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Calculation Notes for Subsurface Leak Resulting in Pool, TWRS FSAR Accident Analysis

Brett Hall

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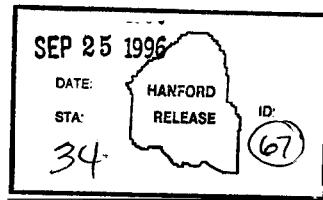
Abstract: This document includes the calculations performed to quantify the risk associated with the unmitigated and mitigated accident scenarios described in the TWRS FSAR for the accident analysis titled: "Subsurface Leaks Resulting in Pool."

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WHC-SD-WM-CN-057, Rev. 0

**CALCULATION NOTES FOR SUBSURFACE LEAK RESULTING IN POOL,
TWRS FSAR ACCIDENT ANALYSIS**

Rev. 0

Safety Analysis and Nuclear Engineering

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1.0 INTRODUCTION

This document contains the calculation notes that support the accident analysis titled "Subsurface Leak Resulting in Pool" in the TWRS FSAR. These calculation notes include the consequence and frequency analysis performed for both the mitigated and unmitigated accident scenarios described in the FSAR.

2.0 UNMITIGATED ACCIDENT ANALYSIS

2.1 SCENARIO DESCRIPTION

The bounding representative accident selected for detailed analysis is a leak from an unencased bermed line during a submersible pump transfer from an SST. The leak is assumed to be catastrophic (i.e., 100 % of pipe flow is assumed to leak into the soil). The leak is postulated to erode away the embankment material. The waste spreads out on the soil surface. It is postulated that the leak occurs when the ambient temperature is significantly below the saturation temperature of the waste. The dissolved salts in the waste crystallize out of solution as the pool spreads, limiting the penetration of waste into the soil. While the pump is running it is assumed the surface of the pool remains liquid.

After the leak is detected and the transfer pump is shut off, it is postulated the remaining liquid waste soaks into the soil. Due to the filtering effect of the soil, the crystallized salts and any entrained solid precipitates carried in the waste stream are postulated to be deposited in the top few centimeters of soil. While the pump is running and the pool is growing, aerosols are resuspended by wind blowing over the liquid surface. After the pump is shutdown and the waste soaks in, particulate surface contamination is resuspended, as the soil dries out. This occurs for an indefinite period of time [until the leak site is remediated (e.g., sprayed with water to limit resuspension, covered with gravel)].

The onsite and offsite receptors are exposed to resuspended aerosols and particulates from the liquid pool and contaminated soil. The onsite receptor is also exposed to significant shine and skyshine dose from the liquid pool and contaminated soil. Shine and skyshine dose to the offsite receptor will be negligible due to the distance to that receptor.

2.2 FREQUENCY CATEGORY

The initiating event, leak from unencased bermed line, is considered to be an Anticipated event (frequency range 1 to 1E-2 events/yr), based on Site experience. The transfer lines used for interim stabilization of SSTs are beyond their intended 5 year design life (WHC-SD-WM-SAR-034 1989). Since these lines are in contact with the soil, they are prone to corrosion failures.

The table in Appendix A lists the events from the occurrence reporting database (between 1972 and 1996) where unencased bermed or buried transfer lines were found to have leaked during waste transfers or pressure testing with water. Eight events occurred over the 24 year period. Four events between 1992 and 1996. The leak frequency over the 1992 to 1996 time frame is judged to be representative of future SST transfers.

2.3 SOURCE TERM DEVELOPMENT

2.3.1 Waste Inventory and Solids Content

Unencased bermed transfer lines will only be used for transfers from SSTs. The radionuclide inventory of SST waste, by solids and liquids fractions, are summarized in Table 2-1. The inhalation unit liter dose (ULD) values for the solids and liquids fractions are summarized in Table 2-2.

For this analysis, it is assumed that the waste being transported contains one-third (by volume) entrained solids. This is the maximum solids loading expected during tank waste transfers. This solids loading is conservative because it is above the upper operating limit of the 242-A evaporator (limit = 30 vol % solids), which handles the most concentrated waste transported in the tank farms.

2.3.2 Final Surface Pool Volume and Dimensions

The size of the pool formed in the unmitigated accident is dependent on several factors, including: the leak flow rate, the topography of the soil, the natural infiltration rate of the soil, the salt content of the waste, the temperature of the waste, and the ambient temperature. Many different leak volumes and pool dimensions can be postulated for the unmitigated accident. Some perspective on the volume and dimensions of a surface pool that can be formed during waste leaks can be gained by examining the occurrence report database. Atlantic Richfield Hanford Company Occurrence report 73-78 (available in the DOE reading Room) and ARH-2977 RD (1974) describe an incident where an estimated 8600 gal of waste escaped from an underground pipeline to the soil surface in the S Tank Farm. The flow covered an irregularly shaped area with maximum dimensions of about 50 feet by 200 ft. The depth of penetration into the soil varied from about 2 inches to 18 inches. In another incident at the 242-S evaporator (described in Rockwell Occurrence Report ROR-80-2788), an estimated 2000 gallons of waste were released from a buried unencased transfer line. This incident created a surface pool containing an estimated 200 gallons of waste (i.e., 10 % of the subsurface leak surfaced).

Neither of the above described leaks, however, adequately represent the magnitude of a waste leak that can be formed in an unmitigated event. Both leaks were manually detected. At a maximum submersible pump flowrate of 50 gpm (see Section 2.3.2), a 2000 gallon leak could occur in just 40 min, a 8600 gal leak in 2.7 hours.

In an incident described in Rockwell Occurrence Report 79-02, raw water was left running inside a service pit in the tank farms. The raw water overflowed the service pit producing a surface pool covering an estimated surface area of 3000 yd² before it was discovered. The leak occurred under freezing conditions. The pool was estimated to contain 37,500 gal of water.

For the unmitigated accident analysis, it is assumed that the leak ultimately results in a surface pool of waste of the same dimensions as the raw water leak described in Rockwell Occurrence Report 79-02. This will result in reasonably conservative dose estimates, as the total leak volume used in the unmitigated accident analysis will be roughly 4 times larger than the largest surface waste leak recorded in the database. Although even larger leak volumes could be postulated, the consequence analysis below shows that the 37,500 gal leak size is sufficient to determine the need for safety class controls. Using the raw water leak incident as a basis, it is assumed the unmitigated waste leak covers a surface area of soil equal to 3000 yd².

For the purposes of estimating shine and skyshine doses the pool is assumed to be circular. The radius of this pool is estimated from the surface area as follows:

$$r = [(3000 \text{ yd}^2)(3 \text{ ft/yd})^2 / (3.14)]^{1/2} = 93 \text{ ft} = 28 \text{ m}$$

2.3.3 Time Required for Pool to Form

It is postulated that the leak occurs during a submersible pump transfer from an SST. The submersible pumps used for interim stabilization of the SSTs pump at much higher flowrates than the SST saltwell pumps. Appendix B contains a calculation that shows that the maximum flowrate that could be expected in a catastrophic pipe break right outside the pump pit will amount to less than 50 gpm for the submersible pump transfer system used to pump out Tank BX-106.

The minimum amount of time required for the unmitigated surface pool to form can be determined by dividing the assumed total leak volume by the maximum submersible pump flowrate:

$$\text{Time required for pool to form} = 37,500 \text{ gal} / 50 \text{ gpm} = 750 \text{ min or } 12.5 \text{ h.}$$

2.3.4 Resuspension Release from Surface of Growing Liquid Pool

The waste is postulated to spread out as a liquid pool until the pump is shutdown at 12.5 hours. A resuspension release flux of 2E-10 kg/s-m² is used to estimate the respirable release from the growing liquid pool. This resuspension flux was estimated from Figure 3-8 in DOE-HDBK-3010-94, corresponding to the mass release for a pond with a 200 m fetch at wind speeds less than 5 m/s. This resuspension flux is conservative for this accident where the radius of the pool is 28 m and 99.5 percentile meteorology is assumed in performing dose calculations. 99.5 percentile meteorology corresponds to wind speeds less than 1 m/s.

The aerosol resuspension release rate from the surface of the growing pool varies with time, as the release rate is proportional to the surface area of the pool. The time integrated dose over the 0- to 12.5 hour time period can be estimated, however, by using the average resuspension release rate over the time period of interest. The average pool surface area is 1500 yd² (one

half the final pool surface area). The resuspension release flux is converted to a volumetric basis using a waste density of 1.4 kg/L. This is the estimated density of waste slurry containing 1/3 vol fraction solids. The volumetric resuspension flux is 1.4E-10 L/m²-s (2E-10/1.4).

The average resuspension release rate while the pool is growing is therefore

$$Q'(\text{res, pool}) = (1.4E-10 \text{ L/m}^2\text{-s})(1500 \text{ yd}^2)(3 \text{ ft/yd})^2(1 \text{ m}/3.28 \text{ ft})^2 \\ = 1.8E-7 \text{ L/s}$$

Multiplying this average release rate by the time period the pool is growing gives the total release over this phase of the accident.

$$Q(\text{res, pool}) = (1.8E-7 \text{ L/s})(60 \text{ s/min})(750 \text{ min}) = 8.1E-3 \text{ L}$$

2.3.5 Resuspension Release from Contaminated Soil After Pool Soaks In

A respirable release fraction of 8.4E-5 over 24 hours is used to estimate the resuspension release after the transfer pump is shut off. This resuspension fraction comes from HDBK-3010-94, Section 3.2.4.4. This resuspension fraction is based on experiments with UNH solutions spilled on soil and dried under low wind speed conditions (< 2.5 mph). These experiments are reported in BNWL-1732 (1973). The data collected during the experiments indicate that the resuspension release is not constant over time. Most of the resuspendable particles can be expected to come off early. Figure 2-1 provides a graph showing measured resuspension rates with time. This graph is reproduced from BNWL-1732. Based on this graph, 50 % of the release can be expected to occur in the first 2 hours after the waste soaks in and dries out. 80 % can be expected to come off in the first 12 hours after soaking in. Ninety nine percent of the total release can be expected to occur within 16 hours after the waste soaks into the soil.

The release quantities, in equivalent L of waste, over the various time periods of interest are calculated by multiplying the total leak volume by the appropriate respirable release fraction, correcting for the time varying nature of the release. The total release from the contaminated soil over a 24 hour time frame after soaking in (i.e., in the 12.5-h to 36.5-h time frame) is:

$$Q(\text{res, soil, tot}) = (37,500 \text{ gal})(3.8 \text{ L/gal})(8.4E-5) = 12 \text{ L}$$

The total release quantity is used in Section 3 to estimate the offsite receptor dose. The resuspension release is essentially over 28.5 hours after the leak breaks through the berm, because 99% of the dose occurs in the first 16 hours after the waste soaks into the ground (which occurs at 12.5 h). Long term resuspension from contaminated soil can be expected to add insignificantly to the release estimated above.

The onsite receptor can be exposed to resuspended particles for a 12 h duration (approximately 1 work shift). The release over the 12 hour time frame is 80 % of the total release:

$$Q(\text{res, soil, 12.5- to 24.5-h}) = (37,500 \text{ gal})(3.8 \text{ L/gal})(8.4E-5)(0.8) \\ = 9.6 \text{ L}$$

2.3.6 Maximum Release Rate For Estimating Toxicological Exposures

The toxicological consequences of the accident are determined based on the maximum release rate (see Section 3). Resuspension from the contaminated soil surface results in a larger release than resuspension of the surface of the liquid pool. The maximum resuspension release rate during the accident occurs in the 12.5 to 14.5 h time frame. During this 2-h time frame, half of the total soil resuspension volume occurs. The maximum resuspension release rate is:

$$Q'(\text{res, soil, max}) = [(6 \text{ L})/(2h)](1h/3600 \text{ s}) = 8.3E-4 \text{ L/s}$$

2.4 CONSEQUENCE ANALYSIS

The onsite and offsite receptor receive an inhalation dose from resuspended aerosols and particulates. In accordance with the methodology outlined in SARR-037, the offsite receptor is assumed to be subject to a 24-h uptake ingestion dose in addition to the inhalation dose. The onsite receptor will receive significant external dose due to direct shine and skyshine. The shine and skyshine dose from the pool and contaminated soil will be negligible to the offsite receptor due to the extreme distance to that receptor. Both receptors can be effected by the chemicals and heavy metals in the release as well as by the radioactive components of the waste.

The following sections estimate the dose and toxicological effects to the onsite and offsite receptors.

2.4.1 Onsite and Offsite Receptor Inhalation Dose Estimates

The onsite and offsite receptor inhalation doses are calculated in accordance with the methodology outlined in WHC-SD-WM-SARR-016 (1996) and WHC-SD-WM-SARR-037 (1996). The inhalation doses are calculated using the following equation:

$$D_{\text{inh}} = (Q)(X/Q')(BR)(ULD_{\text{inh}})$$

Where,

D_{inh} = dose, in Sv (50-yr CEDE)

Q = respirable release volume, in equivalent L of waste

X/Q' = atmospheric dispersion coefficient, in m^3/s

BR = receptor breathing rate, in m^3/s

ULD_{inh} = inhalation unit liter dose, in Sv/L

The respirable release volumes are determined in Section 2.3. The acute X/Q's for the onsite and offsite receptors from SARR-016 are reproduced in Table 2-3. The active man's breathing rate is $3.3E-4 \text{ m}^3/\text{s}$. The inhalation ULDs for each tank waste type, by solids and liquids fractions, are reproduced from SARR-037 in Table 2-2.

