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and E. G. Sherwood

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FIELD REEVALUATION EXPERIMENTS (1991)

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Figure 1

[illegible]

FIELD REVERSEAL EXPERIMENTS (PFX)
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I. INTRODUCTION

The present field experiment is a continuation of the work reported in [1] and [2] on the effect of the external magnetic field on the plasma in the tokamak. The results of the experiment are presented in this paper. The experiment was carried out at the Los Alamos National Laboratory, Los Alamos, New Mexico, in the summer of 1974. The experiment was carried out in the tokamak, which is a toroidal vacuum chamber of major radius 1.5 m and minor radius 0.25 m. The plasma is heated by a radio-frequency (RF) heating system, which is a cyclotron resonance heating system. The plasma is heated by a radio-frequency (RF) heating system, which is a cyclotron resonance heating system. The plasma is heated by a radio-frequency (RF) heating system, which is a cyclotron resonance heating system.

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II. EXPERIMENTAL METHOD

The experiment was carried out in the tokamak, which is a toroidal vacuum chamber of major radius 1.5 m and minor radius 0.25 m. The plasma is heated by a radio-frequency (RF) heating system, which is a cyclotron resonance heating system. The plasma is heated by a radio-frequency (RF) heating system, which is a cyclotron resonance heating system. The plasma is heated by a radio-frequency (RF) heating system, which is a cyclotron resonance heating system. The plasma is heated by a radio-frequency (RF) heating system, which is a cyclotron resonance heating system. The plasma is heated by a radio-frequency (RF) heating system, which is a cyclotron resonance heating system.

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Ion temperatures are inferred from the Doppler broadening of the $H\alpha$ line, which does not suffer from significant Stark effects in the density range of interest. Depending on the strength of the magnetic field, T_i varies between 10 and 20 eV. For the densities of interest, the equilibrium time between the H and deuterium is less than 1 s. The ion temperature is independent of radius and appears to increase by about 10% during the last of the equilibrium and instability phases. Although this may well be due to the uncertainty of the measurement, it is also the case that is consistent with localized equilibration between electrons and ions, provided T_e is fixed.

During the equilibrium and instability phases, the temporal evolution of the electron temperature, T_e , electron temperature gradient, ∇T_e , and ion temperature, T_i , is shown in Fig. 1. The electron temperature is 10 eV at the center of the plasma and decreases to about 5 eV at the edge. The ion temperature is 10 eV at the center and decreases to about 5 eV at the edge. The electron temperature gradient is 10 eV/cm at the center and decreases to about 5 eV/cm at the edge. The ion temperature gradient is 10 eV/cm at the center and decreases to about 5 eV/cm at the edge. The electron temperature is 10 eV at the center and decreases to about 5 eV at the edge. The ion temperature is 10 eV at the center and decreases to about 5 eV at the edge.

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were added and both first- and second-half-cycle operation were used in an effort to duplicate their results. No significant change in the maximum attainable lifetimes was observed.

Quartz and plates were inserted in an effort to control the radial electric field and the associated rotation [14]. The effects were generally detrimental, but the results were clouded by possible interference of the plates with the PI process. Experimentally, the plasma stability is very sensitive to the PI conditions.

The passive mirrors were removed from the end of the main coil in FHX-A and separately powered spher coils were added. With a 1/3 quarter period risetime and a mirror ratio ~ 2 , the unobscured mirrors resulted in RFC lifetime up to 45 μ s. The plasma parameters under these conditions are being measured. The mirror power supply will soon be unobscured to provide additional axial ionization energy, similar to the Kruzhnitskiy experiment. A possible explanation for the improved lifetime is that more rapid closure of the field lines confines a larger fraction of the plasma, and hence, greater inertia against the accelerating force.

The former party held the position of a "Red" officer with the
title of "Red" officer.

1. *Journal of the American Medical Association*, 1990; 263: 1025-1028.

The radial equilibrium profile of the air flow in the primary chamber is significantly different from the lower chamber, which is compared to the radial equilibrium profile of the air flow in the radial equilibrium chamber. The radial equilibrium profile of the air flow in the primary chamber is significantly different from the lower chamber, which is compared to the radial equilibrium profile of the air flow in the radial equilibrium chamber. The radial equilibrium profile of the air flow in the primary chamber is significantly different from the lower chamber, which is compared to the radial equilibrium profile of the air flow in the radial equilibrium chamber.

