

REFLECTED REACTOR KINETICS

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## **ABSTRACT**

The transfer function of a reflected reactor has been derived from a model which is applicable to all reactors. The result is identical to a bare reactor transfer function at low frequencies, provided that the neutron lifetime includes the effect of the reflector. At high frequencies the reflector introduces in effect an additional group of delayed neutrons. A calculation of the effective neutron lifetime in the SRE from the theoretical results was in good agreement with the experimental value.

## I. INTRODUCTION

Kinetic behavior of a bare reactor has been studied extensively, both theoretically and experimentally. These studies have shown that the inhour equation and transfer function derived from the usual form of the kinetics equation give excellent agreement with experimental results.

Reflected reactors have usually been treated by assuming that the same kinetics equations are applicable providing that a modified value of the prompt neutron lifetime is used which includes the effect of the reflector. Measurements of fast transients in the KEWB reactor, however, indicated empirically that treatment of the reflector by an additional delay group (or groups) would give better results.<sup>1</sup>

One theoretical approach to reflected reactor kinetics has been made previously.<sup>2</sup> The method was applied to a coupled fast-thermal critical assembly to calculate the prompt neutron lifetime. The results agreed well with other determinations. However, attempts to apply this method to the Sodium Reactor Experiment (SRE) give unreasonable values for the lifetime. Further examination showed that this method is inapplicable to reactors with reflectors which make a large contribution to reactivity. This report describes a method of analysis which avoids the above difficulty, and is thus applicable to all reflected reactors.

## II. DERIVATION OF THE REFLECTED REACTOR TRANSFER FUNCTION

The general equation governing reactor kinetics is:

$$\text{Rate of Change of Neutron Density} = \text{Production Rate} - \text{Destruction Rate} + \text{Source} \quad \dots (1)$$

This equation is usually applied to a bare reactor by assuming the "flow diagram" for the neutrons shown in Figure 1. Thus Equation 1 becomes, in the absence of a source,

$$\frac{dn}{dt} = \frac{(1 - \beta)k_{\text{eff}}n}{\ell^*} + \sum \lambda_i C_i - \frac{n}{\ell^*},$$

and the associated equations for the delayed neutrons are:

$$\frac{dC_i}{dt} = \frac{\beta_i k_{\text{eff}}n}{\ell^*} - \lambda_i C_i$$

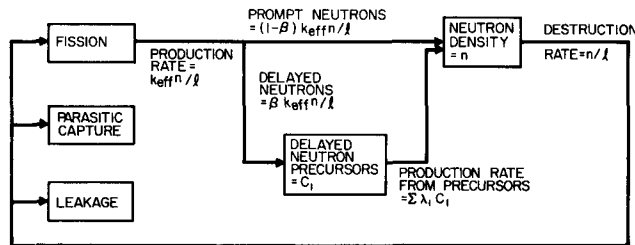


Figure 1. Flow Diagram for Bare Reactor Kinetics

### A. PREVIOUS MODEL

For a reflected reactor, the same approach can be used by treating separately the neutrons in the core and the reflector. However, the neutrons can bounce back and forth from core to reflector many times before being finally absorbed. Thus the diagram shown in Figure 2 applies for a reflected reactor.



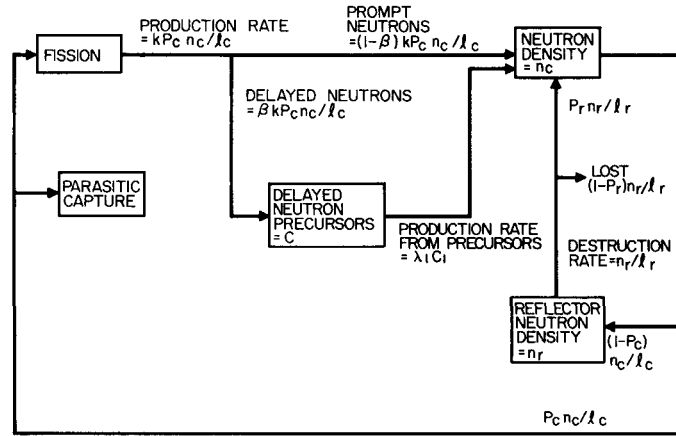


Figure 2. Flow Diagram for Reflected Reactor Kinetics

The definitions of the symbols are:

$n_c$  = neutron density in the core

$n_r$  = neutron density in the reflector

$P_c$  = probability that a neutron will remain in the core

$P_r$  = probability that a disappearing reflector neutron will reappear in the core

$k$  = infinite multiplication constant ( $k_\infty$ ) for the core.

