

LEGIBILITY NOTICE

A major purpose of the Technical Information Center is to provide the broadest dissemination possible of information contained in DOE's Research and Development Reports to business, industry, the academic community, and federal, state and local governments.

Although a small portion of this report is not reproducible, it is being made available to expedite the availability of information on the research discussed herein.

**KWOC INDEX OF U.S. NUCLEAR
REGULATORY COMMISSION REGULATORY
GUIDE SERIES**

Performance Assurance Project Office

S. D. Jennings

DISCLAIMER

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

Prepared by the
OAK RIDGE NATIONAL LABORATORY
Oak Ridge, Tennessee 37831
operated by
MARTIN MARIETTA ENERGY SYSTEMS, INC.
for the
DEPARTMENT OF ENERGY
Under contract No. DE-AC05-84OR21400

MASTER

DISTRIBUTION OF THIS DOCUMENT IS UNLIMITED

88

CONTENTS

	<u>Page</u>
FOREWARD	v
1. INTRODUCTION	1
2. KWOC INDEX OF U.S. NUCLEAR REGULATORY COMMISSION REGULATORY GUIDE SERIES	1

FOREWARD

To meet the objectives of the program funded by the Department of Energy (DOE)-Nuclear Energy (NE) Technology Support Programs, the Performance Assurance Project Office (PAPO) administers a Performance Assurance Information Program that collects, compiles, and distributes program-related information, reports, and publications for the benefit of the DOE-NE program participants. The "KWOC Index of U.S. Nuclear Regulatory Commission Regulatory Guide Series" is prepared as an aid in searching for specific topics in the U.S. Nuclear Regulatory Commission, Regulatory Guide Series. The latest version of active guides may be purchased from the Government Printing Office.

1. INTRODUCTION

One major task of the Nuclear Standards Program funded by the Department of Energy (DOE)-Nuclear Energy (NE) Technology Support Programs is to promote and support the use of standards by providing line managers and standards coordinators with data that facilitates their ability to utilize standards requirements. To meet this task, the Performance Assurance Project Office (PAPO) administers a Performance Assurance Information Program.

This task is carried out in accordance with the principle set forth in DOE Order 1300.2, "Department of Energy Standards Program," December 18, 1980, and DOE memorandum, "Implementation of DOE Orders on Quality Assurance, Standards, and Unusual Occurrence Reporting for Nuclear Energy Programs," March 3, 1982, and with guidance from the DOE-NE Technology Support Programs. The purpose of this information program is to collect, compile, and distribute program-related information, reports, and publications for the benefit of the DOE-NE program participants.

2. KWOC INDEX OF U.S. NUCLEAR REGULATORY COMMISSION REGULATORY GUIDE SERIES

The KWOC (Key-Word-Out-of-Context) Index is prepared as an aid in searching for specific topics in the U.S. Nuclear Regulatory Commission Regulatory Guide Series and is based on the latest available issue, August 1989.

The KWOC Index is an alphabetical listing of the titles used in the guides. This Index facilitates identification of subjects by locating major words of the titles on the left of the alphabetical index listing (*indicates the end of a title). The designations for the specific guide numbers are shown in the right-hand column.

The series lists active and draft regulatory guides and consists of the following ten divisions.

- 1. Power Reactors**
- 2. Research and Test Reactors**

- 3. Fuels and Materials Facilities**
- 4. Environmental and Siting**
- 5. Materials and Plant Protection**
- 6. Products**
- 7. Transportation**
- 8. Occupational Health**
- 9. Antitrust and Financial Review**
- 10. General**

The following pages present the KWOC Index.

KINOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Absorber	Use of Borosilicate-Glass Raschig Rings as a Neutron Absorber in Solutions of	3.1
Academic	Guidance to Academic Institutions Applying for Specific Byproduct Material	10.2
Acceptance	Acceptance Sampling Procedures for Exempted and Generally Licensed Items	6.6
Access	Standard Format and Content Guide for Access Authorization Plans for Nuclear	SG 301-4
Access	Use of Observation (Visual Surveillance) Techniques in Material Access Areas (5.14
Access	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65
Access	Entry/Exit Control for Protected Areas, Vital Areas, and Material Access Areas (5.7
Accessibility	Welder Qualification for Areas of Limited Accessibility*	1.71
Accessibility	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28
Accident	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at	1.145
Accident	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Accident	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Accident	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Accident	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Accident	Control of Combustible Gas Concentrations in Containment Following a Loss-of-	1.7
Accident	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized	1.77
Accident	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant	1.82
Accident	Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and	1.97
Accident	Criticality Accident Alarm Systems (Draft ON 015-4, Proposed Revision 1,	8.12
Accidental	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
Accidental	Protection of Nuclear Power Plant Control Room Operators Against an Accidental	1.95
Accidental	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.33
Accidental	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.34
Accidental	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.35
Accountability	Statistical Terminology and Notation for Special Nuclear Materials Control and	5.3
Accounting	Standard Format and Content for the Special Nuclear Material Control and	5.45
Accounting	Management Review of Nuclear Material Control and Accounting Systems (for	5.51
Accounting	Considerations for Establishing Traceability of Special Nuclear Material	5.58
Acquisition	Onsite Meteorological Measurement Program for Uranium Recovery Facilities- Data	3.63
Activation	Applications of Bioassay for Fission and Activation Products (Draft ON 714-4	8.26
Active	Functional Specification for Active Valve Assemblies in Systems Important to	1.148
Actuation	Periodic Testing of Protection System Actuation Functions (Safety Guide 22)*	1.22
Actuation	Periodic Testing of Fuel Reprocessing Plant Protection System Actuation Functions	3.22
Administrative	Administrative Practices for Nuclear Criticality Safety at Fuels and Materials	3.57
Administrative	Administrative Guide for Packaging and Transporting Radioactive Material*	7.1
Administrative	Administrative Guide for Obtaining Exemptions from Certain NRC Requirements Over	7.5
Administrative	Administrative Guide for Verifying Compliance with Packaging Requirements for	7.7
Administrative	Guide for Administrative Practices in Radiation Monitoring*	8.2
Adsorption	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Adsorption	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
AEC	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust	9.3
Air	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Air	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Air	Preoperational Testing of Instrument and Control Air Systems (Draft RS 709-4, a	1.68.3
air	(Withdrawn - See 47 FR 19258, 5-6-82) Reissued as Regulatory Guide 1.68.3, a	1.80
Air	Efficiency Testing of Air-Cleaning Systems Containing Devices for Removal of	3.2
Air	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air	8.25
Airborne	Calculation Models for Estimating Radiation Doses to Man from Airborne	3.51
Airborne	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
ALARA	(Second Proposed Revision 4 to Regulatory Guide 8.8) Information Relevant to	OP 618-4
Alarm	Perimeter Intrusion Alarm Systems (Draft SG 479-4, Proposed Revision 2,	5.44

NRC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Alarm	Criticality Accident Alarm Systems (Draft NUREG-0154, Proposed Revision 1,	8.12
Alarm	Audible-Alarm Dosimeters (Draft NUREG-0044 published 8-79)*	8.28
Aluminum	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in	2.3
Analyses	Measurements of Radionuclides in the Environment-Strontium-89 and Strontium-90	4.6
Analysis	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power	EM 805-5
Analysis	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power	1.110
Analysis	An Acceptable Model and Related Statistical Methods for the Analysis of Fuel	1.126
Analysis	Laboratory Investigations of Soils for Engineering Analysis and Design of	1.138
Analysis	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis	1.154
Analysis	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (1.70
Analysis	Combining Model Responses and Spatial Components in Seismic Response Analysis*	1.92
Analysis	Standard Format and Content of Safety Analysis Reports for Uranium Enrichment	3.25
Analysis	Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing	3.26
Analysis	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Analysis	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Analysis	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.6*
Analysis	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62
Analysis	Measurements of Radionuclides in the Environment-Sampling and Analysis of	4.5
Analysis	Analysis and Use of Process Data for the Protection of Special Nuclear Material*	5.24
Analysis	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic	5.39
Analysis	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Analysis	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels*	7.6
Analysis	Load Combinations for the Structural Analysis of Shipping Casks for Radioactive	7.8
Analytical	Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF4)	5.4
Analyzers	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 405-4
Antitrust	Regulatory Staff Position Statement on Antitrust Matters*	9.1
Antitrust	Information Needed by the NRC Staff in Connection with Its Antitrust Review of	9.2
Antitrust	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust	9.3
Appendix	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents	1.109
Appendix	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
Appendix	Reporting of Operating Information-Appendix A Technical Specifications (for	1.16
Approvals	Guide for the Preparation of Applications for Licenses and Approvals to	FC 406-4
Aquatic	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
Aqueous	Nuclear Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Area	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65
ASME	Inservice Inspection Code Case Acceptability - ASME Section XI, Division 1 (1.147
ASME	Design and Fabrication Code Case Acceptability-ASME Section III, Division 1*	1.84
ASME	Materials Code Case Acceptability-ASME Section III, Division 1*	1.85
ASME	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors	1.87
Assay	Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste (5.11
Assay	Nondestructive Uranium-235 Enrichment Assay by Gamma-Ray Spectrometry (Draft SG	5.21
Assay	In Situ Assay of Plutonium Residual Holdup (Draft SG 045-4, Proposed Revision 1,	5.23
Assay	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission	5.34
Assay	In Situ Assay of Enriched Uranium Residual Holdup (Draft SG 047-4, Proposed	5.37
Assay	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray	5.38
Assay	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic	5.39
Assay	Qualification, Calibration, and Error Estimation Methods for Nondestructive	5.53
Assessment	Comprehensive Vibration Assessment Program for Reactor Internals During	1.20
Assessment	Assessment of the Assumption of Normality (Employing Individual Observed Values)*	5.22
Assessment	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Assessments	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at	1.145

KODC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Assignments	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify	1.41
Assumption	Assessment of the Assumption of Normality (Employing Individual Observed Values)*	5.22
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Assumptions	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized	1.77
Assumptions	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control	1.78
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.33
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.34
Assumptions	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.35
Assumptions	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program*	8.9
Assuring	Assuring the Availability of Funds for Decommissioning Nuclear Reactors*	DG-1003
Atmosphere	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Atmospheric	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Atmospheric	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at	1.145
Attenuation	Calculation of Radon Flux Attenuation by Earthen Uranium Mill Tailings Covers (3.64
Audit	Qualification of Quality Assurance Program Audit Personnel for Nuclear Power	1.146
Auditing	Auditing of Quality Assurance Programs for Nuclear Power Plants*	1.166
August	Station Blackout (Draft SI 501-4 published 3-86) (Issued June 1988, reissued	1.155
Austenitic	Nonmetallic Thermal Insulation for Austenitic Stainless Steel*	1.36
Austenitic	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Authorization	Standard Forms and Content Guide for Access Authorization Plans for Nuclear	SG 301-4
Authorize	Guide for the Preparation of Applications for Licenses and Approvals to	FC 406-4
Availability	Assuring the Availability of Funds for Decommissioning Nuclear Reactors*	DG-1003
Availability	Availability of Electric Power Sources*	1.93
Backfitting	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	1.11
Badge	Film Badge Performance Criteria*	8.3
Barriers	Confinement Barriers and Systems for Fuel Reprocessing Plants*	3.18
Barriers	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel	3.27
Batteries	Installation Design and Installation of Large Lead Storage Batteries for Nuclear	1.128
Batteries	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for	1.129
Batteries	Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants (1.158
Best	Best-Estimate Calculations of Emergency Core Cooling System Performance (Draft	1.157
Bioassay	Applications of Bioassay for Uranium*	8.11
Bioassay	Applications of Bioassay for I-125 and I-131*	8.20
Bioassay	Bioassay at Uranium Mills (Draft OP 013-4, Proposed Revision 1, published 1-87)*	8.22
Bioassay	Applications of Bioassay for Fission and Activation Products (Draft OM 714-4	8.26
Bioassay	Criteria for Establishing a Tritium Bioassay Program (Draft OP 713-4 published 6-	8.32
Bioassay	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program*	8.9
Biological	Shield Test Program for Evaluation of Installed Biological Shielding in Research	2.1
Biological	Packaging and Transportation of Radioactively Contaminated Biological Materials*	7.2
Blackout	Station Blackout (Draft SI 501-4 published 3-86) (Issued June 1988, reissued	1.155
Bodies	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.4
Boiling	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Boiling	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Boiling	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Boiling	Maintenance of Water Purity in Boiling Water Reactors (for Comment)*	1.56
Boiling	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems	1.68.1

NDOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Boiling	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Boiling	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98
Borosilicate	Use of Borosilicate-Glass Raschig Rings as a Neutron Absorber in Solutions of	3.1
Boundary	Reactor Coolant Pressure Boundary Leakage Detection Systems*	1.45
Brachytherapy	Leak Testing Radioactive Brachytherapy Sources*	6.1
Brachytherapy	Integrity and Test Specifications for Selected Brachytherapy Sources*	6.2
Broad	(Proposed Revision 2 to Regulatory Guide 10.5) Guide for the Preparation of	FC 408-4
Broad	Applications for Type A Licenses of Broad Scope (Errata published 7-84) (Draft	10.5
Byproduct	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.10
Byproduct	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.11
Byproduct	Guidance to Academic Institutions Applying for Specific Byproduct Material	10.2
Byproduct	Guide for the Preparation of Applications for Licenses for Laboratory and	10.7
Byproduct	Acceptance Sampling Procedures for Exempted and Generally Licensed Items	6.6
Byproduct	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and	8.21
Cable	Qualification Test for Cable Penetration Fire Stops for Use in Nuclear Power	RS 809-5
Cables	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Cables	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-	1.131
Calculational	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Calibrating	Guide for the Preparation of Applications for Licenses for the Use of	FC 413-4
Calibration	Test and Calibration of Radiation Protection Instrumentation*	OP 032-5
Calibration	Qualification, Calibration, and Error Estimation Methods for Nondestructive	5.53
Cask	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air	8.25
Cask	Suggested Format for Cask Flow Statements Submitted as Guarantees of Payment of	9.4
Cask	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Cask	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Cask	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Cask	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels*	7.6
Casks	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62
Casks	Load Combinations for the Structural Analysis of Shipping Casks for Radioactive	7.8
Cement	Qualifications for Cement Grouting for Prestressing Tendons in Containment	1.107
Certification	Training and Certification of Independent Spent Fuel Storage Installation	HF 608-4
Channels	Response-Time Testing of Protection System Instrument Channels*	IC 121-5
Characterization	Standard Format and Content of Site Characterization Plans for High-Level-Waste	4.17
Chemical	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control	1.78
Chemical	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Chlorine	Protection of Nuclear Power Plant Control Room Operators Against an Accident!	1.95
Chromatography	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 405-4
Cladding	Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components*	1.63
Class	Service Limits and Loading Combinations for Class 1 Linear-Type Component	1.124
Class	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type	1.130
Class	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors	1.87
Classification	Tornado Design Classification*	1.117
Classification	Seismic Design Classification*	1.29
Classification	Seismic Design Classification for Plutonium Processing and Fuel Fabrication	3.16
Classification	Classification of Containment Properties of Sealed Radioactive Sources*	6.6
Classifications	Quality Group Classifications and Standards for Water-, Steam-, and Radioisotope-	1.26
Cleaning	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated	1.37
Cleaning	Efficiency Testing of Air-Cleaning Systems Containing Devices for Removal of	3.2
Cleanup	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Closure	Materials and Inspections for Reactor Vessel Closure Studs*	1.65
Coatings	Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled	1.54

KHOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Coatings	Quality Assurance Requirements for Protective Coatings Applied to Fuel	3.21
Coatings	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel	3.30
Code	Materials, Construction, and Testing of Concrete Containments (Articles CC-1000,	1.136
Code	Inservice Inspection Code Case Acceptability - ASME Section XI, Division 1 (1.147
Code	Design and Fabrication Code Case Acceptability-ASME Section III, Division 1*	1.84
Code	Materials Code Case Acceptability-ASME Section III, Division 1*	1.85
Code	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors	1.87
Collection	Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance	1.88
Combustible	Control of Combustible Gas Concentrations in Containment Following a Loss-of-	1.7
Combustible	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel	3.7
Commercial	Preparation of Environmental Reports for Commercial Uranium Enrichment Facilities	4.9
Communication	Communication with Transport Vehicles*	5.32
Compliance	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents	1.109
Compliance	Administrative Guide for Verifying Compliance with Packaging Requirements for	7.7
Component	Service Limits and Loading Combinations for Class 1 Linear-Type Component	1.124
Component	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type	1.130
Components	Development of Floor Design Response Spectra for Seismic Design of Floor-	1.122
Components	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.143
Components	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-	1.26
Components	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated	1.37
Components	Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components*	1.43
Components	Design Limits and Loading Combinations for Metal Primary Reactor Containment	1.57
Components	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors	1.87
Components	Combining Modal Responses and Spatial Components in Seismic Response Analysis*	1.92
Components	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Comprehensive	Comprehensive Vibration Assessment Program for Reactor Internals During	1.20
Computer	Criteria for Programmable Digital Computer System Software in Safety-Related	1.152
Concentrations	Control of Combustible Gas Concentrations in Containment Following a Loss-of-	1.7
Concepts	Limit of Error Concepts and Principles of Calculation in Nuclear Materials	5.18
Concepts	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program*	8.9
Concrete	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power	EM 805-5
Concrete	(Proposed Revision 2 to Regulatory Guide 1.96) Quality Assurance Requirements	RS 908-5
Concrete	(Proposed Regulatory Guide 1.35.1) Determining Prestressing Forces for	SC 807-4
Concrete	(Proposed Revision 3 to Regulatory Guide 1.35) Inservice Inspection of Ungrouted	SC 810-4
Concrete	Materials, Construction, and Testing of Concrete Containments (Articles CC-1000,	1.136
Concrete	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor	1.142
Concrete	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containment	1.35
Concrete	Concrete Radiation Shields for Nuclear Power Plants*	1.69
Concrete	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted	1.90
Concrete	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.94
Concrete	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel	3.27
Concrete	Concrete Radiation Shields*	3.9
Condensate	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems	1.68.1
Conduct	Conduct of Nuclear Material Physical Inventories*	5.13
Confinement	Confinement Barriers and Systems for Fuel Processing Plants*	3.18
Consequence	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at	1.145
Construction	(Proposed Revision 2 to Regulatory Guide 1.96) Quality Assurance Requirements	RS 908-5
Construction	Materials, Construction, and Testing of Concrete Containments (Articles CC-1000,	1.136
Construction	Quality Assurance Program Requirements (Design and Construction) (Draft RS 002-	1.28
Construction	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors	1.87
Construction	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.94