It is observed that the confinement time during the equilibrium phase of a shot is τ_{eq} is longer than predicted for both diffusion, but is substantially shorter than the τ_{eq} calculated time, assuming the estimate of ρ_{eq} is correct. The shorter confinement time might be caused by micro-turbulence or ion diffusion [1,2] or by ion-ion diffusion. This latter particle diffusion can dominate electron-ion diffusion for small values of ρ_{eq} where ρ_{eq} is the minor radius of the plasma and ρ_{eq} is the ion gyro-radius in the external field [3]. For the observed values of ρ_{eq} and ρ_{eq} , both sources of anomalous diffusion are viable. The diagnostic studies of the recently observed longer-lived plasmas should increase the accuracy of the τ_{eq} and particle and energy confinement time.

The study of the stability properties of the FHX-produced plasmas is limited by the rotational $m=2$ mode. The rotational modes for a standard tokamak have been theoretically determined by Freidberg and Bernstein [1]. The theory predicts a stable range of normalized rotation $\omega/\omega_{ci} < 1$ for a resist-rotor profile. Freidberg modified the model to handle the resist-rotor RFD. He made a simple, but not unique, analytic continuation in the vicinity of the singular point at $R=0$. The result predicted

the critical rotation of $- / \omega \sim 1.5$ for the mhd mode. Because the finite Larmor radius (FLR) expansion used in the code breaks down near $B=0$, the validity of the result was suspect. However, the results were verified ($- / \omega \sim 1.5$) by using a Vlasov fluid code [22] that correctly calculates the orbits near $B=0$.

The experimental and theoretical results qualitatively agree that a stable range of rotation does exist. However, the theoretically predicted value of the critical rotation is four times higher than experimentally observed. The error bars in Fig. 7 do not include the uncertainty in determining ω from measured quantities, but it is certainly accurate to within a factor of 2. Other possibilities exist to explain the error, such as that the HFC may not have the assumed rigid-rotor equilibrium, or the UV laser may not be rotating at the same velocity as the centerium, even though calculations indicate they should.

The apparent lowering of the rotational threshold for instability may arise from a coupling between the rotational mode and the MHD kink mode. Because of the geometry, the fastest growing kink mode probably results in the same $m=1$ distortion. Even though geometry and FLR effects may reduce the growth rate of the kink so that it is not observed in a non-rotational form during the plasma lifetime, the free energy in the field that drives the kink may lower the threshold of the rotational mode. Since some rotational modes are one dimensional, they do not apply their possible driving force. Evidence is existing the possibility of adding a gravitational term to simulate this effect. In addition, a 2-dimensional MHD code is being used to study the effect of geometry on rotation on the kink mode [23].

The technique to slice the plasma into thin layers has never been observed in the tokamak system. In spite of recent advances in the understanding of tearing modes [24,25], a two-dimensional model may have to be developed to accurately determine the stability of finite length layers as produced in the tokamak system. However, if long layers tend to tear into distinct rings, the result may not be detrimental to fusion concepts [26].

The continuing focus of research is to understand and control the mhd rotational instability. Detailed experimental studies of the dependence of this mode on temperature, geometry, and kink formation processes are being pursued. Advanced theoretical models including geometry and kinetic effects are under development.

ACKNOWLEDGEMENT

The assistance of Dr. C. J. Buchenauer in the development of the UV polychromator is gratefully acknowledged. In addition, the expert technical services of G. L. Stelzer and G. M. Sandoval are greatly appreciated.

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FIGURE CAPTIONS

- Fig. 1. Reversed-field configuration.
- Fig. 2. Comparison of the external B field measured in FRX-A and FRX-B between the quartz tube and the coil at the axial midplane. Traces are shown for both vacuum and a static 5 mTorr D_2 fill pressure.
- Fig. 3. Schematic arrangement for some of the diagnostics on FRX-A.
- Fig. 4. Side-on streak and end-on framing pictures for the (a) dirty mode and (b) clean mode of operation. The framing rate is 1.7 s/frame, with the first frame at $t=3$ s. The thin, outer luminous ring in the framing pictures corresponds to the 20-cm diameter quartz tube.
- Fig. 5. Oscilloscope traces comparing the temporal behavior of the (a) CIII $2297\text{\AA}$ radiation, (b) CV $2271\text{\AA}$ radiation, and (c) the external magnetic field for clean mode operation. The intensities of the two carbon lines are plotted on the same scale.
- Fig. 6. The line density through the RFC di-meter as a function of axial position for three times during the equilibrium phase. The connection lines are added for visual clarity.
- Fig. 7. The temporal history of the CV rotational velocity normalized to the deuterium diamagnetic drift velocity.