The differential equations corresponding to this model are:

$$\frac{dn_c}{dt} = \frac{(1 - \beta)P_c k n_c}{l_c} + \sum \lambda_i C_i + \frac{P_r n_r}{l_r} - \frac{n_c}{l_c} ,$$

$$\frac{dn_r}{dt} = \frac{(1 - P_c)n_c}{l_c} - \frac{n_r}{l_r} ,$$

$$\frac{dC_i}{dt} = \frac{\beta_i k P_c n_c}{l_c} - \lambda_i C_i . \quad \dots (2)$$

These equations are identical with those derived by C. E. Cohn.<sup>2</sup>

Although this model is a logical one, it does not give correct results since some neutrons are permitted to go around the "loop" from core to reflector and

back indefinitely without ever being absorbed. One consequence of this is that the critical equation, obtained by setting all time derivatives in Equation 2 to zero, is

$$kP_c + (1 - P_c)P_r = 1 \quad \dots (2a)$$

This is not in agreement with the general form of the critical equation

$$k\mathcal{L} = 1$$

where  $\mathcal{L}$ , the net nonleakage probability, is some function of  $P_c$  and  $P_r$ . The discrepancy arises because neutrons returning from the reflector are counted the same as "new" neutrons from fission. In reactors where the reflector makes a large contribution to the reactivity, the error in Equation 2a would be significant.

## B. NEW MODEL

A model which avoids this difficulty is shown in Figure 3. In this model, neutrons which enter the reflector are not counted in the core until after their

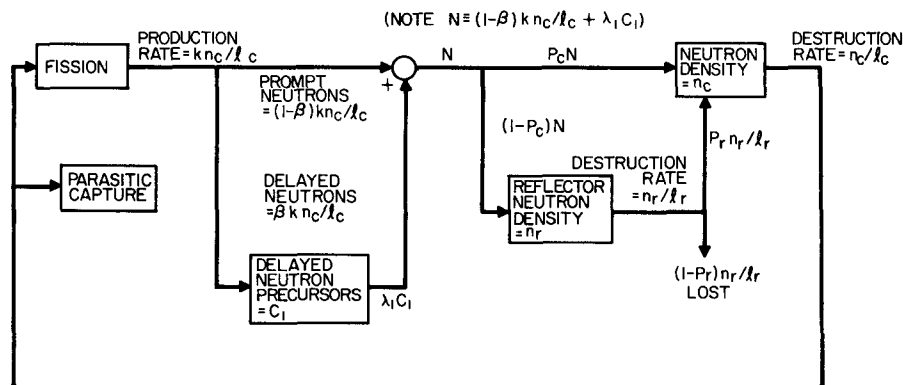


Figure 3. Modified Flow Diagram for Reflected Reactor Kinetics

final exit from the reflector. Although this does not correspond to the physical situation in the reactor, it will give correct results for the kinetic behavior if  $l_r$  and  $l_c$  are defined as the total length of time that neutrons spend in the reflector and core, respectively. In other words the total length of time spent in the two regions is the same in the model as in the reactor, but the chronological order is changed, thus eliminating the "loop."

The differential equations for this model are:

$$\begin{aligned}\frac{dn_c}{dt} &= \frac{(1 - \beta)kP_c n_c}{\ell_c} + \frac{P_r n_r}{\ell_r} + P_c \sum \lambda_i C_i - \frac{n_c}{\ell_c} , \\ \frac{dn_r}{dt} &= \frac{(1 - \beta)k(1 - P_c)n_c}{\ell_c} + (1 - P_c) \sum \lambda_i C_i - \frac{n_r}{\ell_r} , \\ \frac{dC_i}{dt} &= \frac{\beta_i k n_c}{\ell_c} - \lambda_i C_i .\end{aligned}\quad \dots (3)$$

The critical equation, found by setting all time derivatives to zero, is:

$$k \left[ P_c + (1 - P_c)P_r \right] = 1 , \quad \dots (4)$$

which now has the same form as the general critical equation.

