

---

**Pacific Northwest Laboratory**  
**Monthly Report to the**  
**Nuclear Research and**  
**Applications Division**  
**for May 1976**

---

by  
**H. T. Fullam**

**June 1976**

**Prepared for the Energy Research  
and Development Administration  
under Contract E(45-1):1830**



## NOTICE

This report was prepared as an account of work sponsored by the United States Government. Neither the United States nor the Energy Research and Development Administration, nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness or usefulness of any information, apparatus, product or process disclosed, or represents that its use would not infringe privately owned rights.

PACIFIC NORTHWEST LABORATORY  
operated by  
BATTELLE  
for the  
ENERGY RESEARCH AND DEVELOPMENT ADMINISTRATION  
*Under Contract E(45-1)-1830*

Printed in the United States of America  
Available from  
National Technical Information Service  
U.S. Department of Commerce  
5285 Port Royal Road  
Springfield, Virginia 22151  
Price: Printed Copy \$4.50; Microfiche \$1.25

3 3679 00062 1963

BNWL-1845-24  
UC-23

PACIFIC NORTHWEST LABORATORY  
MONTHLY REPORT TO THE  
NUCLEAR RESEARCH AND APPLICATIONS DIVISION  
FOR MAY 1976

H. T. Fullam

June 1976

BATTELLE  
PACIFIC NORTHWEST LABORATORIES  
RICHLAND, WASHINGTON 99352

CONTENTS

STRONTIUM HEAT SOURCE DEVELOPMENT PROGRAM . . . . .	1
LONG-TERM COMPATIBILITY TESTS . . . . .	1
ADDITIONAL SHORT-TERM COMPATIBILITY TESTS . . . . .	3

PACIFIC NORTHWEST LABORATORY  
MONTHLY REPORT TO THE  
NUCLEAR RESEARCH AND APPLICATIONS DIVISION  
FOR APRIL 1976

STRONTIUM HEAT SOURCE DEVELOPMENT PROGRAM

At Hanford, strontium is separated from the high-level waste, then converted to the fluoride, and doubly encapsulated in small, high-integrity containers for subsequent long-term storage. The fluoride conversion, encapsulation and storage take place in the Waste Encapsulation and Storage Facilities (WESF). This encapsulated strontium fluoride represents an economical source of  $^{90}\text{Sr}$  if the WESF capsule can be licensed for heat source applications under anticipated use conditions. The objectives of this program are to obtain the data needed to license  $^{90}\text{SrF}_2$  heat sources and specifically the WESF  $^{90}\text{SrF}_2$  capsules. The information needed for licensing can be divided into three general areas:

1. Long-term  $\text{SrF}_2$  compatibility data.
2. Chemical and physical property data on  $^{90}\text{SrF}_2$ .
3. Capsule property data such as external corrosion resistance, crush strength, etc.

The current program is designed to provide the required information.

LONG-TERM COMPATIBILITY TESTS

The long-term compatibility tests are continuing as scheduled. The 1000-hr  $^{90}\text{SrF}_2$  couples have been sectioned and the test specimens shipped to ORNL for analysis. Metallographic examination of the specimens is now underway. Examination of the 1000-hr nonradioactive  $\text{SrF}_2$  test specimens by scanning electron microscopy (SEM) and electron microprobe (EM) is continuing. Preliminary results indicate that attack of the Haynes Alloy 25 and Hastelloy C-276 by the  $\text{SrF}_2$  involves a general dissolution of the metal surface and selective leaching of chromium from the alloy matrix. Analysis of the  $\text{SrF}_2$  shows a high chromium content confirming the leaching of chromium from the alloys. There was little evidence of grain boundary attack of the two alloys. In the case of the TZM there was a general

dissolution of the metal surface and no evidence of grain boundary attack. A detailed evaluation of metal attack, including micrographs and X-ray scans, will be provided when all of the SEM and EM data are available.

Testing of the two full-size WESF  $^{90}\text{SrF}_2$  capsules is continuing. It has not been possible to obtain capsule temperature measurements during the month, however, because of the strike which is curtailing the hot cell operations at ARHCO. How soon it will be possible to resume taking the temperature measurements is unknown.