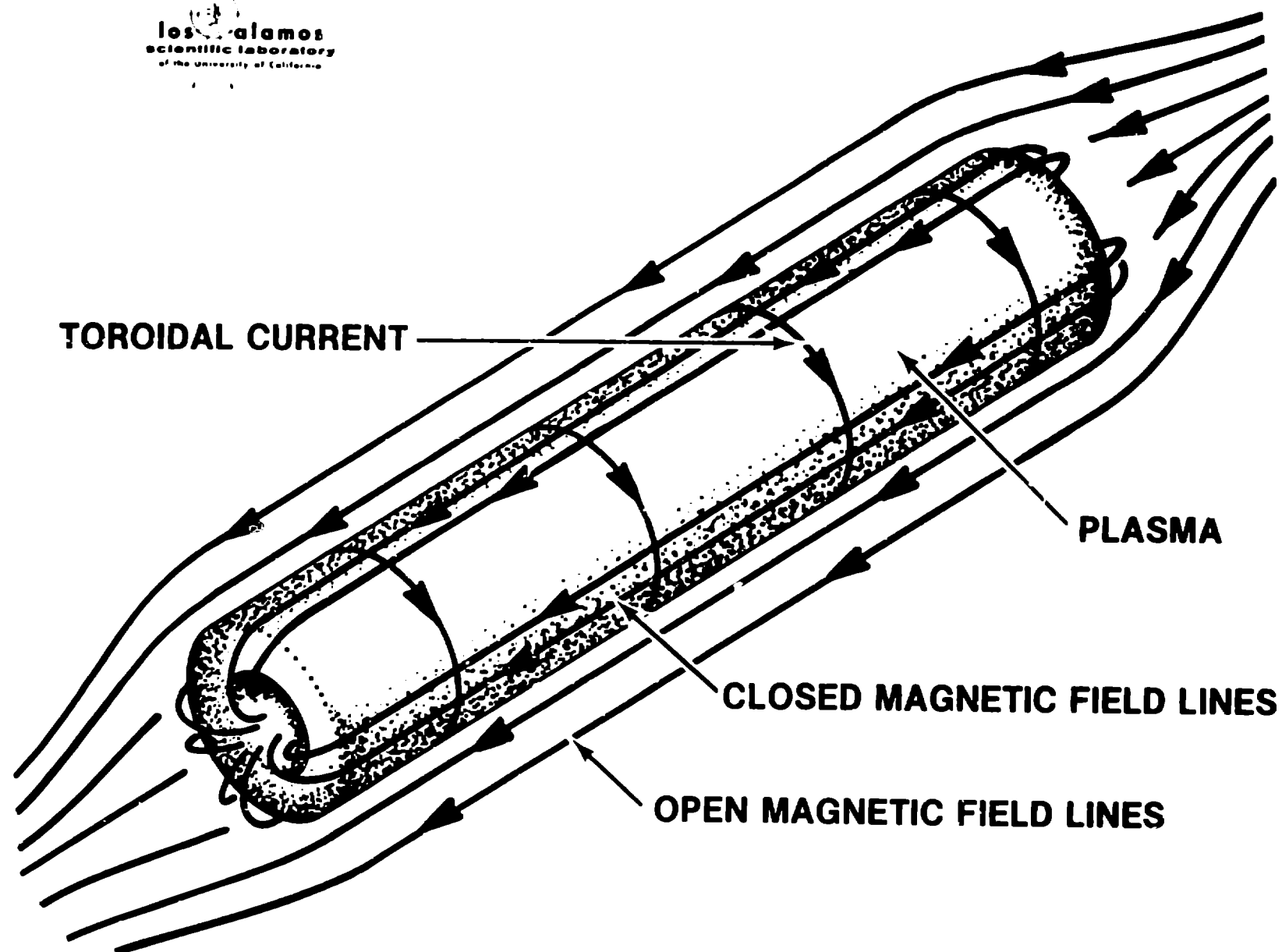


FIG. 1. REVERSED-FIELD CONFIGURATION.