KMOE INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Construction	Design, Construction, and Inspection of Embankment Retention Systems for Uranium	3.11
Construction	Design, Construction, and Use of Radioisotopic Power Generators for Certain Land	6.3
Construction	Information Needed by the NRC Staff in Connection with Its Antitrust Review of	9.2
Containers	Selection and Use of Pressure-Sensitive Seal* on Containers for Onsite Storage	5.10
Containment	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Containment	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Containment	Containment System Leakage Testing*	MS 021-5
Containment	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Containment	Qualifications for Cement Grouting for Prestressing Tendons in Containment	1.107
Containment	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	1.11
Containment	Containment Isolation Provisions for Fluid Systems (for Comment)*	1.141
Containment	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containment	1.35
Containment	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment	1.40
Containment	Design Limits and Loading Combinations for Metal Primary Reactor Containment	1.57
Containment	Electric Penetration Assemblies in Containment Structures for Nuclear Power	1.63
Containment	Control of Combustible Gas Concentrations in Containment Following a Loss-of-	1.7
Containment	Qualification Tests of Electric Valve Operators Installed Inside the Containment	1.73
Containment	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted	1.90
Containment	Classification of Containment Properties of Sealed Radioactive Sources*	6.4
Containments	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels*	7.6
Containments	(Proposed Regulatory Guide 1.35.1) Determining Prestressing Forces for	SC 807-4
Containments	(Proposed Revision 3 to Regulatory Guide 1.35) Inservice Inspection of Ungrouted	SC 810-4
Containments	Materials, Construction, and Testing of Concrete Containments (Articles CC-1000,	1.136
Containments	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor	1.142
Contaminants	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4
Contaminated	Packaging and Transportation of Radioactively Contaminated Biological Materials*	7.2
Contingency	Materials Protection Contingency Measures for Uranium and Plutonium Fuel	5.30
Contingency	Standard Format and Content of Safeguards Contingency Plans for Nuclear Power	5.54
Contingency	Standard Format and Content of Safeguards Contingency Plans for Fuel Cycle	5.55
Contingency	Standard Format and Content of Safeguards Contingency Plans for Transportation (5.56
Control	Design, Installation, and Inspection of Seepage Control Liners at Uranium	MS 166-4
Control	Guidance to Operators at the Controls and to Senior Operators in the Control	1.114
Control	Quality Assurance Requirements for Control of Procurement of Items and Services	1.123
Control	Inspection of Water-Control Structures Associated with Nuclear Power Plants*	1.127
Control	Criteria for Power, Instrumentation, and Control Portions of Safety Systems (1.153
Control	Control of Ferrite Content in Stainless Steel Weld Metal*	1.31
Control	Control of Electroslag Weld Properties*	1.34
Control	Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components*	1.43
Control	Control of the Use of Sensitized Stainless Steel*	1.44
Control	Control of Preheat Temperature for Welding of Low-Alloy Steel*	1.50
Control	Preoperational Testing of Instrument and Control Air Systems (Draft RS 709-4, a	1.68.3
Control	Control of Combustible Gas Concentrations in Containment Following a Loss-of-	1.7
Control	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized	1.77
Control	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control	1.78
control	(Withdrawn - See 47 FR 19258, 5-6-82) Reissued as Regulatory Guide 1.68.3, a	1.80
Control	Protection of Nuclear Power Plant Control Room Operators Against an Accidental	1.95
Control	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Control	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Control	Nuclear Criticality Control and Safety of Homogeneous Plutonium-Uranium Fuel	3.47
Control	General Guidance for Designing, Testing, Operating, and Maintaining Emission	3.56
Control	General Use of Locks in the Protection and Control of Facilities and Special	5.12

KINOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Control	Security Seals for the Protection and Control of Special Nuclear Material*	5.15
Control	Limit of Error Concepts and Principles of Calculation in Nuclear Materials	5.18
Control	Selection of Material Balance Areas and Item Control Areas*	5.26
Control	Nuclear Material Control Systems for Nuclear Power Plants*	5.29
Control	Statistical Terminology and Notation for Special Nuclear Materials Control and	5.3
Control	Standard Format and Content for the Special Nuclear Material Control and	5.45
Control	Management Review of Nuclear Material Control and Accounting Systems (for	5.51
Control	Shipping and Receiving Control of Strategic Special Nuclear Material (Draft SG	5.57
Control	Entry/Exit Control for Protected Areas, Vital Areas, and Material Access Areas (5.7
Controls	Guidance to Operators at the Controls and to Senior Operators in the Control	1.116
Controls	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65
Coolant	Reactor Coolant Pump Flywheel Integrity (for Comment)*	1.14
Coolant	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Coolant	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Coolant	Reactor Coolant Pressure Boundary Leakage Detection Systems*	1.45
Coolant	Control of Combustible Gas Concentrations in Containment Following a Loss-of-	1.7
Coolant	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant	1.82
Cooled	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Cooled	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power	1.110
Cooled	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Cooled	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Cooled	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-	1.131
Cooled	Loose-Part Detection Program for the Primary System of Light-Water-Cooled	1.133
Cooled	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Cooled	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.143
Cooled	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Cooled	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated	1.37
Cooled	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Cooled	Housekeeping Requirements for Water-Cooled Nuclear Power Plants*	1.39
Cooled	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment	1.40
Cooled	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Cooled	Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled	1.54
Cooled	Initial Test Programs for Water-Cooled Reactor Power Plants*	1.68
Cooled	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Lister-	1.68.2
Cooled	Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and	1.97
Cooled	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	5.1
Cooled	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power	8.27
Cooling	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Cooling	Best-Estimate Calculations of Emergency Core Cooling System Performance (Draft	1.157
Cooling	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water	1.79
Cooling	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant	1.82
Core	LWR Core Reloads; Guidance on Applications for Amendments to Operating Licenses	SC 521-4
Core	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Core	Best-Estimate Calculations of Emergency Core Cooling System Performance (Draft	1.157
Core	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water	1.79
Corrosion	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Cost	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power	1.110
Counters	Standard Test Procedure for Geiger-Muller Counters*	8.6
Covers	Calculation of Radon Flux Attenuation by Earthen Uranium Mill Tailings Covers (3.64
Cranes	(Withdrawn - See 44 FR 49321, 8-21-79) See NUREG-0554, "Single-Failure-Proof	1.104
Criteria	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Criteria	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Criteria	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Criteria	Criteria for Programmable Digital Computer System Software in Safety-Related	1.152
Criteria	Criteria for Power, Instrumentation, and Control Portions of Safety Systems (1.153
Criteria	Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants*	1.32
Criteria	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Criteria	General Site Suitability Criteria for Nuclear Power Stations*	4.7
Criteria	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels*	7.6
Criteria	Film Badge Performance Criteria*	8.3
Criteria	Criteria for Establishing a Tritium Bioassay Program (Draft OP 713-4 published 6-	8.32
Criterion	Application of the Single-Failure Criterion to Nuclear Power Plant Protection	1.53
Critical	Guide for the Preparation of Applications for Special Nuclear Material Licenses	10.3
Criticality	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.33
Criticality	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.34
Criticality	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.35
Criticality	Nuclear Criticality Safety in Operations with Fissionable Materials as Fuels and	3.4
Criticality	Nuclear Criticality Safety in the Storage of Fissile Materials*	3.43
Criticality	Nuclear Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Criticality	Nuclear Criticality Control and Safety of Homogeneous Plutonium- Uranium Fuel	3.47
Criticality	Administrative Practices for Nuclear Criticality Safety at Fuels and Materials	3.57
Criticality	Criticality Safety for Handling, Storing, and Transporting LWR Fuel at Fuels and	3.58
Criticality	Criticality Accident Alarm Systems (Draft ON 015-4, Proposed Revision 1,	8.12
Criticality	Criticality and Other Interior Evacuation Signals*	8.5
Damping	Damping Values for Seismic Design of Nuclear Power Plants*	1.61
Decommissioning	Assuring the Availability of Funds for Decommissioning Nuclear Reactors*	DG-1003
Decommissioning	Records Important for Decommissioning for Licensees Under 10 CFR Parts 30, 40,	DG-3001
Decommissioning	Standard Format and Content of Decommissioning Plans for Licensees Under 10 CFR	3.65
Definitions	Quality Assurance Terms and Definitions*	1.74
Degraded	Bases for Plugging Degraded PWR Steam Generator Tubes (for Comment)*	1.121
Demonstrate	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-	1.68.2
Densification	An Acceptable Model and Related Statistical Methods for the Analysis of Fuel	1.126
Design	(Proposed Revision 2 to Regulatory Guide 1.13) Spent Fuel Storage Facility	CE 913-5
Design	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power	EM 865-5
Design	Design, Installation, and Inspection of Seepage Control Liners at Uranium	MS 146-4
Design	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Design	Tornado Design Classification*	1.117
Design	Development of Floor Design Response Spectra for Seismic Design of Floor-	1.122
Design	Physical Models for Design and Operation of Hydraulic Structures and Systems for	1.125
Design	Installation Design and Installation of Large Lead Storage Batteries for Nuclear	1.128
Design	Spent Fuel Storage Facility Design Basis (for Comment) (Draft CE 913-5, Proposed	1.13
Design	Laboratory Investigations of Soils for Engineering Analysis and Design of	1.138
Design	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Design	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.143
Design	Quality Assurance Program Requirements (Design and Construction) (Draft RS 002-	1.28
Design	Seismic Design Classification*	1.29
Design	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Design	Design Limits and Loading Combinations for Metal Primary Reactor Containment	1.57
Design	Design Basis Floods for Nuclear Power Plants (Errata published 7-30-80)*	1.59
Design	Design Response Spectra for Seismic Design of Nuclear Power Plants*	1.60
Design	Damping Values for Seismic Design of Nuclear Power Plants*	1.61
Design	Quality Assurance Requirements for the Design of Nuclear Power Plants*	1.64

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Design	Design Basis Tornado for Nuclear Power Plants*	1.76
Design	Design and Fabrication Code Case Acceptability-ASME Section III, Division 1*	1.14
Design	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Design	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Design	Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel	3.10
Design	Design, Construction, and Inspection of Embankment Retention Systems for Uranium	3.11
Design	General Design Guide for Ventilation Systems of Plutonium Processing and Fuel	3.12
Design	Seismic Design Classification for Plutonium Processing and Fuel Fabrication	3.14
Design	General Design Guide for Ventilation Systems for Fuel Reprocessing Plants (for	3.32
Design	Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing	3.40
Design	Design of an Independent Spent Fuel Storage Installation (Water-Basin Type) (3.49
Design	Applicability of Existing Regulatory Guides to the Design and Operation of an	3.53
Design	Design of an Independent Spent Fuel Storage Installation (Dry Storage) (Draft CE	3.60
Design	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.25
Design	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.42
Design	Design Considerations-Systems for Measuring the Mass of Liquids*	5.48
Design	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.8
Design	Design, Construction, and Use of Radioisotopic Power Generators for Certain Land	6.3
Design	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels*	7.6
Detection	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Detection	Specially Designed Vehicle with Armed Guards for Road Shipment of Special	5.31
Detection	General Guidance for Designing, Testing, Operating, and Maintaining Emission	3.56
Detection	Loose-Part Detection Program for the Primary System of Light-Water-Cooled	1.133
Detection	Reactor Coolant Pressure Boundary Leakage Detection Systems*	1.45
Services	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission	5.34
Devices	(Proposed Revision 2 to Regulatory Guide 10.6) Guide for the Preparation of	FC 401-4
Devices	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 404-4
Devices	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 405-4
Devices	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 407-4
Devices	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.10
Devices	Guide for the Preparation of Applications for Use of Sealed Sources and Devices	10.6
Devices	Efficiency Testing of Air-Cleaning Systems Containing Devices for Removal of	3.2
Devices	General Guidance for Designing, Testing, Operating, and Maintaining Emission	3.56
De-watering	Safety-Related Permanent De-watering Systems for Nuclear Power Plants*	FP 811-4
Diesel	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Diesel	Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems	1.108
Diesel	Fuel-Oil Systems for Standby Diesel Generators*	1.137
Diesel	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Differences	Evaluation of Shipper-Receiver Differences in the Transfer of Special Nuclear	5.28
Digital	Criteria for Programmable Digital Computer System Software in Safety-Related	1.152
Dioxide	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Discharge	Normal Water Level and Discharge at Nuclear Power Plants (for Comment)*	1.135
Dispersion	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Dispersion	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
Dispersion	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at	1.145
Dispersion	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.4
Disposal	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4
Disposal	Standard Format and Content of Environmental Reports for Near-Surface Disposal	4.18
Disposal	Guidance for Selecting Sites for Near-Surface Disposal of Low-Level Radioactive	4.19
Distribution	Guide for the Preparation of Applications for Licenses and Approvals to	FC 406-4

KWN INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Distribution	Independence Between Redundant, Standby (Onsite) Power Sources and Between Their	1.6
Distribution	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic	5.39
Division	Inservice Inspection Code Case Acceptability - ASME Section XI, Division 1 (1.147
Division	Design and Fabrication Code Case Acceptability-ASME Section III, Division 1*	1.84
Division	Materials Code Case Acceptability-ASME Section III, Division 1*	1.85
Doorway	Special Nuclear Material Doorway Monitors*	5.27
Dose	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Doses	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents	1.109
Doses	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Dosimeters	(Proposed Revision 2 to Regulatory Guide 8.14) Personnel Neutron Dosimeters*	ON 940-4
Dosimeters	Personnel Neutron Dosimeters (Draft ON 940-4, Proposed Revision 2, published 2-	8.14
Dosimeters	Audible-Alarm Dosimeters (Draft ON 304-4 published 8-79)*	8.28
Dosimeters	Direct-Reading and Indirect-Reading Pocket Dosimeters*	8.4
Dosimetry	Performance, Testing, and Procedural Specifications for Thermoluminescence	4.13
Dry	Guide for the Preparation of Applications for Licenses for the Use of Panoramic	FC 403-4
Dry	Guide for the Preparation of Applications for Licenses for the Use of Self-	10.9
Dry	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Dry	Design of an Independent Spent Fuel Storage Installation (Dry Storage) (Draft CE	3.60
Dry	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Dry	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.42
Drying	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.8
Duty	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment	1.40
Earthen	Calculation of Radon Flux Attenuation by Earthen Uranium Mill Tailings Covers (3.64
Earthquake	Earthquake Instrumentation for Fuel Reprocessing Plants*	3.17
Earthquakes	(Proposed Revision 2 to Regulatory Guide 1.12) Nuclear Power Plant	MS 140-5
Earthquakes	Instrumentation for Earthquakes (Draft MS 140-5, Proposed Revision 2, published	1.12
Edition	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (1.70
Effluent	Radiological Effluent and Environmental Monitoring at Uranium Mills*	4.14
Effluent	Quality Assurance for Radiological Monitoring Programs (Normal Operations)-	4.15
Effluent	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.4
Effluents	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents	1.109
Effluents	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Effluents	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Effluents	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
Effluents	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Effluents	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Ejection	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized	1.77
Electric	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Electric	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Electric	Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power	1.100
Electric	Thermal Overload Protection for Electric Motors on Motor-Operated Valves*	1.106
Electric	Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems	1.108
Electric	Periodic Testing of Electric Power and Protection Systems*	1.118
Electric	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-	1.131
Electric	Quality Assurance Requirements for the Installation, Inspection, and Testing of	1.30
Electric	Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants*	1.32
Electric	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify	1.41
Electric	Electric Penetration Assemblies in Containment Structures for Nuclear Power	1.63
Electric	Qualification Tests of Electric Valve Operators Installed Inside the Containment	1.73
Electric	Physical Independence of Electric Systems*	1.75
Electric	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power	1.81

KINOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Electric	Environmental Qualification of Certain Electric Equipment Important to Safety	1.89
Electric	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Electric	Availability of Electric Power Sources*	1.93
Electroslag	Control of Electroslag Weld Properties*	1.36
Embankment	Design, Construction, and Inspection of Embankment Retention Systems for Uranium	3.11
Embankment	Operational Inspection and Surveillance of Embankment Retention Systems for	3.11.1
Embrittlement	Radiation Embrittlement of Reactor Vessel Materials (Draft NE 305-4, Proposed	1.99
Emergency	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Emergency	Emergency Planning and Preparedness for Nuclear Power Reactors (Revision 1 to	1.101
Emergency	Best-Estimate Calculations of Emergency Core Cooling System Performance (Draft	1.157
Emergency	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water	1.79
Emergency	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power	1.81
Emergency	Emergency Planning for Research and Test Reactors (Draft NF 201-4, Proposed	2.6
Emergency	Emergency Water Supply Systems for Fuel Reprocessing Plants*	3.31
Emergency	Emergency Planning for Fuel Cycle Facilities and Plants Licensed Under 10 CFR	3.42
Emission	General Guidance for Designing, Testing, Operating, and Maintaining Emission	3.56
Employed	Radiation Protection: Training for Personnel Employed in Medical Facilities*	OP 212-4
Engineered	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Engineering	Laboratory Investigations of Soils for Engineering Analysis and Design of	1.138
Enriched	In Situ Assay of Enriched Uranium Residual Holdup (Draft SG 047-4, Proposed	5.37
Enriched	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel	8.24
Enrichment	Standard Format and Content of Safety Analysis Reports for Uranium Enrichment	3.25
Enrichment	Preparation of Environmental Reports for Commercial Uranium Enrichment Facilities	4.9
Enrichment	Nondestructive Uranium-235 Enrichment Assay by Gamma-Ray Spectrometry (Draft SG	5.21
Enrichment	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray	5.38
Enrichment	Standard Format and Content for the Special Nuclear Material Control and	5.45
Entry	Entry/Exit Control for Protected Areas, Vital Areas, and Material Access Areas (5.7
Environment	Quality Assurance for Radiological Monitoring Programs (Normal Operations)-	4.15
Environment	Measurements of Radionuclides in the Environment-Sampling and Analysis of	4.5
Environment	Measurements of Radionuclides in the Environment-Strontium-89 and Strontium-90	4.6
Environmental	Environmental Qualification of Connection Assemblies for Nuclear Power Plants (1.156
Environmental	Environmental Qualification of Certain Electric Equipment Important to Safety	1.89
Environmental	Standard Format and Content of License Applications, Including Environmental	3.46
Environmental	Preparation of Environmental Reports for Uranium Mills*	3.8
Environmental	Terrestrial Environmental Studies for Nuclear Power Stations*	4.11
Environmental	Performance, Testing, and Procedural Specifications for Thermoluminescence	4.13
Environmental	Radiological Effluent and Environmental Monitoring at Uranium Mills*	4.14
Environmental	Standard Format and Content of Environmental Reports for Near-Surface Disposal	4.18
Environmental	Preparation of Environmental Reports for Nuclear Power Stations*	4.2
Environmental	Environmental Technical Specification* for Nuclear Power Plants (for Comment)*	4.8
Environmental	Preparation of Environmental Reports for Commercial Uranium Enrichment Facilities	4.9
Environmental	Preparation of an Environmental Report to Support a Rule Making Petition Seeking	6.7
Equations	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program*	8.9
Equipment	Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power	1.100
Equipment	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.116
Equipment	Development of Floor Design Response Spectra for Seismic Design of Floor-	1.122
Equipment	Quality Assurance Requirements for the Installation, Inspection, and Testing of	1.30
Equipment	Environmental Qualification of Certain Electric Equipment Important to Safety	1.89
Equipment	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.25
Equipment	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.42
Equipment	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65

KINOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Error	Limit of Error Concepts and Principles of Calculation in Nuclear Materials	5.18
Error	Qualification, Calibration, and Error Estimation Methods for Nondestructive	5.53
Error	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air	8.25
Establishing	Criteria for Establishing a Tritium Bioassay Program (Draft OP 713-4 published 6-	8.32
Estimate	Best-Estimate Calculations of Emergency Core Cooling System Performance (Draft	1.157
Estimates	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Estimating	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Estimating	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
Estimating	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Estimating	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
Estimation	Qualification, Calibration, and Error Estimation Methods for Nondestructive	5.53
Evacuation	Criticality and Other Interior Evacuation Signals*	8.5
Evaluations	Evaluations of Explosions Postulated To Occur on Transportation Routes Near	1.91
Examinations	Nuclear Power Plant Simulation Facilities for Use in Operator License	1.149
Examinations	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice	1.150
Exempted	Acceptance Sampling Procedures for Exempted and Generally Licensed Items	6.6
Exemption	Preparation of an Environmental Report to Support a Rule Making Petition Seeking	6.7
Exemptions	Administrative Guide for Obtaining Exemptions from Certain NRC Requirements Over	7.5
Exhaust	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Existing	Applicability of Existing Regulatory Guides to the Design and Operation of an	3.53
Exit	Entry/Exit Control for Protected Areas, Vital Areas, and Material Access Areas (5.7
Explosions	Evaluations of Explosions Postulated To Occur on Transportation Routes Near	1.91
Exposure	Instruction Concerning Prenatal Radiation Exposure (Draft OP 031-4, Proposed	8.13
Exposure	Instruction Concerning Risks from Occupational Radiation Exposure (Draft OP 902-	8.29
Exposure	Occupational Radiation Exposure Records System*	8.7
Exposures	(Second Proposed Revision 4 to Regulatory Guide 8.8) Information Relevant to	OP 612-4
Exposures	Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As	8.10
Exposures	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.18
Exposures	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.31
Exposures	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.8
Fabrication	Design and Fabrication Code Case Acceptability-ASME Section III, Division 1*	1.84
Fabrication	Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel	3.10
Fabrication	General Design Guide for Ventilation Systems of Plutonium Processing and Fuel	3.12
Fabrication	Seismic Design Classification for Plutonium Processing and Fuel Fabrication	3.14
Fabrication	General Fire Protection Guide for Plutonium Processing and Fuel Fabrication	3.16
Fabrication	Quality Assurance Requirements for Protective Coatings Applied to Fuel	3.21
Fabrication	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28
Fabrication	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Fabrication	Quality Assurance Program Requirements for Fuel Reprocessing Plants and for	3.3
Fabrication	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.34
Fabrication	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.35
Fabrication	Standard Format and Content of License Applications for Plutonium Processing and	3.39
Fabrication	Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing	3.40
Fabrication	Standard Format and Content for the Health and Safety Sections of License	3.52
Fabrication	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel	3.7
Fabrication	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Fabrication	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel	8.24
Failure	(Withdrawn - See 44 FR 49321, 8-22-79) See NUREG-0554, "Single-Failure-Proof	1.104
Failure	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Failure	Application of the Single-Failure Criterion to Nuclear Power Plant Protection	1.53
Failure	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Feedwater	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems	1.68.1
Ferrite	Control of Ferrite Content in Stainless Steel Weld Metal*	1.31
Ferritic	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Ferritic	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Fiberglass	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin*	1.72
Film	Film Badge Performance Criteria*	8.3
Filtration	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Filtration	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Fire	Qualification Test for Cable Penetration Fire Stops for Use in Nuclear Power	RS 809-5
Fire	Fire Protection Guidelines for Nuclear Power Plants (for Comment)*	1.120
Fire	General Fire Protection Guide for Plutonium Processing and Fuel Fabrication	3.16
Fire	General Fire Protection Guide for Fuel Reprocessing Plants (for Comment)*	3.38
Fissile	Use of Borosilicate-Glass Raschig Rings as a Neutron Absorber in Solutions of	3.1
Fissile	Nuclear Criticality Safety in the Storage of Fissile Materials*	3.43
Fissile	Nuclear Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Fissile	Standard Format and Content of Part 71 Applications for Approval of Packaging of	7.9
Fission	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission	5.34
Fission	Applications of Bioassay for Fission and Activation Products (Draft DN 714-6	8.26
Fissionable	Nuclear Criticality Safety in Operations with Fissionable Materials at Fuels and	3.6
Fixed	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Fixed	Intent and Scope of the Physical Protection Upgrade Rule Requirements for Fixed	5.61
Flood	Flood Protection for Nuclear Power Plants*	1.102
Floods	Design Basis Floods for Nuclear Power Plants (Errata published 7-30-80)*	1.59
Floods	Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing	3.40
Floor	Development of Floor Design Response Spectra for Seismic Design of Floor-	1.122
Flow	Suggested Format for Cash Flow Statements Submitted as Guarantees of Payment of	9.6
Fluid	Containment Isolation Provisions for Fluid Systems (for Comment)*	1.141
Fluid	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated	1.37
Fluidized	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.8
Fluorescence	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 405-4
Flux	Calculation of Radon Flux Attenuation by Earthen Uranium Mill Tailings Covers (3.64
Flywheel	Reactor Coolant Pump Flywheel Integrity (for Comment)*	1.14
Force	Plant Security Force Duties*	5.43
Forces	(Proposed Regulatory Guide 1.35.1) Determining Prestressing Forces for	SC 807-4
Format	(Proposed Revision 2 to Regulatory Guide 7.9) Standard Format and Content of	FC 416-4
Format	Standard Format and Content Guide for Access Authorization Plans for Nuclear	SG 301-4
Format	(Proposed Revision 2 to Regulatory Guide 3.5) Standard Format and Content of	WM 039-4
Format	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis	1.154
Format	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (1.70
Format	Standard Format and Content of License Applications for Storage Only of	3.15
Format	Standard Format and Content of Safety Analysis Reports for Uranium Enrichment	3.25
Format	Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing	3.26
Format	Standard Format and Content of License Applications for Plutonium Processing and	3.39
Format	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Format	Standard Format and Content of License Applications, Including Environmental	3.46
Format	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Format	Standard Format and Content of License Applications for Uranium Mills (for	3.5
Format	Standard Format and Content for the Health and Safety Sections of License	3.52
Format	Standard Format and Content for the Health and Safety Sections of License	3.55
Format	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Format	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62

KWN INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Format	Standard Format and Content of Decommissioning Plans for Licensees Under 10 CFR	3.65
Format	Standard Format and Content of Site Characterization Plans for High-Level Waste	4.17
Format	Standard Format and Content of Environmental Reports for Near-Surface Disposal	4.18
Format	Standard Format and Content for the Special Nuclear Material Control and	5.45
Format	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Format	Standard Format and Content of Safeguards Contingency Plans for Nuclear Power	5.54
Format	Standard Format and Content of Safeguards Contingency Plans for Fuel Cycle	5.55
Format	Standard Format and Content of Safeguards Contingency Plans for Transportation	5.56
Format	Standard Format and Content for a Licensee Physical Security Plan for the	5.59
Format	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.60
Format	Standard Format and Content of Part 71 Applications for Approval of Packaging of	7.9
Format	Suggested Format for Cash Flow Statements Submitted as Guarantees of Payment of	9.4
Format	(Proposed Revision 2 to Regulatory Guide 1.96) Quality Assurance Requirements	RS 908-5
Foundations	Site Investigations for Foundations of Nuclear Power Plants*	1.132
Foundations	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Fracture	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Fuel	(Proposed Revision 1 to Regulatory Guide 3.50) Guidance on Preparing a License	CE 402-4
Fuel	(Proposed Revision 2 to Regulatory Guide 1.13) Spent Fuel Storage Facility	CE 913-5
Fuel	Training and Certification of Independent Spent Fuel Storage Installation	NF 608-4
Fuel	Qualifications for the Radiation Safety Officer in a Large-Scale Non-Fuel-Cycle	OP 722-4
Fuel	An Acceptable Model and Related Statistical Methods for the Analysis of Fuel	1.126
Fuel	Spent Fuel Storage Facility Design Basis (for Comment) (Draft CE 913-5, Proposed	1.13
Fuel	Fuel-Oil Systems for Standby Diesel Generators*	1.137
Fuel	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Fuel	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in	2.3
Fuel	Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel	3.10
Fuel	General Design Guide for Ventilation Systems of Plutonium Processing and Fuel	3.12
Fuel	Seismic Design Classification for Plutonium Processing and Fuel Fabrication	3.14
Fuel	Standard Format and Content of License Applications for Storage Only of	3.15
Fuel	General Fire Protection Guide for Plutonium Processing and Fuel Fabrication	3.16
Fuel	Earthquake Instrumentation for Fuel Reprocessing Plants*	3.17
Fuel	Containment Barriers and Systems for Fuel Reprocessing Plants*	3.18
Fuel	Reporting of Operating Information for Fuel Reprocessing Plants*	3.19
Fuel	Process Offgas Systems for Fuel Reprocessing Plants*	3.20
Fuel	Quality Assurance Requirements for Protective Coatings Applied to Fuel	3.21
Fuel	Periodic Testing of Fuel Reprocessing Plant Protection System Actuation Functions	3.22
Fuel	Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing	3.26
Fuel	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel	3.27
Fuel	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28
Fuel	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Fuel	Quality Assurance Program Requirements for Fuel Reprocessing Plants and for	3.3
Fuel	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel	3.30
Fuel	Emergency Water Supply Systems for Fuel Reprocessing Plants*	3.31
Fuel	General Design Guide for Ventilation Systems for Fuel Reprocessing Plants (for	3.32
Fuel	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.33
Fuel	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.34
Fuel	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.35
Fuel	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Fuel	General Fire Protection Guide for Fuel Reprocessing Plants (for Comment)*	3.38
Fuel	Standard Format and Content of License Applications for Plutonium Processing and	3.39
Fuel	Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing	3.40

KWDG INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Fuel	Emergency Planning for Fuel Cycle Facilities and Plants Licensed Under 10 CFR	3.42
Fuel	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Fuel	Nuclear Criticality Control and Safety of Homogeneous Plutonium-Uranium Fuel	3.47
Fuel	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Fuel	Design of an Independent Spent Fuel Storage Installation (Water-Basin Type) (3.49
Fuel	Guidance on Preparing a License Application to Store Spent Fuel in an	3.50
Fuel	Standard Format and Content for the Health and Safety Sections of License	3.52
Fuel	Applicability of Existing Regulatory Guides to the Design and Operation of an	3.53
Fuel	Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation (3.54
Fuel	Criticality Safety for Handling, Storing, and Transporting LWR Fuel at Fuels and	3.58
Fuel	Content of Technical Specifications for Fuel Reprocessing Plants*	3.6
Fuel	Design of an Independent Spent Fuel Storage Installation (Dry Storage) (Draft CE	3.60
Fuel	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Fuel	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62
Fuel	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel	3.7
Fuel	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Fuel	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	5.1
Fuel	Materials Protection Contingency Measures for Uranium and Plutonium Fuel	5.30
Fuel	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray	5.38
Fuel	Standard Format and Content of Safeguards Contingency Plans for Fuel Cycle	5.55
Fuel	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel	8.24
Fuels	Nuclear Criticality Safety in Operations with Fissionable Materials at Fuels and	3.4
Fuels	Administrative Practices for Nuclear Criticality Safety a. Fuels and Materials	3.57
Fuels	Criticality Safety for Handling, Storing, and Transporting LWR Fuel at Fuels and	3.58
Functional	Functional Specification for Active Valve Assemblies in Systems Important to	1.148
Functions	Periodic Testing of Protection System Actuation Functions (Safety Guide 22)*	1.22
Functions	Periodic Testing of Fuel Reprocessing Plant Protection System Actuation Functions	3.22
Funds	Assuring the Availability of Funds for Decommissioning Nuclear Reactors*	DG-1003
Gamma	Guide for the Preparation of Applications for Licenses for the Use of Self-	10.9
Gamma	Nondestructive Uranium-235 Enrichment Assay by Gamma-Ray Spectrometry (Draft SG	5.21
Gamma	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray	5.38
Gamma	General Safety Standard for Installations Using Nonmedical Sealed Gamma-Ray	6.5
Gas	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 405-4
Gas	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Gas	Control of Combustible Gas Concentrations in Containment Following a Loss-of-	1.7
Gaseous	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Gaseous	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Gaseous	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Gaseous	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Gases	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel	3.7
Gauges	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-4
Gauges	Safety Features of Gauges Containing Radioactive Material*	TP 102-5
Gauging	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 406-4
Gauging	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 407-4
Geiger	Standard Test Procedure for Geiger-Muller Counters*	8.6
Generally	Acceptance Sampling Procedures for Exempted and Generally Licensed Items	6.6
Generation	Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation (3.54
Generator	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Generator	Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems	1.108
Generator	Bases for Plugging Degraded PWR Steam Generator Tubes (for Comment)*	1.121
Generator	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes*	1.83

INDOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Generator	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Generators	Fuel-Oil Systems for Standby Diesel Generators*	1.137
Generators	Design, Construction, and Use of Radioisotopic Power Generators for Certain Land	6.3
Geologic	Standard Format and Content of Site Characterization Plans for High-Level-Waste	4.17
Germanium	Guidelines for Germanium Spectroscopy Systems for Measurement of Special Nuclear	5.9
Glass	Use of Borosilicate-Glass Raschig Rings as a Neutron Absorber in Solutions of	3.1
Ground	Guidelines for Ground-Water Monitoring at In Situ Uranium Solution Mines*	ES 114-4
Ground	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4
Grouted	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted	1.90
Grouting	Qualifications for Cement Grouting for Prestressing Tendons in Containment	1.107
Guarantees	Suggested Format for Cash Flow Statements Submitted as Guarantees of Payment of	9.4
Guards	Training, Equipping, and Qualifying of Guards and Watchmen*	5.20
Guards	Specially Designed Vehicle with Armed Guards for Road Shipment of Special	5.31
Habitability	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control	1.78
Hazardous	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control	1.78
Head	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Health	Standard Format and Content for the Health and Safety Sections of License	3.52
Health	Standard Format and Content for the Health and Safety Sections of License	3.55
Health	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and	8.21
Health	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel	8.24
Health	Health Physics Surveys in Uranium Mills (Draft OH 710-4 published 8-80)*	8.30
Heat	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Heat	Guidance for Residual Heat Removal (for Comment)*	1.139
Heat	Ultimate Heat Sink for Nuclear Power Plants (for Comment)*	1.27
Heat	Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation (3.54
Heated	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.4
Hexafluoride	Standard Format and Content for the Health and Safety Sections of License	3.55
Hexafluoride	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Hexafluoride	Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF4)	5.4
Holdup	In Situ Assay of Plutonium Residual Holdup (Draft SG 045-4, Proposed Revision 1,	5.23
Holdup	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.25
Holdup	In Situ Assay of Enriched Uranium Residual Holdup (Draft SG 047-4, Proposed	5.37
Holdup	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.42
Holdup	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.8
Housekeeping	Housekeeping Requirements for Water-Cooled Nuclear Power Plants*	1.39
Hydraulic	Physical Models for Design and Operation of Hydraulic Structures and Systems for	1.125
Identification	Identification of Valves for Inclusion in Inservice Testing Programs*	MS 901-4
Identification	Truck Identification Markings*	5.17
Identification	Identification Plaque for Irretrievable Well-Logging Sources (for Comment)*	6.8
Implementing	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
Impurity	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic	5.39
Independence	Independence Between Redundant Standby (Onsite) Power Sources and Between Their	1.6
Independence	Physical Independence of Electric Systems*	1.75
Indication	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems*	1.47
Indirect	Direct-Reading and Indirect-Reading Pocket Dosimeters*	8.4
Individual	Assessment of the Assumption of Normality (Employing Individual Observed Values)*	5.22
Industrial	(Proposed Revision 2 to Regulatory Guide 10.6) Guide for the Preparation of	FC 401-4
Industrial	Protection of Nuclear Power Plants Against Industrial Sabotage*	1.17
Industrial	Guide for the Preparation of Applications for Use of Sealed Sources and Devices	10.6
Industrial	Guides for the Preparation of Applications for Licenses for Laboratory and	10.7
Inoperable	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems*	1.47

KINOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Inservice	Identification of Valves for Inclusion in Inservice Testing Programs*	MS 901-4
Inservice	(Proposed Revision 3 to Regulatory Guide 1.35) Inservice Inspection of Ungrouted	SC 810-4
Inservice	Inservice Inspection Code Case Acceptability - ASME Section XI, Division 1 (1.147
Inservice	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice	1.150
Inservice	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containment	1.35
Inservice	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes*	1.83
Inservice	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted	1.90
Inspection	Design, Installation, and Inspection of Seepage Control Liners at Uranium	MS 146-4
Inspection	(Proposed Revision 2 to Regulatory Guide 1.94) Quality Assurance Requirements	RS 908-5
Inspection	(Proposed Regulatory Guide 1.35.1) Determining Prestressing Forces for	SC 807-4
Inspection	(Proposed Revision 3 to Regulatory Guide 1.35) Inservice Inspection of Ungrouted	SC 810-4
Inspection	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.116
Inspection	Inspection of Water-Control Structures Associated with Nuclear Power Plants*	1.127
Inspection	Inservice Inspection Code Case Acceptability - ASME Section XI, Division 1 (1.147
Inspection	Quality Assurance Requirements for the Installation, Inspection, and Testing of	1.30
Inspection	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containment	1.35
Inspection	Qualification of Nuclear Power Plant Inspection, Examination, and Testing	1.58
Inspection	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes*	1.83
Inspection	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted	1.90
Inspection	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.94
Inspection	Design, Construction, and Inspection of Embankment Retention Systems for Uranium	3.11
Inspection	Operational Inspection and Surveillance of Embankment Retention Systems for	3.11.1
Inspection	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel	3.30
Inspections	Materials and Inspections for Reactor Vessel Closure Studs*	1.65
Installation	Training and Certification of Independent Spent Fuel Storage Installation	NF 608-4
Installation	Installation of Transducers*	IC 131-5
Installation	Design, Installation, and Inspection of Seepage Control Liners at Uranium	MS 146-4
Installation	(Proposed Revision 2 to Regulatory Guide 1.94) Quality Assurance Requirements	RS 908-5
Installation	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.116
Installation	Installation Design and Installation of Large Lead Storage Batteries for Nuclear	1.128
Installation	Quality Assurance Requirements for the Installation, Inspection, and Testing of	1.30
Installation	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.94
Installation	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Installation	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Installation	Design of an Independent Spent Fuel Storage Installation (Water-Basin Type) (3.49
Installation	Guidance on Preparing a License Application to Store Spent Fuel in an	3.50
Installation	Applicability of Existing Regulatory Guides to the Design and Operation of an	3.53
Installation	Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation (3.54
Installation	Design of an Independent Spent Fuel Storage Installation (Dry Storage) (Draft CE	3.60
Installations	General Safety Standard for Installations Using Nonmedical Sealed Gamma-Ray	6.5
Institutions	Guidance to Academic Institutions Applying for Specific Byproduct Material	10.2
Institutions	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.18
Institutions	Radiation Safety Surveys at Medical Institutions*	8.23
Instrument	Response-Time Testing of Protection System Instrument Channels*	IC 121-5
Instrument	Instrument Setpoints for Safety-Related Systems (Draft IC 010-5, Proposed	1.105
Instrument	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	1.11
Instrument	Instrument Sensing Lines (Draft IC 126-5, published 3-82)*	1.151
Instrument	Preoperational Testing of Instrument and Control Air Systems (Draft RS 709-4, a	1.68.3
Instrumentation	(Proposed Revision 2 to Regulatory Guide 1.12) Nuclear Power Plant	MS 140-5
Instrumentation	Test and Calibration of Radiation Protection Instrumentation*	OP 032-5
Instrumentation	Instrumentation for Earthquakes (Draft MS 140-5, Proposed Revision 2, published	1.12

NWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Instrumentation	Criteria for Power, Instrumentation, and Control Portions of Safety Systems (1.153
Instrumentation	Quality Assurance Requirements for the Installation, Inspection, and Testing of	1.30
Instrumentation	Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and	1.97
Instrumentation	Earthquake Instrumentation for Fuel Reprocessing Plants*	5.17
Instruments	Guide for the Preparation of Applications for Licenses for the Use of	FC 413-4
Instruments	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air	8.25
Insulation	Nonmetallic Thermal Insulation for Austenitic Stainless Steel*	1.36
Integrity	Reactor Coolant Pump Flywheel Integrity (for Comment)*	1.14
Integrity	Integrity and Test Specifications for Selected Brachytherapy Sources*	6.2
Intergranular	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Internal	Internal Transfers of Special Nuclear Material (for Comment)*	5.49
Internals	Comprehensive Vibration Assessment Program for Reactor Internals During	1.20
Interpass	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Intersections	Nuclear Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Intrusion	Perimeter Intrusion Alarm Systems (Draft SG 479-4, Proposed Revision 2,	5.44
Inventories	Conduct of Nuclear Material Physical Inventories*	5.13
Irradiators	Guide for the Preparation of Applications for Licenses for the Use of Panoramic	FC 403-4
Irradiators	Guide for the Preparation of Applications for Licenses for the Use of Self-	10.9
Irretrievable	Identification Plaque for Irretrievable Well-Logging Sources (for Comment)*	6.8
Isolation	Containment Isolation Provisions for Fluid Systems (for Comment)*	1.141
Isoletion	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Isotopic	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic	5.39
Item	Selection of Material Balance Areas and Item Control Areas*	5.26
Items	Guide for the Preparation of Applications for Licenses and Approvals to	FC 606-4
Items	Quality Assurance Requirements for Control of Procurement of Items and Services	1.123
Items	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Items	Acceptance Sampling Procedures for Exempted and Generally Licensed Items	6.6
Key	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65
Laboratory	Laboratory Investigations of Soils for Engineering Analysis and Design of	1.138
Laboratory	Guide for the Preparation of Applications for Licenses for Laboratory and	10.7
Land	Design, Construction, and Use of Radioisotopic Power Generators for Certain Land	6.3
Lead	Installation Design and Installation of Large Lead Storage Batteries for Nuclear	1.128
Lead	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for	1.129
Lead	Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants (1.158
Leak	Guide for the Preparation of Applications for Licenses for the Use of	FC 412-4
Leak	Leak Testing Radioactive Brachytherapy Sources*	6.1
Leakage	Containment System Leakage Testing*	MS 021-5
Leakage	Reactor Coolant Pressure Boundary Leakage Detection Systems*	1.45
Leakage	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Leakage	Leakage Tests on Packages for Shipment of Radioactive Materials (for Comment)*	7.4
License	(Proposed Revision 1 to Regulatory Guide 3.50) Guidance on Preparing a License	CE 402-4
License	(Proposed Revision 2 to Regulatory Guide 3.5) Standard Format and Content of	WM 039-4
License	Nuclear Power Plant Simulation Facilities for Use in Operator License	1.149
License	Standard Format and Content of License Applications for Storage Only of	3.15
License	Standard Format and Content of License Applications for Plutonium Processing and	3.39
License	Standard Format and Content of License Applications, Including Environmental	3.46
License	Standard Format and Content of License Applications for Uranium Mills (for	3.5
License	Guidance on Preparing a License Application to Store Spent Fuel in an	3.50
License	Standard Format and Content for the Health and Safety Sections of License	3.52
License	Standard Format and Content for the Health and Safety Sections of License	3.55
License	Standard Format and Content for the Special Nuclear Material Control and	5.45

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

License	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust	9.3
Licensed	Medical Evaluation of Licensed Personnel for Nuclear Power Plants (Draft OL 401-	1.134
Licensed	Emergency Planning for Fuel Cycle Facilities and Plants Licensed Under 10 CFR	3.42
Licensed	Acceptance Sampling Procedures for Exempted and Generally Licensed Items	6.6
Licensed	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and	8.21
Licensee	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Licensee	Standard Format and Content for a Licensee Physical Security Plan for the	5.59
Licensee	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.60
Licensees	Records Important for Decommissioning for Licensees Under 10 CFR Parts 30, 40,	DG-3001
Licensees	Guide for the Preparation of Applications for Licenses and Approvals to	FC 406-4
Licensees	Standard Format and Content of Decommissioning Plans for Licensees Under 10 CFR	3.65
Licenses	Guide for the Preparation of Applications for Licenses for the Use of Panoramic	FC 403-4
Licenses	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 404-4
Licenses	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 405-4
Licenses	Guide for the Preparation of Applications for Licenses and Approvals to	FC 406-4
Licenses	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 407-4
Licenses	(Proposed Revision 2 to Regulatory Guide 10.5) Guide for the Preparation of	FC 408-4
Licenses	Guide for the Preparation of Applications for Nuclear Pharmacy Licenses*	FC 410-4
Licenses	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-4
Licenses	Guide for the Preparation of Applications for Licenses for the Use of	FC 412-4
Licenses	Guide for the Preparation of Applications for Licenses for the Use of	FC 413-4
Licenses	Guide for the Preparation of Applications for Licenses for Medical Teletherapy	FC 414-4
Licenses	LWR Core Reloads; Guidance on Applications for Amendments to Operating Licenses	SC 521-4
Licenses	Termination of Operating Licenses for Nuclear Reactors*	1.86
Licenses	Guidance to Academic Institutions Applying for Specific Byproduct Material	10.2
Licenses	Guide for the Preparation of Applications for Special Nuclear Material Licenses	10.3
Licenses	Guide for the Preparation of Applications for Licenses to Process Source	10.4
Licenses	Applications for Type A Licenses of Broad Scope (Errata published 7-84) (Draft	10.5
Licenses	Guide for the Preparation of Applications for Licenses for Laboratory and	10.7
Licenses	Guide for the Preparation of Applications for Licenses for the Use of Self-	10.9
Light	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Light	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power	1.110
Light	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Light	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Light	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-	1.131
Light	Loose-Part Detection Program for the Primary System of Light-Water-Cooled	1.133
Light	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Light	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.143
Light	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Light	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Light	Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and	1.97
Light	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	5.1
Light	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Light	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power	8.27
Lightning	Lightning Protection for Nuclear Power Plants*	RS 705-4
Limit	Limit of Error Concepts and Principles of Calculation in Nuclear Materials	5.18
Limited	Welder Qualification for Areas of Limited Accessibility*	1.71
Limited	Guidance to Academic Institutions Applying for Specific Byproduct Material	10.2
Limited	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28
Limits	Service Limits and Loading Combinations for Class 1 Linear-Type Component	1.124
Limits	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type	1.130

KDOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Limits	Design Limits and Loading Combinations for Metal Primary Reactor Containment	1.57
Limits	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air	8.25
Line	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Linear	Service Limits and Loading Combinations for Class 1 Linear-Type Component	1.124
Liners	Design, Installation, and Inspection of Seepage Control Liners at Uranium	MS 146-4
Liners	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel	3.27
Lines	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	1.11
Lines	Instrument Sensing Lines (Draft IC 126-5, published 3-82)*	1.151
Liquid	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Liquid	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Liquid	Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel	3.10
Liquid	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Liquids	Design Considerations-Systems for Measuring the Mass of Liquids*	5.48
Load	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify	1.41
Load	Load Combinations for the Structural Analysis of Shipping Casks for Radioactive	7.8
Loading	Service Limits and Loading Combinations for Class 1 Linear-Type Component	1.124
Loading	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type	1.130
Loading	Design Limits and Loading Combinations for Metal Primary Reactor Containment	1.57
Lock	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65
Locks	General Use of Locks in the Protection and Control of Facilities and Special	5.12
Logging	Identification Plaque for Irretrievable Well-Logging Sources (for Comment)*	6.8
Loose	Loose-Part Detection Program for the Primary System of Light-Water-Cooled	1.133
Loss	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Loss	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Loss	Control of Combustible Gas Concentrations in Containment Following a Loss-of-	1.7
Loss	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant	1.82
LWR	LWR Core Reloads; Guidance on Applications for Amendments to Operating Licenses	SC 521-4
LWR	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (1.70
LWR	Criticality Safety for Handling, Storing, and Transporting LWR Fuel at Fuels and	3.58
Maintaining	General Guidance for Designing, Testing, Operating, and Maintaining Emission	3.56
Maintaining	Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As	8.10
Maintenance	Maintenance Programs for Nuclear Power Plants*	DG-100?
Maintenance	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for	1.129
Maintenance	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Maintenance	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Maintenance	Maintenance of Water Purity in Boiling Water Reactors (for Comment)*	1.56
Maintenance	Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance	1.88
Man	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents	1.109
Man	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Man	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Management	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.143
Management	Management Review of Nuclear Material Control and Accounting Systems (for	5.51
Manual	Manual Initiation of Protective Actions*	1.62
Manufacturing	Materials Protection Contingency Measures for Uranium and Plutonium Fuel	5.30
Manufacturing	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and	8.21
Markings	Truck Identification Markings*	5.17
Mass	Guide for the Preparation of Applications for Special Nuclear Material Licenses	10.3
Mass	Design Considerations-Systems for Measuring the Mass of Liquids*	5.48
Mass	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Material	(Proposed Revision 2 to Regulatory Guide 7.9) Standard Format and Content of	FC 416-4
Material	Safety Features of Gauges Containing Radioactive Material*	TP 102-5

KHOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Material	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.10
Material	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.11
Material	Guidance to Academic Institutions Applying for Specific Byproduct Material	10.2
Material	Guide for the Preparation of Applications for Special Nuclear Material Licenses	10.3
Material	Guide for the Preparation of Applications for Licenses to Process Source	10.4
Material	Guide for the Preparation of Applications for Licenses for Laboratory and	10.7
Material	Use of Borosilicate-Glass Raschig Rings as a Neutron Absorber in Solutions of	3.1
Material	Standard Format and Content of License Applications for Storage Only of	3.15
Material	Selection and Use of Pressure-Sensitive Seals on Containers for Onsite Storage	5.10
Material	Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste (5.11
Material	Conduct of Nuclear Material Physical Inventories*	5.13
Material	Use of Observation (Visual Surveillance) Techniques in Material Access Areas (5.14
Material	Security Seals for the Protection and Control of Special Nuclear Material*	5.15
Material	Analysis and Use of Process Data for the Protection of Special Nuclear Material*	5.24
Material	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.25
Material	Selection of Material Balance Areas and Item Control Areas*	5.26
Material	Special Nuclear Material Doorway Monitors*	5.27
Material	Nuclear Material Control Systems for Nuclear Power Plants*	5.29
Material	Specially Designed Vehicle with Armed Guards for Road Shipment of Special	5.31
Material	Statistical Evaluation of Material Unaccounted For*	5.33
Material	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission	5.34
Material	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.42
Material	Standard Format and Content for the Special Nuclear Material Control and	5.45
Material	Internal Transfers of Special Nuclear Material (for Comment)*	5.49
Material	Management Review of Nuclear Material Control and Accounting Systems (for	5.51
Material	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Material	Shipping and Receiving Control of Strategic Special Nuclear Material (Draft SG	5.57
Material	Considerations for Establishing Traceability of Special Nuclear Material	5.58
Material	Standard Format and Content for a Licensee Physical Security Plan for the	5.59
Material	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.60
Material	Entry/Exit Control for Protected Areas, Vital Areas, and Material Access Areas (5.7
Material	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.8
Material	Guidelines for Germanium Spectroscopy Systems for Measurement of Special Nuclear	5.9
Material	Acceptance Sampling Procedures for Exempted and Generally Licensed Items	6.6
Material	Administrative Guide for Packaging and Transporting Radioactive Material*	7.1
Material	Establishing Quality Assurance Programs for Packaging Used in the Transport of	7.10
Material	Procedures for Picking Up and Receiving Packages of Radioactive Material (for	7.3
Material	Administrative Guide for Obtaining Exemptions from Certain NRC Requirements Over	7.5
Material	Load Combinations for the Structural Analysis of Shipping Casks for Radioactive	7.8
Material	Standard Format and Content of Part 71 Applications for Approval of Packaging of	7.9
Material	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and	8.21
Materials	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-4
Materials	Guide for the Preparation of Applications for Licenses for the Use of	FC 412-4
Materials	Guide for the Preparation of Applications for Licenses for the Use of	FC 413-4
Materials	Measurement of Radiation Levels on Surfaces of Packages of Radioactive Materials*	TP 914-4
Materials	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Materials	Materials, Construction, and Testing of Concrete Containments (Articles CC-1000,	1.136
Materials	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Materials	Materials and Inspections for Reactor Vessel Closure Studs*	1.65
Materials	Materials Code Case Acceptability-ASME Section III, Division 1*	1.85
Materials	Radiation Embrittlement of Reactor Vessel Materials (Draft ME 305-4, Proposed	1.99

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Materials	Nuclear Criticality Safety in Operations with Fissionable Materials at Fuels and	3.4
Materials	Nuclear Criticality Safety in the Storage of Fissile Materials*	3.43
Materials	Nuclear Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Materials	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Materials	Administrative Practices for Nuclear Criticality Safety at Fuels and Materials	3.57
Materials	Criticality Safety for Handling, Storing, and Transporting LWR Fuel at Fuels and	3.58
Materials	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Materials	General Use of Locks in the Protection and Control of Facilities and Special	5.12
Materials	Limit of Error Concepts and Principles of Calculation in Nuclear Materials	5.18
Materials	Evaluation of Shipper-Receiver Differences in the Transfer of Special Nuclear	5.28
Materials	Statistical Terminology and Notation for Special Nuclear Materials Control and	5.3
Materials	Materials Protection Contingency Measures for Uranium and Plutonium Fuel	5.30
Materials	Packaging and Transportation of Radioactively Contaminated Biological Materials*	7.2
Materials	Leakage Tests on Packages for Shipment of Radioactive Materials (for Comment)*	7.4
Materials	Administrative Guide for Verifying Compliance with Packaging Requirements for	7.7
Mathematical	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.4
Matters	Regulatory Staff Position Statement on Antitrust Matters*	9.1
Maximum	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Measurement	(Second Proposed Revision 1 to Regulatory Guide 1.23) Meteorological Measurement	ES 926-4
Measurement	Measurement of Radiation Levels on Surfaces of Packages of Radioactive Materials*	TP 914-4
Measurement	Onsite Meteorological Measurement Program for Uranium Recovery Facilities- Data	3.63
Measurement	Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF4)	5.4
Measurements	Guidelines for Germanium Spectroscopy Systems for Measurement of Special Nuclear	5.9
Measurements	Measurements of Radionuclides in the Environment-Sampling and Analysis of	4.5
Measurements	Measurements of Radionuclides in the Environment-Strontium-89 and Strontium-90	4.6
Measures	Considerations for Establishing Traceability of Special Nuclear Material	5.58
Measuring	Materials Protection Contingency Measures for Uranium and Plutonium Fuel	5.30
Measuring	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-4
Measuring	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Mechanical	Design Considerations-Systems for Measuring the Mass of Liquids*	5.48
Mechanical	Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power	1.100
Medical	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.116
Medical	Guide for the Preparation of Applications for Licenses and Approvals to	FC 406-4
Medical	Guide for the Preparation of Applications for Licenses for Medical Teletherapy	FC 414-4
Medical	Radiation Protection Training for Personnel Employed in Medical Facilities*	OP 212-4
Medical	Medical Evaluation of Licensed Personnel for Nuclear Power Plants (Draft OL 401-	1.134
Medical	Guide for Preparation of Applications for Medical Use Programs (Draft FC 415-4,	10.8
Medical	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.18
Medical	Radiation Safety Surveys at Medical Institutions*	8.23
Metal	Control of Ferrite Content in Stainless Steel Weld Metal*	1.31
Metal	Design Limits and Loading Combinations for Metal Primary Reactor Containment	1.57
Meteorological	(Second Proposed Revision 1 to Regulatory Guide 1.23) Meteorological Measurement	ES 926-4
Meteorological	Onsite Meteorological Programs (Safety Guide 23) (Draft SS 926-4, Proposed	1.23
Meteorological	Onsite Meteorological Measurement Program for Uranium Recovery Facilities- Data	3.63
Hill	Operational Inspection and Surveillance of Embankment Retention Systems for	3.11.1
Hill	Calculation of Radon Flux Attenuation by Earthen Uranium Mill Tailings Covers (3.64
Milling	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Milling	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
Mills	(Proposed Revision 2 to Regulatory Guide 3.5) Standard Format and Content of	WM 039-4
Mills	Design, Construction, and Inspection of Embankment Retention Systems for Uranium	3.11
Mills	Standard Format and Content of License Applications for Uranium Mills (for	3.5

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Mills	General Guidance for Designing, Testing, Operating, and Maintaining Emission	3.56
Mills	Preparation of Environmental Reports for Uranium Mills*	3.8
Mills	Radiological Effluent and Environmental Monitoring at Uranium Mills*	4.14
Mills	Bioassay at Uranium Mills (Draft OP 013-4, Proposed Revision 1, published 1-87)*	8.22
Mills	Health Physics Surveys in Uranium Mills (Draft OH 710-4 published 8-80)*	8.30
Mills	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.31
Mines	Guidelines for Ground-Water Monitoring at In Situ Uranium Solution Mines*	ES 114-4
Minimizing	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.25
Minimizing	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.42
Minimizing	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.8
Mining	Standard Format and Content of License Applications, Including Environmental	3.46
Missiles	Protection Against Low-Trajectory Turbine Missiles*	1.115
Mixtures	Nuclear Criticality Control and Safety of Homogeneous Plutonium- Uranium Fuel	3.47
Modal	Combining Modal Responses and Spatial Components in Seismic Response Analysis*	1.92
Modal	An Acceptable Model and Related Statistical Methods for the Analysis of Fuel	1.126
Modeling	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4
Models	Physical Models for Design and Operation of Hydraulic Structures and Systems for	1.125
Models	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at	1.145
Models	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Models	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.6
Models	Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program*	8.9
Monitored	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Monitoring	Guidelines for Ground-Water Monitoring at In Situ Uranium Solution Mines*	ES 114-4
Monitoring	Guide for the Preparation of Applications for Licenses for the Use of	FC 413-4
Monitoring	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel	3.7
Monitoring	Programs for Monitoring Radioactivity in the Environs of Nuclear Power Plants*	4.1
Monitoring	Radiological Effluent and Environmental Monitoring at Uranium Mills*	4.14
Monitoring	Quality Assurance for Radiological Monitoring Programs (Normal Operations)-	4.15
Monitoring	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Monitoring	Guide for Administrative Practices in Radiation Monitoring*	8.2
Monitors	Special Nuclear Material Doorway Monitors*	5.27
Motor	Thermal Overload Protection for Electric Motors on Motor-Operated Valves*	1.106
Motors	Thermal Overload Protection for Electric Motors on Motor-Operated Valves*	1.106
Motors	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment	1.40
Muller	Standard Test Procedure for Geiger-Muller Counters*	8.6
Natural	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.6
Net	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Neutron	(Proposed Revision 2 to Regulatory Guide 8.14) Personnel Neutron Dosimeters*	OH 940-4
Neutron	Use of Borosilicate-Glass Rasching Rings as a Neutron Absorber in Solutions of	3.1
Neutron	Personnel Neutron Dosimeters (Draft OH 940-4, Proposed Revision 2, published 2-	8.14
Nitrate	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic	5.39
Nondestructive	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel	3.27
Nondestructive	Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste (5.11
Nondestructive	Nondestructive Uranium-235 Enrichment Assay by Gamma-Ray Spectrometry (Draft SG	5.21
Nondestructive	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission	5.34
Nondestructive	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray	5.38
Nondestructive	Qualification, Calibration, and Error Estimation Methods for Nondestructive	5.53
Nonmedical	General Safety Standard for Installations Using Nonmedical Sealed Gamma-Ray	6.5
Nonmetallic	Nonmetallic Thermal Insulation for Austenitic Stainless Steel*	1.36
Nonportable	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 404-4
Nonradioactive	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4