The material released in this accident is SST waste with an estimated maximum solids content of 33 vol %. The composite ULD_{inh} for the SST slurry is estimated by combining the SST ULDs for solids and liquids (from Table 2-2) in their appropriate volume ratios, as follows:

$$ULD_{inh}(\text{SST slurry}) = (0.67)(1.1E4 \text{ Sv/L}) + (0.33)(2.2E5 \text{ Sv/L}) \\ = 8.0E4 \text{ Sv/L}$$

2.4.1.1 Onsite Receptor Inhalation Dose. The total duration of the accident is 26.5 h. The onsite receptor is assumed to be exposed for a duration of 12 hours (approximately 1 shift). The worst case release occurs in the 12 hours after the liquid pool soaks into the ground. From Section 2.3.4, the 12 h resuspension release from the contaminated soil is 9.6 L. To simplify the calculation, the acute X/Q', with plume meander, from Table 2-3 is used. It is appropriate to apply plume meander because the release period exceeds 1 hour. The inhalation dose to the maximally exposed onsite receptor therefore is:

$$D(\text{on, inh}) = (9.6 \text{ L})(1.13E-2 \text{ s/m}^3)(3.3E-4 \text{ m}^3/\text{s})(8.0E4 \text{ Sv/L}) \\ = 2.9 \text{ Sv (290 rem)}$$

2.4.1.2 Offsite Receptor Inhalation Dose. The offsite receptor is exposed for the duration of the accident. This receptor receives an inhalation dose from aerosols resuspended from the surface of the pool while it is growing and in liquid form and an inhalation dose from resuspended contaminated soil after the pool soaks in. From Section 2.3.3, the release from the liquid pool is $8.1E-3 \text{ L}$. From Section 2.3.4, the total release from contaminated soil is 12 L. Using the acute offsite X/Q', with plume meander, the inhalation dose to the offsite receptor is:

$$D(\text{off, inh}) = (8.1E-3 \text{ L} + 12 \text{ L})(2.12E-5 \text{ s/m}^3)(3.3E-4 \text{ m}^3/\text{s})(8.0E4 \text{ Sv/L}) \\ = 6.7E-3 \text{ Sv (0.67 rem)}$$

2.4.2 Offsite Receptor Ingestion Dose Estimate

The offsite receptor is assumed to be exposed to a 24 h uptake ingestion dose as well as to an inhalation dose. This is in accordance with SARR-037. The dose effect due to ingestion is estimated with the following equation:

$$D_{ing} = (Q)(X/Q')(ULD_{ing})$$

Where,

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D_{ing} = 24-h uptake ingestion dose, in Sv (50-yr CEDE)
 Q = respirable release volume, in equivalent L of waste
 X/Q' = atmospheric dispersion coefficient, in s/m^3
 ULD_{ing} = inhalation unit liter dose, in $Sv \cdot m^3/s \cdot L$

The material released in this accident is SST waste with an estimated maximum solids content of 33 vol %. The composite ULD_{ing} for the SST slurry is estimated by combining the SST ULDs for solids and liquids (from Table 2-2) in their appropriate volume ratios, as follows:

$$ULD_{ing}(\text{SST slurry}) = (0.67)(0.052 \text{ Sv} \cdot m^3/s \cdot L) + (0.33)(4.1 \text{ Sv} \cdot m^3/s \cdot L) \\ = 1.4 \text{ Sv} \cdot m^3/s \cdot L$$

This ULD_{ing} includes external exposure due to shine from particles deposited on the ground around the receptor. The offsite X/Q and the total resuspension release volume are the same as used previously. The ingestion dose to the offsite receptor is:

$$D(\text{ing, off}) = (8.1E-3 \text{ L} + 12 \text{ L})(2.12E-5 \text{ s/m}^3)(1.4 \text{ Sv} \cdot m^3/s \cdot L) \\ = 3.6E-4 \text{ Sv (3.6E-2 rem)}$$

2.4.3 Direct Shine Dose to Onsite Receptor

The direct shine dose to the onsite receptor is estimated using the MICROSHEILD computer program (Grove 1992). WHC-SD-WM-SARR-016 (1996) contains a description of the calculational methods used in this codes. Two MICROSHEILD runs are performed. The first run is made to calculate the dose contribution from the important gamma emitters. The second run is made to calculate the dose contribution from bremsstrahlung radiation produced during the decay of Sr-90/Y-90.

2.4.3.1 Shine Dose Due to Gamma Emitters. The onsite receptor receives the maximum inhalation dose during the 12 hours after the pool soaks into the ground. The direct shine dose is therefore calculated for the same time period.

2.4.3.1.1 MICROSHEILD Input. The shine dose at the receptor location is dependent on the source geometry, the location of the receptor with respect to the source volume, the activity of gamma emitters in the source volume, and the source and shield material characteristics and densities.

Source Geometry. As discussed previously the pool is postulated to assume a circular shape with a radius of 93 ft (28 m). The source volume is therefore modelled as a disk 28 m in diameter. The height of the disk (i.e., depth of penetration of waste into the soil) is given by the following equation:

$$d = V/(\pi)(r^2)(v)$$

Where d = depth of waste penetration into the soil, V = volume of waste in circular size pool, $\pi = 3.14$, r = radius, and v = void volume of the soil. The void volume of Hanford soil is estimated to range from 0.3 to 0.4. For this analysis, a void volume (porosity) of 0.4 is assumed. This is conservative because it results in the shallowest penetration of waste into the soil. Inserting the appropriate parameters gives:

$$d = (37,500 \text{ gal})(1 \text{ ft}^3/7.48 \text{ gal})/[(3.14)(93 \text{ ft})^2(0.4)] \\ = 0.46 \text{ ft} = 5.5 \text{ in (14 cm)}$$

Onsite Receptor Location. The onsite receptor is assumed to be 100 m from the site of the leak. The circular pool is assumed to form between the berm and the receptor. The receptor is therefore 72 m [100 - 28] from the center of the pool and 44 m [100 - 2(28)] from the leading edge of the pool. The dose rate is calculated for a point 1.5 m off the ground to determine effects to the trunk of the receptor.

Source Activities. The SST radionuclide inventories from Table 2-1 are used to define the gamma emitter activities in the source volume. For the composite SST material containing 33 vol % solids, the following activities are estimated by multiplying the solids and liquids concentrations by the appropriate volume fraction. The daughter Ba-137m is in equilibrium with its parent Cs-137 at a 0.946 ratio. The source volume activities for the important gamma emitters are as follows:

$$\text{Cs-137 activity} = [(1.0E11 \text{ Bq/L})(0.33) + 2.2E10 \text{ Bq/L}(0.67)] \\ \times (37,500 \text{ gal})(3.79 \text{ L/gal}) = 6.8E15 \text{ Bq}$$

$$\text{Ba-137m activity} = (0.946)(\text{Cs-137 activity}) = 6.4E15 \text{ Bq}$$

$$\text{Eu-154 activity} = [(5.8E9 \text{ Bq/L})(0.33) + (2.4E9 \text{ Bq/L})(0.67)] \\ \times (37,500 \text{ gal})(3.79 \text{ L/gal}) = 5.0E14 \text{ Bq}$$

$$\text{Co-60 activity} = [(4.2E8 \text{ Bq/L})(0.33) + (9.5E6 \text{ Bq/L})(0.67)] \\ \times (37,500 \text{ gal})(3.79 \text{ L/gal}) = 2.1E13 \text{ Bq}$$

Shield Materials. The waste is modelled in the shine runs as water with a density of 1.4 g/cc (the estimated density of a waste slurry containing 33 vol % solids). Normalizing across the soil volume gives an overall waste density of $(0.4)(1.4 \text{ g/cc}) = 0.56 \text{ g/cc}$. The soil in the source volume is modelled as concrete with a density of 1.6 g/cc (the estimated soil bulk density).

The soil between the edge of the pool and the receptor is modelled as a side clad shield consisting a concrete with an overall density of 1.6 g/cc as in the source volume.

2.4.3.1.2 MICROSHIELD Results. The input and output file from the MICROSHIELD run are included in Appendix C, Case 1. The cylindrical source

was divided into 10 radial, 10 circumferential, and 10 axial kernels or segments.

The total exposure rate, with buildup, was found to be 275 mR/h. Exposure in Roentgen is conservatively converted to effective dose equivalent, in rem, using a conversion factor of 1. Multiplying the exposure rate by the exposure duration of 12 h gives a total shine dose, due to gamma emitters, of 3300 mrem, or 33 mSv.

2.4.3.2 Shine Dose Due to Bremsstrahlung Radiation. The shine dose due to brehmstrahlung radiation was estimated with MICROSCHILD using the same geometries and shield material densities used to estimate the direct shine dose.

Appendix D includes output generated by BREMCALC (Rittman 1992) of the photon production rates for 1 Ci (3.7E10 Bq) of Sr-90/Y-90, in both water and concrete. The source term in this scenario is made up of a mixture of concrete (to simulate soil) and water. It is conservative to use the photon production rate values for concrete since they are higher than those of water.

The photon production rates in the source pool can be estimated by scaling up the photon production rates for concrete in Appendix D by the total activity of Sr-90/Y-90 in the source volume. From Table 2-1, the activity concentration of Sr-90/Y-90 in the solids phase of SST waste is 1.6E12 Bq/L. In the liquids phase of SST waste, the Sr-90/Y-90 concentration is 1.1E10 Bq/L. Multiplying by the appropriate solids and liquids fractions and the total spill quantity gives the total activity of Sr-90/Y-90 in the source volume:

$$\begin{aligned}\text{Sr-90/Y-90 Activity} &= [(1.6E12 \text{ Bq/L})(0.33) + (1.1E10 \text{ Bq/L})(0.67)] \\ &= x (37,500 \text{ gal})(3.79 \text{ L/gal}) \\ &= 7.6E16 \text{ Bq} (2.05E6 \text{ Ci})\end{aligned}$$

To estimate the photon production rate of the contaminated soil, the photon production rates for concrete, from Appendix D, were scaled up by a factor of 2.05E+06. The photon production rates entered into the code, for each of the energies bins of concern, are summarized in Table 2-4.

The MICROSCHILD output file is contained in Appendix C, Case 2. From the output file, the estimated exposure rate is 40.3 mR/h, with buildup in air. Over a 12 h time period, the total dose (EDE) is approximately 480 mrem, or 4.8 mSv.

2.4.4 Skyshine Dose to Onsite Receptor

The skyshine dose from the contaminated soil is estimated using the MICROSKYSHINE computer code (1987). WHC-SD-WM-SARR-016 contains a description of the calculational methods used in this code. Again, the dose due to gamma emitters and brehmstrahlung effect are calculated separately.

2.4.4.1 Skyshine Dose Due to Gamma Emitters. The source volume geometry, receptor location, gamma emitter activities, and shield material densities are the same as in the direct shine calculations. In the MICROSKYSHINE input, a hypothetical shield wall is inserted between the source volume and the receptor that is just tall enough to preclude line of sight radiation to the receptor.

The geometrical parameters that must be entered into the MICROSKYSHINE code are depicted in Figure 2-2. For this analysis, the shield wall was arbitrarily located 1 m from the edge of the pool (the location of the wall is not important as long as it precludes line of sight radiation but is not too high to mask skyshine). For this analysis, $W = 28$ m (section 2.3.1), $R = 28 + 1 = 29$ m, $X = 100 - W - R = 71$ m, and $L = 14.5$ cm (Section 2.4.3.1.1). The dose is modelled at a point 1.5 m off the ground (Section 2.4.3.1.1). Therefore, $-H = 1.5 - Y$. The parameter Y was determined using similar triangles, with the following equation:

$$Y/(W+R) = -1.5/(W+R+X)$$

Rearranging and solving gives $Y = 0.855$ m and $H = -0.645$ m.

The results of the SKYSHINE run are included in Appendix E as Case 1. The estimated exposure rate is 1.31 R/h. Multiplying by the 12 h exposure duration and converting to absorbed dose, using a conservative conversion factor of 1 (R to rem), gives a total dose of 16 rem (160 mSv).

2.4.4.2 Skyshine Dose Due to Bremsstrahlung Radiation. The skyshine dose due to bremsstrahlung radiation is measured using the same geometrical parameters as in the previous section. The photon production rates and energy groups entered into the code are the same as those used for the direct shine doses in Section 2.4.3.3.

The output file from the MICROSKYSHINE run are included in Appendix E as Case 2. The estimated exposure rate is 0.39 R/h. Multiplying by the 12 h exposure period, using a conservative conversion factor of 1 (R to rem), gives a total dose of 4.7 rem (47 mSv).

2.4.5 Summary of Dose Estimates, by Pathway

Table 2-5 provides a summary of the onsite and offsite dose estimates, by pathway, for the unmitigated accident scenario. The shine and skyshine dose estimates in the table are the combined total from gamma emitters and bremsstrahlung radiation. The total dose to the onsite receptor due to the inhalation, shine, and skyshine pathways is 3100 mSv (310 rem). The total dose to the offsite receptor from the inhalation and ingestion pathways is 7.1E-3 Sv (0.71 rem).

2.4.6 Toxicological Exposure Estimates for the Onsite and Offsite Receptors

Onsite and offsite toxicological exposures are assessed using the sum-of-fractions methodology outlined in WHC-CM-SARR-011 (1996). Table 2-6 gives the unit liter sum-of-fractions multipliers for the various waste types, broken into liquids values and solids values. Toxicological exposure is assessed by multiplying the release rate by the appropriate sum-of-fractions multiplier from Table 2-6. (Note that the sum-of-fraction multipliers are dependent on accident frequency). Products less than one are considered to indicate acceptable risk (i.e., indicate exposures below evaluation guidelines).

The sum of fractions multipliers for SST waste containing 33 vol % solids is estimated by combining the SST solids and liquids values from Table 2-6. For the Anticipated frequency range, the sum of the fraction multiplier for the onsite receptor is $2.0E4 \text{ s/L} [(0.33)(4.0E4 \text{ s/L}) + (0.67)(1.0E4)]$. For the offsite receptor and the Anticipated frequency category, the sum of the fraction multiplier for SST waste containing 33 vol % solids is $36 \text{ s/L} [(0.33)(94) + (0.67)(8)]$.