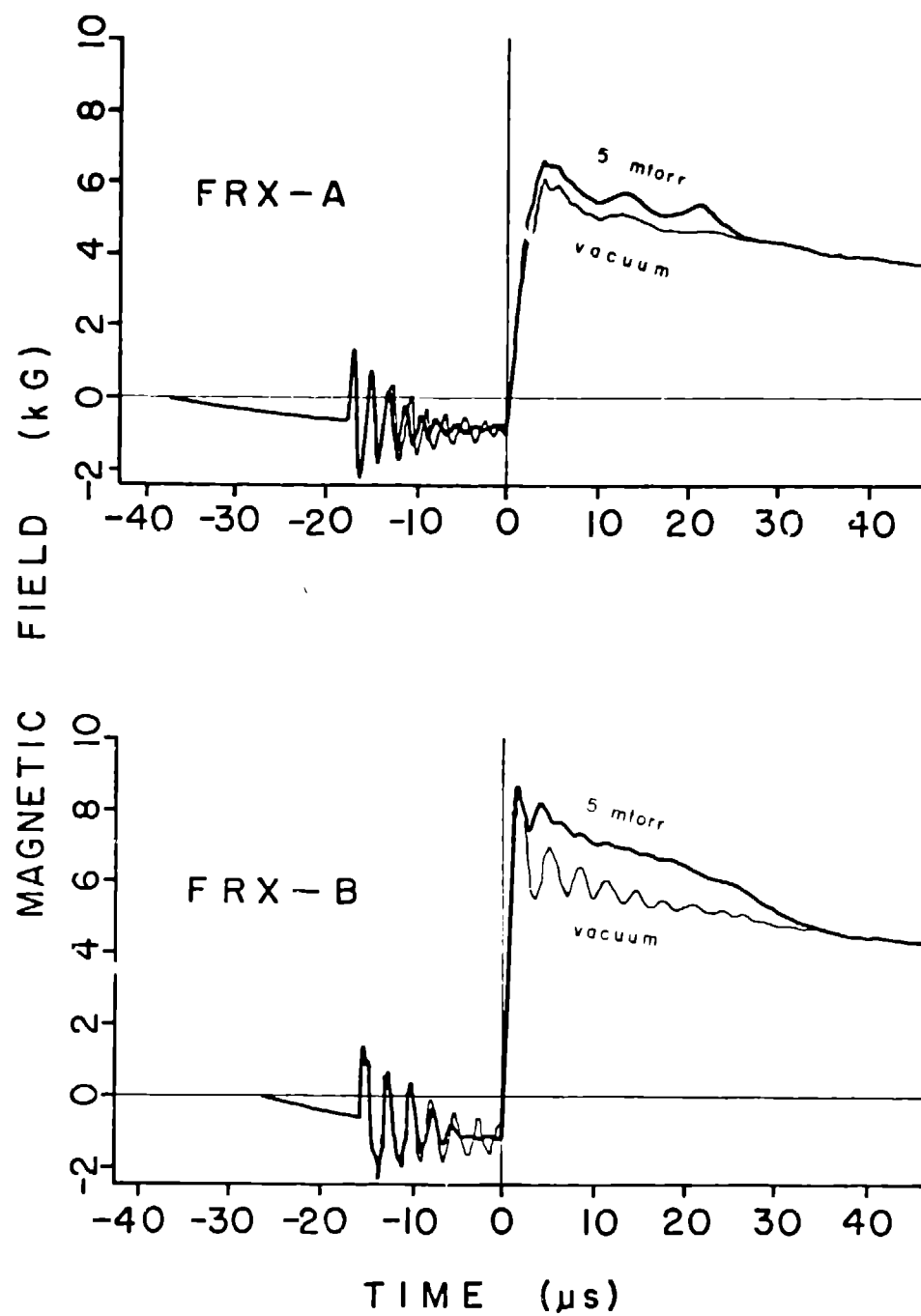


FIG. 2. COMPARISON OF THE EXTERNAL B FIELD MEASURED IN FRX-A AND FRX-B BETWEEN THE QUARTZ TUBE AND THE COIL AT THE AXIAL MIDPLANE. TRACES ARE SHOWN FOR BOTH VACUUM AND A STATIC 5 mTorr D_2 FILL PRESSURE.