A test has been started to determine the effect of a thermal gradient on the compatibility of Hastelloy C-276 with nonradioactive  $\text{SrF}_2$ . A single test couple is being subjected to a thermal gradient of approximately  $560^\circ\text{C}$  with a maximum temperature of about  $920^\circ\text{C}$ . The test will last for 6 months after which the test couple will be sectioned and examined to determine metal-fluoride interaction at various locations. The test couple consists of a section of Hastelloy C-276 pipe containing  $\text{SrF}_2$  compacted to a density of about 70% of theoretical density. The metal surface to fuel volume ratio (S/V) of the test couple is  $1.0 \text{ cm}^{-1}$  which is slightly greater than that of the WESF  $^{90}\text{SrF}_2$  capsule. The nonradioactive  $\text{SrF}_2$  being used in the test is the same as that used in the long-term compatibility tests. Composition of the  $\text{SrF}_2$  is given in Table 1.

The test capsule was fabricated from 1-1/2 in. Sch-40 Hastelloy C-276 pipe and was 26 in. long. The end pieces were 1/8 in. thick Hastelloy C-276 plate. The  $\text{SrF}_2$  was compacted into the capsule in increments by cold pressing. The filled capsule was sealed by TIG welding the end plate in place. Chromel-alumel thermocouples were welded to the surface of the capsules at various locations to monitor the capsule surface temperature during the test. The test couple was placed in a jacket of 304L stainless steel and inserted into a three-zone tube furnace. Each of the three heating zones of the furnace could be controlled independently allowing a temperature gradient to be maintained over the length of the capsule.

Figure 1 shows the capsule surface temperature several days after the test was started. A fairly uniform temperature gradient is maintained across approximately two-thirds of the capsule length, but the system is designed to give a nonuniform gradient across the colder third of the capsule.

TABLE 1. Composition of the Nonradioactive  $\text{SrF}_2$  Used in the Compatibility Tests

Component	Wt%
$\text{SrF}_2$	94.50
$\text{AlF}_3$	0.55
$\text{BaF}_2$	0.88
$\text{CaF}_2$	0.36
$\text{CdF}_2$	0.003
$\text{CrF}_3$	0.08
$\text{FeF}_3$	0.03
$\text{MgF}_2$	0.24
NaF	2.28
$\text{NdF}_3$	0.92
$\text{NiF}_2$	0.02
$\text{PbF}_2$	0.05
$\text{H}_2\text{O}$	<0.01
$\text{NO}_3^-$	<0.01
$\text{O}^{2-}$	<0.01

#### ADDITIONAL SHORT-TERM COMPATIBILITY TESTS

Additional compatibility tests are underway to evaluate potential containment materials not covered in the original short-term tests. A total of 29 different materials are being tested at 800°C for 1500 and 4400 hr using nonradioactive  $\text{SrF}_2$  of the composition shown in Table 1. Hastelloy C-276, Haynes Alloy 25 and TZM are also being tested as reference materials. The 1500-hr tests have been completed and examination of the test specimens is underway. Metallographic examination of most of the specimens has been completed, and estimates of metal attack, based on the micrographs, are given in Table 2. A detailed discussion of the metal attack will be provided at a later date when all the test results are available.

