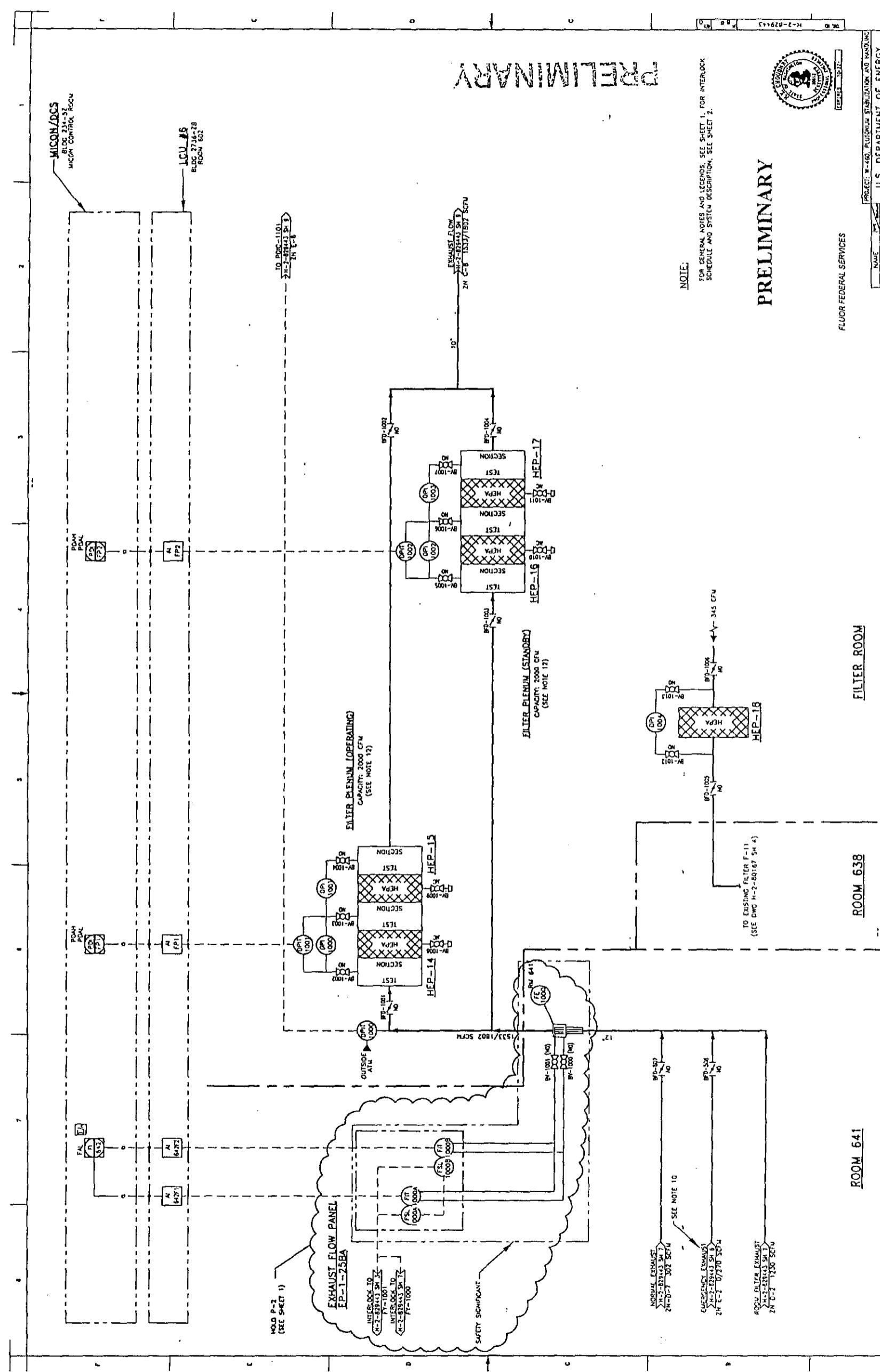
PROJECT: #1450, PLUTONIUM STABILIZATION AND HANDLING									
U.S. DEPARTMENT OF ENERGY									
National Defense Complex									
HVAC/CONTROL SYSTEMS									
AIRFLOW & CONTROL DIAG									
INTERLOCKS & SYS DESC									
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NAME: _____									
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DRAFTING TRACABILITY LIST									
ENG. NO.	TITLE	REF. NUMBER	REFERENCES	NEXT USED ON	REVISIONS				
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PRELIMINARY