### C. DERIVATION OF TRANSFER FUNCTION

To solve equation 3 for the reactor transfer function, the time-varying quantities are replaced by their equilibrium values plus small variations:

$$\begin{aligned}n_c &= n_{c0} + \delta n_c \\ n_r &= n_{r0} + \delta n_r \\ P_c &= P_{c0} + \delta P_c \\ k &= k_0 + \delta k \\ C_i &= C_{i0} + \delta C_i .\end{aligned}\quad \dots (5)$$

This assumes no variation in  $P_r$ , which is reasonable for small changes in the core.

The equilibrium values, which are the values at criticality, also satisfy Equation 3. Thus, substituting Equation 5 into Equation 3, subtracting Equation 3 with the critical values inserted, neglecting the products of differentials, and taking Laplace transforms, gives:

$$\begin{aligned}
s \delta n_c &= \frac{-\delta n_c}{\ell_c} + \frac{(1-\beta)}{\ell_c} \left[ P_{c0} k_0 \delta n_c + n_0 (P_{c0} \delta k + k_0 \delta P_c) \right] \\
&+ \frac{P_r \delta n_r}{\ell_r} + \sum \lambda_i (C_{i0} \delta P_c + P_{c0} \delta C_i) \quad , \\
s \delta n_r &= \frac{-\delta n_r}{\ell_r} + \frac{(1-\beta)}{\ell_c} (1 - P_{c0}) (n_{c0} \delta k + k_0 \delta n_c) - k_0 n_0 \delta P_c \\
&+ \sum \lambda_i \left[ (1 - P_{c0}) \delta C_i - C_{i0} \delta P_c \right] \quad , \\
s \delta C_i &= -\lambda_i \delta C_i + \frac{\beta_i}{\ell_c} (n_{c0} \delta k + k_0 \delta n_c) \quad . \quad \dots (6)
\end{aligned}$$

The last one of Equation 6 yields

$$\delta C_i = \frac{\beta_i (n_{c0} \delta k + k_0 \delta n_c)}{\ell_c (s + \lambda_i)} \quad . \quad \dots (7)$$

However,  $n_{c0} \delta k$  can be neglected since at low frequencies it is much smaller than  $k_0 \delta n_c$ , and at higher frequencies both terms are negligible. (This same approximation is made in the derivation of the bare reactor transfer function.)

$C_{i0}$  in Equation 6 is the equilibrium solution of the third of Equations 3 where  $dC_i/dt = 0$ ,  $k = k_0$  and  $n_c = n_{c0}$ ; so

$$\lambda_i C_{i0} = \frac{\beta_i k_0 n_{c0}}{\ell_c} \quad . \quad \dots (8)$$

Now using Equation 8 in the first two equations of 6, solving the second for  $\delta n_r$  and substituting the result and Equation 7 into the first of Equation 6 gives:

$$\begin{aligned}
& \frac{\delta n_c}{n_{c0}} \left\{ s + \frac{1}{\ell_c} - \frac{(1 - \beta)k_0}{\ell_c} \left[ P_{c0} + \frac{(1 - P_{c0})P_r}{\ell_r s + 1} \right] - \frac{k_0}{\ell_c} \left[ P_{c0} + \frac{(1 - P_{c0})P_r}{\ell_r s + 1} \right] \sum \frac{\lambda_i \beta_i}{s + \lambda_i} \right\} \\
&= \frac{(1 - \beta)}{\ell_c} \left[ P_{c0} \delta k + k_0 \delta P_c \right] + \frac{P_r (1 - \beta)}{\ell_c (\ell_r s + 1)} \left[ (1 - P_{c0}) \delta k - k_0 \delta P_c \right] \\
&+ \frac{\beta k_0 \delta P_c}{\ell_c} \left[ 1 - \frac{P_r}{\ell_r s + 1} \right] , \quad \dots (9)
\end{aligned}$$

where all terms involving  $\delta n_c$  have been taken to the left hand side.

The term

$$\frac{\beta k_0}{\ell_c} \left[ P_{c0} + \frac{(1 - P_{c0})P_r}{\ell_r s + 1} \right]$$

can be combined with the term including the summation, since  $\beta = \sum \beta_i$ , yielding:

$$\frac{k_0}{\ell_c} \left[ P_{c0} + \frac{(1 - P_{c0})P_r}{\ell_r s + 1} \right] \sum \frac{\beta_i s}{s + \lambda_i} .$$

Now, in this term only, let  $(\ell_r s + 1) = 1$ . Although this is valid only at low frequencies, at higher frequencies the entire term is very small, so the error in the transfer function is less than 1% at any frequency. This approximation is equivalent to neglecting the additional delay of the reflector on the delayed neutrons which enter the reflector.