Toxicological exposures are assessed using the maximum release rate. From Section 2.3.5, the maximum release rate of waste during the unmitigated accident is $8.3E-4 \text{ L/s}$. The onsite and offsite sum of fractions results are therefore:

$$\text{Onsite sum of fractions} = (8.3E-4 \text{ L/s})(2.0E4 \text{ s/L}) = 16.7$$

$$\text{Offsite sum of fractions} = (8.3E-4 \text{ L/s})(36 \text{ s/L}) = 3.0E-2$$

3.0 MITIGATED ACCIDENT ANALYSIS

3.1 SCENARIO

In the mitigated scenario, operator action is credited with limiting the duration of the release and limiting the volume of waste material spilled. The unencased bermed lines that may be used for interim stabilization of SSTs are of relatively short length. It is feasible to have operators or radiation protection technicians survey these lines every 30 minutes during SST transfers to detect pipe failures and berm washouts. With radiation monitors it is likely the leak can be detected before the berm washes out.

For this analysis it is assumed that the leak occurs and washes out the berm shortly after the operator passes. Flow continues for 30 minutes until the operator passes by that point at the next surveillance interval. It is also assumed that it takes 30 minutes for the appropriate transfer pump to be shutdown following detection of the leak. It is also assumed that emergency response procedures will be implemented to evacuate workers in the vicinity of the leak to upwind staging areas within 30 minutes of detection of the leak. It is assumed that radiation surveys will be performed to establish a safe distance around the leak site and that access controls will be implemented to minimize onsite worker exposures. A human factors analysis will be performed as part of the FSAR effort to verify that the transfer pump can be shutoff and emergency response procedures implemented within the 30 minute time period.

The leak occurs at the maximum flowrate of 50 gpm for 1 h, releasing 3000 gal of waste. Line holdup is also assumed to flow through the leakage path. During the first hour while the pump is running aerosols are released from the growing liquid pool. After the pump is shutoff, it is conservatively assumed, for modelling purposes, that the waste instantaneously soaks into the ground. The onsite receptor is exposed to resuspended aerosols from the growing liquid pool for one hour before being evacuated to an upwind staging area. In addition, the onsite receptor is exposed to direct shine and skyshine radiation from the growing pool for a 1 h time period. The offsite receptor is exposed to resuspended aerosols from the growing liquid pool for one hour and to resuspended particulates from the contaminated ground for an additional 24 hours. No credit is taken for emergency response to cover up the leak to minimize resuspension within the 24 h period.

3.2 FREQUENCY CATEGORY

The frequency category for the mitigated accident scenario is Anticipated, the same as in the unmitigated scenario, as no credit is taken for controls to reduce the likelihood of the accident.

3.3 SOURCE TERM DEVELOPMENT

3.3.1 Final Surface Pool Volume and Dimensions

As in the unmitigated analysis, it is conservatively assumed that the leak is catastrophic, with 100 % of pipe flow leaking onto the soil surface. The amount of waste required to saturate the berm, resulting in washout, is conservatively neglected. At the maximum flowrate of 50 gpm, the total flow over the 1 h time frame required for detection of the leak and shut off of the pump is 3000 gal [(50 gpm)(60 min)]. It is conservatively assumed that the leak occurs at the low point of the bermed pipe and that the entire line holdup drains back through the leakage path during the 1 hour time frame required to shutdown the pump. It is assumed that the transfer is made through a 3 in schedule 40 pipe that is 1000 ft long. This gives a conservative estimate for line holdup. The inner diameter of a 3 in schedule 40 pipe is 3.068 in. The volume of liquid waste that can drain back through this line is given by the following equation:

$$V(\text{drainback}) = (\pi)(d)^2 L/4$$

Where, $\pi = 3.14$, d = diameter of the pipe, L = length of the pipe. Solving gives:

$$V(\text{drainback}) = (3.14)(3.068/12 \text{ ft})^2(1000 \text{ ft})/4 = 51.3 \text{ ft}^3$$

The drainback in gallons is 384 [(51.3 ft³)(7.48 gal/ft³)].

The total volume spilled is less than 3400 gal. It is assumed that the waste forms a circular shape with an average waste depth of 1 in. This assumed pool depth is judged to be conservative because some of the waste would be expected to soak in reducing the spread of the waste, and the unevenness of Hanford terrain makes average pool depths less than 1 in unlikely. The surface area covered by the pool can be calculated by dividing the pool volume by the average depth. This gives a surface area of:

$$A = (3400 \text{ gal})(1 \text{ ft}^3/7.48 \text{ gal})/(1/12 \text{ ft}) = 5450 \text{ ft}^2$$

The radius of the pool is given by:

$$r = [5450 \text{ ft}^2/(\pi)]^{1/2} = 41.7 \text{ ft} = 12.7 \text{ m}$$

3.3.2 Resuspension Release from Growing Liquid Pool

A resuspension mass flux of 2E-10 kg/m²-s is used to estimate the release from the liquid pool while the pump is running. This is the same resuspension flux as used in the unmitigated analysis in Section 2.3.3. Converting to a volumetric basis, using a waste density of 1.4 kg/L gives a resuspension flux of 1.4E-10 L/m²-s [(2E-10 kg/m²-s)/(1.4 kg/L)]. The resuspension release rate varies with time. The total resuspension release volume over the first hour of the accident is estimated based on the average surface area of the pool over that time frame. The average surface area is 5450 ft²/2 = 2725 ft² = 830 m². Multiplying the volumetric resuspension flux by the average surface area gives the average resuspension rate of the first hour of the accident:

$$Q'(\text{res, pool}) = (1.4E-10 \text{ L/m}^2\text{-s})(830 \text{ m}^2) = 1.2E-7 \text{ L/s}$$

Multiplying by the release duration gives the total release volume.

$$Q(\text{res, pool}) = (1.16E-7 \text{ L/s})(1 \text{ h})(3600 \text{ s/h}) = 4.2E-4 \text{ L}$$

3.3.3 Resuspension Release from Contaminated Ground After Pool Soaks In

As in Section 2.3.4, a resuspension respirable fraction of 8.4E-5 over 24 hours is used to estimate the release from the pool after it soaks into the ground. Multiplying by the total release volume gives:

$$Q(\text{res, soil}) = (3400 \text{ gal})(3.79 \text{ L/gal})(8.4E-5) = 1.1 \text{ L}$$

3.3.4 Maximum Release Rate for Estimating Toxicological Exposures

For the offsite receptor, the maximum release rate occurs after the waste soaks into the soil and dries out. Based on the discussion in Section 2.3.4, 50 % of the total release above could be expected to occur over a two hour time frame. This gives a maximum release rate for the offsite receptor of 1.5E-4 L/s [(1.1 L/2h)(1 h/3600 s)].

During the hour the onsite receptor is exposed the release exists as a pool. Although some of the pool may soak into the ground while the pump is running, the ground will remain damp. The ground is not expected to dry out giving a release rate as high as estimated above for the offsite receptor. The maximum release rate for the onsite receptor (over the first hour of the accident) is 1.2E-7 L/s (Section 3.3.2).

3.4 CONSEQUENCE ANALYSIS

The receptor exposure pathways are the same as in the unmitigated analysis, Section 2.4.

3.4.1 Onsite and Offsite Receptor Inhalation Dose Estimates

3.4.1.1 Onsite Receptor Inhalation Dose. This receptor is exposed only during the first hour of the accident. The receptor is evacuated after 1 hour. The resuspension release volume during this period is 4.2E-4 L. Over this short time period, it is necessary to use the acute X/Q' , without plume meander. From Table x, this X/Q' is 3.41E-2 s/m³. Using the SST composite slurry ULD_{inh} developed earlier, and the active receptor breathing rate of 3.3E-4 m³/s, the dose to the onsite receptor is:

$$D(\text{on, inh}) = (4.2E-4 \text{ L})(3.41E-2 \text{ s/m}^3)(3.3E-4 \text{ m}^3/\text{s})(8.0E4 \text{ Sv/L}) \\ = 3.8E-4 \text{ Sv (3.8E-2 rem)}$$

3.4.1.1 Offsite Receptor Inhalation Dose. The offsite receptor receives inhalation dose due to resuspension off the liquid pool for 1 hour and due to resuspension off the contaminated soil surface for an additional 24 hours. Using the acute offsite X/Q' (with plume meander since the release is long term) from Table 2-3, the offsite receptor dose is:

$$D(\text{off, inh}) = (4.2E-4 \text{ L} + 1.1 \text{ L})(2.12E-5 \text{ s/m}^3)(3.3E-4 \text{ m}^3/\text{s})(8.0E4 \text{ Sv/L}) \\ = 6.2E-4 \text{ Sv (6.2E-2 rem)}$$

3.4.2 Offsite Receptor Ingestion Dose Estimate

From Section 2.4.2, $ULD_{\text{ing}}(\text{SST slurry}) = 1.4 \text{ Sv-m}^3/\text{s-L}$. The dose to the offsite receptor due to 24-h uptake of his own fruits and vegetables is:

$$D(\text{off, ing}) = (4.2E-4 \text{ L} + 1.1 \text{ L})(2.12E-5 \text{ s/m}^3)(1.4 \text{ Sv-m}^3/\text{s-L}) \\ = 3.3E-5 \text{ Sv (3.3E-3 rem)}$$

3.4.3 Estimate of Direct Shine Dose to the Onsite Receptor

The dose rate to the onsite receptor will vary considerably over the receptor's 1 h exposure duration, because the pool dimensions and activity change over the course of the hour. The shine dose to the onsite receptor is conservatively estimated based on the maximum dose rate for the fully formed pool, when the edge of the pool is closest to the receptor and the pool activity is maximized. As in the unmitigated analysis, the pool is postulated to be located between the berm and the receptor. The pool is postulated to assume a circular shape as it grows. The onsite receptor is located 100 m from the location on the berm where the leak occurs.

The shine doses due to gamma and bremsstrahlung radiation are estimated using the MICROSHIELD computer code, as in unmitigated accident analysis (Section 2.4.3). The dose rate is estimated for a surface pool 1 in deep (i.e., no soaking in). The pool is modelled as a disk. In this case, the maximum pool size is 3400 gal spread out over a surface area of 5450 ft². The radius of the disk is 12.7 m (Section 3.3.1). To estimate self shielding effects, the waste pool is modelled as water with a density of 1.4 g/cc. For the direct shine dose estimate, the soil between the pool and the receptor is modelled as a side clad shield of concrete with a density of 1.6 g/cc. The source volume activities for the important gamma emitters are estimated from the inventories reported in Table 2-1, as follows, for entry into the code:

$$\text{Cs-137 activity} = [(1.0E11 \text{ Bq/L})(0.33) + 2.2E10 \text{ Bq/L}(0.67)] \\ \times (3400 \text{ gal})(3.79 \text{ L/gal}) = 6.2E14 \text{ Bq}$$

$$\text{Ba-137m activity} = (0.946)(\text{Cs-137 activity}) = 5.8E14 \text{ Bq}$$

$$\text{Eu-154 activity} = [(5.8E9 \text{ Bq/L})(0.33) + (2.4E9 \text{ Bq/L})(0.67)] \\ \times (3400 \text{ gal})(3.79 \text{ L/gal}) = 4.5E13 \text{ Bq}$$

$$\text{Co-60 activity} = [(4.2E8 \text{ Bq/L})(0.33) + (9.5E6 \text{ Bq/L})(0.67)]$$

$$x (3400 \text{ gal})(3.79 \text{ L/gal}) = 1.9E12 \text{ Bq}$$

For estimating the dose due to bremsstrahlung radiation, the Sr-90/Y-90 source volume activity is estimated in a similar manner:

$$\begin{aligned} \text{Sr-90/Y-90 Activity} &= [(1.6E12 \text{ Bq/L})(0.33) + (1.1E10 \text{ Bq/L})(0.67)] \\ &= x (3400 \text{ gal})(3.79 \text{ L/gal}) \\ &= 6.9E15 \text{ Bq (1.86E5 Ci)} \end{aligned}$$

To estimate the photon production rate of the surface pool due to bremsstrahlung effect, the photon production rates for concrete, from the BREMCALC results in Appendix D, were scaled up by a factor of 1.86E+06. Using the concrete photon production rates is conservative for the liquid pool. The photon production rates entered into the code are summarized in Table 3-1.

The output files from the MICROSHIELD runs are included in Appendix C as Cases 3 and 4. The output files summarize the input parameters as well as the results of the code calculations. From the output file shown as Case 3, the maximum dose rate due to direct gamma radiation is 88 mR/h, with buildup. From the output file shown as case 4, the maximum exposure rate, in air, due to bremsstrahlung radiation is 13 mR/h. Multiplying the maximum dose rate by the exposure duration of 1 h gives a conservative estimate of the integrated exposure to the onsite receptor. The exposure rate in Roentgen is converted to effective dose equivalent, in rem, using a conservative conversion factor of 1:

$$D(\text{on, shine, gamma radiation}) = (88 \text{ mR/h})(1 \text{ h}) = 88 \text{ mrem (8.8E-01 mSv)}$$

$$D(\text{on, shine, bremsstrahlung}) = (13 \text{ mR/h})(1 \text{ h}) = 13 \text{ mrem (13E-01 mSv)}$$

3.4.4 Estimate of Skyshine Dose to the Onsite Receptor

As in the unmitigated accident analysis, the MICROSKYSHINE code was used to estimate to dose to the onsite receptor due to gamma radiation and bremsstrahlung radiation.

The source volume geometry, receptor location with respect to the source volume, gamma emitter activities, photon production rates (for estimating bremsstrahlung skyshine) and shield material densities are the same as in the direct shine calculations. The geometrical parameters required by the MICROSKYSHINE code are depicted in Figures 2-2. For this analysis, the shield wall was arbitrarily located 1 m from the edge of the pool (the location of the wall is not important as long as it precludes line of sight radiation but is not too high to mask skyshine). For this analysis, $W = 12.7 \text{ m}$ (section 3.3.1), $R1 = 12.7 + 1 = 13.7 \text{ m}$, $X = 100 - W - R1 = 73.6 \text{ m}$, and $L = 2.54 \text{ cm}$ (Section 3.3.1). The dose is modelled at a point 1.5 m off the ground. Therefore, $-H = 1.5 - Y$. The parameter Y was determined using similar triangles, with the following equation:

$$Y/(W+R1) = -1.5/(W+R1+X)$$

Rearranging and solving gives $Y = 0.396$ m and $H = -1.10$ m.