KODC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Normality	Assessment of the Assumption of Normality (Employing Individual Observed Values)*	5.22
Notation	Statistical Terminology and Notation for Special Nuclear Materials Control and	5.3
NRC	Compilation of Reporting Requirements for Persons Subject to NRC Regulations*	10.1
NRC	Administrative Guide for Obtaining Exemptions from Certain NRC Requirements Over	7.5
NRC	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and	8.21
NRC	Information Needed by the NRC Staff in Connection with Its Antitrust Review of	9.2
Numbering	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	5.1
Occupational	(Second Proposed Revision 4 to Regulatory Guide 8.8) Information Relevant to	OP 618-4
Occupational	Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As	8.10
Occupational	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.18
Occupational	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Occupational	Instruction Concerning Risks from Occupational Radiation Exposure (Draft OH 902-	8.29
Occupational	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.31
Occupational	Occupational Radiation Exposure Records Systems*	8.7
Occupational	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.8
Offgas	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98
Offgas	Process Offgas Systems for Fuel Reprocessing Plants*	3.20
Officer	Qualifications for the Radiation Safety Officer in a Large-Scale Non-Fuel-Cycle	OP 722-4
Oil	Fuel-Oil Systems for Standby Diesel Generators*	1.137
Onsite	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Onsite	Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems	1.108
Onsite	Onsite Meteorological Programs (Safety Guide 23) (Draft SS 926-4, Proposed	1.23
Onsite	Independence Between Redundant Standby (Onsite) Power Sources and Between Their	1.6
Onsite	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Onsite	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62
Onsite	Onsite Meteorological Measurement Program for Uranium Recovery Facilities- Data	3.63
Onsite	Selection and Use of Pressure-Sensitive Seals on Containers for Onsite Storage	5.10
Operating	LWR Core Reloads; Guidance on Applications for Amendments to Operating Licenses	SC 521-4
Operating	Reporting of Operating Information-Appendix A Technical Specifications (for	1.16
Operating	Termination of Operating Licenses for Nuclear Reactors*	1.86
Operating	Reporting of Operating Information for Fuel Reprocessing Plants*	3.19
Operating	General Guidance for Designing, Testing, Operating, and Maintaining Emission	3.56
Operating	Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As	8.10
Operating	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust	9.3
Operation	(Second Proposed Revision 3 to Regulatory Guide 1.33) Quality Assurance Program	RS 902-4
Operation	Physical Models for Design and Operation of Hydraulic Structures and Systems for	1.125
Operation	Quality Assurance Program Requirements (Operation) (Draft RS 902-4, Proposed	1.33
Operation	Applicability of Existing Regulatory Guides to the Design and Operation of an	3.53
Operational	Operational Inspection and Surveillance of Environment Retention Systems for	3.11.1
Operations	Nuclear Criticality Safety in Operations with Fissionable Materials at Fuels and	3.4
Operations	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Operations	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
Operations	Quality Assurance for Radiological Monitoring Programs (Normal Operations)-	4.15
Operations	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.25
Operations	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.42
Operations	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.8
Operator	Nuclear Power Plant Simulation Facilities for Use in Operator License	1.149
Operators	Training and Certification of Independent Spent Fuel Storage Installation	HF 608-4
Operators	Guidance to Operators at the Controls and to Senior Operators in the Control	1.114
Operators	Qualification Tests of Electric Valve Operators Installed Inside the Containment	1.73
Operators	Protection of Nuclear Power Plant Control Room Operators Against an Accidental	1.95

KODC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Overload	Thermal Overload Protection for Electric Motors on Motor-Operated Valves*	1.106
Packages	Measurement of Radiation Levels on Surfaces of Packages of Radioactive Materials* TP 914-4	
Packages	Procedures for Picking Up and Receiving Packages of Radioactive Material (for	7.3
Packages	Leakage Tests on Packages for Shipment of Radioactive Materials (for Comment)*	7.4
Packaging	(Proposed Revision 2 to Regulatory Guide 7.9) Standard Format and Content of	
Packaging	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Packaging	Administrative Guide for Packaging and Transporting Radioactive Material*	7.1
Packaging	Establishing Quality Assurance Programs for Packaging Used in the Transport of	7.10
Packaging	Packaging and Transportation of Radioactively Contaminated Biological Materials*	7.2
Packaging	Administrative Guide for Verifying Compliance with Packaging Requirements for	7.7
Packaging	Standard Format and Content of Part 71 Applications for Approval of Packaging of	7.9
Paints	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel	3.30
Panoramic	Guide for the Preparation of Applications for Licenses for the Use of Panoramic	
Particles	Efficiency Testing of Air-Cleaning Systems Containing Devices for Removal of	3.2
Payment	Suggested Format for Cash Flow Statements Submitted as Guarantees of Payment of	9.4
Pellets	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Penetrating	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	1.11
Penetration	Qualification Test for Cable Penetration Fire Stops for Use in Nuclear Power	RS 809-5
Penetration	Electric Penetration Assemblies in Containment Structures for Nuclear Power	1.63
Performing	(Proposed Revision 2 to Regulatory Guide 10.6) Guide for the Preparation of	
Performing	Guide for the Preparation of Applications for Use of Sealed Sources and Devices	10.6
Perimeter	Perimeter Intrusion Alarm Systems (Draft SG 479-4, Proposed Revision 2,	5.44
Permanent	Safety-Related Permanent Dewatering Systems for Nuclear Power Plants*	
Permit	Information Needed by the NRC Staff in Connection with Its Antitrust Review of	9.2
Personnel	(Proposed Revision 2 to Regulatory Guide 8.14) Personnel Neutron Dosimeters*	OH 940-4
Personnel	Radiation Protection Training for Personnel Employed in Medical Facilities*	OP 212-4
Personnel	Medical Evaluation of Licensed Personnel for Nuclear Power Plants (Draft OL 401-	1.134
Personnel	Qualification of Quality Assurance Program Audit Personnel for Nuclear Power	1.146
Personnel	Qualification of Nuclear Power Plant Inspection, Examination, and Testing	1.58
Personnel	Qualification and Training of Personnel for Nuclear Power Plants (Draft RS 807.	1.8
Personnel	Personnel Neutron Dosimeters (Draft OH 940-4, Proposed Revision 2, published 2-	8.14
Personnel	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power	8.27
Persons	Compilation of Reporting Requirements for Persons Subject to NRC Regulations*	10.1
Petition	Preparation of an Environmental Report to Support a Rule Making Petition Seeking	6.7
Pharmacy	Guide for the Preparation of Applications for Nuclear Pharmacy Licenses*	FC 410-4
Physics	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and	8.21
Physics	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel	8.24
Physics	Health Physics Surveys in Uranium Mills (Draft OH 710-4 published 8-80)*	8.30
Pipe	Nuclear Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Piping	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin*	1.72
Plan	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Plan	Standard Format and Content for a Licensee Physical Security Plan for the	5.59
Plan	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.60
Planning	Emergency Planning and Preparedness for Nuclear Power Reactors (Revision 1 to	1.101
Planning	Emergency Planning for Research and Test Reactors (Draft HF 201-4, Proposed	2.6
Planning	Emergency Planning for Fuel Cycle Facilities and Plants Licensed Under 10 CFR	3.42
Plans	Standard Format and Content Guide for Access Authorization Plans for Nuclear	SG 301-4
Plans	Standard Format and Content of Decommissioning Plans for Licensees Under 10 CFR	3.65
Plans	Standard Format and Content of Site Characterization Plans for High-Level-Waste	4.17
Plans	Standard Format and Content of Safeguards Contingency Plans for Nuclear Power	5.54
Plans	Standard Format and Content of Safeguards Contingency Plans for Fuel Cycle	5.55

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Plans	Standard Format and Content of Safeguards Contingency Plans for Transportation (5.56
Plaque	Identification Plaque for Irretrievable Well-Logging Sources (for Comment)*	6.8
Plate	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type	1.130
Plate	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in	2.3
Plates	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray	5.38
Plugging	Bases for Plugging Degraded PWR Steam Generator Tubes (for Comment)*	1.121
Plutonium	Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel	3.10
Plutonium	General Design Guide for Ventilation Systems of Plutonium Processing and Fuel	3.12
Plutonium	Seismic Design Classification for Plutonium Processing and Fuel Fabrication	3.14
Plutonium	General Fire Protection Guide for Plutonium Processing and Fuel Fabrication	3.16
Plutonium	Quality Assurance Requirements for Protective Coatings Applied to Fuel	3.21
Plutonium	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28
Plutonium	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Plutonium	Quality Assurance Program Requirements for Fuel Reprocessing Plants and for	3.3
Plutonium	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.35
Plutonium	Standard Format and Content of License Applications for Plutonium Processing and	3.39
Plutonium	Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing	3.40
Plutonium	Nuclear Criticality Control and Safety of Homogeneous Plutonium-Uranium Fuel	3.47
Plutonium	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel	3.7
Plutonium	Measurements of Radionuclides in the Environment-Sampling and Analysis of	4.5
Plutonium	In Situ Assay of Plutonium Residual Holdup (Draft SG 045-4, Proposed Revision 1,	5.23
Plutonium	Materials Protection Contingency Measures for Uranium and Plutonium Fuel	5.30
Plutonium	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission	5.34
Pocket	Direct-Reading and Indirect-Reading Pocket Dosimeters*	8.4
Pond	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin*	1.72
Portable	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 407-4
Portions	Criteria for Power, Instrumentation, and Control Portions of Safety Systems (1.153
Position	Regulatory Staff Position Statement on Antitrust Matters*	9.1
Positive	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Postaccident	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Postulated	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control	1.78
Postulated	Evaluations of Explosions Postulated To Occur on Transportation Routes Near	1.91
Potential	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at	1.145
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	3.33
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	3.34
Potential	Assumptions Used for Evaluating the Potential Radiological Consequences of a	3.35
Powders	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Power	Maintenance Programs for Nuclear Power Plants*	OG-1001
Power	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power	EM 805-5
Power	(Second Proposed Revision 1 to Regulatory Guide 1.23) Meteorological Measurement	ES 926-4
Power	Safety-Related Permanent Dewatering Systems for Nuclear Power Plants*	FP 811-4
Power	(Proposed Revision 2 to Regulatory Guide 1.12) Nuclear Power Plant	MS 140-5
Power	(Second Proposed Revision 4 to Regulatory Guide 8.8) Information Relevant to	OP 618-4
Power	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Power	Lightning Protection for Nuclear Power Plants*	RS 705-4

KYOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Power	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification, Qualification Test for Cable Penetrations, and Fire Stops for Use in Nuclear Power	RS 802-5
Power	(Proposed Revision 2 to Regulatory Guide 1.94) Quality Assurance Requirements	RS 908-5
Power	Standard Format and Content Guide for Access Authorization Plans for Nuclear	SG 301-4
Power	Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power	1.100
Power	Emergency Planning and Preparedness for Nuclear Power Reactors (Revision 1 to	1.101
Power	Flood Protection for Nuclear Power Plants*	1.102
Power	(Withdrawn - See 44 FR 49321, 8-22-79) See NUREG-0554, "Single-Failure-Proof	1.104
Power	Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems	1.108
Power	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power	1.110
Power	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Power	Guidance to Operators at the Controls and to Senior Operators in the Control	1.114
Power	Periodic Testing of Electric Power and Protection Systems*	1.118
Power	Fire Protection Guidelines for Nuclear Power Plants (for Comment)*	1.120
Power	Quality Assurance Requirements for Control of Procurement of Items and Services	1.123
Power	Physical Models for Design and Operation of Hydraulic Structures and Systems for	1.125
Power	Inspection of Water-Control Structures Associated with Nuclear Power Plants*	1.127
Power	Installation Design and Installation of Large Lead Storage Batteries for Nuclear	1.128
Power	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for	1.129
Power	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-	1.131
Power	Site Investigations for Foundations of Nuclear Power Plants*	1.132
Power	Medical Evaluation of Licensed Personnel for Nuclear Power Plants (Draft OL 401-	1.134
Power	Normal Water Level and Discharge at Nuclear Power Plants (for Comment)*	1.135
Power	Laboratory Investigations of Soils for Engineering Analysis and Design of	1.138
Power	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Power	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor	1.142
Power	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.143
Power	Auditing of Quality Assurance Programs for Nuclear Power Plants*	1.144
Power	Atmospheric Dispersion Models for Potential Accident Consequence Assessments at	1.145
Power	Qualification of Quality Assurance Program Audit Personnel for Nuclear Power	1.146
Power	Functional Specification for Active Valve Assemblies in Systems Important to	1.148
Power	Nuclear Power Plant Simulation Facilities for Use in Operator License	1.149
Power	Criteria for Programmable Digital Computer System Software in Safety-Related	1.152
Power	Criteria for Power, Instrumentation, and Control Portions of Safety Systems (1.153
Power	Environmental Qualification of Connection Assemblies for Nuclear Power Plants (1.156
Power	Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants (1.158
Power	Protection of Nuclear Power Plants Against Industrial Sabotage*	1.17
Power	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Power	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-	1.26
Power	Ultimate Heat Sink for Nuclear Power Plants (for Comment)*	1.27
Power	Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants*	1.32
Power	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated	1.37
Power	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Power	Housekeeping Requirements for Water-Cooled Nuclear Power Plants*	1.39
Power	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment	1.40
Power	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify	1.41
Power	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems*	1.47
Power	Power Levels of Nuclear Power Plants*	1.49
Power	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Power	Application of the Single-Failure Criterion to Nuclear Power Plant Protection	1.53
Power	Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled	1.54

KDOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Power	Qualification of Nuclear Power Plant Inspection, Examination, and Testing	1.58
Power	Design Basis Floods for Nuclear Power Plants (Errata published 7-30-80)*	1.59
Power	Independence Between Redundant Standby (Onsite) Power Sources and Between Their	1.6
Power	Design Response Spectra for Seismic Design of Nuclear Power Plants*	1.60
Power	Damping Values for Seismic Design of Nuclear Power Plants*	1.61
Power	Electric Penetration Assemblies in Containment Structures for Nuclear Power	1.63
Power	Quality Assurance Requirements for the Design of Nuclear Power Plants*	1.64
Power	Initial Test Programs for Water-Cooled Reactor Power Plants*	1.68
Power	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems	1.68.1
Power	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-	1.68.2
Power	Concrete Radiation Shields for Nuclear Power Plants*	1.69
Power	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (1.70
Power	Qualification Tests of Electric Valve Operators Installed Inside the Containment	1.73
Power	Design Basis Tornado for Nuclear Power Plants*	1.76
Power	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control	1.78
Power	Qualification and Training of Personnel for Nuclear Power Plants (Draft RS 807.	1.8
Power	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power	1.81
Power	Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance	1.88
Power	Environmental Qualification of Certain Electric Equipment Important to Safety	1.89
Power	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Power	Evaluations of Explosions Postulated To Occur on Transportation Routes Near	1.91
Power	Availability of Electric Power Sources*	1.93
Power	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.94
Power	Protection of Nuclear Power Plant Control Room Operators Against an Accidental	1.95
Power	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Power	Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and	1.97
Power	Standard Format and Content of License Applications for Storage Only of	3.15
Power	Programs for Monitoring Radioactivity in the Environs of Nuclear Power Plants*	4.1
Power	Terrestrial Environmental Studies for Nuclear Power Stations*	4.11
Power	Preparation of Environmental Reports for Nuclear Power Stations*	4.2
Power	General Site Suitability Criteria for Nuclear Power Stations*	4.7
Power	Environmental Technical Specifications for Nuclear Power Plants (for Comment)*	4.8
Power	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	5.1
Power	Nuclear Material Control Systems for Nuclear Power Plants*	5.29
Power	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Power	Standard Format and Content of Safeguards Contingency Plans for Nuclear Power	5.54
Power	Design, Construction, and Use of Radioisotopic Power Generators for Certain Land	6.3
Power	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Power	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power	8.27
Power	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.8
Power	Information Needed by the NRC Staff in Connection with Its Antitrust Review of	9.2
Power	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust	9.3
Practice	Recommended Practice for Dealing with Outlying Observations*	5.36
Practices	Administrative Practices for Nuclear Criticality Safety at Fuels and Materials	3.57
Practices	Guide for Administrative Practices in Radiation Monitoring*	8.2
Predict	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.6
Preheat	Control of Preheat Temperature for Welding of Low-Alloy Steel*	1.50
Preheat	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Premiums	Suggested Format for Cash Flow Statements Submitted as Guarantees of Payment of	9.4
Prenatal	Instruction Concerning Prenatal Radiation Exposure (Draft OP 031-4, Proposed	8.13
Preoperational	Comprehensive Vibration Assessment Program for Reactor Internals During	1.20

KWD INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Preoperational	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify	1.41
Preoperational	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems	1.68.1
Preoperational	Preoperational Testing of Instrument and Control Air Systems (Draft RS 709-4, a	1.68.3
Preoperational	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water	1.79
Preparedness	Emergency Planning and Preparedness for Nuclear Power Reactors (Revision 1 to	1.101
Preparing	(Proposed Revision 1 to Regulatory Guide 3.50) Guidance on Preparing a License	CE 402-4
Preparing	Guidance on Preparing a License Application to Store Spent Fuel in an	3.50
Preregistered	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-6
Preservice	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice	1.150
Pressure	Thermal Shock to Reactor Pressure Vessels (Safety Guide 2)*	1.2
Pressure	Reactor Coolant Pressure Boundary Leakage Detection Systems*	1.45
Pressure	Selection and Use of Pressure-Sensitive Seals on Containers for Onsite Storage	5.10
Pressurized	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis	1.154
Pressurized	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Pressurized	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Pressurized	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Pressurized	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized	1.77
Pressurized	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water	1.79
Pressurized	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes*	1.83
Prestressed	(Proposed Regulatory Guide 1.35.1) Determining Prestressing Forces for	SC 807-4
Prestressed	(Proposed Revision 3 to Regulatory Guide 1.35) Inservice Inspection of Ungrouted	SC 810-4
Prestressed	Inservice Inspection of Ungrouted Tendons in F- stressed Concrete Containment	1.35
Prestressed	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted	1.90
Prestressing	(Proposed Regulatory Guide 1.35.1) Determining Prestressing Forces for	SC 807-4
Prestressing	Qualifications for Cement Grouting for Prestressing Tendons in Containment	1.107
Primary	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	1.11
Primary	Loose-Part Detection Program for the Primary System of L -ht-Water-Cooled	1.133
Primary	Design Limits and Loading Combinations for Metal Primary Reactor Containment	1.57
Procedural	Performance, Testing, and Procedural Specifications for Thermoluminescence	4.13
Procurement	Quality Assurance Requirements for Control of Procurement of Items and Services	1.123
Products	Applications of Bioassay for Fission and Activation Products (Draft OH 714-4	8.26
Programmable	Criteria for Programmable Digital Computer System Software in Safety-Related	1.152
Programs	Maintenance Programs for Nuclear Power Plants*	DG-1001
Programs	Guide for the Preparation of Applications for Licenses for Medical Teletherapy	FC 414-4
Programs	Identification of Valves for Inclusion in Inservice Testing Programs*	MS 901-4
Programs	Auditing of Quality Assurance Programs for Nuclear Power Plants*	1.144
Programs	Onsite Meteorological Programs (Safety Guide 23) (Draft SS 926-4, Proposed	1.23
Programs	Initial Test Programs for Water-Cooled Reactor Power Plants*	1.68
Programs	Guide for Preparation of Applications for Medical Use Programs (Draft FC 415-4,	10.8
Programs	Programs for Monitoring Radioactivity in the Environs of Nuclear Power Plants*	4.1
Programs	Quality Assurance for Radiological Monitoring Programs (Normal Operations)-	4.15
Programs	Establishing Quality Assurance Programs for Packaging Used in the Transport of	7.10
Programs	Acceptable Programs for Respiratory Protection*	8.15
Proof	(Withdrawn - See 44 FR 49321, 8-22-79) See NUREG-0554, "Single-Failure-Proof	1.104
Protected	Entry/Exit Control for Protected Areas, Vital Areas, and Material Access Areas (5.7
Protection	Response-Time Testing of Protection System Instrument Channels*	IC 121-5
Protection	Test and Calibration of Radiation Protection Instrumentation*	OP 032-5
Protection	Radiation Protection Training for Personnel Employed in Medical Facilities*	OP 212-4
Protection	Lightning Protection for Nuclear Power Plants*	RS 705-4
Protection	Flood Protection for Nuclear Power Plants*	1.102
Protection	Thermal Overload Protection for Electric Motors on Motor-Operated Valves*	1.106