From Equation 4

$$k_o P_{c0} + k_o (1 - P_{c0}) P_r = 1 ,$$

so solving Equation 9 for  $\delta n/n_0$  gives, letting  $\lambda_r = 1/\ell_r$ , and  $1 - \beta = 1$ :

$$\frac{\delta n_c}{n_{c0}} = \frac{P_{c0}\delta k + k_0\delta P_c + \frac{(1 - P_{c0})P_r\delta k - P_r k_0\delta P_c}{\ell_r(s + \lambda_r)}}{s\ell_c \left[ 1 + \frac{k_{c0}(1 - P_{c0})P_r}{\ell_c(s + \lambda_r)} + \frac{1}{\ell_c} \sum \frac{\beta_i}{s + \lambda_i} \right]} \quad \dots (10)$$

Equation 4 implies that

$$k_{\text{eff}} = kP_c + k(1 - P_c)P_r \quad ,$$

so,

$$\delta\rho = \frac{\delta k_{\text{eff}}}{k_{\text{eff}0}} = P_{c0}\delta k + k_0\delta P_c + (1 - P_{c0})P_r\delta k - P_r k_0\delta P_c \quad , \quad \dots (11)$$

assuming, as before, no change in  $P_r$ .

At low frequencies the numerator in Equation 10 is therefore  $\delta\rho$ . At higher frequencies the numerator is a dynamic reactivity, in which a change in the probability of neutrons escaping to the reflector is affected by the frequency of the change. This is physically reasonable, since a very fast change will have less effect due to the delay associated with the reflector.

Thus the transfer function of a reflected reactor is:

$$G_r(s) = \frac{\delta n_c/n_{c0}}{\delta\rho/\beta} = \frac{1}{\frac{s\ell_c}{\beta} \left[ 1 + \frac{k_{c0}(1 - P_{c0})P_r}{\ell_c(s + \lambda_r)} + \frac{\beta}{\ell_c} \sum_i \frac{\beta_i/\beta}{s + \lambda_i} \right]} \quad \dots (12)$$

#### D. DISCUSSION

At low frequencies where  $\omega \ll 1/\ell_r$ , the transfer function is identical to the bare reactor transfer function:

$$G(s) = \frac{1}{\frac{s\ell^*}{\beta} \left[ 1 + \frac{\beta}{\ell^*} \sum_i \frac{\beta_i/\beta}{s + \lambda_i} \right]} , \quad \dots (13)$$

providing that we define

$$\ell^* = \ell_c + k_{c0}(1 - P_{c0})P_r\ell_r . \quad \dots (14)$$

Thus the effective lifetime is the core lifetime plus the reflector lifetime weighted by the fraction of neutrons which enter and return from the reflector.

At higher frequencies the reflector in effect introduces a seventh group of delayed neutrons:

$$G_r(s) = \frac{1}{\frac{\ell_c s}{\beta} \left[ 1 + \frac{\beta}{\ell_c} \sum_{i=1}^{i=7} \frac{\beta_i/\beta}{s + \lambda_i} \right]} , \quad \dots (15)$$

where

$$\lambda_7 \equiv \lambda_r = \frac{1}{\ell_r} ,$$

$$\beta_7 \equiv k_{c0}(1 - P_{c0})P_r .$$

Note that in Equation 15,

$$\beta \equiv \sum_{i=1}^{i=6} \beta_i ,$$

i. e. , it does not include the neutrons from the reflector. This is physically reasonable because the delay associated with the reflector is very much shorter than that of the real delay groups. Thus the reactivity required for prompt criticality is not radically changed by the presence of a reflector.

Figure 4 shows a plot of the transfer function of a typical graphite-moderated and -reflected thermal reactor, in which  $\ell_c$  and  $\ell_r$  are the same order of magnitude. In this case, the only perturbation of the bare reactor transfer function is well above the region usually considered for lifetime measurements. However, if the reflector lifetime is much longer than the core lifetime, then the transfer function may be distorted in the region of the  $\ell^*/\beta$  break-off, and the bare reactor transfer function would not represent an adequate approximation.