The results of the MICROSKYSHINE runs are included in Appendix E as Cases 3 and 4. The estimated exposure rate from gamma emitters was found to be 220 mR/h. The estimated dose rate from bremsstrahlung was found to be 65 mR/h. Multiplying by the onsite exposure duration of 1 hour, and converting to effective dose equivalent using a conversion factor of 1, gives the following dose estimates:

$$D(\text{on, skyshine, gamma radiation}) = (220 \text{ mR/h})(1 \text{ h}) = 220 \text{ mrem (2.2 mSv)}$$

$$D(\text{on, skyshine, bremsstrahlung}) = (65 \text{ mR/h})(1 \text{ h}) = 65 \text{ mrem (0.65 mSv)}$$

These dose estimates are conservative because they are based on the maximum exposure rate for the fully formed pool.

3.4.5 Summary of Mitigated Radiological Dose Estimates, by Pathway

Table 3-2 provides a summary of the onsite and offsite dose estimates, by pathway, for the unmitigated accident scenario. The total dose to the onsite receptor due to the inhalation, shine, and skyshine pathways is 4.4 mSv (0.44 rem). The total dose to the offsite receptor from the inhalation and ingestion pathways is 0.65 mSv (6.5E-02 rem).

3.4.6 Toxicological Exposure Estimates for the Onsite and Offsite Receptors

From Section 3.3.4, the maximum release rate for the onsite receptor (during the first hour of the accident) is 1.2E-7 L/s. The maximum release for the offsite receptor is 1.5E-4 L/s. From Section 2.4.6, onsite sum of fractions multiplier for the anticipated frequency category is 2.0E4 s/L. The offsite sum of fractions multiplier for the anticipated frequency category is 36 s/L. The onsite and offsite sum of fractions results are therefore:

$$\text{Onsite sum of fractions} = (1.2E-7 \text{ L/s})(2.0E4 \text{ s/L}) = 2.4E-3$$

$$\text{Offsite sum of fractions} = (1.5E-4 \text{ L/s})(36 \text{ s/L}) = 5.4E-3$$

4.0 COMPARISON TO EVALUATION GUIDELINES

Table 4-1 summarizes the results of the consequence analyses above and compares the consequences with the TWRS evaluation guidelines. Table 4-1 shows that the unmitigated accident produces onsite and offsite dose consequences in excess of the evaluation guideline for the Anticipated frequency category (the frequency category of the accident). The unmitigated onsite toxicological consequences are also shown to be above the evaluation guideline.

Table 4-1 shows that the controls selected to mitigate the consequences of the accident are sufficient to keep onsite and offsite radiological and toxicological exposures below the evaluation guidelines for the Anticipated frequency category.

5.0 SUMMARY OF KEY ASSUMPTIONS

This section summarizes the key parameters and assumptions used in the above unmitigated and mitigated accident analyses. The list of key parameters and assumptions define the safety envelope of the analyses and will be useful for future USQ determinations.

5.1 KEY ASSUMPTIONS, UNMITIGATED ACCIDENT ANALYSIS

In the unmitigated accident analysis, the representative case analyzed is a leak from an unencased bermed line during an SST transfer which utilizes a submersible pump (as opposed to a saltwell pump). The key assumptions in the analysis are as follows:

- The waste leaked is assumed to be SST waste, as defined in WHC-SD-WM-SARR-037 (1996), containing 33 vol % entrained solids.
- The leak is assumed to cause a washout of the berm due to "piping" phenomena.
- The leak is assumed to be catastrophic, with 100 % of pipe flow after berm washout assumed to be available to form a surface pool.
- The leak is assumed to occur at the maximum submersible pump flowrate of 50 gpm.
- The surface pool is assumed to ultimately cover a soil surface area of 3000 yd², based on the above mentioned raw water leak.
- The transfer pump is assumed to be shut off after 37,500 gal of waste has leaked to the soil surface (see Section 2.3.1 for basis), approximately 12.5 hours after the leak is initiated.
- After the transfer pump is shut off, the waste is assumed to soak into the soil, leaving entrained and dissolved salts as surface contamination.
- The maximally exposed onsite receptor is assumed to be exposed to particles resuspended from the contaminated soil surface, after the waste has soaked in, for a period of 12 hours (1 work shift with overtime).
- The maximally exposed onsite receptor is assumed to be exposed to shine and skyshine from the contaminated soil, after the waste has soaked in, for a period of 12 hours.
- The maximally exposed offsite receptor is assumed to be exposed to resuspended waste from the surface of the growing liquid pool for the first 12.5 hours of the accident and to waste particles resuspended from the contaminated soil for an additional 24 hours.

- For calculating shine and skyshine doses to the onsite receptor, the surface pool is assumed to be circular, located between the berm and the receptor.
- For calculating shine and skyshine doses, the onsite receptor is assumed to 100 m from the site of the leak (100 m from the far edge of the circular pool).
- For calculating shine and skyshine doses to the onsite receptor, the waste is assumed to soak in to the soil to an average depth of 5.5 in, based on a soil porosity of 0.4. The waste is assumed to occupy the void spaces in the soil.
- For calculating shine and skyshine doses to the onsite receptor, the soil is assumed to have shielding properties similar to concrete with a density of 1.6 g/cc.
- For calculating shine and skyshine doses to the onsite receptor, the waste is assumed to have self shielding properties similar to water with a density of 1.4 g/cc.
- Shine and skyshine doses to the offsite receptor are assumed to be negligible due to distance.

5.2 KEY ASSUMPTIONS, MITIGATED ACCIDENT ANALYSIS

The mitigated scenario credits operator surveillances performed every 30 minutes with detecting the leak. Credit is also taken for emergency response actions to evacuate the onsite receptor to an upwind staging area at a safe distance within 30 minutes of detection of the leak. The key assumptions used in the mitigated analysis are summarized as follows:

- The leak is assumed to occur in an unencased bermed transfer line during an SST transfer using a submersible transfer pump.
- The waste leaked is assumed to be SST waste, as defined in WHC-SD-WM-SARR-037 (1996), containing 33 vol % entrained solids.
- The leak is assumed to washout the berm due to "piping" phenomena.
- The leak is assumed to occur at the maximum submersible pump flowrate of 50 gpm.
- The leak is assumed to be catastrophic, with 100% of pipe flow after berm washout available to form surface pool.
- Berm washout is assumed to occur shortly after an operator surveillance.

- The leak is assumed to continue undetected until the next operator surveillance 30 minutes later.
- 30 minutes is assumed for operator action to turn off the appropriate transfer pump and to evacuate the onsite receptor to a safe distance upwind.
- 400 gal of line holdup is assumed to drain back through the leakage path after the transfer pump is shut off.
- The waste is assumed to form a liquid pool on the surface for a period of 1 hour until the transfer pump is shut off.
- The surface pool is assumed to soak into the ground after the transfer pump is shut off.
- The onsite receptor is assumed to be exposed to resuspended material from the surface of the growing liquid waste pool for a period of 1 hour. The onsite receptors exposure after 1 hour is assumed to be negligible due to emergency response.
- The onsite receptor is assumed to be exposed to shine and skyshine from the growing liquid pool for a period of 1 hour.
- The maximum offsite receptor is assumed to be exposed to resuspended aerosols from the pool surface for 1 hour and to resuspended particulates from the contaminated soil for an additional 24 hours after the waste soaks into the ground.
- For calculating shine and skyshine doses to the onsite receptor, the surface pool is assumed to be circular, located between the berm and the receptor.
- For calculating shine and skyshine doses to the onsite receptor, the pool is assumed to spread to an average thickness of 1 in.
- For calculating shine and skyshine doses, the onsite receptor is assumed to 100 m from the site of the leak (100 m from the far edge of the circular pool).
- For calculating shine and skyshine doses to the onsite receptor, the waste is assumed to have self shielding properties similar to water with a density of 1.4 g/cc.
- For calculating shine and skyshine doses to the onsite receptor, the soil between the pool and the receptor is assumed to have immersion shielding properties similar to concrete with a density of 1.6 g/cc.
- Shine and skyshine doses to the offsite receptor are assumed to be negligible due to distance.

6.0 PEER REVIEW

Appendix F contains the signed off peer review checklist for this document.

7.0 REFERENCES

ARH-2977 RD, 1974, *Report On the Cleanup Activities Following the 241-S Tank Farm Contamination Occurrence at the Hanford Reservation, Richland, Washington on November 14, 1973*, Atlantic Richfield Hanford Company, Richland, Washington.

DOE-HDBK-3010-94, 1994, *Airborne Release Fractions/Rates and Respirable Fractions for Nonreactor Nuclear Facilities*, U.S. Department of Energy, Washington, D.C.

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Rittman, P. D., 1992, *BREMCALC - A Computer Program for Calculating Electron and Positron Bremsstrahlung*, WHC-SA-1435-FP, Westinghouse Hanford Company, Richland, Washington.

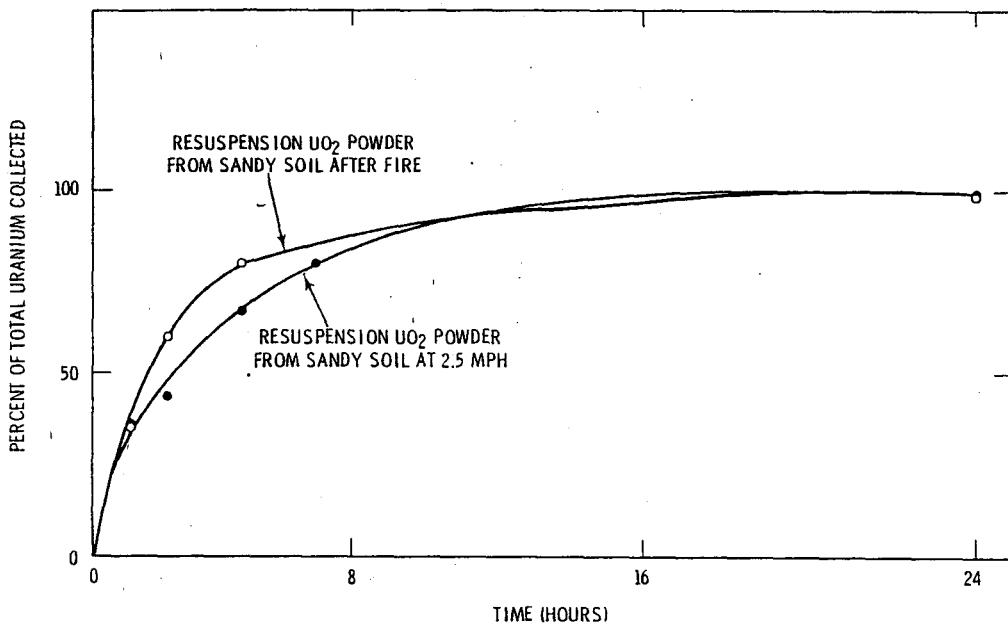
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WHC-SD-WM-SARR-016, 1996, *Tank Waste Compositions and Atmospheric Dispersion Coefficients for Use in Accelerated Safety Analysis Consequence Assessments*, Rev. 2, Westinghouse Hanford Company, Richland, Washington.

WHC-SD-WM-SARR-037, 1996, *Development of Radiological Concentrations and Unit Liter Doses for TWRS FSAR Radiological Consequence Calculations*, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

Figure 2-1. Graph of Resuspension Fraction Vs. Time



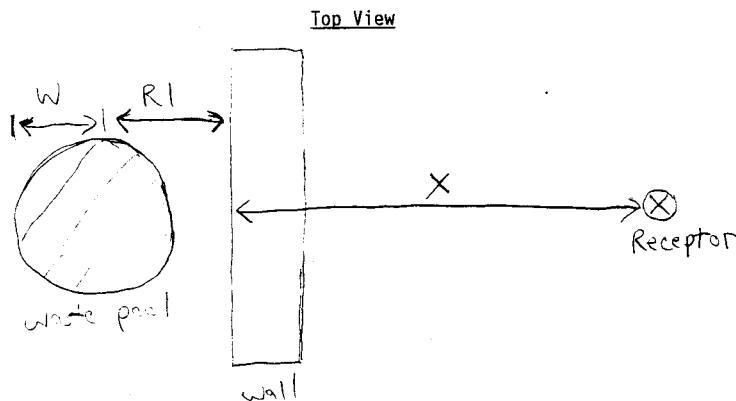
AERODYNAMIC ENTRAINMENT OF URANIUM DIOXIDE POWDER FROM SMOOTH, SANDY SOIL AT AN AIR VELOCITY OF 2.5 MPH

* From Figure 14 of BNWL-1732

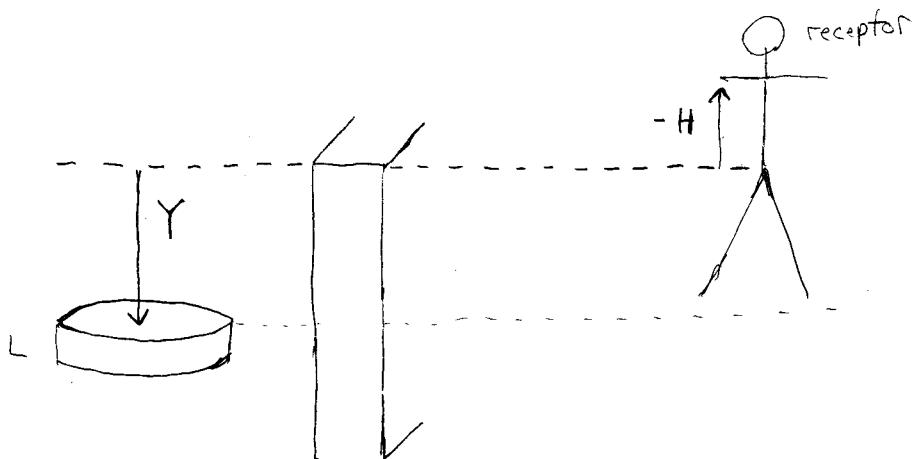
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Figure 2-2. Geometrical Parameters for MICROSKYSHINE Calculations