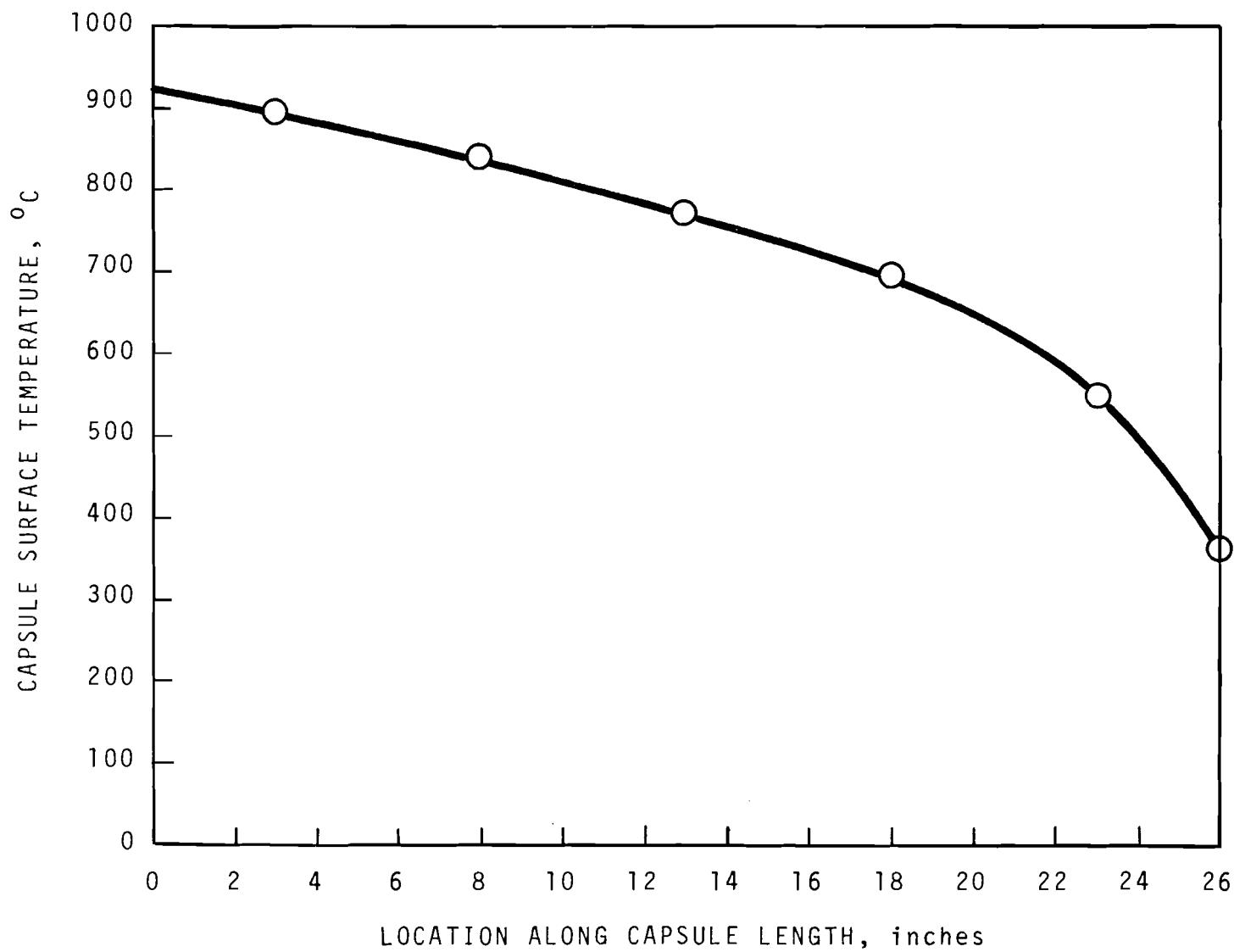


FIGURE 1. Capsule Surface Temperature at Various Locations

TABLE 2. Metal Attack in Test Specimens Exposed to  
Nonradioactive  $\text{SrF}_2$  at 800°C for 1500 Hr

Material	Depth of Metal Affected, mils	
	Chemical Attack	Change in Microstructure
Hastelloy C-276	3	7
Hastelloy C-4	5	12
Hastelloy B	4	15
Hastelloy B-2		
Hastelloy S	7	15
Haynes Alloy 25	2	3
Haynes Alloy 556		
Inconel 617		
Inconel 671	15	25
Incoloy 800		
Rene 41	10	14
Udimet 700	>25	-
Monel 400		
Nickel 200	7	10
Ingot Iron	3	-
Ductile Cast Iron	CR	CR
316L SS	6	-
JS 777		
Copper	>25	-
Titanium	>25	-
Hafnialoy 2525	>25	-
Molybdenum	2	-
TZM	1	-
Niobium	3	2
Ta-10% W	10	-
Mo-50% Re	2	-
W-26% Re	2	-
Rhenium	<1	-
Iridium		
Ir-0.3% W		
Platinum	>25	-
Gold	>25	-

- = no apparent change in microstructure.  
CR = complete reaction.