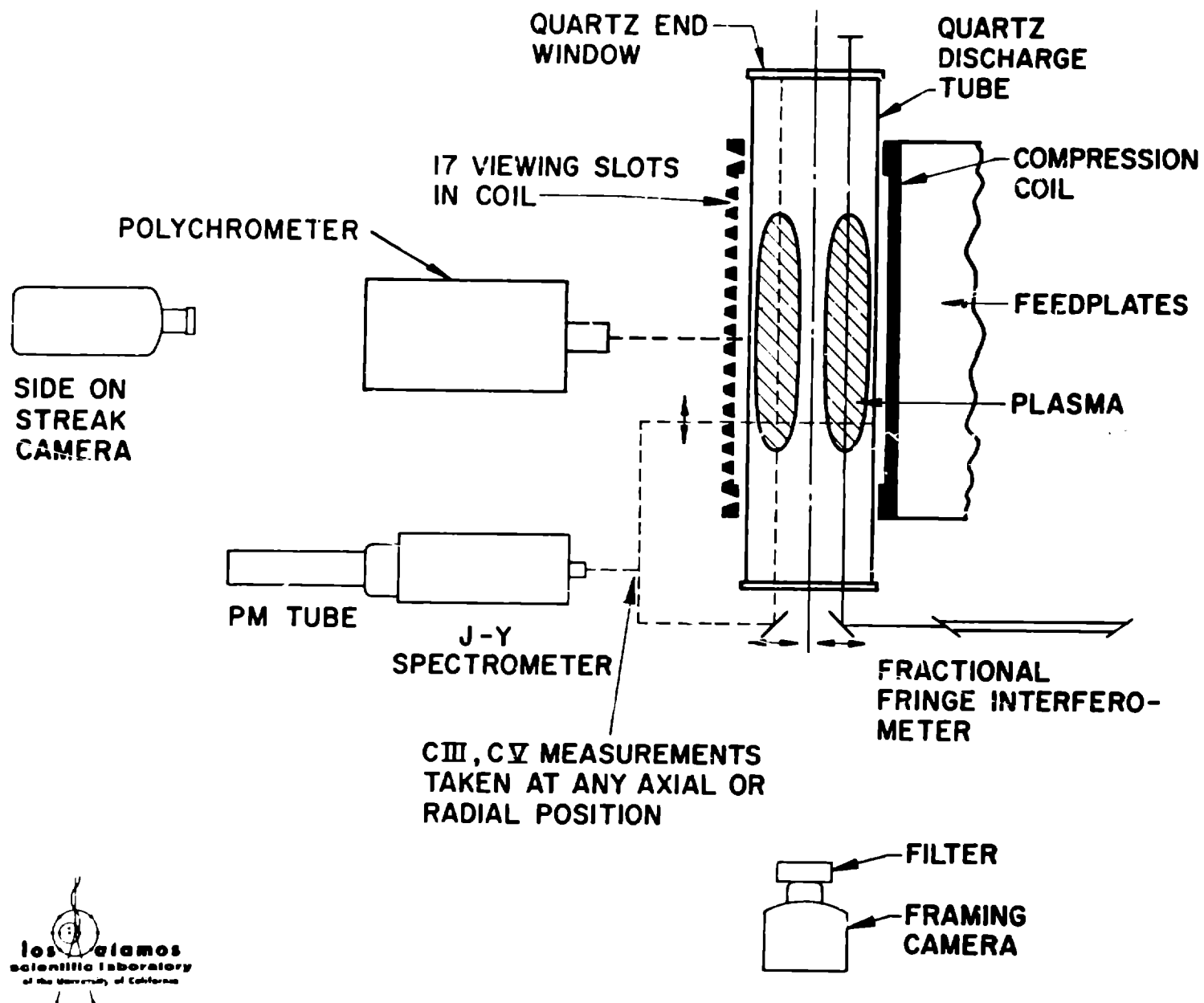


FIG. 3. SCHEMATIC ARRANGEMENT FOR SOME OF THE DIAGNOSTICS ON FRX-A.