PRELIMINARY

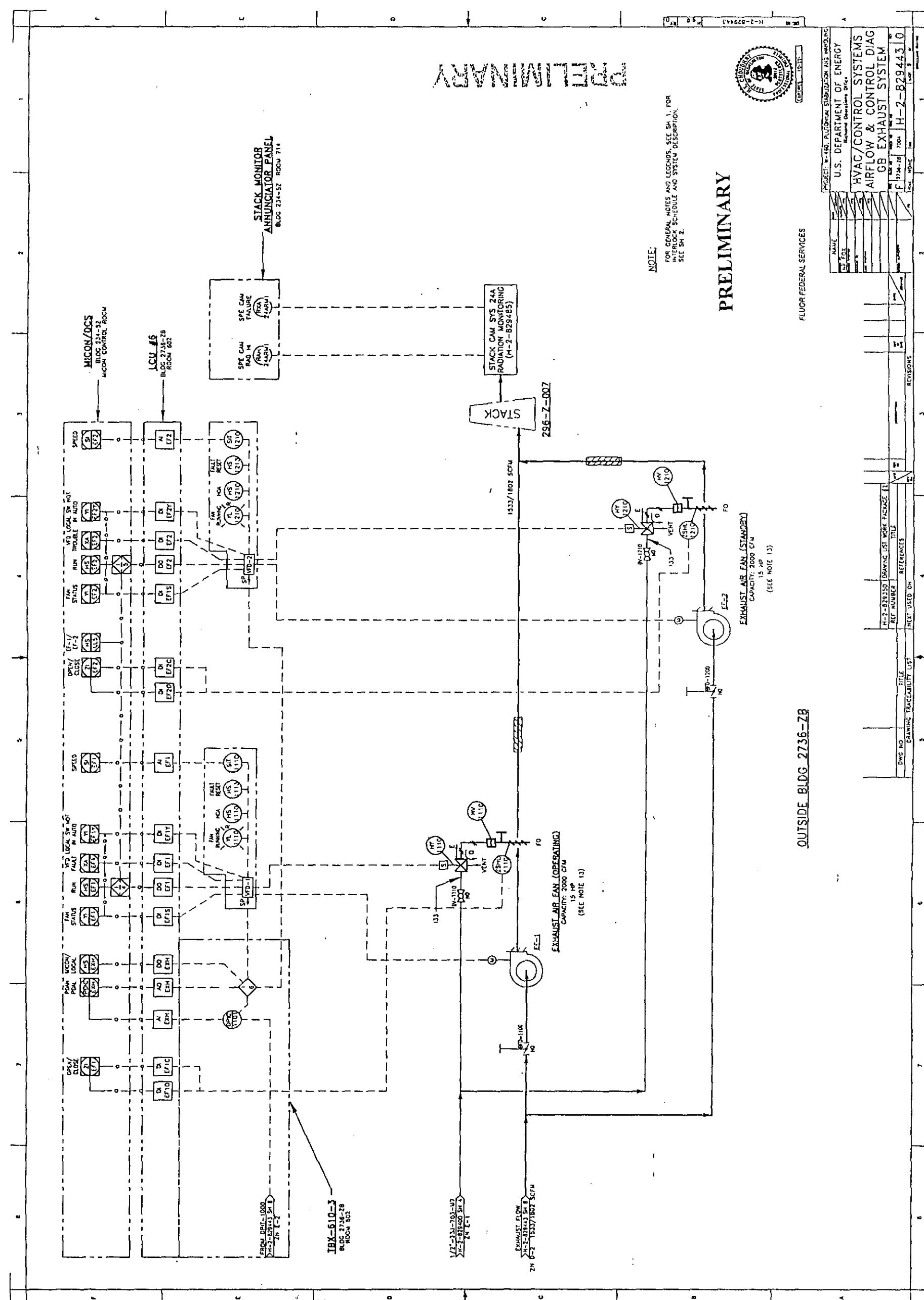
NOTE: FOR GENERAL NOTES AND LEGENDS, SEE SHEET 1, FOR INTERLOCK SCHEDULE AND SYSTEM DESCRIPTION, SEE SHEET 2.



EUROPEAN SERVICES

PROJECT: W-460, PLUTONIUM STERILIZATION AND HANDLING	
U. S. DEPARTMENT OF ENERGY	
National Defense Office	
HVAC/CONTROL SYSTEMS	
AIRFLOW & CONTROL DIAG	
GLOVEBOX FILTERS	
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ATT A-16

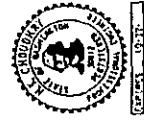


PRELIMINARY

EUROPEAN SERVICES

OUTSIDE B10C 2716 78

FOR GENERAL NOTES AND LEGENDS, SEE SH. 1. FOR
INTERLOCK SCHEDULE AND SYSTEM DESCRIPTION,
SEE SH. 2.