Side View



~~26 of 79~~

Table 2-1 Bounding Activity Concentrations, by Waste Type*

Isotope	Concentration (Bq/L)					
	SST Liquids	SST Solids	DST Liquids	DST Solids	Aging Waste Liquids	Aging Waste Solids
⁶⁰ Co	9.53 E+06	4.2 E+08	6.97 E+06	1.5 E+07	7.71 E+05	4.9 E+08
⁹⁰ Sr	1.05 E+10	1.6 E+12	4.59 E+09	5.2 E+10	5.60 E+09	2.9 E+12
⁹⁰ Y	1.05 E+10	1.6 E+12	4.59 E+09	5.2 E+10	5.60 E+09	2.9 E+12
¹³⁷ Cs	2.21 E+10	1.0 E+11	5.86 E+10	5.9 E+10	8.84 E+10	9.8 E+10
¹⁵⁴ Eu	2.35 E+09	5.8 E+09	4.18 E+07	3.0 E+08	0.00	0.00
²³⁷ Np	0.00	0.00	2.3 E+05	8.1 E+05	9.20 E+04	9.9 E+08
²³⁸ Pu	9.21 E+04	1.9 E+08	1.78 E+06	7.2 E+07	2.75 E+03	6.7 E+07
²³⁹ Pu	3.62 E+07	4.4 E+08	7.65 E+06	1.6 E+09	1.20 E+06	4.4 E+08
²⁴¹ Pu	2.57 E+08	3.2 E+09	1.84 E+07	3.8 E+09	3.39 E+05	1.7 E+09
²⁴¹ Am	4.23 E+07	2.3 E+08	3.40 E+07	2.7 E+09	1.10 E+06	1.1 E+10
²⁴⁴ Cm	4.23 E+05	2.3 E+06	1.22 E+05	1.6 E+07	1.10 E+04	6.1 E+07

*From WHC-SD-WM-SARR-037 (1996)

Table 2-2. Unit Liter Doses for Inhalation and Ingestion.

Composite	Inhalation ULD (Sv/L)	Ingestion ULD (Sv-m ³ /s-L)
Single-shell tank liquids	1.1 E+04	0.052
Single-shell tank solids	2.2 E+05	4.1
Double-shell tank liquids	6.1 E+03	0.068
Double-shell tank solids	5.3 E+05	0.48
Aging waste facility liquids	1.4 E+03	0.092
Aging waste facility solids	1.7 E+06	8.1

NOTE: The information in this table is from WHC-SD-WM-SARR-037, 1996, Development of Radiological Concentrations and Unit Liter Doses for TWRS FSAR Radiological Consequence Calculations, Westinghouse Hanford Company, Richland, Washington.

ULD = unit liter dose.

Table 2-3. Centerline Atmospheric Dispersion Coefficients for 200-Area Tank Farm Acute, Ground Level Release

Maximum individual	Bounding integrated χ/Q' (s/m^3)	Bounding integrated χ/Q' (s/m^3) with plume meander	Maximum puff χ/Q ($1/m^3$)
Onsite sector and distance	3.41 E-02 E 100 m	1.13 E-02 ESE 100 m	9.85 E-03 E 100 m
Offsite sector and distance	2.83 E-5 N 8,760 m	2.12 E-05 N 8,760 m	1.14 E-7 NNW 8,690 m

NOTE: The information in this table is from WHC-SD-WM-SARR-016, 1996, Tank Waste Compositions and Atmospheric Dispersion Coefficients for Use in Accelerated Safety Analysis Consequence Assessments, Westinghouse Hanford Company, Richland, Washington.

E = east
ESE = east, southeast.
N = north
NNW = north north west

Table 2-4. Photon Production Rates Entered in the MICROSHIELD and MICROSKYSHINE Codes, Unmitigated Accident Analysis

Group Midpoint Energy (MeV)	Photon Production Rate for 1 Ci Sr-90 in Concrete (photons/s)	Photon Production Rate Entered into Code for 2.05E6 Ci of Sr-90 (photons/s)
0.015	1.24E+09	2.54E+15
0.025	6.26E+08	1.28E+15
0.035	3.99E+08	8.18E+14
0.045	2.83E+08	5.80E+14
0.055	2.14E+08	4.39E+14
0.065	1.69E+08	3.46E+14
0.075	1.37E+08	2.81E+14
0.085	1.14E+08	2.34E+14
0.095	9.61E+07	1.97E+14
0.150	5.03E+08	1.03E+15
0.250	1.95E+08	4.0E+14
0.350	9.98E+07	2.05E+14
0.475	7.84E+07	1.61E+14
0.650	4.89E+07	1.00E+14
0.825	1.82E+07	3.73E+13
1.000	1.28E+07	2.62E+13
1.225	6.95E+06	1.42E+13
1.475	2.46E+06	5.04E+12
1.700	6.12E+05	1.25E+12
1.900	1.51E+05	3.10E+11
2.1	1.59E+04	3.26E+10
2.3	8.51E+01	1.74E+08

Table 2-5. Summary of Unmitigated Dose Estimates, by Pathway

Pathway	Onsite Receptor	Offsite Receptor
Inhalation	2900 mSv (290 rem)	6.7 mSv (0.67 rem)
Ingestion	NA	0.36 mSv (3.6E-2 rem)
Shine, gamma	33 mSv (3.3 rem)	negligible
Shine, bremsstrahlung	4.8 mSv (0.48 rem)	negligible
Skyshine, gamma	160 mSv (16 rem)	negligible
Skyshine, bremsstrahlung	47 mSv (4.7 rem)	negligible
Total Dose	3100 mSv (310 rem)	7.1 mSv (0.71 rem)

Table 2-6. Sum-of-Fraction of Risk Guidelines for a Unit Release of Chemicals and Gases.

*The sum of fractions are multiplied by the release rate for continuous release and release amount for a puff releases. Release rates for continuous releases are in units of liters per second for liquids and solids, and m^3/s for gases. Puff release quantities are in units of liters for solids and liquids and m^3 for gases.

Tank waste type (Units of sum of fractions follow tank waste type)	Maximum individual	Accident frequency, 1/yr		
		1 - 10^{-2}	$10^{-2} -$ 10^{-4}	$10^{-4} - 10^{-6}$
DST or SST solid or liquid continuous release				
Single-shell liquids(s/L)	Onsite	9.6 E+03	7.5 E+02	2.0 E+02
Single-shell liquids(s/L)	Offsite	8.0 E+00	8.0 E+00	6.2 E-01
Single-shell solids(s/L)	Onsite	4.0 E+04	2.1 E+04	1.0 E+03
Single-shell solids(s/L)	Offsite	9.4 E+01	3.3 E+01	1.7 E+01
Double-shell liquids(s/L)	Onsite	1.0 E+04	7.5 E+02	2.1 E+02
Double-shell liquids(s/L)	Offsite	8.4 E+00	8.4 E+00	6.2 E-01
Double-shell solids(s/L)	Onsite	1.8 E+04	3.3 E+03	6.3 E+02
Double-shell solids(s/L)	Offsite	1.9 E+02	1.5 E+01	2.8 E+00
DST or SST liquid or solid puff release				
Single-shell liquids (L^{-1})	Onsite	2.8 E+03	2.2 E+02	5.7 E+01
Single-shell liquids (L^{-1})	Offsite	3.2 E-02	3.2 E-02	2.5 E-03
Single-shell solids (L^{-1})	Onsite	1.2 E+04	6.0 E+03	2.9 E+02
Single-shell solids (L^{-1})	Offsite	3.8 E-01	1.3 E-01	6.9 E-02
Double-shell liquids (L^{-1})	Onsite	2.9 E+03	2.2 E+02	6.0 E+01

Table 2-6. Sum-of-Fraction of Risk Guidelines for a Unit Release of Chemicals and Gases.

*The sum of fractions are multiplied by the release rate for continuous release and release amount for a puff releases. Release rates for continuous releases are in units of liters per second for liquids and solids, and m^3/s for gases. Puff release quantities are in units of liters for solids and liquids and m^3 for gases.

Tank waste type (Units of sum of fractions follow tank waste type)	Maximum individual	Accident frequency, 1/yr		
		1 - 10^{-2}	$10^{-2} -$ 10^{-4}	$10^{-4} - 10^{-6}$
Double-shell liquids (L^{-1})	Offsite	3.4 E-02	3.4 E-02	2.5 E-03
Double-shell solids (L^{-1})	Onsite	5.2 E+03	9.7 E+02	1.8 E+02
Double-shell solids (L^{-1})	Onsite	7.7 E-01	5.9 E-02	1.1 E-02

Table 3-1. Photon Production Rates Entered in MICROSHIELD and MICROSKYSHINE, Mitigated Accident Analysis

Group Midpoint Energy (MeV)	Photon Production Rate for 1 Ci Sr-90 in Concrete (photons/s)	Photon Production Rate Entered into Code for 1.86E5 Ci of Sr-90 (photons/s)
0.015	1.24E+09	2.30E+14
0.025	6.26E+08	1.16E+14
0.035	3.99E+08	7.42E+13
0.045	2.83E+08	5.26E+13
0.055	2.14E+08	3.98E+13
0.065	1.69E+08	3.14E+13
0.075	1.37E+08	2.54E+13
0.085	1.14E+08	2.12E+13
0.095	9.61E+07	1.79E+13
0.150	5.03E+08	9.36E+13
0.250	1.95E+08	3.63E+13
0.350	9.98E+07	1.46E+13
0.475	7.84E+07	9.10E+12
0.650	4.89E+07	9.10E+12
0.825	1.82E+07	3.39E+12
1.000	1.28E+07	2.38E+12
1.225	6.95E+06	1.29E+12
1.475	2.46E+06	4.58E+11
1.700	6.12E+05	1.14E+11
1.900	1.51E+05	2.81E+10
2.1	1.59E+04	2.96E+09
2.3	8.51E+01	1.58E+07

Table 3-2. Summary of Mitigated Dose Estimates, by Pathway

Pathway	Onsite Receptor	Offsite Receptor
Inhalation	0.38 mSv (3.8E-02 rem)	0.62 mSv (6.2E-02 mrem)
Ingestion	NA	3.3E-02 mSv (3.3E-03 rem)
Shine, gamma	0.88 mSv (8.8E-02 rem)	negligible
Shine, bremsstrahlung	0.13 mSv (1.3E-02 rem)	negligible
Skyshine, gamma	2.2 mSv (0.22 rem)	negligible
Skyshine, bremsstrahlung	0.65 mSv (6.5E-02 rem)	negligible
Total Dose	4.2 mSv (0.42 rem)	0.65 mSv (6.5E-02 rem)

Table 4-1. Consequences of Subsurface Leak Resulting in Pool, Compared to Evaluation Guidelines

Receptor/Hazard	Calculated Dose/Exposure		Evaluation guideline		
	Unmitigated	Mitigated	Anticipated	Unlikely	Extremely Unlikely
Offsite/radiological	7.1 mSv	0.65 mSv	1 mSv	5 mSv	40 mSv
Onsite/radiological	3100 mSv	4.2 mSv	5 mSv	50 mSv	100 mSv
Offsite/toxicological sum-of-fractions	3.0E-2	5.4E-3	1	1	1
Onsite/toxicological sum-of-fractions	16.7	2.4E-3	1	1	1

APPENDICES

APPENDIX A

Historical Subsurface Leaks from Tank Farm Transfer Lines

Historical Subsurface Leaks from Tank Farm Transfer Lines

Occurrence Number	Date	Leak Type	Description
73-45	7/05/73	Subsurface leak	Backhoe ran over and broke a flange on a below grade riser on a transfer line at T tank farm. Watery liquid was seen seeping from the ground. Cause: pipe flanges on transfer lines at T farm were not adequately marked.
75-145	12/19/75	Subsurface leak	Possible transfer piping leakage during transfer from Tank 103-B to Tank 110-SX. Cause unknown. Material balance discrepancy was not resolved.
79-61	5/25/79	Subsurface leak	Slurry line SL-113 leaked in 241-S farm when being readied for a pressure test. Cause: mechanical failure of 2 inch carbon steel line.
ROR-80-2788	11/11/80	Subsurface leak forming surface pool	Radioactive leak at east side of 242-S evaporator. Direct buried (unencased) line failed due to galvanic corrosion. 2000 gal of slurry were spilled. 200 gal pool formed at the surface.
WHC-TANKFARM-1992-09	5/05/92	Subsurface leak forming surface pool	While conducting a hydrostatic pressure test of underground waste transfer lines in the 241-S Tank Farm, water was heard draining into a 241-S valve pit. SL-119 leaked to ground forming a pool of water at the surface. SL-119 is direct buried (unencased) line. The water eroded through the overlying berm.

WHC-TANKFARM-1992-40	6/14/92	Subsurface leak	Transfer line SL-116 in 241-S farm failed hydro test, waste came out buried portion of heat trace conduit near the line. Cause of line leak was likely heat stress induced fatigue. Heat trace conduit likely failed due to corrosion. SL-116 is direct buried line.
WHC-TANKFARM-1992-45	6/10/92	Suspected subsurface leak	Attempting to hydro test direct buried transfer line SL-115, 200 gallons more than the calculated line holdup was added. Assume line failed.
WHC-TANKFARM-1995-109	11/21/94	Suspected subsurface leak	Pressure test of supernate transfer line SN-246 failed, indicating loss of integrity. This line is unencased along some of its route.