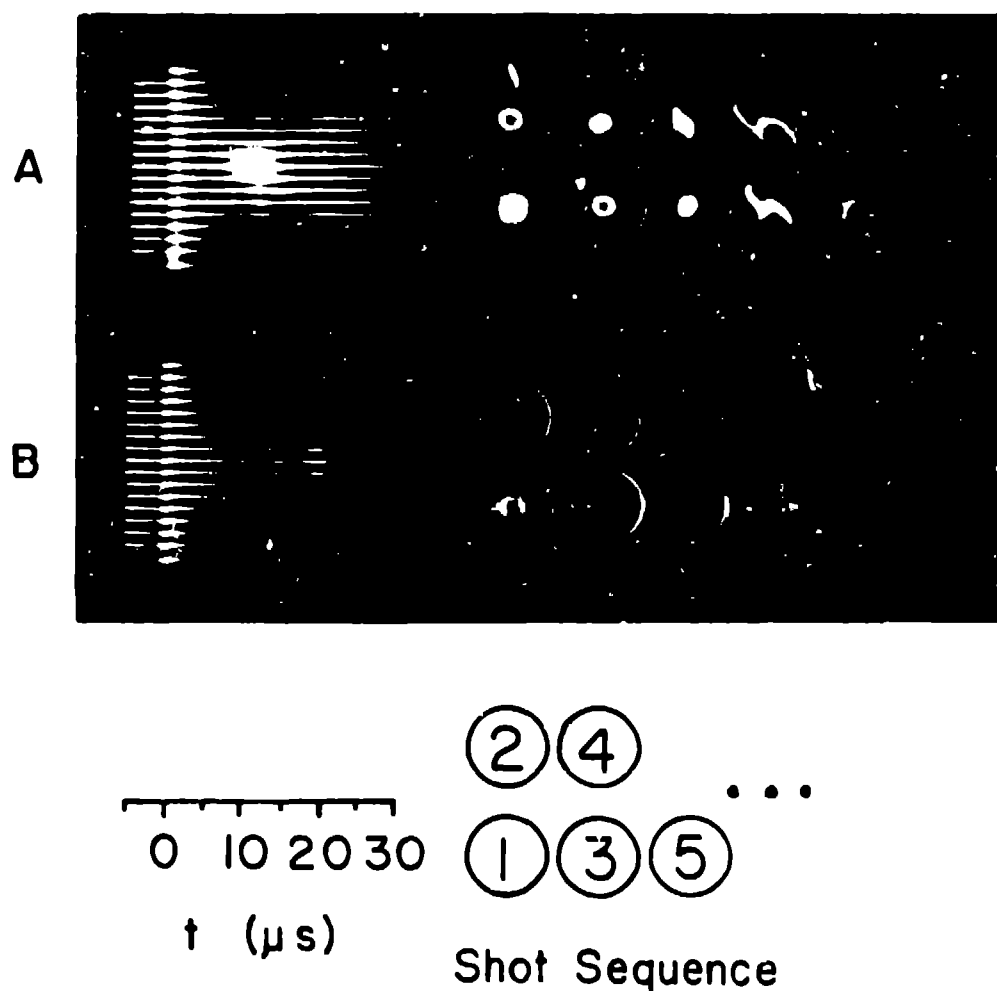


FIG. 4. SIDE-ON STREAK AND END-ON FRAMING PICTURES FOR THE (A) DIRTY MODE AND (B) CLEAN MODE OF OPERATION. THE FRAMING RATE IS 1.7 ns/FRAME, WITH THE FIRST FRAME AT $t = 3$ ns. THE THIN, OUTER LUMINOUS RING IN THE FRAMING PICTURES CORRESPONDS TO THE 20-CM DIAMETER QUARTZ TUBE.