PROJECT # 450 FULTON STABILIZATION AND INNOVATION	
U. S. DEPARTMENT OF ENERGY	
Building Operations Office	
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ATT A-17

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ATTACHMENT B

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SUMMARY AND SYNOPSIS OF CAP88-PC MODELING

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C A P 8 8 - P C

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Version 2.00

Clean Air Act Assessment Package - 1988

S Y N O P S I S R E P O R T

Non-Radon Individual Assessment
Apr 26, 2000 02:58 pm

Facility: Z-Plant
Address: Hanford Site
City: Richland
State: WA Zip: 99336

Source Category:
Source Type: Stack
Emission Year: 2000

Comments: Project W-460

Effective Dose Equivalent
(mrem/year)

1.67E+03

At This Location: 19230 Meters East Southeast
Dataset Name: Project W-460
Dataset Date: Apr 26, 2000 02:58 pm
Wind File: C:\CAP88PC2\WNDFILES\HS200W10.WND

Apr 26, 2000 02:58 pm

SYNOPSIS
Page 1

MAXIMALLY EXPOSED INDIVIDUAL

Location Of The Individual: 19230 Meters East Southeast
Lifetime Fatal Cancer Risk: 1.17E-02

ORGAN DOSE EQUIVALENT SUMMARY

Organ	Dose Equivalent (mrem/y)
GONADS	2.79E+02
BREAST	2.83E+01
R MAR	1.77E+03
LUNGS	3.68E+03
THYROID	2.78E+01
ENDOST	2.18E+04
RMNDR	9.71E+02
EFFEC	1.67E+03

Apr 26, 2000 02:58 pm

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SYNOPSIS
Page 2

RADIOMUCLIDE EMISSIONS DURING THE YEAR 2000

Nuclide	Class	Size	Source		TOTAL
			#1	Ci/y	
PU-239	Y	1.00	9.9E+01	9.9E+01	
U-233	Y	1.00	1.1E+01	1.1E+01	
AM-241	W	1.00	4.5E+01	4.5E+01	

SITE INFORMATION

Temperature: 12 degrees C
Precipitation: 16 cm/y
Mixing Height: 1000 m

Apr 26, 2000 02:58 pm

SYNOPSIS
Page 3

SOURCE INFORMATION

Source Number: 1

Stack Height (m): 15.
Diameter (m): 0.

Plume Rise
Momentum (m/s): 7.
(Exit Velocity)

AGRICULTURAL DATA

	Vegetable	Milk	Meat
Fraction Home Produced:	0.000	0.000	0.000
Fraction From Assessment Area:	1.000	1.000	1.000
Fraction Imported:	0.000	0.000	0.000

Food Arrays were not generated for this run.
Default Values used.

DISTANCES (M) USED FOR MAXIMUM INDIVIDUAL ASSESSMENT

19230 24380

C A P 8 8 - P C

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Version 2.00

Clean Air Act Assessment Package - 1988

D O S E A N D R I S K E Q U I V A L E N T S U M M A R I E S

Non-Radon Individual Assessment
Apr 26, 2000 02:58 pm

Facility: Z-Plant
Address: Hanford Site
City: Richland
State: WA Zip: 99336

Source Category:
Source Type: Stack
Emission Year: 2000

Comments: Project W-460

Dataset Name: Project W-460
Dataset Date: Apr 26, 2000 02:58 pm
Wind File: C:\CAP88PC2\WNDFILES\HS200W10.WND

Apr 26, 2000 02:58 pm

SUMMARY
Page 1

ORGAN DOSE EQUIVALENT SUMMARY

Organ	Selected Individual (mrem/y)
GONADS	2.79E+02
BREAST	2.83E+01
R MAR	1.77E+03
LUNGS	3.68E+03
THYROID	2.78E+01
ENDOST	2.18E+04
RMNDR	9.71E+02
EFFEC	1.67E+03

PATHWAY EFFECTIVE DOSE EQUIVALENT SUMMARY

Pathway	Selected Individual (mrem/y)
INGESTION	3.86E+01
INHALATION	1.63E+03
AIR IMMERSION	1.41E-05
GROUND SURFACE	5.65E-01
INTERNAL	1.67E+03
EXTERNAL	5.65E-01
TOTAL	1.67E+03

Apr 26, 2000 02:58 pm

SUMMARY
Page 2

NUCLIDE EFFECTIVE DOSE EQUIVALENT SUMMARY

Nuclide	Selected Individual (mrem/y)
PU-239	9.58E+02
U-233	3.97E+01
AM-241	6.77E+02
TOTAL	1.67E+03