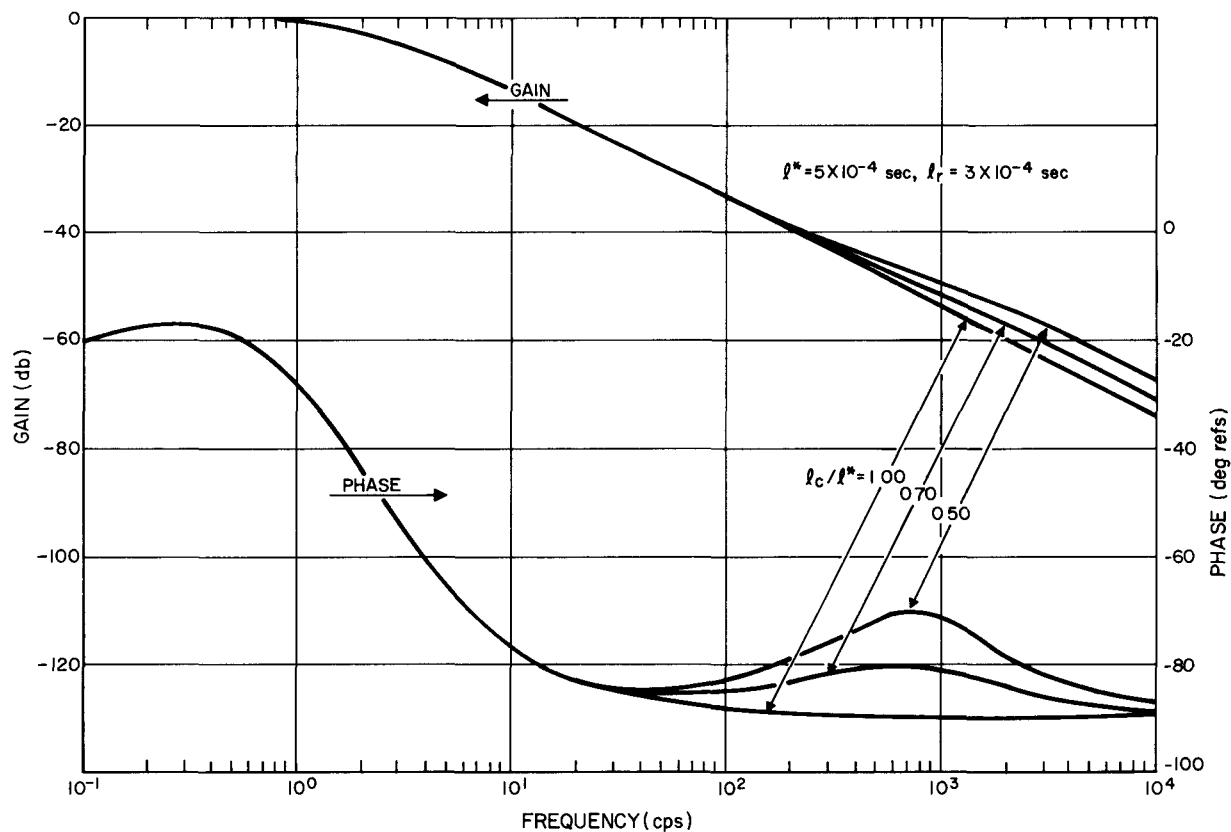


Figure 4. Reflected Reactor Transfer Function

The transfer function derived above is identical to that of Cohn<sup>2</sup> except for the expression for effective lifetime, as discussed in the following section.



### III. CALCULATION OF $\ell^*$ AND $\beta_{\text{eff}}$ IN THE SRE

A calculation of  $\ell^*$  and  $\beta_{\text{eff}}$  for the second core loading of the SRE was used for comparison with the results of the transfer function measurement. The method was a combination of the results from the preceeding derivation with a two-group analysis of the kinetics of a bare reactor,<sup>3</sup> which yields the following expressions:

$$\beta_{\text{eff}} = \beta(k_d) \quad , \quad \dots (16)$$

$$\ell_c = \ell_{\text{cth}} \left( \frac{1 - \nu \Sigma_{1f}}{\Sigma_{1r}} \right) + \ell_{\text{cf}} \quad , \quad \dots (17)$$

where

$k_d$  = effective multiplication constant for delayed neutrons

$\ell_{\text{cf}}, \ell_{\text{cth}}$  = prompt neutron lifetime in the core for fast and thermal groups, respectively

$\ell_c$  = total mean prompt neutron lifetime in the core

$\Sigma_{1f}$  = fission cross section in fast group

$\Sigma_{1r}$  = total removal cross section from fast group

$\nu$  = number of fission neutrons per fission.