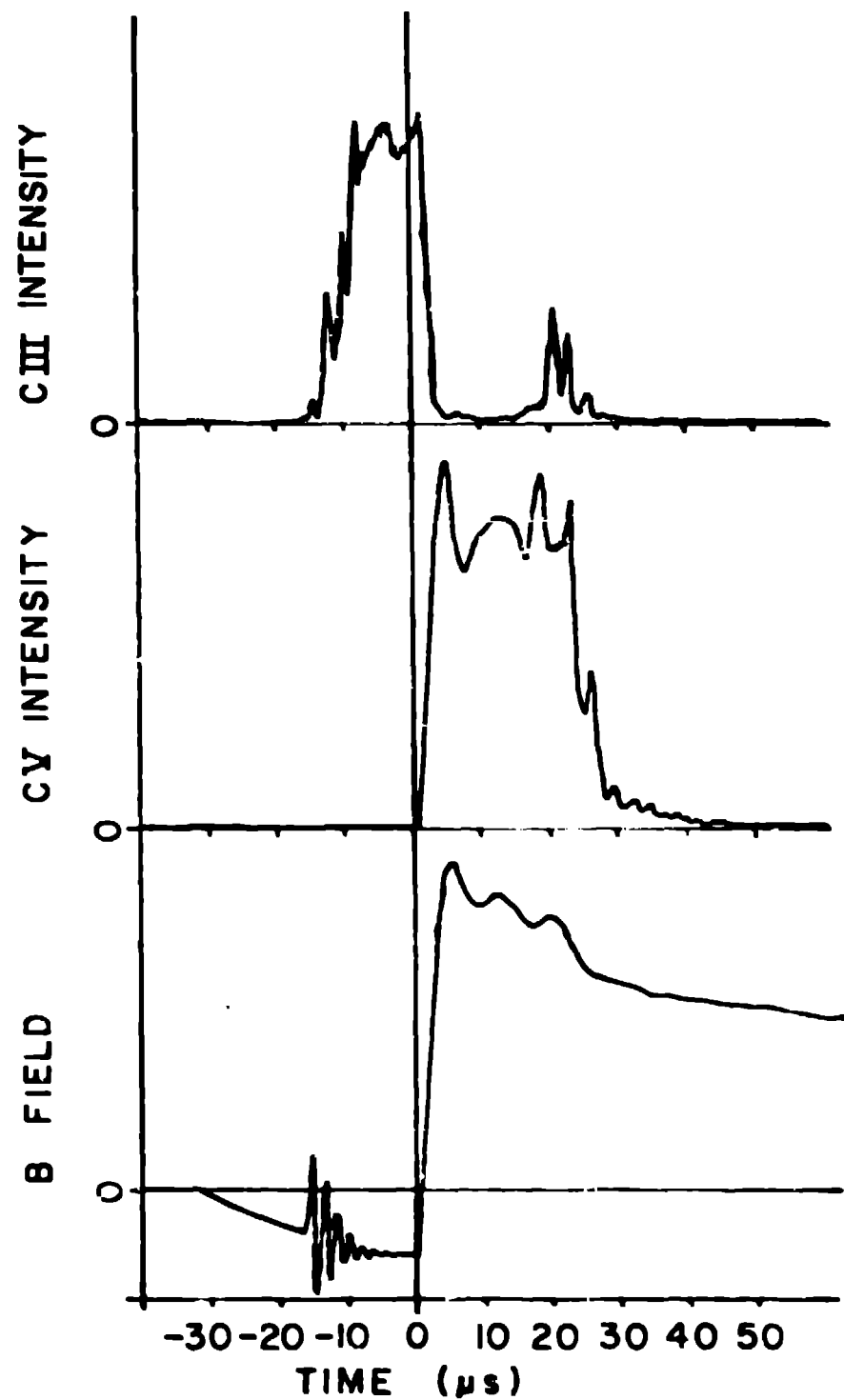


FIG. 5. CIII and CV INTENSITY AND B FIELD. THE TEMPORAL BEHAVIOR OF THE CIII AND CV INTENSITY AND B FIELD DURING THE PULSED RADIATION AND THE EXTERNAL MAGNETIC FIELD FOR CLEAN MHD OPERATION. THE INTENSITIES OF THE TWO CATION LINES ARE PLotted ON THE SAME SCALE.

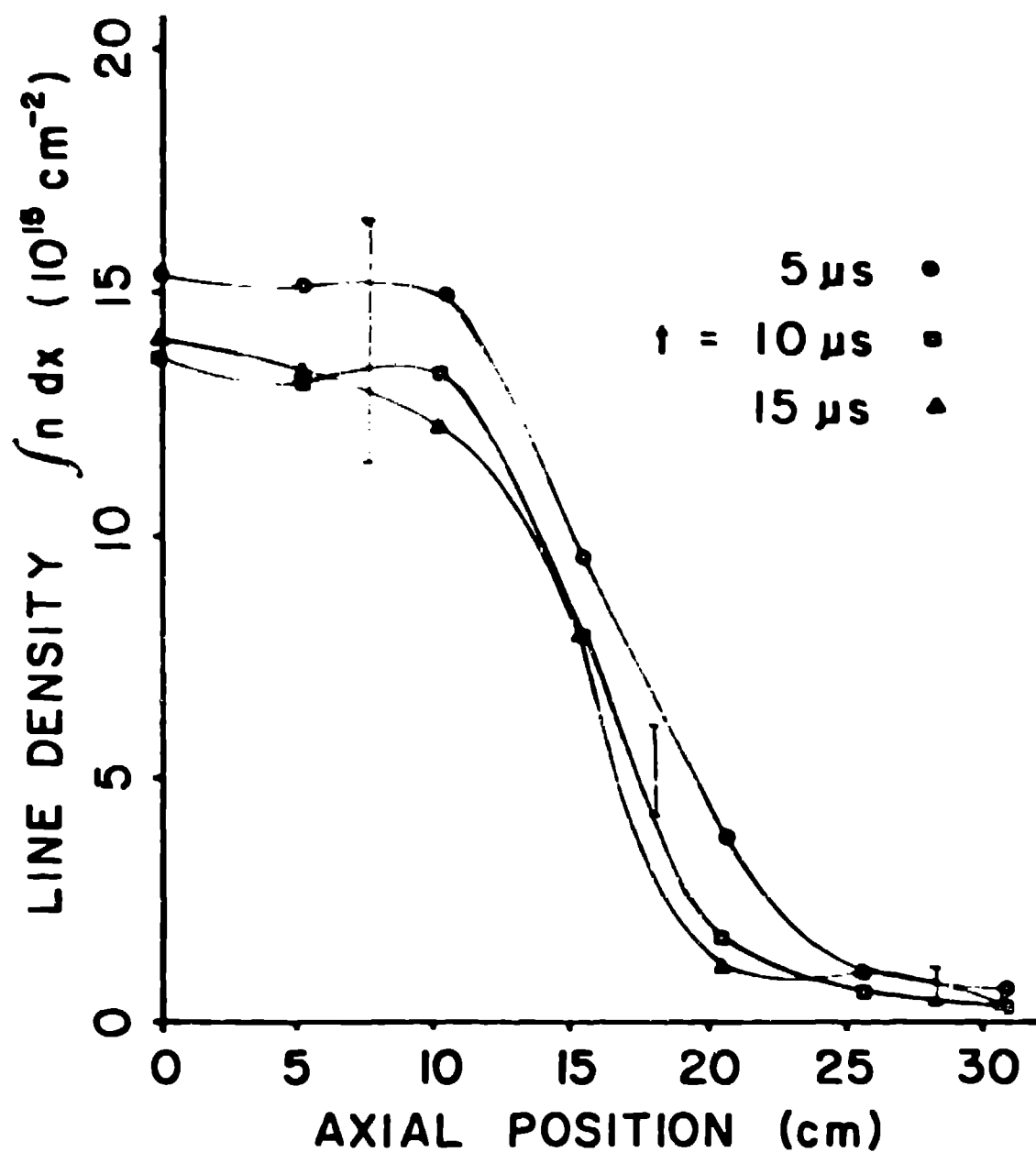


FIG. 6. THE LINE DENSITY THROUGH THE RFC DIAMETER AS A FUNCTION OF AXIAL POSITION FOR THREE TIMES DURING THE EQUILIBRIUM PHASE. THE CONNECTING LINES ARE ADDED FOR VISUAL CLARITY.