KWN INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Protection	Protection Against Low-Trajectory Turbine Missiles*	1.115
Protection	Periodic Testing of Electric Power and Protection Systems*	1.118
Protection	Fire Protection Guidelines for Nuclear Power Plants (for Comment)*	1.120
Protection	Protection of Nuclear Power Plants Against Industrial Sabotage*	1.17
Protection	Periodic Testing of Protection System Actuation Functions (Safety Guide 22)*	1.22
Protection	Application of the Single-Failure Criterion to Nuclear Power Plant Protection	1.53
Protection	Protection of Nuclear Power Plant Control Room Operators Against an Accidental	1.95
Protection	General Fire Protection Guide for Plutonium Processing and Fuel Fabrication	3.16
Protection	Periodic Testing of Fuel Reprocessing Plant Protection System Actuation Functions	3.22
Protection	General Fire Protection Guide for Fuel Reprocessing Plants (for Comment)*	3.38
Protection	General Use of Locks in the Protection and Control of Facilities and Special	5.12
Protection	Security Seals for the Protection and Control of Special Nuclear Material*	5.15
Protection	Analysis and Use of Process Data for the Protection of Special Nuclear Material*	5.24
Protection	Materials Protection Contingency Measures for Uranium and Plutonium Fuel	5.30
Protection	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Protection	Standard Format and Content for a Licensee Physical Security Plan for the	5.59
Protection	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.60
Protection	Intent and Scope of the Physical Protection Upgrade Rule Requirements for Fixed	5.61
Protection	Physical Protection for Transient Shipments (Draft SG 126-4 published 9-81)*	5.63
Protection	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65
Protection	Acceptable Programs for Respiratory Protection*	8.15
Protection	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power	8.27
Protective	Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled	1.54
Protective	Manual Initiation of Protective Actions*	1.62
Protective	Quality Assurance Requirements for Protective Coatings Applied to Fuel	3.21
Protective	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel	3.30
Provisions	Containment Isolation Provisions for Fluid Systems (for Comment)*	1.161
Pump	Reactor Coolant Pump Flywheel Integrity (for Comment)*	1.14
Pumps	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Purity	Maintenance of Water Purity in Boiling Water Reactors (for Comment)*	1.56
Purpose	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents	1.109
Purpose	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
PWR	Bases for Plugging Degraded PWR Steam Generator Tubes (for Comment)*	1.121
Qualification	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Qualification	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Qualification	Qualification Test for Cable Penetration Fire Stops for Use in Nuclear Power	RS 809-5
Qualification	Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power	1.100
Qualification	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-	1.131
Qualification	Qualification of Quality Assurance Program Audit Personnel for Nuclear Power	1.146
Qualification	Environmental Qualification of Connection Assemblies for Nuclear Power Plants (1.156
Qualification	Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants (1.158
Qualification	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment	1.40
Qualification	Qualification of Nuclear Power Plant Inspection, Examination, and Testing	1.58
Qualification	Welder Qualification for Areas of Limited Accessibility*	1.71
Qualification	Qualification Tests of Electric Valve Operators Installed Inside the Containment	1.73
Qualification	Qualification and Training of Personnel for Nuclear Power Plants (Draft RS 807.	1.8
Qualification	Environmental Qualification of Certain Electric Equipment Important to Safety	1.89
Qualification	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Qualification	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28
Qualification	Qualification, Calibration, and Error Estimation Methods for Nondestructive	5.53
Qualifications	Qualifications for the Radiation Safety Officer in a Large-Scale Non-Fuel-Cycle	OP 722-4

INOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Qualifications	Qualifications for Cement Grouting for Prestressing Tendons in Containment	1.107
Qualifying	Training, Equipping, and Qualifying of Guards and Watchmen*	5.20
Quality	(Second Proposed Revision 3 to Regulatory Guide 1.33) Quality Assurance Program	RS 902-4
Quality	(Proposed Revision 2 to Regulatory Guide 1.94) Quality Assurance Requirements	RS 908-5
Quality	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.116
Quality	Quality Assurance Requirements for Control of Procurement of Items and Services	1.123
Quality	Auditing of Quality Assurance Programs for Nuclear Power Plants*	1.144
Quality	Qualification of Quality Assurance Program Audit Personnel for Nuclear Power	1.146
Quality	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-	1.26
Quality	Quality Assurance Program Requirements (Design and Construction) (Draft RS 002-	1.28
Quality	Quality Assurance Requirements for the Installation, Inspection, and Testing of	1.30
Quality	Quality Assurance Program Requirements (Operation) (Draft RS 902-4, Proposed	1.33
Quality	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated	1.37
Quality	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Quality	Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled	1.56
Quality	Quality Assurance Requirements for the Design of Nuclear Power Plants*	1.64
Quality	Quality Assurance Terms and Definitions*	1.74
Quality	Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance	1.88
Quality	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.96
Quality	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in	2.3
Quality	Quality Assurance Program Requirements for Research Reactors*	2.5
Quality	Quality Assurance Requirements for Protective Coatings Applied to Fuel	3.21
Quality	Quality Assurance Program Requirements for Fuel Reprocessing Plants and for	3.3
Quality	Quality Assurance for Radiological Monitoring Programs (Normal Operations)-	4.15
Quality	Establishing Quality Assurance Programs for Packaging Used in the Transport of	7.10
Radiation	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power	EN 805-5
Radiation	Guide for the Preparation of Applications for Licenses for the Use of	FC 413-4
Radiation	Test and Calibration of Radiation Protection Instrumentation*	OP 032-5
Radiation	Radiation Protection Training for Personnel Employed in Medical Facilities*	OP 212-4
Radiation	(Second Proposed Revision 4 to Regulatory Guide 8.8) Information Relevant to	OP 618-4
Radiation	Qualifications for the Radiation Safety Officer in a Large-Scale Non-Fuel-Cycle	OP 722-4
Radiation	Measurement of Radiation Levels on Surfaces of Packages of Radioactive Materials*	TP 914-4
Radiation	Concrete Radiation Shields for Nuclear Power Plants*	1.69
Radiation	Radiation Embrittlement of Reactor Vessel Materials (Draft ME 305-4, Proposed	1.99
Radiation	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.10
Radiation	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.11
Radiation	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Radiation	Concrete Radiation Shields*	3.9
Radiation	Radiation Symbol*	8.1
Radiation	Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As	8.10
Radiation	Instruction Concerning Prenatal Radiation Exposure (Draft OP 031-4, Proposed	8.13
Radiation	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.18
Radiation	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Radiation	Guide for Administrative Practices in Radiation Monitoring*	8.2
Radiation	Radiation Safety Surveys at Medical Institutions*	8.23
Radiation	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power	8.27
Radiation	Instruction Concerning Risks from Occupational Radiation Exposure (Draft OH 902-	8.29
Radiation	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.31
Radiation	Occupational Radiation Exposure Record Systems*	8.7
Radiation	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.8
Radioactive	(Proposed Revision 1 to Regulatory Guide 3.50) Guidance on Preparing a License	CE 402-4

INDC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Radioactive	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4
Radioactive	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-4
Radioactive	Guide for the Preparation of Applications for Licenses for the Use of	FC 412-4
Radioactive	Guide for the Preparation of Applications for Licenses for the Use of	FC 413-4
Radioactive	(Proposed Revision 2 to Regulatory Guide 7.9) Standard Format and Content of	FC 416-4
Radioactive	Safety Features of Gauges Containing Radioactive Material*	TP 102-5
Radioactive	Measurement of Radiation Levels on Surfaces of Packages of Radioactive Materials*	TP 914-6
Radioactive	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Radioactive	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.143
Radioactive	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Radioactive	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Radioactive	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-	1.26
Radioactive	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98
Radioactive	Standard Format and Content of License Applications for Storage Only of	3.15
Radioactive	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Radioactive	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
Radioactive	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Radioactive	Standard Format and Content of Environmental Reports for Near-Surface Disposal	4.18
Radioactive	Guidance for Selecting Sites for Near-Surface Disposal of Low-Level Radioactive	4.19
Radioactive	Leak Testing Radioactive Brachytherapy Sources*	6.1
Radioactive	Classification of Containment Properties of Sealed Radioactive Sources*	6.4
Radioactive	Administrative Guide for Packaging and Transporting Radioactive Material*	7.1
Radioactive	Establishing Quality Assurance Programs for Packaging Used in the Transport of	7.10
Radioactive	Procedures for Picking Up and Receiving Packages of Radioactive Material (for	7.3
Radioactive	Leakage Tests on Packages for Shipment of Radioactive Materials (for Comment)*	7.4
Radioactive	Administrative Guide for Obtaining Exemptions from Certain NRC Requirements Over	7.5
Radioactive	Administrative Guide for Verifying Compliance with Packaging Requirements for	7.7
Radioactive	Load Combinations for the Structural Analysis of Shipping Casks for Radioactive	7.8
Radioactive	Standard Format and Content of Part 71 Applications for Approval of Packaging of	7.9
Radioactivity	Packaging and Transportation of Radioactively Contaminated Biological Materials*	7.2
Radioactivity	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Radioactivity	Programs for Monitoring Radioactivity in the Environs of Nuclear Power Plants*	4.1
Radioactivity	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Radiography	(Proposed Revision 2 to Regulatory Guide 10.6) Guide for the Preparation of	FC 401-4
Radiography	Guide for the Preparation of Applications for Use of Sealed Sources and Devices	10.6
Radioisotopic	Design, Construction, and Use of Radioisotopic Power Generators for Certain Land	6.3
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.33
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.34
Radiological	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.35
Radiological	Radiological Effluent and Environmental Monitoring at Uranium Mills*	4.14
Radiological	Quality Assurance for Radiological Monitoring Programs (Normal Operations)-	4.15
Radionuclide	Qualifications for the Radiation Safety Officer in a Large-Scale Non-Fuel-Cycle	OP 722-4
Radionuclide	Preparation of an Environmental Report to Support a Rule Making Petition Seeking	6.7
Radionuclides	Measurements of Radionuclides in the Environment-Sampling and Analysis of	4.5
Radionuclides	Measurements of Radionuclides in the Environment-Strontium-89 and Strontium-90	4.6

KNOE INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Radon	Calculation of Radon Flux Attenuation by Earthen Uranium Mill Tailings Covers (3.64
Radiaste	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power	1.110
Raschig	Use of Borosilicate-Glass Raschig Rings as a Neutron Absorber in Solutions of	3.1
Reactor	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents	1.109
Reactor	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	1.11
Reactor	Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor	1.113
Reactor	Materials, Construction, and Testing of Concrete Containments (Articles CC-1CJ0,	1.136
Reactor	Reactor Coolant Pump Flywheel Integrity (for Comment)*	1.14
Reactor	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor	1.142
Reactor	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice	1.150
Reactor	Thermal Shock to Reactor Pressure Vessels (Safety Guide 2)*	1.2
Reactor	Comprehensive Vibration Assessment Program for Reactor Internals During	1.20
Reactor	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Reactor	Reactor Coolant Pressure Boundary Leakage Detection Systems*	1.45
Reactor	Design Limits and Loading Combinations for Metal Primary Reactor Containment	1.57
Reactor	Materials and Inspections for Reactor Vessel Closure Studs*	1.65
Reactor	Initial Test Programs for Water-Cooled Reactor Power Plants*	1.68
Reactor	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems	1.68.1
Reactor	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes*	1.83
Reactor	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Reactor	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98
Reactor	Radiation Embrittlement of Reactor Vessel Materials (Draft ME 305-4, Proposed	1.99
Reactor	Standard Format and Content of License Applications for Storage Only of	3.15
Reactor	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Reactors	Assuring the Availability of Funds for Decommissioning Nuclear Reactors*	DG-1003
Reactors	Emergency Planning and Preparedness for Nuclear Power Reactors (Revision 1 to	1.101
Reactors	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power	1.110
Reactors	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Reactors	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Reactors	Loose-Part Detection Program for the Primary System of Light-Water-Cooled	1.133
Reactors	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis	1.154
Reactors	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Reactors	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Reactors	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Reactors	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Reactors	Maintenance of Water Purity in Boiling Water Reactors (for Comment)*	1.56
Reactors	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized	1.77
Reactors	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water	1.79
Reactors	Termination of Operating Licenses for Nuclear Reactors*	1.86
Reactors	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors	1.87
Reactors	Shield Test Program for Evaluation of Installed Biological Shielding in Research	2.1
Reactors	Development of Technical Specifications for Experiments in Research Reactors*	2.2
Reactors	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in	2.3
Reactors	Review of Experiments for Research Reactors*	2.4
Reactors	Quality Assurance Program Requirements for Research Reactors*	2.5
Reactors	Emergency Planning for Research and Test Reactors (Draft HF 201-4, Proposed	2.6
Reactors	Nuclear Criticality Control and Safety of Homogeneous Plutonium-Uranium Fuel	3.47
Reactors	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	5.1
Receiver	Evaluation of Shipper-Receiver Differences in the Transfer of Special Nuclear	5.28
Receiving	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Receiving	Shipping and Receiving Control of Strategic Special Nuclear Material (Draft SG	5.57

KINOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Receiving	Procedures for Picking Up and Receiving Packages of Radioactive Material (for Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant)	7.3
Recirculation	Recommended Practice for Dealing with Outlying Observations*	1.82
Recommended	Records Important for Decommissioning for Licensees Under 10 CFR Parts 30, 40, 5.36	
Records	Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance	DG-3001
Records	Occupational Radiation Exposure Records Systems*	1.88
Redundant	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify 8.7	
Redundant	Independence Between Redundant Standby (Onsite) Power Sources and Between Their	1.41
Refueling	LWR Core Reloads; Guidance on Applications for Amendments to Operating Licenses	SC 521-4
Registration	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.10
Registration	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.11
Regulations	Compilation of Reporting Requirements for Persons Subject to NRC Regulations*	10.1
Regulatory	(Proposed Revision 1 to Regulatory Guide 3.50) Guidance on Preparing a License	CE 402-4
Regulatory	(Proposed Revision 2 to Regulatory Guide 1.13) Spent Fuel Storage Facility	CE 913-5
Regulatory	(Second Proposed Revision 1 to Regulatory Guide 1.23) Meteorological Measurement	ES 926-4
Regulatory	(Proposed Revision 2 to Regulatory Guide 10.6) Guide for the Preparation of	FC 401-4
Regulatory	(Proposed Revision 2 to Regulatory Guide 10.5) Guide for the Preparation of	FC 408-4
Regulatory	(Proposed Revision 2 to Regulatory Guide 7.9) Standard Format and Content of	FC 416-4
Regulatory	(Proposed Revision 2 to Regulatory Guide 1.12) Nuclear Power Plant	MS 140-5
Regulatory	(Proposed Revision 2 to Regulatory Guide 8.14) Personnel Neutron Dosimeters*	ON 940-4
Regulatory	(Second Proposed Revision 4 to Regulatory Guide 8.8) Information Relevant to	OP 618-4
Regulatory	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Regulatory	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Regulatory	(Second Proposed Revision 3 to Regulatory Guide 1.33) Quality Assurance Program	RS 902-4
Regulatory	(Proposed Revision 2 to Regulatory Guide 1.94) Quality Assurance Requirements	RS 908-5
Regulatory	(Proposed Regulatory Guide 1.35.1) Determining Prestressing Forces for	SC 807-4
Regulatory	(Proposed Revision 3 to Regulatory Guide 1.35) Inservice Inspection of Ungrouted	SC 810-4
Regulatory	(Proposed Revision 2 to Regulatory Guide 3.5) Standard Format and Content of	WM 039-4
Regulatory	Preoperational Testing of Instrument and Control Air Systems (Draft RS 709-4, a	1.68.3
Regulatory	(Withdrawn - See 47 FR 19258, 5-4-82) Reissued as Regulatory Guide 1.68.3, a	1.80
Regulatory	Applicability of Existing Regulatory Guides to the Design and Operation of an	3.53
Regulatory	Regulatory Staff Position Statement on Antitrust Matters*	9.1
Regulatory	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust	9.3
Reinforced	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin*	1.72
Reliability	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Reloade	LWR Core Reloads; Guidance on Applications for Amendments to Operating Licenses	SC 521-4
Rem	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Remote	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-	1.68.2
Renewal	Standard Format and Content for the Health and Safety Sections of License	3.52
Renewal	Standard Format and Content for the Health and Safety Sections of License	3.55
Replacement	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for	1.129
Reporting	Reporting of Operating Information-Appendix A Technical Specifications (for	1.16
Reporting	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Reporting	Compilation of Reporting Requirements for Persons Subject to NRC Regulations*	10.1
Reporting	Reporting of Operating Information for Fuel Reprocessing Plants*	3.19
Reporting	Onsite Meteorological Measurement Program for Uranium Recovery Facilities- Data	3.63
Reporting	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Reporting	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.4
Reporting	Reporting of Safeguards Events (Draft SG 901-4 published 10-79) (Draft SG 901-4,	5.62
Reporting	Standard Format and Content of Site Characterization Plans for High-Level-Waste	4.17
Repositories	Earthquake Instrumentation for Fuel Reprocessing Plants*	3.17
Reprocessing		

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Reprocessing	Confinement Barriers and Systems for Fuel Reprocessing Plants*	3.18
Reprocessing	Reporting of Operating Information for Fuel Reprocessing Plants*	3.19
Reprocessing	Process Offgas Systems for Fuel Reprocessing Plants*	3.20
Reprocessing	Quality Assurance Requirements for Protective Coatings Applied to Fuel	3.21
Reprocessing	Periodic Testing of Fuel Reprocessing Plant Protection System Actuation Functions	3.22
Reprocessing	Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing	3.26
Reprocessing	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel	3.27
Reprocessing	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28
Reprocessing	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Reprocessing	Quality Assurance Program Requirements for Fuel Reprocessing Plants and for	3.3
Reprocessing	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel	3.30
Reprocessing	Emergency Water Supply Systems for Fuel Reprocessing Plants*	3.31
Reprocessing	General Design Guide for Ventilation Systems for Fuel Reprocessing Plants (for	3.32
Reprocessing	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.33
Reprocessing	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Reprocessing	General Fire Protection Guide for Fuel Reprocessing Plants (for Comment)*	3.38
Reprocessing	Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing	3.40
Reprocessing	Content of Technical Specifications for Fuel Reprocessing Plants*	3.6
Research	Shield Test Program for Evaluation of Installed Biological Shielding in Research	2.1
Research	Development of Technical Specifications for Experiments in Research Reactors*	2.2
Research	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in	2.3
Research	Review of Experiments for Research Reactors*	2.4
Research	Quality Assurance Program Requirements for Research Reactors*	2.5
Research	Emergency Planning for Research and Test Reactors (Draft HF 201-4, Proposed	2.6
Residual	Guidance for Residual Heat Removal (for Comment)*	1.139
Residual	In Situ Assay of Plutonium Residual Holdup (Draft SG 045-4, Proposed Revision 1,	5.23
Residual	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.25
Residual	In Situ Assay of Enriched Uranium Residual Holdup (Draft SG 047-4, Proposed	5.37
Residual	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.42
Residual	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.8
Resin	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin*	1.72
Respiratory	Acceptable Programs for Respiratory Protection*	8.15
Response	Response-Time Testing of Protection System Instrument Channels*	IC 121-5
Response	Development of Floor Design Response Spectra for Seismic Design of Floor-	1.122
Response	Design Response Spectra for Seismic Design of Nuclear Power Plants*	1.60
Response	Combining Modal Responses and Spatial Components in Seismic Response Analysis*	1.92
Responses	Combining Modal Responses and Spatial Components in Seismic Response Analysis*	1.92
Retention	Design, Construction, and Inspection of Embankment Retention Systems for Uranium	3.11
Retention	Operational Inspection and Surveillance of Embankment Retention Systems for	3.11.1
Retrievable	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Retrospective	Suggested Format for Cash Flow Statements Submitted as Guarantees of Payment of	9.4
Review	Review of Experiments for Research Reactors*	2.4
Review	Management Review of Nuclear Material Control and Accounting Systems (for	5.51
Review	Information Needed by the NRC Staff in Connection with Its Antitrust Review of	9.2
Review	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust	9.3
Rings	Use of Borosilicate-Glass Rasching Rings as a Neutron Absorber in Solutions of	3.1
Risks	Instruction Concerning Risks from Occupational Radiation Exposure (Draft OH 902-	8.29
Road	Specially Designed Vehicle with Armed Guards for Road Shipment of Special	5.31
Rod	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized	1.77
Room	Guidance to Operators at the Controls and to Senior Operators in the Control	1.114
Room	Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control	1.78