Equation 16 can be applied directly to the SRE, since the reflector affects  $\beta_{\text{eff}}$  only in the value of  $k_d$ . Since  $k_d$  is identical to  $k_{\text{eff}}$  except for the fast non-leakage probability, then for a critical reactor where  $k_{\text{eff}} = 1$ :

$$k_d = e^{B^2(\tau_p - \tau_d)} \quad ,$$

where  $\tau_p$  and  $\tau_d$  are the fission to thermal ages for prompt and delayed neutrons, respectively, and  $B^2$  is the effective buckling for a reflected reactor. The values of  $\tau$  were obtained by the usual method,<sup>4</sup> and  $B^2$  was determined from a two-group

calculation on the reflected reactor by requiring that  $e^{-B^2\tau_{\text{eff}}}$  [where  $\tau_{\text{eff}} = (1 - \beta)\tau_p + \beta\tau_d$ ] give the correct fast nonleakage probability. Using these values,  $k_d = 1.058$  and  $\beta_{\text{eff}} = 0.0069$ .

The calculation of  $\ell^*$  used the results of the derivation of the reflected transfer function:

$$\ell^* = \ell_c + k_{c0}(1 - P_{c0})P_r\ell_r \quad \dots (14)$$

The value of  $\ell_c$  is obtained from Equation 17 with:<sup>5</sup>

$$\ell_{\text{cth}} = \frac{1}{V_{\text{th}}(\Sigma_{\text{ath}} + D_{\text{th}}B_c^2)} \quad ,$$

$$\ell_{\text{fth}} = \frac{\lambda_s \sqrt{2m}}{\xi} \left[ \frac{1}{\sqrt{E_{\text{th}}}} - \frac{1}{\sqrt{E_{\text{fission}}}} \right] \quad , \quad \dots (18)$$

where:

$V_{\text{th}}$  = average velocity of thermal neutrons

$\Sigma_{\text{ath}}$ ,  $D_{\text{th}}$  = thermal absorption cross section and diffusion coefficient, respectively, in the core

$\lambda_s$  = mean free path of fast neutrons

$m$  = mass of neutron =  $1.66 \times 10^{-24}$  gm

$\xi$  = logarithmic decrement of energy lost per collision

$E_{\text{th}}$ ,  $E_{\text{fission}}$  = energy of thermal and fission neutrons, respectively

$B_c^2$  = geometric buckling of bare core:

$$B_c^2 = \left( \frac{2.405}{R_{\text{core}}} \right)^2 + \left( \frac{\pi}{H_{\text{core}}} \right)^2 \quad \dots (19)$$

Using previously reported values for the SRE,<sup>6</sup>  $\ell_{cth} = 5.1 \times 10^{-4}$  sec and  $\ell_{fth} = 0.5 \times 10^{-4}$  sec.

The term  $\Sigma_{1f}/\Sigma_{1r}$  is the fraction of neutrons which causes fissions before becoming thermal. This value is very difficult to calculate accurately in the SRE. A rough estimate was made using the infinite dilution resonance integral for  $U^{235}$  as the effective cross section in the fuel, and then computing a fuel resonance disadvantage factor by the same method as is used for thermal flux calculations.<sup>6</sup> The result was  $\Sigma_{1f}/\Sigma_{1r} = 0.06$ . Using the above values in Equation 17:

$$\ell_c = 4.8 \times 10^{-4} \text{ sec.}$$

The thermal neutron lifetime in the reflector was obtained by a Monte Carlo calculation which followed 200 neutrons injected into the reflector. This calculation showed that the average number of mean free paths traveled before returning to the core is 17, and the probability of return is 0.89. A similar calculation showed that the neutrons leaving the reflector have a probability of 0.60 of returning again to the reflector before being absorbed in the core.