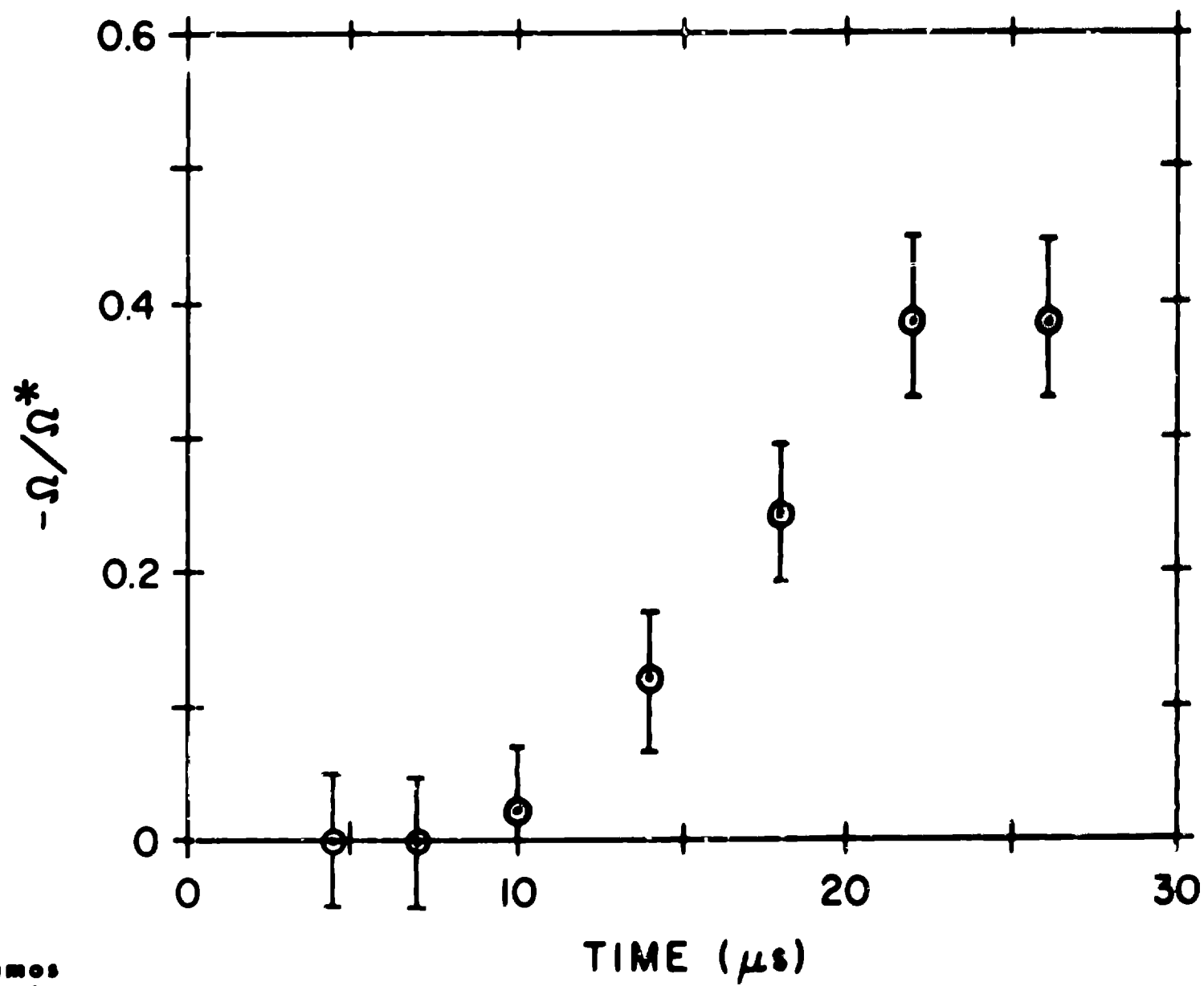


FIG. 7. THE TIME-HISTORY OF THE UV ROTATIONAL VELOCITY NORMALIZED TO
 THE DEUTERIUM DIAMAGNETIC EARTH VELOCITY.