KWN INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Room	Protection of Nuclear Power Plant Control Room Operators Against an Accidental	1.95
Routes	Evaluations of Explosions Postulated To Occur on Transportation Routes Near	1.91
Rule	Intent and Scope of the Physical Protection Upgrade Rule Requirements for Fixed	5.61
Rule	Preparation of an Environmental Report to Support a Rule Making Petition Seeking	6.7
Sabotage	Protection of Nuclear Power Plants Against Industrial Sabotage*	1.17
Safeguards	Standard Format and Content of Safeguards Contingency Plans for Nuclear Power	5.54
Safeguards	Standard Format and Content of Safeguards Contingency Plans for Fuel Cycle	5.55
Safeguards	Standard Format and Content of Safeguards Contingency Plans for Transportation (5.56
Safeguards	Reporting of Safeguards Events (Draft SG 901-4 published 10-79) (Draft SG 901-4,	5.62
Safety	Safety-Related Permanent Dewatering Systems for Nuclear Power Plants*	FP 811-4
Safety	Qualifications for the Radiation Safety Officer in a Large-Scale Non-Fuel-Cycle	OP 722-4
Safety	Safety Features of Gauges Containing Radioactive Material*	TP 102-5
Safety	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Safety	Instrument Setpoints for Safety-Related Systems (Draft IC 010-5, Proposed	1.105
Safety	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11)	1.11
Safety	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor	1.142
Safety	Functional Specification for Active Valve Assemblies in Systems Important to	1.148
Safety	Criteria for Programmable Digital Computer System Software in Safety-Related	1.152
Safety	Criteria for Power, Instrumentation, and Control Portions of Safety Systems (1.153
Safety	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis	1.154
Safety	Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants (1.158
Safety	Thermal Shock to Reactor Pressure Vessels (Safety Guide 2)*	1.2
Safety	Periodic Testing of Protection System Actuation Functions (Safety Guide 22)*	1.22
Safety	Onsite Meteorological Programs (Safety Guide 23) (Draft SS 926-4, Proposed	1.23
Safety	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Safety	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Safety	Quality Assurance Requirements for the Installation, Inspection, and Testing of	1.30
Safety	Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants*	1.32
Safety	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems*	1.47
Safety	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Safety	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Safety	Independence Between Redundant Standby (Onsite) Power Sources and Between Their	1.6
Safety	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (1.70
Safety	Environmental Qualification of Certain Electric Equipment Important to Safety	1.89
Safety	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.10
Safety	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.11
Safety	Standard Format and Content of Safety Analysis Reports for Uranium Enrichment	3.25
Safety	Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing	3.26
Safety	Nuclear Criticality Safety in Operations with Fissionable Materials at Fuels and	3.4
Safety	Nuclear Criticality Safety in the Storage of Fissile Materials*	3.43
Safety	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Safety	Nuclear Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Safety	Nuclear Criticality Control and Safety of Homogeneous Plutonium- Uranium Fuel	3.47
Safety	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Safety	Standard Format and Content for the Health and Safety Sections of License	3.52
Safety	Standard Format and Content for the Health and Safety Sections of Licens	3.55
Safety	Administrative Practices for Nuclear Criticality Safety at Fuels and Materials	3.57
Safety	Criticality Safety for Handling, Storing, and Transporting LWR Fuel at Fuels and	3.58
Safety	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Safety	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62
Safety	General Safety Standard for Installations Using Nonmedical Sealed Gamma-Ray	6.5

KNOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Safety	Radiation Safety Surveys at Medical Institutions*	8.23
Sampled	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air	8.25
Sampling	Measurements of Radionuclides in the Environment-Sampling and Analysis of	4.5
Sampling	Acceptance Sampling Procedures for Exempted and Generally Licensed Items	6.6
Sampling	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air	8.25
Scale	Qualifications for the Radiation Safety Officer in a Large-Scale Non-Fuel-Cycle	OP 722-4
Scope	(Proposed Revision 2 to Regulatory Guide 10.5) Guide for the Preparation of	FC 408-4
scope	(Withdrawn - See 47 FR 19258, 5-4-82) Reissued as Regulator; Guide 1.68.3, a	1.80
Scope	Guidance to Academic Institutions Applying for Specific Byproduct Material	10.2
Scope	Applications for Type A Licenses of Broad Scope (Errata published 7-84) (Draft	10.5
Scope	Intent and Scope of the Physical Protection Upgrade Rule Requirements for Fixed	5.61
Scrap	Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste (5.11
Scrap	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission	5.34
See	Design, Construction, and Use of Radioisotopic Power Generators for Certain Land	6.3
Sealed	(Proposed Revision 2 to Regulatory Guide 10.6) Guide for the Preparation of	FC 401-4
Sealed	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 404-4
Sealed	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 405-4
Sealed	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 407-4
Sealed	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-4
Sealed	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.11
Sealed	Guide for the Preparation of Applications for Use of Sealed Sources and Devices	10.6
Sealed	Classification of Containment Properties of Sealed Radioactive Sources*	6.4
Sealed	General Safety Standard for Installations Using Nonmedical Sealed Gamma-Ray	6.5
Seals	Selection and Use of Pressure-Sensitive Seals on Containers for Onsite Storage	5.10
Seals	Security Seals for the Protection and Control of Special Nuclear Material*	5.15
Security	Security Seals for the Protection and Control of Special Nuclear Material*	5.15
Security	Plant Security Force Duties*	5.43
Security	Standard Format and Content for a Licensee Physical Security Plan for the	5.59
Security	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65
Seeking	Preparation of an Environmental Report to Support a Rule Making Petition Seeking	6.7
Seepage	Design, Installation, and Inspection of Seepage Control Liners at Uranium	MS 146-4
Seismic	Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power	1.100
Seismic	Development of Floor Design Response Spectra for Seismic Design of Floor-	1.122
Seismic	Seismic Design Classification*	1.29
Seismic	Design Response Spectra for Seismic Design of Nuclear Power Plants*	1.60
Seismic	Damping Values for Seismic Design of Nuclear Power Plants*	1.61
Seismic	Combining Modal Responses and Spatial Components in Seismic Response Analysis*	1.92
Seismic	Seismic Design Classification for Plutonium Processing and Fuel Fabrication	3.14
Selected	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.4
Selected	Integrity and Test Specifications for Selected Brachytherapy Sources*	6.2
Selecting	Guidance for Selecting Sites for Near-Surface Disposal of Low-Level Radioactive	4.19
Selection	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Selection	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Selection	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel	3.30
Selection	Selection and Use of Pressure-Sensitive Seals on Containers for Onsite Storage	5.10
Selection	Selection of Material Balance Areas and Item Control Areas*	5.26
Self	Guide for the Preparation of Applications for Licenses for the Use of Panoramic	FC 403-4
Self	Guide for the Preparation of Applications for Licenses for the Use of Self-	10.9
Senior	Guidance to Operators at the Controls and to Senior Operators in the Control	1.114
Sensing	Instrument Sensing Line; (Draft IC 126-5, published 3-82)*	1.151
Sensitive	Selection and Use of Pressure-Sensitive Seals on Containers for Onsite Storage	5.10

KWN INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Sensitized	Control of the Use of Sensitized Stainless Steel*	1.44
September	Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems	1.108
Serial	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	5.1
Service	Service Limits and Loading Combinations for Class 1 Linear-Type Component	1.124
Service	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type	1.130
Services	Guide for the Preparation of Applications for Licenses for the Use of	FC 412-4
Services	Quality Assurance Requirements for Control of Procurement of Items and Services	1.123
Servicing	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-4
Setpoints	Instrument Setpoints for Safety-Related Systems (Draft IC 010-5, Proposed	1.105
Shared	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power	1.81
Shell	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type	1.130
Shield	Shield Test Program for Evaluation of Installed Biological Shielding in Research	2.1
Shielding	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power	EN 805-5
Shielding	Shield Test Program for Evaluation of Installed Biological Shielding in Research	2.1
Shields	Concrete Radiation Shields for Nuclear Power Plants*	1.69
Shields	Concrete Radiation Shields*	3.9
Shipment	Specially Designed Vehicle with Armed Guards for Road Shipment of Special	5.31
Shipment	Leakage Tests on Packages for Shipment of Radioactive Materials (for Comment)*	7.4
Shipments	Physical Protection for Transient Shipments (Draft SG 126-4 published 9-81)*	5.63
Shipments	Administrative Guide for Obtaining Exemptions from Certain NRC Requirements Over	7.5
Shipments	Administrative Guide for Verifying Compliance with Packaging Requirements for	7.7
Shipper	Evaluation of Shipper-Receiver Differences in the Transfer of Special Nuclear	5.28
Shipping	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Shipping	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Shipping	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Shipping	Shipping and Receiving Control of Strategic Special Nuclear Material (Draft SG	5.57
Shipping	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels*	7.6
Shipping	Load Combinations for the Structural Analysis of Shipping Casks for Radioactive	7.8
Shock	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis	1.154
Shock	Thermal Shock to Reactor Pressure Vessels (Safety Guide 2)*	1.2
Shutdown	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-	1.68.2
Shutdown	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power	1.81
Signals	Criticality and Other Interior Evacuation Signals*	8.5
Simulation	Nuclear Power Plant Simulation Facilities for Use in Operator License	1.149
Single	(Withdrawn - See 44 FR 49321, 8-22-79) See NUREG-0554, "Single-Failure-Proof	1.104
Single	Application of the Single-Failure Criterion to Nuclear Power Plant Protection	1.53
Sink	Ultimate Heat Sink for Nuclear Power Plants (for Comment)*	1.27
Site	Site Investigations for Foundations of Nuclear Power Plants*	1.132
Site	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify	1.41
Site	Standard Format and Content of Site Characterization Plans for High-Level-Waste	4.17
Site	General Site Suitability Criteria for Nuclear Power Stations*	4.7
Sites	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4
Sites	Guidance for Selecting Sites for Near-Surface Disposal of Low-Level Radioactive	4.19
Sites	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Sites	Intent and Scope of the Physical Protection Upgrade Rule Requirements for Fixed	5.61
Software	Criteria for Programmable Digital Computer System Software in Safety-Related	1.152
Soil	Measurements of Radionuclides in the Environment-Sampling and Analysis of	4.5
Soils	(Proposed Revision 2 to Regulatory Guide 1.94) Quality Assurance Requirements	RS 908-5
Soils	Laboratory Investigations of Soils for Engineering Analysis and Design of	1.138
Solid	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Solution	Guidelines for Ground-Water Monitoring at In Situ Uranium Solution Mines*	ES 114-4

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Solution	Standard Format and Content of License Applications, Including Environmental	3.46
Solutions	Use of Borosilicate-Glass Rasching Rings as a Neutron Absorber in Solutions of	3.1
Solutions	Nucleus Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Solutions	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic	5.39
Source	Guide for the Preparation of Applications for Licenses for the Use of Panoramic	FC 403-4
Source	Guide for the Preparation of Applications for Licenses to Process Source	10.4
Source	Guide for the Preparation of Applications for Licenses for the Use of Self-	10.9
Source	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
Sources	(Proposed Revision 2 to Regulatory Guide 10.6) Guide for the Preparation of	FC 401-4
Sources	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 404-4
Sources	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 405-4
Sources	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 407-4
Sources	Guide for the Preparation of Applications for Licenses for the Use of	FC 411-4
Sources	Independence Between Redundant Standby (Onsite) Power Sources and Between Their	1.6
Sources	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant	1.82
Sources	Availability of Electric Power Sources*	1.93
Sources	Guide for the Preparation of Applications for Radiation Safety Evaluation and	10.11
Sources	Guide for the Preparation of Applications for Use of Sealed Sources and Devices	10.6
Sources	Leak Testing Radioactive Brachytherapy Sources*	6.1
Sources	Integrity and Test Specifications for Selected Brachytherapy Sources*	6.2
Sources	Classification of Containment Properties of Sealed Radioactive Sources*	6.4
Sources	General Safety Standard for Installations Using Nonmedical Sealed Gamma-Ray	6.5
Sources	Identification Plaque for Irretrievable Well-Logging Sources (for Comment)*	6.8
Spatial	Combining Modal Responses and Spatial Components in Seismic Response Analysis*	1.92
Specification	Functional Specification for Active Valve Assemblies in Systems Important to	1.148
Specifications	Reporting of Operating Information-Appendix A Technical Specifications (for	1.16
Specifications	Development of Technical Specifications for Experiments in Research Reactors*	2.2
Specifications	Content of Technical Specifications for Fuel Reprocessing Plants*	3.6
Specifications	Performance, Testing, and Procedural Specifications for Thermoluminescence	4.13
Specifications	Environmental Technical Specifications for Nuclear Power Plants (for Comment)*	4.8
Specifications	Integrity and Test Specifications for Selected Brachytherapy Sources*	6.2
Spectra	Development of Floor Design Response Spectra for Seismic Design of Floor-	1.122
Spectra	Design Response Spectra for Seismic Design of Nuclear Power Plants*	1.60
Spectrochemical	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Spectrometric	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Spectrometry	Nondestructive Uranium-235 Enrichment Assay by Gamma-Ray Spectrometry (Draft SG	5.21
Spectrometry	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray	5.38
Spectroscopy	Guidelines for Germanium Spectroscopy Systems for Measurement of Special Nuclear	5.9
Spent	(Proposed Revision 1 to Regulatory Guide 3.50) Guidance on Preparing a License	CE 402-4
Spent	(Proposed Revision 2 to Regulatory Guide 1.13) Spent Fuel Storage Facility	CE 913-5
Spent	Training and Certification of Independent Spent Fuel Storage Installation	HF 608-4
Spent	Spent Fuel Storage Facility Design Basis (for Comment) (Draft CE 913-5, Proposed	1.13
Spent	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Spent	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Spent	Design of an Independent Spent Fuel Storage Installation (Water-Basin Type) (3.49
Spent	Guidance on Preparing a License Application to Store Spent Fuel in an	3.50
Spent	Applicability of Existing Regulatory Guides to the Design and Operation of an	3.53
Spent	Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation (3.54
Spent	Design of an Independent Spent Fuel Storage Installation (Dry Storage) (Draft CE	3.60
Spent	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Spent	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62

Splices	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Splices	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-	1.131
Spontaneous	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission	5.34
Spray	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin*	1.72
Staff	Regulatory Staff Position Statement on Antitrust Matters*	9.1
Staff	Information Needed by the NRC Staff in Connection with Its Antitrust Review of	9.2
Staff	Information Needed by the AEC Regulatory Staff in Connection with Its Antitrust	9.3
Stainless	Control of Ferrite Content in Stainless Steel Weld Metals*	1.31
Stainless	Nonmetallic Thermal Insulation for Austenitic Stainless Steel*	1.36
Stainless	Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components*	1.43
Stainless	Control of the Use of Sensitized Stainless Steel*	1.44
Stainless	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Standard	(Proposed Revision 2 to Regulatory Guide 7.9) Standard Format and Content of	FC 416-4
Standard	Standard Format and Content Guide for Access Authorization Plans for Nuclear	SG 301-4
Standard	(Proposed Revision 2 to Regulatory Guide 3.5) Standard Format and Content of	WM 039-4
Standard	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (1.70
Standard	Standard Format and Content of License Applications for Storage Only of	3.15
Standard	Standard Format and Content of Safety Analysis Reports for Uranium Enrichment	3.25
Standard	Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing	3.26
Standard	Standard Format and Content of License Applications for Plutonium Processing and	3.39
Standard	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Standard	Standard Format and Content of License Applications, Including Environmental	3.46
Standard	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Standard	Standard Format and Content of License Applications for Uranium Mills (for	3.5
Standard	Standard Format and Content for the Health and Safety Sections of License	3.52
Standard	Standard Format and Content for the Health and Safety Sections of License	3.55
Standard	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Standard	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62
Standard	Standard Format and Content of Decommissioning Plans for Licensees Under 10 CFR	3.65
Standard	Standard Format and Content of Site Characterization Plans for High-Level-Waste	4.17
Standard	Standard Format and Content of Environmental Reports for Near-Surface Disposal	4.18
Standard	Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF4)	5.4
Standard	Standard Format and Content for the Special Nuclear Material Control and	5.45
Standard	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Standard	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Standard	Standard Format and Content of Safeguards Contingency Plans for Nuclear Power	5.54
Standard	Standard Format and Content of Safeguards Contingency Plans for Fuel Cycle	5.55
Standard	Standard Format and Content of Safeguards Contingency Plans for Transportation (5.56
Standard	Standard Format and Content for a Licensee Physical Security Plan for the	5.59
Standard	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.60
Standard	General Safety Standard for Installations Using Nonmedical Sealed Gamma-Ray	6.5
Standard	Standard Format and Content of Part 71 Applications for Approval of Packaging of	7.9
Standard	Standard Test Procedure for Geiger-Muller Counters*	8.6
Standards	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-	1.26
Standby	Fuel-Oil Systems for Standby Diesel Generators*	1.137
Standby	Independence Between Redundant Standby (Onsite) Power Sources and Between Their	1.6
Standby	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Startup	LWR Core Reloads; Guidance on Applications for Amendments to Operating Licenses	SC 521-4
Startup	Comprehensive Vibration Assessment Program for Reactor Internals During	1.20
Startup	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems	1.68.1
Startup	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-	1.68.2