Let  $f_r$  be the probability that a reflector neutron will return to the core, and let  $f_c$  be the probability that a neutron leaving the reflector will return to the reflector before being absorbed in the core. Then the probability of making exactly one pass through the reflector and returning to the core is  $f_r(1 - f_c)$ . For two passes, the probability is  $f_r f_c f_r(1 - f_c)$ , etc. Thus the total mean distance traveled in the reflector before finally being absorbed in the core is:

$$D_r = \frac{f_r(1 - f_c)d_r + (f_r f_c)f_r(1 - f_c)2d_r + (f_r f_c)(f_r f_c)f_r(1 - f_c)3d_r + \dots}{f_r(1 - f_c) + f_r f_c f_r(1 - f_c) + f_r f_c f_r f_r(1 - f_c) + \dots}$$

$$= \frac{d_r}{1 - f_r f_c}, \quad \dots (20)$$

where

$d_r$  = mean distance traveled during one pass through the reflector

$D_r$  = mean total distance traveled in the reflector before the final return to the core.

Thus the lifetime of thermal neutrons in the reflector is

$$\ell_{r_{th}} = \frac{D_r}{V_{th}} = \frac{d_r}{V_{th}(1 - f_r f_c)} , \quad \dots (21)$$

where  $V_{th}$  is the mean velocity of thermal neutrons in the reflector.

Neutrons which spend time in the reflector only as fast neutrons have the same lifetime as those which remain in the core, since the slowing-down times in the core and reflector are the same. For computational ease this effect is taken into account by assuming that the slowing-down time in the reflector is zero, and that a neutron which slows down in the reflector has the same lifetime in the core as those which slow down in the core.

The Monte Carlo calculation indicated that about 25% of the fast neutrons which enter the reflector never appear there as a thermal neutron, so for these neutrons  $\ell_r = 0$ . The remaining 75% of those which enter as fast neutrons also appear there as thermal neutrons, and have the same mean lifetime in the reflector as those which first enter as thermal neutrons. The probability that a neutron will remain in the core is:

$$P_{c0} = \frac{e^{-B_c^2 \tau}}{1 + L^2 B_c^2} ,$$

and the probability of entering the reflector as a fast neutron is:

$$P_f = 1 - e^{-B_c^2 \tau} ,$$

so the probability of first entering as a thermal neutron is:

$$P_{th} = (1 - P_{c0}) - P_f = \frac{(L^2 B_c^2) e^{-B_c^2 \tau}}{1 + L^2 B_c^2} .$$

Therefore, the mean 'lifetime' in the reflector is obtained by weighting the respective lifetimes by the fraction of neutrons:

$$\ell_r = \frac{\left(1 - e^{-B_c^2 \tau}\right)}{1 - \frac{e^{-B_c^2 \tau}}{1 + L^2 B_c^2}} \left[ 0.25(0) + 0.75 \left( \ell_{r_{th}} \right) \right] + \frac{\frac{(L^2 B_c^2) e^{-B_c^2 \tau}}{1 + L^2 B_c^2}}{1 - \frac{e^{-B_c^2 \tau}}{1 + L^2 B_c^2}} \ell_{r_{th}} = 2.9(10)^{-4} \text{ sec} .$$

Using this and the calculated lifetime in the core in Equation 14:

$$\ell^* = 5.5(10)^{-4} \text{ sec} .$$

This agrees very well with the experimental value<sup>7</sup> of  $5.2(10)^{-4}$  sec. Cohn's<sup>2</sup> expression for the effective lifetime is (in the present notation):

$$\ell^* = \frac{\ell_c + k_{c0}(1 - P_{c0})P_r \ell_r}{k_{c0} P_{c0}} ,$$

yielding a value of  $\ell^* = 7.2(10)^{-4}$  sec, which is not in agreement with the experimental value. Thus, the present derivation yields a better value for the effective neutron lifetime.

#### IV. CONCLUSIONS

- a) Bare reactor kinetics provides an accurate method of treating reflected reactors at frequencies more than one decade below the break frequency of the reflector, which is about 1000 cps for graphite reflectors.
- b) At higher frequency the reflector introduces, in effect, an additional group of delayed neutrons.
- c) The reactivity corresponding to prompt criticality is not affected by the reflector.
- d) The method of calculating prompt neutron lifetime discussed in this report yields results in good agreement with experiment.

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