DISTRIBUTION

NO. OF  
COPIES

OFFSITE

1      ERDA Chicago Patent Attorney  
          9800 S. Cass Avenue  
          Argonne, IL    60439

          A. A. Churm

1      ERDA Division of Biomedical and Environmental Research  
          Washington, DC   20545

          J. N. Maddox

2      ERDA Division of Production and Materials Management  
          Washington, DC   20545

          F. P. Baranowski  
          R. W. Ramsey, Jr.

11     ERDA Nuclear Research and Applications Division  
          Washington, DC   20545

          R. T. Carpenter  
          G. P. Dix  
          T. J. Dobry, Jr.  
          N. Goldenberg  
          A. P. Litman (3)  
          J. J. Lombardo  
          W. C. Remini  
          B. J. Rock  
          E. J. Wahlquist

1      ERDA Oak Ridge Operations Office  
          P. O. Box E  
          Oak Ridge, TN   37830

          D. C. Davis, Jr.

3      ERDA Savannah River Operations Office  
          P. O. Box A  
          Aiken, SC   29801

          R. H. Bass  
          T. B. Hindman  
          R. K. Huntoon

NO. OF  
COPIES

27 ERDA Technical Information Center

1 Department of the Army  
Headquarters, U.S. Army  
Facilities Engineering Support Agency  
Fort Belvoir, VA 22060

H. Musselman, Technical Director

1 Electronics and Applied Physics Division  
Building 347.3, AERE Harwell  
Oxfordshire OX11 ORA  
Great Britain

E. H. Cooke-Yarborough

1 General Atomic Company  
P. O. Box 81601  
San Diego, CA 92138

H. C. Carney

1 General Electric Company MSVD  
P. O. Box 8555  
Philadelphia, PA 19101

P. E. Brown

1 General Electric Company, Vallecitos Laboratory  
P. O. Box 846  
Pleasanton, CA 94566

G. E. Robinson

3 Los Alamos Scientific Laboratory  
P. O. Box 1663  
Los Alamos, NM 87544

S. E. Bronisz  
R. A. Kent  
R. N. Mulford

1 Monsanto Research Corporation  
Mound Laboratory (ERDA)  
Nuclear Operations  
P. O. Box 32  
Miamisburg, OH 45342

W. T. Cave

NO. OF  
COPIES

1      Naval Nuclear Power Unit  
          P. O. Box 96  
          Fort Belvoir, VA 22060  
          F. E. Rosell

1      Naval Facilities Engineering Command  
          Nuclear Power Division (FAC04N)  
          200 Stovall Street  
          Alexandria, VA 22332  
          G. E. Krauter

1      Navy Office of the Chief of Naval Operations  
          Washington, DC 20390  
          Head, Reactor Branch

4      Holifield National Laboratory  
          Oak Ridge, TN 37830  
          R. S. Crouse  
          J. R. DiStefano  
          E. Lamb  
          A. C. Schaffhauser

3      Teledyne Energy Systems  
          110 W. Timonium Road  
          Timonium, MD 21093  
          P. Dick  
          R. Hannah  
          P. Vogelberger

1      Westinghouse Astronuclear Laboratory  
          P. O. Box 10864  
          Pittsburgh, PA 15236  
          C. C. Silverstein

ONSITE

2      ERDA Richland Operations  
          W. C. Johnson  
          H. A. House

NO. OF  
COPIES

8

Atlantic Richfield Hanford Company

L. I. Brecke  
H. H. Hopkins  
R. E. Isaacson  
R. M. Knights  
C. W. Malody  
J. D. Moore  
G. C. Oberg  
H. P. Shaw

25

Battelle-Northwest

J. W. Bartlett  
T. D. Chikalla  
M. O. Cloninger  
R. L. Dillon  
H. T. Fullam (3)  
S. R. Gano  
K. M. Harmon  
A. J. Haverfield  
J. H. Jarrett  
R. S. Kemper  
R. P. Marshall  
R. W. McKee  
J. M. Nielsen  
R. E. Nightingale  
D. E. Olesen  
L. D. Perrigo  
A. M. Platt  
H. H. Van Tuyl  
R. E. Westerman  
Technical Information Files (3)  
Technical Publications