APPENDIX B

Calculation of Maximum Submersible Pump Transfer Flowrate

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Calculations

Page 1

Engineering

Subject: BX-106 FLYGT B-2600 PUMP RUN-IN PERFORMANCE AGAINST DERIVED
OGT SYSTEM CURVE OF THE CONCEPTUAL CONFIGURATION

References:

1. QC INSPECTION RECORD OF WORK ORDER NUMBER 2H9500114F, p. 29 AND ATTACHED CALCULATIONS
2. SCHEMATIC OF BX-106 PUMP PIT PIPING CONFIGURATION FOR OGT SALTWELL PUMPING (H-2-821842 SH 1)
3. FLEXONICS INC., FLEXIBLE METAL HOSE AND FITTINGS, p. 14
4. CAMERON HYDRAULIC DATA, p. 3-8, 3-112, 3-113, 3-118
5. TANK FARM SURVEILLANCE AND WASTE STATUS SUMMARY REPORT FOR SEPTEMBER 1992, p. C-11
6. H-2-41348 and H-2-37852
7. WHC-SD-WM-SAR-034, Rev. 0-A

Objective of Analysis: To show that the pump performance-system configuration operating points for BX-106 OGT saltwell pumping fall within the bounds of WHC-SD-WM-SAR-034, REV 0-A, SECTION 9.0 (ACCIDENT SAFETY ANALYSIS)

Design Input and their Sources: All of the above references

Results of Literature Review: N.A.

Calculations: (Description of calculations to be performed, models and methods used.)

BX-106 SYSTEM CURVE

The model being developed is the system curve (see attached) for the proposed BX-106 pumping configuration. A submersible pump will be lowered into the tank and then pump waste solution out of the top of the heel pit and transfer the waste to 244-BX. The model developed is for the flow that would result if a catastrophic failure occurred at the location where the piping leaves the heel pit. This is the worst case scenario for the accident analysis described in Section 9.2.1.1 of WHC-SD-WM-SAR-034, Rev 0-A.

The derived system curve will be comprised of the lift from the top of the waste to the top of the heel pit, along with all the resistances along the flow path to the top of the heel pit. Shown are the following calculations for the derived system curve:

Prepared by:
Date:

Terry J. Bowtron
4/12/95

Checked by:
Date:

John J. [Signature]
05/25/95

Hanford Operations and Engineering Contractor for the US Department of Energy

Westinghouse
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Calculations

Engineering

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Total static head (referenced from beginning waste level in BX-106)

Tank bottom elevation = 614 ft. (ref. H-2-37852)

Waste level from tank bottom = 1.4 ft. (ref. #5)

Top elevation of BX-106 heel pit = 654.83 ft. (ref. H-2-41348)

Total static head = 654.83 - (614 + 1.4) = 39.43 ft

Friction Intake Side

0

Friction Discharge Side

2 inch pipe:

35 ft standard pipe, C=130 (ref. H-2-68521, 2 and ref. #4 p. 3-8)

one elbow, 5.17 ft standard pipe (ref #4 p. 3-120)

1 inch pipe:

6.8 ft standard pipe, C=130 (ref. #2 and ref. #4 p. 3-8)

4 ft flexible metal hose, at 3 times loss at C=130 (ref. #3 and ref. #4 p.3-8)

Valve, K=0.69 (ref. #4 p. 3-112)

Two long radius elbows, r/d=4, K=0.32 (ref. #4 p. 3-113)

One standard elbow, K=0.69

two reducers, 2 in. to 1 in., 45 deg angle, K=0.23 (ref. #4 p. 3-118)

exit loss, K=1.0 (ref. #4 p. 3-116)

Friction loss in terms of pipe length, ft. = $1044 * (\text{flow,gpm})^{1.85} * \text{Equivalent length, ft.} / 100$ / (C=130, friction factor for new steel pipe)^{1.85} / (d=1.05 for 1 inch Sch 40 pipe, or d=2.07 for 2 inch Sch 40 pipe)^{4.8655} (ref. #4 p-3-7) Note: This loss calculation method is conservative for the desired analysis. The above formula is for water at 60 degrees with new pipe. The BX-106 waste solution is a more viscous than water and would result in more friction loss per unit flow and when combined with the diminished pump performance due to viscosity the resulting flow at the catastrophic break location decreases.

Friction loss in terms of velocity head, ft. = $K * (\text{velocity,ft/sec})^2 / 2 / (g = \text{gravity constant} = 32.2)$, where velocity, ft/sec = $0.408 * (\text{flow,gpm}) / (\text{d,inside diameter, in.})^2$.

Total system head required of the pump to produce flow at the top of the heel pit during a catastrophic primary line failure = Total static head + friction losses = 39.43 ft. + 1 inch line losses + 2 inch line losses = 39.43 ft + [(6.8 + 4*3), straight 1" pipe + (K=.69+2*.32+.69+2*.23+1), velocity head] + [(35+5.17), straight 2 in pipe]

The above equation reduces to the following:

Total system head = $39.43 + 0.0190 * Q^{1.85} + 0.0074 * Q^2 + 0.00149 * Q^{1.85}$ where Q,gpm

Prepared by:

Terry J. Bowman

Date:

04/29/95

Checked by:

Date:

[Handwritten signatures and initials]

Westinghouse
Hanford Company
CalculationsPage 3
Engineering

From the total system head equation will be developed a system curve to compare against the run-in tests. Q is varied from 0 to 60 gpm by increments of 10 gpm.

GPM	<u>Total system head, ft.</u>
0	39.43
10	41.6
20	47.6
30	57.2
40	70.1
50	86.4
60	106.0

These results are plotted and compared against the pump run-in curve of 4/10/95 (see attached). The run-in test developed the performance curve of the pump, i.e., total head against flow. The intersection of both curves is the operating point of the pump at the specified parameters, i.e., catastrophic break at top of heel pit.
Note: Both the run-in test and the system curve derivation are for water. The waste solution in the tank has a greater viscosity than water. The resulting intersection of the true field curves would result in less flow and less head than the water-based analysis.

Assumptions:

Drawing H-2-41348 has the most accurate and up to date elevation data. This elevation was surveyed for the drawing and went through QA.

Summary of Results and Conclusions:

The discharge flow rate at the worst case scenario of a pipe break point right outside the pump pit will amount to less than 50 gpm. The pressure head (as water) of the pumping system deadhead is 35 psig. The pump-system configuration of BX-106 is within the bounds of WHC-SD-WM-SAR-034, REV. 0-A, SECTION 9.0 (ACCIDENT SAFETY ANALYSIS), where maximum pressure is 60 psig and maximum flow is 60 gpm.

Reviews and Approvals:

Prepared by:
Date:

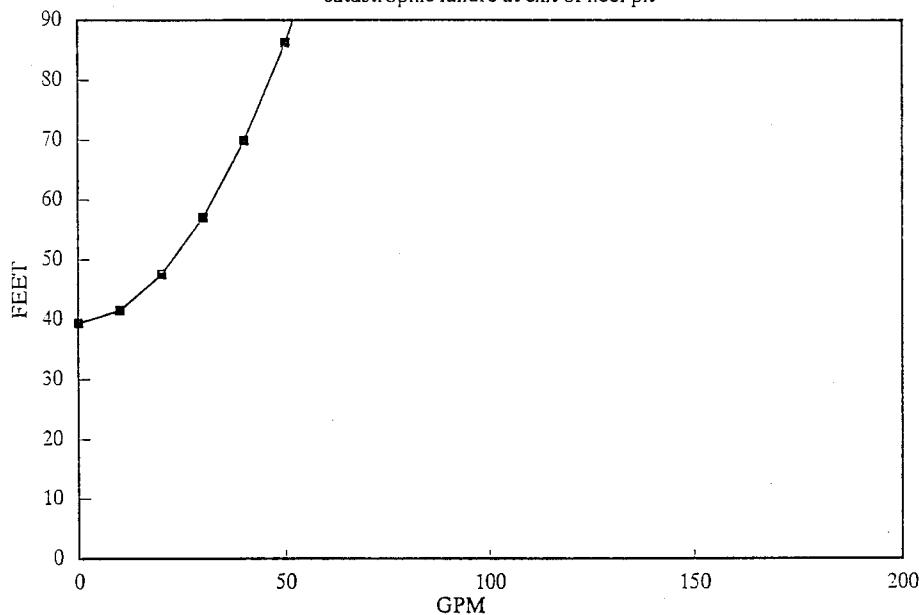
Perry J. Bowles
4/28/95

Checked by:
Date:

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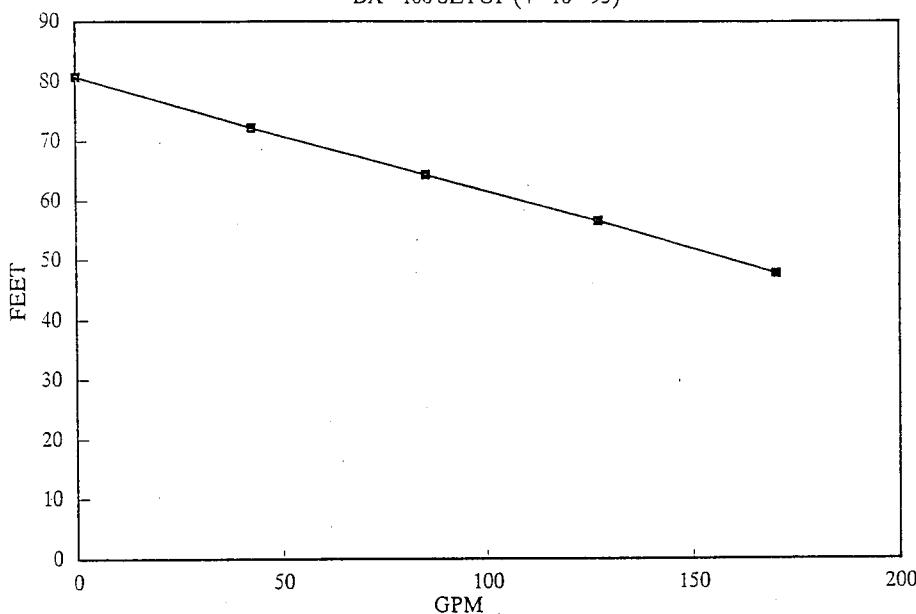
BX-106 SYSTEM CURVE

catastrophic failure at exit of heel pit



FLYGT B-2060 RUN-IN

BX-106 SETUP (4-10-95)



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APPENDIX C
MICROSHIELD Output Files

CASE 1. MICROSHIELD Input and Output for Direct Shine Due to
Gamma Emitters, Unmitigated Accident Scenario

MicroShield 4.00 - Serial #4.00-00128
Westinghouse Hanford Company

Page : 1 File Ref: _____
 DOS File: USUBG.MS4 Date: ____/____/_____
 Run Date: September 20, 1996 By: _____
 Run Time: 11:40 a.m. Friday Checked: _____
 Duration: 0:00:33

Case Title: unmitigated subsurface leak, dose from gamma emitters

GEOMETRY 7 - Cylinder Volume - Side Shields
centimeters feet and inches

Dose point coordinate X:	7200.0	236.0	2.6
Dose point coordinate Y:	164.0	5.0	4.6
Dose point coordinate Z:	0.0	0.0	.0
Cylinder height:	14.0	0.0	5.5
Cylinder radius:	2800.0	91.0	10.4
Side Clad:	4400.0	144.0	4.3

Source Volume: 3.44821e+8 cm^3 12177.2 cu ft. 2.10423e+7 cu in.

Material	Source	MATERIAL DENSITIES (g/cm^3)			
		Transition	Side Clad	Immersion	Shield
Air	Shield	0.00122			0.00122
Concrete	1.6		1.6		
Water	0.56				

BUILDUP
Method: Buildup Factor Tables
The material reference is Transition

INTEGRATION PARAMETERS
Quadrature Order

Radial	10
Circumferential	10
Axial (along Z)	10

SOURCE NUCLIDES

Nuclide	curies	$\mu\text{Ci}/\text{cm}^3$	Nuclide	curies	$\mu\text{Ci}/\text{cm}^3$
Ba-137m	1.7297e+005	5.0163e+002	Co-60	5.6757e+002	1.6460e+000
Cs-137	1.8378e+005	5.3298e+002	Eu-154	1.3514e+004	3.9190e+001

Page : 2

DOS File: USUBG.MS4

Run Date: September 20, 1996

Run Time: 11:40 a.m. Friday

Title : unmitigated subsurface leak, dose from gamma emitters

Energy (MeV)	Activity (photons/sec)	RESULTS		Exposure Rate In Air (mR/hr)	
		No Buildup (MeV/sq cm/sec)	With Buildup	No Buildup (MeV/sq cm/sec)	With Buildup
0.1	2.023e+014	2.199e+001	7.391e+002	3.365e-002	1.131e+000
0.2	3.415e+013	1.737e+001	2.581e+002	3.065e-002	4.555e-001
0.4	3.567e+012	7.131e+000	4.857e+001	1.390e-002	9.463e-002
0.5	1.083e+012	3.320e+000	1.825e+001	6.517e-003	3.582e-002
0.6	5.799e+015	2.512e+004	1.176e+005	4.903e+001	2.296e+002
0.8	1.950e+014	1.447e+003	5.393e+003	2.752e+000	1.026e+001
1.0	1.748e+014	1.957e+003	6.272e+003	3.608e+000	1.156e+001
1.5	2.161e+014	5.040e+003	1.284e+004	8.479e+000	2.159e+001
TOTAL:	6.626e+015	3.362e+004	1.432e+005	6.396e+001	2.747e+002

CASE 2. MICROSHIELD Input and Output for Direct Shine Due to
Bremsstrahlung Radiation, Unmitigated Accident Scenario

MicroShield 4.00 - Serial #4.00-00128
Westinghouse Hanford Company

Page : 1 File Ref: _____
 DOS File: USUBBR.MS4 Date: ____/____/
 Run Date: September 20, 1996 By: _____
 Run Time: 12:56 p.m. Friday Checked: _____
 Duration: 0:01:24

Case Title: unmitigated subsurface leak, dose from bremsstrahlung effect

GEOMETRY 7 - Cylinder Volume - Side Shields
centimeters feet and inches

Dose point coordinate X:	7200.0	236.0	2.6
Dose point coordinate Y:	164.0	5.0	4.6
Dose point coordinate Z:	0.0	0.0	.0
Cylinder height:	14.0	0.0	5.5
Cylinder radius:	2800.0	91.0	10.4
Side Clad:	4400.0	144.0	4.3

Source Volume: 3.44821e+8 cm^3 12177.2 cu. ft. 2.10423e+7 cu. in.