Apr 26, 2000 02:58 pm

SUMMARY
Page 3

CANCER RISK SUMMARY

Cancer	Selected Individual Total Lifetime Fatal Cancer Risk
LEUKEMIA	1.46E-03
BONE	9.32E-04
THYROID	3.15E-06
BREAST	2.34E-05
LUNG	6.01E-03
STOMACH	1.68E-05
BOWEL	9.13E-06
LIVER	3.18E-03
PANCREAS	1.24E-05
URINARY	9.01E-06
OTHER	1.52E-05
TOTAL	1.17E-02

PATHWAY RISK SUMMARY

Pathway	Selected Individual Total Lifetime Fatal Cancer Risk
INGESTION	1.84E-04
INHALATION	1.15E-02
AIR IMMERSION	2.93E-10
GROUND SURFACE	1.18E-05
INTERNAL	1.17E-02
EXTERNAL	1.18E-05
TOTAL	1.17E-02

Apr 26, 2000 02:58 pm

SUMMARY
Page 4

NUCLIDE RISK SUMMARY

Nuclide	Selected Individual Total Lifetime Fatal Cancer Risk
PU-239	7.68E-03
U-233	5.23E-04
AM-241	3.46E-03
TOTAL	1.17E-02

Apr 26, 2000 02:58 pm

SUMMARY
Page 5

INDIVIDUAL EFFECTIVE DOSE EQUIVALENT RATE (mrem/y)
(All Radionuclides and Pathways)

Distance (m)

Direction 19230 24380

N	3.5E+02	2.4E+02
NNW	3.9E+02	2.6E+02
NW	4.3E+02	2.9E+02
WNW	3.1E+02	2.1E+02
W	2.4E+02	1.7E+02
WSW	2.1E+02	1.5E+02
SW	2.4E+02	1.7E+02
SSW	3.1E+02	2.1E+02
S	4.3E+02	3.0E+02
SSE	6.2E+02	4.2E+02
SE	1.2E+03	8.1E+02
ESE	1.7E+03	1.1E+03
E	1.2E+03	7.7E+02
ENE	7.0E+02	4.7E+02
NE	4.9E+02	3.3E+02
NNE	3.6E+02	2.5E+02

Apr 26, 2000 02:58 pm

SUMMARY
Page 6

INDIVIDUAL LIFETIME RISK (deaths)
(All Radionuclides and Pathways)

Distance (m)

Direction 19230 24380

N	2.4E-03	1.6E-03
NNW	2.6E-03	1.7E-03
NW	2.9E-03	2.0E-03
WNW	2.1E-03	1.4E-03
W	1.6E-03	1.1E-03
WSW	1.4E-03	9.5E-04
SW	1.6E-03	1.1E-03
SSW	2.1E-03	1.4E-03
S	3.0E-03	2.0E-03
SSE	4.3E-03	2.8E-03
SE	8.4E-03	5.6E-03
ESE	1.2E-02	7.7E-03
E	8.0E-03	5.3E-03
ENE	4.8E-03	3.2E-03
NE	3.4E-03	2.2E-03
NNE	2.5E-03	1.6E-03

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ATTACHMENT C

DISCUSSION OF BEST AVAILABLE RADIONUCLIDE CONTROL TECHNOLOGY

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1 **DISCUSSION OF BEST AVAILABLE RADIONUCLIDE CONTROL TECHNOLOGY**
2 (**REQUIREMENT 16**)
3 --
4

5 Pursuant to WAC 246-247-110(16), providing cost factors for construction, operation, and maintenance
6 of the proposed control technology components is not required because the following best available
7 radionuclide control technology (BARCT) discussion is provided. The BARCT is defined by WAC
8 246-247-030 as follows:

9 “Technology that will result in a radionuclide emission limitation based on the maximum degree
10 of reduction for radionuclides from any proposed newly constructed or significantly modified
11 emission units that the licensing authority determines is achievable on a case-by-case basis. A
12 BARCT compliance demonstration must consider energy, environmental, and economic impacts,
13 and other costs through examination of production processes, and available methods, systems and
14 techniques for control of radionuclide emissions. A BARCT compliance demonstration is the
15 conclusion of an evaluative process that results in the selection of the most effective control
16 technology from all know feasible alternatives. In no event shall application of BARCT result in
17 emissions of radionuclides that could exceed the applicable standards of WAC 246-247-040.
18 Control technology that meets BARCT requirements also meets ALARCT requirements.”

19
20 As stated in WAC 246-247-120, only those radionuclides comprising more than 10 percent of the
21 unabated dose need to be evaluated. All of the dose is due to particulate radionuclides. The Washington
22 State Department of Health has provided guidance that HEPA filters generally are considered BARCT for
23 particulate emissions (AIR 92-107).

24
25 It is proposed, pursuant to the quoted citation and the cited WDOH guidance that the ventilation system
26 described in Section 8.0 and the controls (engineering and administrative) described in Section 9.0 be
27 approved as BARCT for the proposed activities.

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