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Statement	Regulatory Staff Position Statement on Antitrust Matters*	9.1
Statements	Suggested Format for Cash Flow Statements Submitted as Guarantees of Payment of	9.4
Station	Station Blackout (Draft SI 501-4 published 3-86) (Issued June 1988, reissued	1.155
Stations	(Second Proposed Revision 4 to Regulatory Guide 8.8) Information Relevant to	OP 618-4
Stations	Terrestrial Environmental Studies for Nuclear Power Stations*	4.11
Stations	Preparation of Environmental Reports for Nuclear Power Stations*	4.2
Stations	General Site Suitability Criteria for Nuclear Power Stations*	4.7
Stations	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.8
Statistical	An Acceptable Model and Related Statistical Methods for the Analysis of Fuel	1.126
Statistical	Statistical Terminology and Notation for Special Nuclear Materials Control and	5.3
Statistical	Statistical Evaluation of Material Unaccounted For*	5.33
Status	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems*	1.47
Steam	Bases for Plugging Degraded PWR Steam Generator Tubes (for Comment)*	1.121
Steam	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-	1.26
Steam	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Steam	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes*	1.83
Steam	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Steel	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Steel	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Steel	(Proposed Revision 2 to Regulatory Guide 1.94) Quality Assurance Requirements	RS 908-5
Steel	Control of Ferrite Content in Stainless Steel Weld Metal*	1.31
Steel	Nonmetallic Thermal Insulation for Austenitic Stainless Steel*	1.36
Steel	Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components*	1.43
Steel	Control of the Use of Sensitized Stainless Steel*	1.44
Steel	Control of Preheat Temperature for Welding of Low-Alloy Steel*	1.50
Steel	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.94
Steel	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Steel	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Steel	Nuclear Criticality Safety for Steel-Pipe Intersections Containing Aqueous	3.45
Stop	Qualification Test for Cable Penetration Fire Stops for Use in Nuclear Power	RS 809-5
Storage	(Proposed Revision 2 to Regulatory Guide 1.13) Spent Fuel Storage Facility	CE 913-5
Storage	Guide for the Preparation of Applications for Licenses for the Use of Panoramic	FC 403-4
Storage	Training and Certification of Independent Spent Fuel Storage Installation	HF 608-4
Storage	Installation Design and Installation of Large Lead Storage Batteries for Nuclear	1.128
Storage	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for	1.129
Storage	Spent Fuel Storage Facility Design Basis (for Comment) (Draft CE 913-5, Proposed	1.13
Storage	Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants (1.158
Storage	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Storage	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Storage	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Storage	Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance	1.88
Storage	Guide for the Preparation of Applications for Licenses for the Use of Self-	10.9
Storage	Guide for Acceptable Waste Storage Methods at UF6 Production Plants*	3.13
Storage	Standard Format and Content of License Applications for Storage Only of	3.15
Storage	Nuclear Criticality Safety in the Storage of Fissile Materials*	3.43
Storage	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Storage	Standard Format and Content for the Safety Analysis Report for an Independent	3.48
Storage	Design of an Independent Spent Fuel Storage Installation (Water-Basin Type) (3.49
Storage	Guidance on Preparing a License Application to Store Spent Fuel in an	3.50
Storage	Applicability of Existing Regulatory Guides to the Design and Operation of an	3.53
Storage	Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation (3.54

KHOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Storage	Design of an Independent Spent Fuel Storage Installation (Dry Storage) (Draft CE	3.60
Storage	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Storage	Standard Format and Content for the Safety Analysis Report for Onsite Storage of	3.62
Storage	Selection and Use of Pressure-Sensitive Seals on Containers for Onsite Storage	5.10
Store	(Proposed Revision 1 to Regulatory Guide 3.50) Guidance on Preparing a License	CE 402-4
Store	Guidance on Preparing a License Application to Store Spent Fuel in an	3.50
Storing	Criticality Safety for Handling, Storing, and Transporting LWR Fuel at Fuels and	3.58
Strategic	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.52
Strategic	Shipping and Receiving Control of Strategic Special Nuclear Material (Draft SG	5.57
Strategic	Standard Format and Content for a Licensee Physical Security Plan for the	5.59
Strategic	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.60
Streams	Quality Assurance for Radiological Monitoring Programs (Normal Operations)-	4.15
Stress	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic	3.37
Strontium	Measurements of Radionuclides in the Environment-Strontium-89 and Strontium-90	4.6
Structural	(Proposed Revision 2 to Regulatory Guide 1.96) Quality Assurance Requirements	RS 908-5
Structural	Quality Assurance Requirements for Installation, Inspection, and Testing of	1.94
Structural	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels*	7.6
Structures	Load Combinations for the Structural Analysis of Shipping Casks for Radioactive	7.8
Structures	Qualifications for Cement Grouting for Prestressing Tendons in Containment	1.107
Structures	Physical Models for Design and Operation of Hydraulic Structures and Systems for	1.125
Structures	Inspection of Water-Control Structures Associated with Nuclear Power Plants*	1.127
Structures	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor	1.142
Structures	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.163
Structures	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containment	1.35
Structures	Electric Penetration Assemblies in Containment Structures for Nuclear Power	1.63
Structures	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted	1.90
Studs	Materials and Inspections for Reactor Vessel Closure Studs*	1.65
Suction	Net Positive Suction Head for Emergency Core Cooling and Containment Heat	1.1
Supply	Emergency Water Supply Systems for Fuel Reprocessing Plants*	3.31
Support	Preparation of an Environmental Report to Support a Rule Making Petition Seeking	6.7
Supported	Development of Floor Design Response Spectra for Seismic Design of Floor-	1.122
Supports	Service Limits and Loading Combinations for Class 1 Linear-Type Component	1.124
Supports	Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type	1.130
Surface	Standard Format and Content of Environmental Reports for Near-Surface Disposal	4.18
Surface	Guidance for Selecting Sites for Near-Surface Disposal of Low-Level Radioactive	4.19
Surfaces	Measurement of Radiation Levels on Surfaces of Packages of Radioactive Materials*	TP 914-4
Surveillance	Operational Inspection and Surveillance of Embankment Retention Systems for	3.11.1
Surveillance	Use of Observation (Visual Surveillance) Techniques in Material Access Areas (5.14
Survey	Guide for the Preparation of Applications for Licenses for the Use of	FC 413-4
Surveys	Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and	8.21
Surveys	Radiation Safety Surveys at Medical Institutions*	8.23
Surveys	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel	8.24
Surveys	Health Physics Surveys in Uranium Mills (Draft OH 710-4 published 8-80)*	8.30
Symbol	Radiation Symbol*	8.1
Tailings	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4
Tailings	Operational Inspection and Surveillance of Embankment Retention Systems for	3.11.1
Tailings	Calculation of Radon Flux Attenuation by Earthen Uranium Mill Tailings Covers (3.64
Tank	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Task	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Task	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Technical	Reporting of Operating Information-Appendix A Technical Specifications (for	1.16

KINOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Technical	Development of Technical Specifications for Experiments in Research Reactors*	2.2
Technical	Content of Technical Specifications for Fuel Reprocessing Plants*	3.6
Technical	Environmental Technical Specifications for Nuclear Power Plants (for Comment)*	4.8
Teletherapy	Guide for the Preparation of Applications for Licenses for Medical Teletherapy	FC 414-4
Temperature	Control of Preheat Temperature for Welding of Low-Alloy Steel*	1.50
Temperature	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors	1.87
Temperature	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for (Proposed Revision 3 to Regulatory Guide 1.35) Inservice Inspection of Ungrouted	3.29
Tendons	Qualifications for Cement Grouting for Prestressing Tendons in Containment	SC 810-4
Tendons	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containment	1.35
Tendons	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted	1.90
Term	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant	1.82
Termination	Termination of Operating Licenses for Nuclear Reactors*	1.86
Terminology	Statistical Terminology and Notation for Special Nuclear Materials Control and	5.3
Terms	Quality Assurance Terms and Definitions*	1.74
Terms	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
Terrestrial	Terrestrial Environmental Studies for Nuclear Power Stations*	4.11
Tetrafluoride	Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF ₄)	5.4
Thermal	Thermal Overload Protection for Electric Motors on Motor-Operated Valves*	1.106
Thermal	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis	1.154
Thermal	Thermal Shock to Reactor Pressure Vessels (Safety Guide 2)*	1.2
Thermal	Nonmetallic Thermal Insulation for Austenitic Stainless Steel*	1.36
Thermoluminescence	Performance, Testing, and Procedural Specifications for Thermoluminescence	4.13
Thermosetting	Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin*	1.72
Thickness	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Thickness	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Third	Qualification and Training of Personnel for Nuclear Power Plants (Draft RS 807.	1.8
Time	Response-Time Testing of Protection System Instrument Channels*	IC 121-5
Topical	Standard Format and Content for a Topical Safety Analysis Report for a Spent	3.61
Tornado	Tornado Design Classification*	1.117
Tornado	Design Basis Tornado for Nuclear Power Plants*	1.76
Toughness	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Toughness	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Toxic	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
Traceability	Considerations for Establishing Traceability of Special Nuclear Material	5.58
Training	Training and Certification of Independent Spent Fuel Storage Installation	HF 608-4
Training	Radiation Protection Training for Personnel Employed in Medical Facilities*	OP 212-4
Training	Qualification and Training of Personnel for Nuclear Power Plants (Draft RS 807.	1.8
Training	Shield Test Program for Evaluation of Installed Biological Shielding in Research	2.1
Training	Training, Equipping, and Qualifying of Guards and Watchmen*	5.20
Training	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power	8.27
Trajectory	Protection Against Low-Trajectory Turbine Missiles*	1.115
Transducers	Installation of Transducers*	IC 131-5
Transfer	Evaluation of Shipper-Receiver Differences in the Transfer of Special Nuclear	5.28
Transfers	Internal Transfers of Special Nuclear Material (for Comment)*	5.49
Transient	Physical Protection for Transient Shipments (Draft SG 126-4 published 9-81)*	5.63
Transit	Standard Format and Content of a Licensee Physical Protection Plan for Strategic	5.60
Transport	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-4
Transport	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Transport	Communication with Transport Vehicles*	5.32
Transport	Establishing Quality Assurance Programs for Packaging Used in the Transport of	7.10

KJOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Transportation	Evaluations of Explosions Postulated To Occur on Transportation Routes Near	1.91
Transportation	Standard Format and Content of Safeguards Contingency Plans for Transportation (5.56
Transportation	Packaging and Transportation of Radioactively Contaminated Biological Materials*	7.2
Transporting	Criticality Safety for Handling, Storing, and Transporting LWR Fuel at Fuels and	3.58
Transporting	Administrative Guide for Packaging and Transporting Radioactive Material*	7.1
Tritium	Criteria for Establishing a Tritium Ricsessay Program (Draft OP 713-4 published 6-	8.32
Truck	Truck Identification Markings*	5.17
Tubes	Bases for Plugging Degraded PWR Steam Generator Tubes (for Comment)*	1.121
Tubes	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes*	1.83
Turbine	Protection Against Low-Trajectory Turbine Missiles*	1.115
Ultimate	Ultimate Heat Sink for Nuclear Power Plants (for Comment)*	1.27
Ultrasonic	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice	1.150
Unaccounted	Statistical Evaluation of Material Unaccounted For*	5.33
Ungrouted	(Proposed Revision 3 to Regulatory Guide 1.35) Inservice Inspection of Ungrouted	SC 810-4
Ungrouted	Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containment	1.35
Unirradiated	Standard Format and Content of License Applications for Storage Only of	3.15
Unit	Guidance to Operators at the Controls and to Senior Operators in the Control	1.114
Unit	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power	1.81
Units	(Proposed Revision 3 to Regulatory Guide 1.9) Selection, Design, Qualification,	RS 802-5
Units	Periodic Testing of Diesel Generator Units Used as Onsite Electric Power Systems	1.108
Units	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Units	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Units	Selection, Design, and Qualification of Diesel-Generator Units Used as Standby (1.9
Upgrade	Intent and Scope of the Physical Protection Upgrade Rule Requirements for Fixed	5.61
Uranium	Guidelines for Ground-Water Monitoring at In Situ Uranium Solution Mines*	ES 114-4
Uranium	Design, Installation, and Inspection of Seepage Control Liners at Uranium	MS 146-4
Uranium	(Proposed Revision 2 to Regulatory Guide 3.5) Standard Format and Content of	WM 039-4
Uranium	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in	2.3
Uranium	Design, Construction, and Inspection of Embankment Retention Systems for Uranium	3.11
Uranium	Operational Inspection and Surveillance of Embankment Retention Systems for	3.11.1
Uranium	Standard Format and Content of Safety Analysis Reports for Uranium Enrichment	3.25
Uranium	Assumptions Used for Evaluating the Potential Radiological Consequences of	3.34
Uranium	Standard Format and Content of License Applications, Including Environmental	3.46
Uranium	Nuclear Criticality Control and Safety of Homogeneous Plutonium- Uranium Fuel	3.67
Uranium	Standard Format and Content of License Applications for Uranium Mills (for	3.5
Uranium	Calculational Models for Estimating Radiation Doses to Man from Airborne	3.51
Uranium	Standard Format and Content for the Health and Safety Sections of License	3.52
Uranium	Standard Format and Content for the Health and Safety Sections of License	3.55
Uranium	General Guidance for Designing, Testing, Operating, and Maintaining Emission	3.56
Uranium	Methods for Estimating Radioactive and Toxic Airborne Source Terms for Uranium	3.59
Uranium	Onsite Meteorological Measurement Program for Uranium Recovery Facilities- Data	3.63
Uranium	Calculation of Radon Flux Attenuation by Earthen Uranium Mill Tailings Covers (3.64
Uranium	Preparation of Environmental Reports for Uranium Mills*	3.8
Uranium	Radiological Effluent and Environmental Monitoring at Uranium Mills*	4.14
Uranium	Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in	4.16
Uranium	Preparation of Environmental Reports for Commercial Uranium Enrichment Facilities	4.9
Uranium	Nondestructive Uranium-235 Enrichment Assay by Gamma-Ray Spectrometry (Draft SG	5.21
Uranium	Materials Protection Contingency Measures for Uranium and Plutonium Fuel	5.30
Uranium	In Situ Assay of Enriched Uranium Residual Holdup (Draft SG 047-4, Proposed	5.37
Uranium	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray	5.38
Uranium	Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF6)	5.4

INDEX OF NUCLEAR REGULATORY GUIDE TITLES

Uranium	Standard Format and Content for the Special Nuclear Material Control and	5.45
Uranium	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis	5.5
Uranium	Applications of Bioassay for Uranium*	8.11
Uranium	Bioassay at Uranium Mills (Draft OP 013-4, Proposed Revision 1, published 1-87)*	8.22
Uranium	Health Physics Surveys During Enriched Uranium-235 Processing and Fuel	8.24
Uranium	Health Physics Surveys in Uranium Mills (Draft OM 710-4 published 8-80)*	8.30
Uranium	Information Relevant to Ensuring that Occupational Radiation Exposures at	8.31
Uranyl	General Methods for the Analysis of Uranyl Nitrate Solutions for Assay, Isotopic	5.39
Values	Damping Values for Seismic Design of Nuclear Power Plants*	1.61
Values	Assessment of the Assumption of Normality (Employing Individual Observed Values)*	5.22
Valve	Functional Specification for Active Valve Assemblies in Systems Important to	1.168
Valve	Qualification Tests of Electric Valve Operators Installed Inside the Containment	1.73
Valve	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Valves	Identification of Valves for Inclusion in Inservice Testing Programs*	MS 901-4
Valves	Thermal Overload Protection for Electric Motors on Motor-Operated Valves*	1.106
Vapors	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel	3.7
Vehicle	Specially Designed Vehicle with Armed Guards for Road Shipment of Special	5.31
Vehicles	Communication with Transport Vehicles*	5.32
Ventilation	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Ventilation	General Design Guide for Ventilation Systems of Plutonium Processing and Fuel	3.12
Ventilation	General Design Guide for Ventilation Systems for Fuel Reprocessing Plants (for	3.32
Verification	Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in	2.3
Verify	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify	1.61
Verifying	Administrative Guide for Verifying Compliance with Packaging Requirements for	7.7
Vessel	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice	1.150
Vessel	Materials and Inspections for Reactor Vessel Closure Studs*	1.65
Vessel	Radiation Embrittlement of Reactor Vessel Materials (Draft ME 305-4, Proposed	1.99
Vessels	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Vessels	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Vessels	Materials, Construction, and Testing of Concrete Containments (Articles CC-1000,	1.136
Vessels	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor	1.142
Vessels	Thermal Shock to Reactor Pressure Vessels (Safety Guide 2)*	1.2
Vessels	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels*	7.6
Vibration	Comprehensive Vibration Assessment Program for Reactor Internals During	1.20
Visual	Use of Observation (Visual Surveillance); Techniques in Material Access Areas (5.14
Vital	Vital Area Access Controls, Protection of Physical Security Equipment, and Key	5.65
Vital	Entry/Exit Control for Protected Areas, Vital Areas, and Material Access Areas (5.7
Volume	Calibration and Error Limits of Air Sampling Instruments for Total Volume of Air	8.25
Wall	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7001
Wall	Fracture Toughness Criteria for Ferritic Steel Shipping Cask Containment Vessels	DG-7002
Waste	(Proposed Revision 1 to Regulatory Guide 3.50) Guidance on Preparing a License	CE 402-4
Waste	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.143
Waste	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-	1.26
Waste	Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel	3.10
Waste	Guide for Acceptable Waste Storage Methods at UF6 Production Plants*	3.13
Waste	Standard Format and Content of Site Characterization Plans for High-Level-Waste	4.17
Waste	Standard Format and Content of Environmental Reports for Near-Surface Disposal	4.18
Waste	Guidance for Selecting Sites for Near-Surface Disposal of Low-Level Radioactive	4.19
Waste	Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste (5.11
Waste	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Watchmen	Training, Equipping, and Qualifying of Guards and Watchmen*	5.20

KWN INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Water	Guidelines for Ground-Water Monitoring at In Situ Uranium Solution Mines*	ES 116-6
Water	Guidelines for Modeling Ground-Water Transport of Radioactive and Nonradioactive	ES 115-6
Water	(Proposed Revision 1 to Regulatory Guide 1.131) Qualification Tests of Electric	RS 050-2
Water	Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power	1.10
Water	Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents	1.111
Water	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents	1.112
Water	Inspection of Water-Control Structures Associated with Nuclear Power Plants*	1.127
Water	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-	1.131
Water	Loose-Part Detection Program for the Primary System of Light-Water-Cooled	1.133
Water	Normal Water Level and Discharge at Nuclear Power Plants (for Comment)*	1.135
Water	Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System	1.140
Water	Design Guidance for Radioactive Waste Management Systems, Structures, and	1.163
Water	Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis	1.154
Water	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases	1.21
Water	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.24
Water	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.25
Water	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-	1.26
Water	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.3
Water	Quality Assurance Requirements for Cleaning of Fluid Systems and Associated	1.37
Water	Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and	1.38
Water	Housekeeping Requirements for Water-Cooled Nuclear Power Plants*	1.39
Water	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.4
Water	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment	1.40
Water	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.5
Water	Design, Testing, and Maintenance Criteria for Postaccident Engineered-Safety-	1.52
Water	Quality Assurance Requirements for Protective Coatings Applied to Water-Cooled	1.54
Water	Maintenance of Water Purity in Boiling Water Reactors (for Comment)*	1.56
Water	Initial Test Programs for Water-Cooled Reactor Power Plants*	1.68
Water	Preoperational and Initial Startup Testing of Feedwater and Condensate Systems	1.68.1
Water	Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-	1.68.2
Water	Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized	1.77
Water	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water	1.79
Water	Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant	1.82
Water	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes*	1.83
Water	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water	1.96
Water	Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and	1.97
Water	Assumptions Used for Evaluating the Potential Radiological Consequences of a	1.98
Water	Emergency Water Supply Systems for Fuel Reprocessing Plants*	3.31
Water	Standard Format and Content for the Safety Analysis Report for an Independent	3.44
Water	Design of an Independent Spent Fuel Storage Installation (Water-Basin Type) (3.49
Water	Reporting Procedure for Mathematical Models Selected to Predict Heated Effluent	4.6
Water	Serial Numbering of Fuel Assemblies for Light-Water-Cooled Nuclear Power Reactors	5.1
Water	Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants-	8.19
Water	Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power	8.27
Weld	Control of Ferrite Content in Stainless Steel Weld Metal*	1.31
Weld	Control of Electroslag Weld Properties*	1.34
Weld	Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components*	1.43
Welder	Welder Qualification for Areas of Limited Accessibility*	1.71
Welder	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28
Welding	Control of Preheat Temperature for Welding of Low-Alloy Steel*	1.50
Welding	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel	3.28

KWOC INDEX ON NUCLEAR REGULATORY GUIDE TITLES

Welding	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for	3.29
Welds	Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice	1.150
Welds	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel	3.27
Met	Guide for the Preparation of Applications for Licenses for the Use of Panoramic	FC 403-4
Met	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material	5.25
X-Ray	Guide for the Preparation of Applications for Licenses for the Use of Sealed	FC 05-4