Material	Source Shield	MATERIAL DENSITIES (g/cm^3)			
		Transition Shield	Side Clad Shield	Immersion Shield	
Air		0.00122		0.00122	
Concrete	1.6		1.6		
Water	0.56				

BUILDUP

Method: Buildup Factor Tables
The material reference is Transition

INTEGRATION PARAMETERS
Quadrature Order

Radial	10
Circumferential	10
Axial (along Z)	10

SOURCE WAS ENTERED AS ENERGIES ONLY

Page : 2

DOS File: USUBBR.MS4

Run Date: September 20, 1996

Run Time: 12:56 p.m. Friday

Title : unmitigated subsurface leak, dose from bremsstrahlung effect

Energy (MeV)	Activity (photons/sec)	RESULTS		Exposure Rate In Air	
		Energy (MeV/sq cm/sec)	Fluence Rate (MeV/sq cm/sec)	No Buildup	With Buildup
0.015	2.540e+015	2.131e-035	1.033e-021	1.828e-036	8.864e-023
0.025	1.280e+015	1.355e-007	9.211e-007	2.338e-009	1.589e-008
0.035	8.180e+014	1.874e-002	3.160e-001	1.187e-004	2.002e-003
0.045	5.800e+014	7.955e-001	2.388e+001	2.646e-003	7.941e-002
0.055	4.390e+014	3.772e+000	1.476e+002	8.490e-003	3.321e-001
0.065	3.460e+014	8.070e+000	3.401e+002	1.465e-002	6.173e-001
0.075	2.810e+014	1.227e+001	5.058e+002	2.000e-002	8.244e-001
0.085	2.340e+014	1.585e+001	6.105e+002	2.462e-002	9.484e-001
0.095	1.970e+014	1.861e+001	6.559e+002	2.848e-002	1.004e+000
0.15	1.030e+015	2.893e+002	6.121e+003	4.764e-001	1.008e+001
0.25	4.000e+014	3.180e+002	3.622e+003	5.867e-001	6.682e+000
0.35	2.050e+014	3.162e+002	2.472e+003	6.099e-001	4.768e+000
0.475	1.610e+014	4.478e+002	2.581e+003	8.786e-001	5.064e+000
0.65	1.000e+014	5.036e+002	2.206e+003	9.777e-001	4.282e+000
0.825	3.730e+013	2.931e+002	1.068e+003	5.554e-001	2.024e+000
1.0	2.620e+013	2.934e+002	9.399e+002	5.407e-001	1.733e+000
1.225	1.420e+013	2.302e+002	6.524e+002	4.074e-001	1.155e+000
1.475	5.040e+012	1.141e+002	2.929e+002	1.928e-001	4.951e-001
1.7	1.250e+012	3.638e+001	8.750e+001	5.910e-002	1.421e-001
1.9	3.100e+011	1.096e+001	2.516e+001	1.723e-002	3.954e-002
2.1	3.260e+010	1.372e+000	3.026e+000	2.089e-003	4.607e-003
2.3	1.740e+008	8.563e-003	1.824e-002	1.266e-005	2.698e-005
TOTAL:	8.695e+015	2.914e+003	2.235e+004	5.403e+000	4.028e+001

CASE 3. MICROSHIELD Input and Output For Direct Shine Due to
Gamma Emitters, Mitigated Accident Scenario

MicroShield 4.00 - Serial #4.00-00128
Westinghouse Hanford Company

Page : 1 File Ref: _____
 DOS File: USUBG.MS4 Date: ____/____/
 Run Date: September 20, 1996 By: _____
 Run Time: 1:18 p.m. Friday Checked: _____
 Duration: 0:00:28

Case Title: unmitigated subsurface leak, dose from gamma emitters

GEOMETRY 7 - Cylinder Volume - Side Shields
centimeters feet and inches

Dose point coordinate X:	8730.0	286.0	5.0
Dose point coordinate Y:	152.24	4.0	11.9
Dose point coordinate Z:	0.0	0.0	.0
Cylinder height:	2.54	0.0	1.0
Cylinder radius:	1270.0	41.0	8.0
Side Clad:	7460.0	244.0	9.0

Source Volume: 1.28704e+7 cm³ 454.513 cu ft. 785398. cu in.

Material	Source	MATERIAL DENSITIES (g/cm ³)			
		Transition Shield	Side Clad Shield	Immersion Shield	
Air		0.00122		0.00122	
Concrete			1.6		
Water	1.4				

BUILDUP

Method: Buildup Factor Tables
The material reference is Transition

INTEGRATION PARAMETERS
Quadrature Order

Radial	10
Circumferential	10
Axial (along Z)	10

SOURCE NUCLIDES

Nuclide	curies	$\mu\text{Ci}/\text{cm}^3$	Nuclide	curies	$\mu\text{Ci}/\text{cm}^3$
Ba-137m	1.5676e+004	1.2180e+003	Co-60	5.1351e+001	3.9899e+000
Cs-137	1.6757e+004	1.3020e+003	Eu-154	1.2162e+003	9.4497e+001

Page : 2

DOS File: USUBG.MS4

Run Date: September 20, 1996

Run Time: 1:18 p.m. Friday

Title : unmitigated subsurface leak, dose from gamma emitters

RESULTS							
Energy (MeV)	Activity (photons/sec)	Energy (MeV/sq cm/sec)	Fluence Rate	Exposure Rate In Air (mR/hr)			
		No Buildup	With Buildup	No Buildup	With Buildup	No Buildup	With Buildup
0.0575	7.192e+013	1.573e+001	2.825e+002	3.311e-002	5.945e-001		
0.246	3.073e+012	9.280e+000	8.545e+001	1.707e-002	1.572e-001		
0.4426	4.185e+011	3.608e+000	2.013e+001	7.066e-003	3.941e-002		
0.5907	2.805e+012	4.064e+001	1.828e+002	7.939e-002	3.570e-001		
0.6631	5.343e+014	9.539e+003	3.958e+004	1.849e+001	7.671e+001		
0.8723	6.314e+012	1.851e+002	6.449e+002	3.483e-001	1.213e+000		
1.0024	1.279e+013	4.826e+002	1.552e+003	8.892e-001	2.859e+000		
1.2618	1.855e+013	1.063e+003	3.022e+003	1.869e+000	5.312e+000		
1.3325	1.900e+012	1.202e+002	3.324e+002	2.085e-001	5.767e-001		
1.5767	1.586e+012	1.360e+002	3.476e+002	2.257e-001	5.769e-001		
TOTAL:	6.537e+014	1.159e+004	4.605e+004	2.217e+001	8.840e+001		

CASE 4. MICROSHIELD Input and Output for Direct Shine Due to
Bremsstrahlung Radiation, Mitigated Accident Scenario

MicroShield 4.00 - Serial #4.00-00128
Westinghouse Hanford Company

Page : 1 File Ref: _____
 DOS File: MSUBBR.MS4 Date: ____/____/
 Run Date: September 23, 1996 By: _____
 Run Time: 8:54 a.m. Monday Checked: _____
 Duration: 0:01:02

Case Title: mitigated subsurface leak, dose from bremsstrahlung effect

GEOMETRY 7 - Cylinder Volume - Side Shields
centimeters feet and inches

Dose point coordinate X:	8730.0	286.0	5.0
Dose point coordinate Y:	152.54	5.0	.1
Dose point coordinate Z:	0.0	0.0	.0
Cylinder height:	2.54	0.0	1.0
Cylinder radius:	1270.0	41.0	8.0
Side Clad:	7460.0	244.0	9.0

Source Volume: 1.28704e+7 cm³ 454.513 cu ft. 785398. cu in.

Material	Source	MATERIAL DENSITIES (g/cm ³)			
		Transition	Side Clad	Immersion	Shield
Air	Shield	0.00122	0.00122	0.00122	0.00122
Concrete			1.6		
Water	1.4				

BUILDUP

Method: Buildup Factor Tables
The material reference is Transition

INTEGRATION PARAMETERS
Quadrature Order

Radial	10
Circumferential	10
Axial (along Z)	10

SOURCE WAS ENTERED AS ENERGIES ONLY

Page : 2

DOS File: MSUBBR.MS4

Run Date: September 23, 1996

Run Time: 8:54 a.m. Monday

Title : mitigated subsurface leak, dose from bremsstrahlung effect

RESULTS						
Energy (MeV)	Activity (photons/sec)	Energy Fluence Rate (MeV/sq cm/sec)	Exposure Rate In Air (mR/hr)			
			No Buildup	With Buildup	No Buildup	With Buildup
0.015	2.300e+014	4.459e-007	7.424e-007	3.824e-008	6.368e-008	
0.025	1.160e+014	2.975e-001	1.043e+000	5.131e-003	1.800e-002	
0.035	7.420e+013	2.958e+000	2.114e+001	1.874e-002	1.339e-001	
0.045	5.260e+013	5.938e+000	7.290e+001	1.975e-002	2.424e-001	
0.055	3.980e+013	7.857e+000	1.337e+002	1.768e-002	3.010e-001	
0.065	3.140e+013	8.993e+000	1.796e+002	1.633e-002	3.261e-001	
0.075	2.540e+013	9.641e+000	2.032e+002	1.571e-002	3.312e-001	
0.085	2.120e+013	1.012e+001	2.126e+002	1.573e-002	3.303e-001	
0.095	1.790e+013	1.041e+001	2.110e+002	1.593e-002	3.228e-001	
0.15	9.360e+013	1.197e+002	1.746e+003	1.972e-001	2.875e+000	
0.25	3.630e+013	1.130e+002	1.026e+003	2.086e-001	1.892e+000	
0.35	1.860e+013	1.055e+002	7.118e+002	2.035e-001	1.373e+000	
0.475	1.460e+013	1.432e+002	7.566e+002	2.809e-001	1.484e+000	
0.65	9.100e+012	1.570e+002	6.604e+002	3.048e-001	1.282e+000	
0.825	3.390e+012	9.004e+001	3.243e+002	1.706e-001	6.146e-001	
1.0	2.380e+012	8.958e+001	2.884e+002	1.651e-001	5.317e-001	
1.225	1.290e+012	7.018e+001	2.026e+002	1.242e-001	3.585e-001	
1.475	4.580e+011	3.488e+001	9.191e+001	5.896e-002	1.553e-001	
1.7	1.140e+011	1.122e+001	2.775e+001	1.822e-002	4.508e-002	
1.9	2.810e+010	3.375e+000	7.978e+000	5.303e-003	1.253e-002	
2.1	2.960e+009	4.251e-001	9.660e-001	6.474e-004	1.471e-003	
2.3	1.580e+007	2.668e-003	5.855e-003	3.945e-006	8.659e-006	
TOTAL:	7.884e+014	9.943e+002	6.879e+003	1.863e+000	1.263e+001	

APPENDIX D

BREMCALC Data

Photon Production Rates for Sr-90/Y-90 in Mixtures using
BREMCALC (photons per second from 1 curie of each isotope)

Midpoint Energy	Water Z = 6.60	Paper Z = 6.35	Air Z = 7.36	Concrete Z = 10.99	AlZr Z = 33.38
0.015	7.83E+08	7.59E+08	8.56E+08	1.24E+09	3.72E+09
0.025	3.99E+08	3.87E+08	4.35E+08	6.26E+08	1.88E+09
0.035	2.56E+08	2.48E+08	2.79E+08	3.99E+08	1.19E+09
0.045	1.82E+08	1.77E+08	1.98E+08	2.83E+08	8.42E+08
0.055	1.38E+08	1.34E+08	1.50E+08	2.14E+08	6.34E+08
0.065	1.09E+08	1.06E+08	1.19E+08	1.69E+08	4.98E+08
0.075	8.91E+07	8.66E+07	9.67E+07	1.37E+08	4.03E+08
0.085	7.42E+07	7.21E+07	8.05E+07	1.14E+08	3.34E+08
0.095	6.29E+07	6.11E+07	6.82E+07	9.61E+07	2.81E+08
0.15	3.33E+08	3.24E+08	3.60E+08	5.03E+08	1.45E+09
0.25	1.32E+08	1.29E+08	1.42E+08	1.95E+08	5.50E+08
0.35	6.88E+07	6.71E+07	7.37E+07	9.98E+07	2.74E+08
0.475	5.50E+07	5.38E+07	5.87E+07	7.84E+07	2.10E+08
0.65	3.53E+07	3.46E+07	3.75E+07	4.89E+07	1.26E+08
0.825	1.36E+07	1.33E+07	1.43E+07	1.82E+07	4.49E+07
1	9.82E+06	9.66E+06	1.03E+07	1.28E+07	3.02E+07
1.225	5.54E+06	5.46E+06	5.75E+06	6.95E+06	1.53E+07
1.475	2.06E+06	2.03E+06	2.12E+06	2.46E+06	4.94E+06
1.7	5.34E+05	5.29E+05	5.45E+05	6.12E+05	1.11E+06
1.9	1.36E+05	1.35E+05	1.38E+05	1.51E+05	2.46E+05
2.1	1.49E+04	1.48E+04	1.50E+04	1.59E+04	2.31E+04
2.3	8.25E+01	8.23E+01	8.28E+01	8.51E+01	1.05E+02

APPENDIX E

MICROSKYSHINE Output Files

CASE 1. MICROSKYSHINE Output for Unmitigated Accident,
Dose due to Gamma Emitters

MicroSkyshine

=====

(Nuclear & Radiological Safety Analysis - 1.16-007)

Page: 1

File Ref:

File: UPOOLG.SKY

Date: ____/____/____

Run: 9:49 a.m.

By:

: August 6, 1996

Checked: _____

CASE: pool 28 m in radius, dose due to gamma emitters

GEOMETRY: Vertical cylinder area source behind a wall

DIMENSIONS (meters):

Distance between wall and detector.....	X	43.
Depth of source behind wall.....	Y	0.855
Offset of detector.....	Z	0.
Depth of dose point.....	H	0.645
Distance between center of source and wall...	R1	29.
Thickness of cover slab.....	T1	0.
Thickness of second shield.....	T2	0.
Radius of source.....	W	28.
Height of source.....	L	0.14

INTEGRATION PARAMETERS:

Number of Radial Segments.....	M	10
Number of Circumferential Segments.....	N	10
Number of Vertical Segments.....	C	10
Quadrature Order.....		16

MATERIAL DENSITIES (g/cc):

Ambient air: .0012

Material	Cover Slab	Lower Shield	Volume Source
Air	-----	-----	-----
Water			0.56
Concrete			1.6
Iron			
Lead			
Zirconium			
Urania			

Buildup factor based on: AIR.

CASE: pool 28 m in radius, dose due to gamma emitters

SOURCE NUCLIDES:

Nuclide	Curies	Nuclide	Curies
Ba-137m	1.7297e+05	Co-60	5.6757e+02
Cs-137	1.8378e+05	Eu-154	1.3514e+04

RESULTS:

Group #	Energy (mev)	Activity (photons/sec)	Dose point rads/photon	Dose rate (mr/hr)
1	1.30	2.212e+14	4.452e-20	4.062e+01
2	1.03	1.656e+14	4.690e-20	3.202e+01
3	.84	9.181e+13	4.529e-20	1.714e+01
4	.66	5.905e+15	4.837e-20	1.178e+03
5	.48	4.881e+12	5.144e-20	1.035e+00
6	.40	1.047e+12	4.968e-20	2.145e-01
7	.24	3.301e+13	5.133e-20	6.987e+00
8	.20	1.136e+12	5.008e-20	2.346e-01
9	.12	2.023e+14	3.954e-20	3.299e+01
10				
11				
12				
13				
14				
15				
16				
17				
18				
19				
20				
TOTALS:		6.626e+15		1.309e+03

CASE 2. MICROSKYSHINE Output for Unmitigated Accident,
Dose Due to Bremsstrahlung Radiation

MicroSkyshine

=====

(Nuclear & Radiological Safety Analysis - 1.16-007)

Page: 1

File Ref:

File: USUBBR.SKY

Date: ____/____/____

Run: 12:41 p.m.

By: _____

: August 12, 1996

Checked: _____

CASE: pool 28 m in radius, 14 cm deep, dose due to bremmstahlung

GEOMETRY: Vertical cylinder area source behind a wall

DIMENSIONS (meters):

Distance between wall and detector.....	X	43.
Depth of source behind wall.....	Y	0.855
Offset of detector.....	Z	0.
Depth of dose point.....	H	0.645
Distance between center of source and wall...	R1	29.
Thickness of cover slab.....	T1	0.
Thickness of second shield.....	T2	0.
Radius of source.....	W	28.
Height of source.....	L	0.14

INTEGRATION PARAMETERS:

Number of Radial Segments.....	M	5
Number of Circumferential Segments.....	N	5
Number of Vertical Segments.....	C	5
Quadrature Order.....		16

MATERIAL DENSITIES (g/cc):

Ambient air: .0012

Material	Cover Slab	Lower Shield	Volume Source
Air	-----	-----	-----
Water			0.56
Concrete			1.6
Iron			
Lead			
Zirconium			
Urania			

Buildup factor based on: AIR.

69 of 78c 79

CASE: pool 28 m in radius, 14 cm deep, dose due to bremmstahlung

SOURCE NUCLIDES:

Source was entered by energy groups.

RESULTS:

Group #	Energy (mev)	Activity (photons/sec)	Dose point rads/photon	Dose rate (mr/hr)
1	1.90	3.100e+11	4.051e-20	5.178e-02
2	1.70	1.250e+12	4.015e-20	2.070e-01
3	1.48	5.040e+12	3.966e-20	8.242e-01
4	1.23	1.420e+13	4.576e-20	2.679e+00
5	1.00	2.620e+13	4.670e-20	5.045e+00
6	.82	3.730e+13	4.495e-20	6.914e+00
7	.65	1.000e+14	4.898e-20	2.020e+01
8	.47	1.610e+14	5.152e-20	3.420e+01
9	.35	2.050e+14	4.815e-20	4.070e+01
10	.25	4.000e+14	5.158e-20	8.508e+01
11	.15	1.030e+15	4.651e-20	1.975e+02
12				
13				
14				
15				
16				
17				
18				
19				
20				
TOTALS:		1.980e+15		3.934e+02

CASE 3. MICROSKYSHINE Output for Mitigated Accident,
Dose Due to Gamma Emitters

MicroSkyshine

=====

(Nuclear & Radiological Safety Analysis - 1.16-007)

Page: 1

File Ref: _____

File: MPOOLG.SKY

Date: _____

Run: 11:36 a.m.

By: _____

: August 8, 1996

Checked: _____

CASE: mit pool, $r = 12.7$ m, dose from gamma radiation

GEOMETRY: Vertical cylinder area source behind a wall

DIMENSIONS (meters):

Distance between wall and detector.....	X	73.6
Depth of source behind wall.....	Y	0.396
Offset of detector.....	Z	0.
Depth of dose point.....	H	-1.1
Distance between center of source and wall...	R1	13.7
Thickness of cover slab.....	T1	0.
Thickness of second shield.....	T2	0.
Radius of source.....	W	12.7
Height of source.....	L	0.0254

INTEGRATION PARAMETERS:

Number of Radial Segments.....	M	10
Number of Circumferential Segments.....	N	10
Number of Vertical Segments.....	C	10
Quadrature Order.....		16

MATERIAL DENSITIES (g/cc):

Ambient air: .0012

Material	Cover Slab	Lower Shield	Volume Source
Air			
Water			1.4
Concrete			
Iron			
Lead			
Zirconium			
Urania			

Buildup factor based on: AIR.

72
67 of 78 79

CASE: mit pool, r = 12.7 m, dose from gamma radiation

SOURCE NUCLIDES:

Nuclide	Curies	Nuclide	Curies
Ba-137m	1.5676e+04	Co-60	5.1351e+01
Cs-137	1.6757e+04	Eu-154	1.2162e+03

RESULTS:

Group #	Energy (mev)	Activity (photons/sec)	Dose point rads/photon	Dose rate (mr/hr)
1	1.30	1.992e+13	8.491e-20	6.975e+00
2	1.03	1.491e+13	8.870e-20	5.453e+00
3	.84	8.263e+12	8.625e-20	2.939e+00
4	.66	5.351e+14	9.004e-20	1.986e+02
5	.48	4.392e+11	9.404e-20	1.703e-01
6	.40	9.424e+10	9.139e-20	3.551e-02
7	.24	2.971e+12	9.101e-20	1.115e+00
8	.20	1.022e+11	8.892e-20	3.748e-02
9	.12	1.821e+13	7.763e-20	5.828e+00
10				
11				
12				
13				
14				
15				
16				
17				
18				
19				
20				
TOTALS:		6.000e+14		2.212e+02

CASE 4. MICROSKYSHINE Output Mitigated Accident,
Dose Due to Bremsstrahlung Radiation

MicroSkyshine

=====

(Nuclear & Radiological Safety Analysis - 1.16-007)

Page: 1

File Ref:

File: MSUBBR.SKY

Date: ____/____/____

Run: 9:53 a.m.

By:

: September 23, 1996

Checked: _____

CASE: mit pool, r = 12.7 m, dose from bremmstrahlung radiation

GEOMETRY: Vertical cylinder area source behind a wall

DIMENSIONS (meters):

Distance between wall and detector.....	X	73.6
Depth of source behind wall.....	Y	0.396
Offset of detector.....	Z	0.
Depth of dose point.....	H	-1.1
Distance between center of source and wall...	R1	13.7
Thickness of cover slab.....	T1	0.
Thickness of second shield.....	T2	0.
Radius of source.....	W	12.7
Height of source.....	L	0.0254

INTEGRATION PARAMETERS:

Number of Radial Segments.....	M	10
Number of Circumferential Segments.....	N	10
Number of Vertical Segments.....	C	10
Quadrature Order.....		16

MATERIAL DENSITIES (g/cc):

Ambient air: .0012

Material	Cover Slab	Lower Shield	Volume Source
Air	-----	-----	-----
Water	-----	-----	1.4
Concrete	-----	-----	-----
Iron	-----	-----	-----
Lead	-----	-----	-----
Zirconium	-----	-----	-----
Urania	-----	-----	-----

Buildup factor based on: AIR.

CASE: mit pool, r = 12.7 m, dose from bremmstrahlung radiation

SOURCE NUCLIDES:

Nuclide	Curies	Nuclide	Curies
Ba-137m	1.5676e+04	Co-60	5.1351e+01
Cs-137	1.6757e+04	Eu-154	1.2162e+03

RESULTS:

Group #	Energy (mev)	Activity (photons/sec)	Dose point rads/photon	Dose rate (mr/hr)
1	1.90	2.810e+10	7.909e-20	9.164e-03
2	1.70	1.140e+11	7.861e-20	3.695e-02
3	1.48	4.580e+11	7.782e-20	1.470e-01
4	1.23	1.290e+12	8.694e-20	4.624e-01
5	1.00	2.380e+12	8.855e-20	8.690e-01
6	.82	3.390e+12	8.582e-20	1.200e+00
7	.65	9.100e+12	9.084e-20	3.409e+00
8	.47	1.460e+13	9.396e-20	5.657e+00
9	.35	1.860e+13	8.881e-20	6.811e+00
10	.25	3.630e+13	9.102e-20	1.362e+01
11	.15	9.360e+13	8.417e-20	3.249e+01
12				
13				
14				
15				
16				
17				
18				
19				
20				
TOTALS:		1.799e+14		6.471e+01

APPENDIX F
Peer Review Checklist

CHECKLIST FOR TECHNICAL PEER REVIEW

A. Calculation Notes for Subsurface Leak Resulting in Pool, TWRS FSAR Accident Analysis, WHC-SD-WM-CN-057, Rev. 0, Brett Hall, 9/19/96

B. Scope of Review: Entire document

Yes No* NA

- [X] [] [] Problem completely defined.
- [X] [] [] Accident scenarios developed in a clear and logical manner.
- [X] [] [] Necessary assumptions explicitly stated and supported.
- [X] [] [] Computer codes and data files documented.
- [X] [] [] Data used in calculations explicitly stated in document.
- [X] [] [] Data checked for consistency with original source information as applicable.
- [X] [] [] Mathematical derivations checked including dimensional consistency of results.
- [X] [] [] Models appropriate and used within range of validity or use outside range of established validity justified.
- [X] [] [] Hand calculations checked for errors. Spreadsheet results should be treated exactly the same as hand calculations.
- [X] [] [] Software input correct and consistent with document reviewed.
- [X] [] [] Software output consistent with input and with results reported in document reviewed.
- [X] [] [] Limits/criteria/guidelines applied to analysis results are appropriate and referenced. Limits/criteria/guidelines checked against references.
- [X] [] [] Safety margins consistent with good engineering practices.
- [X] [] [] Conclusions consistent with analytical results and applicable limits.
- [X] [] [] Results and conclusions address all points required in the problem statement.
- [X] [X] ** Review calculations, comments, and/or notes are attached.
- [X] [] [] Document approved (i.e., the reviewer affirms the technical accuracy of the document).
- [X] [] [] Traceability

Donald R. Porten
Reviewer (Printed Name and Signature)

Donald R. Porten

9/19/96
Date

* All "NO" responses must be explained below or on an additional page.

** Any calculations, comments, or notes generated as part of this review should be signed, dated and attached to this checklist. Such material should be labeled and recorded in such a manner as to be intelligible to a technically qualified third party.

PEER REVIEW CHECKLIST

Document Reviewed: WHC-SD-WM-CN-057

Author: B. Hall

Date: September 1996

Scope of Review: Direct shine and skyshine dose calculations, (Section 2.4.3, 2.4.4, 3.4.3, 3.4.4 and Appendices C, D, and E

Yes No NA

[] [] Previous reviews complete and cover analysis, up to scope of this review, with no gaps.

[] [] Problem completely defined.

[] [] Accident scenarios developed in a clear and logical manner.

[] [] Necessary assumptions explicitly stated and supported.

[] [] Computer codes and data files documented.

[] [] Data used in calculations explicitly stated in document.

[] [] Data checked for consistency with original source information as applicable.

[] [] Mathematical derivations checked including dimensional consistency of results.

[] [] Models appropriate and used within range of validity or use outside range of established validity justified.

[] [] Hand calculations checked for errors. Spreadsheet results should be treated exactly the same as hand calculations.

[] [] Software input correct and consistent with document reviewed.

[] [] Software output consistent with input and with results reported in document reviewed.

[] [] Limits/criteria/guidelines applied to analysis results are appropriate and referenced. Limits/criteria/guidelines checked against references.

[] [] Safety margins consistent with good engineering practices.

[] [] Conclusions consistent with analytical results and applicable limits.

[] [] Results and conclusions address all points required in the problem statement.

[] [] Format consistent with appropriate NRC Regulatory Guide or other standards

[] [] Review calculations, comments, and/or notes are attached.

[] [] Document approved.

J. C. Van Keuren J.C. Van Keuren
Reviewer (Printed Name and Signature)9/23/96
Date

DISTRIBUTION SHEET

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Project Title/Work Order				EDT No. 619204	
Calculation Notes for Subsurface Leak Resulting in Pool, TWRS FSAR Accident Analysis				ECN No. NA	
Name	MSIN	Text With All Attach.	Text Only	Attach./ Appendix Only	EDT/ECN Only
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