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Radiochemical Techniques

**Neutron Activation Techniques  
for the Measurement  
of Trace Metals  
in Environmental Samples**

MASTER

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# **Neutron Activation Techniques for the Measurement of Trace Metals in Environmental Samples**

by

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**Prepared for Subcommittee on Radiochemistry  
National Academy of Sciences-National Research Council**

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## Foreword

The Subcommittee on Radiochemistry is one of a number of subcommittees working under the Committee on Nuclear Science within the National Academy of Sciences—National Research Council. Its members represent government, industrial, and university laboratories in the areas of nuclear chemistry and analytical chemistry.

The Subcommittee has concerned itself with those areas of nuclear science which involve the chemist, such as the collection and distribution of radiochemical procedures, the radiochemical purity of reagents, the place of radiochemistry in college and university programs, and radiochemistry in environmental science.

This series of monographs has grown out of the need for compilations of radiochemical information, procedures, and techniques. The Subcommittee has endeavored to present a series that will be of maximum use to the working scientist. Each monograph presents pertinent information required for radiochemical work with an individual element or with a specialized technique.

Experts in the particular radiochemical technique have written the monographs. The Atomic Energy Commission has sponsored the printing of the series.

The Subcommittee is confident these publications will be useful not only to radiochemists but also to research workers in other fields such as physics, biochemistry, or medicine who wish to use radiochemical techniques to solve specific problems.

Gregory R. Choppin, *Chairman*  
Subcommittee on Radiochemistry

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## I. INTRODUCTION

Anthropogenic wastes enter the environment via the atmosphere, and from direct discharges to aquatic and terrestrial surroundings. Once these pollutants have entered the environment they become entrained in complex biogeochemical and physical processes which serve to either disperse or reconcentrate the various contaminants. An essential feature of an effective environmental quality monitoring program is the capability for measuring these contaminants at near-background levels in the main matrices involved in pollution--fresh waters, sea water, atmospheric particulates, biological organisms and sediments and soils. Because the oceans are a major recipient of anthropogenic pollutants, the analyses of heavy metals in the marine environment will be emphasized in this treatise.

In monitoring the marine environment for metallic pollutants, few analytical methods offer the versatility and sensitivity achieved by neutron activation analysis (NAA). With the recent technological advances in developing high resolution solid state Ge(Li) detectors and their associated electronics, it is now possible to measure a wide spectrum of elemental constituents in virtually any environmental media by nondestructive, instrumental neutron activation analysis (INAA). Multi-element analyses by this method is advantageous in two respects: (1) many of the potentially hazardous pollutant trace metals in various environmental matrices can be simultaneously measured and (2) many of the innocuous elements which greatly affect the biological availability and the biogeochemistry of the pollutant metals can likewise simultaneously be measured. When

the concentrations of metallic pollutants are below the detection limits afforded by nondestructive INAA, either preconcentration or postirradiation separations of the elements of interest can be performed which, in most cases, increases the sensitivity for their measurement by orders of magnitude.

This monograph describes the technology and applications of NAA for the measurement of trace metal constituents in environmental samples, with emphasis placed on the marine environment. The contents are organized to indicate which trace elements can normally be measured by nondestructive INAA in the five matrices considered and, then when necessary, the chemistry required for preconcentrations and separations is provided. Emphasis has been placed on the measurement of the most potentially hazardous trace metals Hg, Cd, Cu, Ag, As, Sb, Cr, Se and Zn. Lead and tin are not included since NAA is not a sensitive method for these elements.

We stress the importance of analyzing only samples which have been collected and preserved to minimize contamination during these operations. Elaborate analyses of samples unrepresentative in time or space can also lead to erroneous conclusions. For both of these reasons we stress the need for good communication between the collector and the analyst. Precautions found effective in minimizing such undesirable artifacts of handling are discussed by Robertson (1972).

## II. PRINCIPLES OF NEUTRON ACTIVATION ANALYSIS

Neutron activation analysis is a nuclear method of elemental analysis in which the elements in a sample to be analyzed are made radioactive by irradiation with neutrons and the induced radioactive species are then identified and measured. The amount of a given neutron activation product that is formed during neutron irradiation is directly proportional to the amount of its parent isotope. Measurement of the radionuclide provides a measure of the total concentration of the parent element.

The basic equation for activation is

$$W = \frac{AM}{\sigma f \phi (1 - e^{-\lambda t}) \times 6.02 \times 10^{23}}$$

where  $W$  = weight of element irradiated, in grams

$A$  = induced activity in disintegrations/sec at  
the end of irradiation

$\phi$  = flux of neutrons used in the irradiation,  
in neutrons/cm<sup>2</sup>/sec

$\sigma$  = the activation cross section for the nuclear  
reaction concerned, in cm<sup>2</sup>

$f$  = fractional abundance of the particular isotope  
of the element concerned

$M$  = atomic weight of that element

$\lambda$  = the decay constant of the induced radionuclide,  
in sec<sup>-1</sup>

$t$  = irradiation time, in seconds

From the above equation it is obvious that the detection sensitivity for a given element by this method can be improved by increasing the neutron flux and/or the irradiation time. The relative detection sensitivities for all of the elements in the periodic table are a function of their activation cross section for the nuclear reaction concerned, the decay constant of the induced radionuclide, the atomic weight of that element and the fractional abundance of the particular isotope of the element concerned. For most elements, NAA is an extremely sensitive method of analysis, as evidenced in Tables 1 through 7. A notable exception is lead, an important ocean pollutant which is best measured by isotope dilution-mass spectrometry, or possibly atomic absorption spectrometry.

The concentration of an element in a sample can be calculated from the above equation using a flux monitor and if the efficiency of the detector is known for the neutron activation product of interest. However, the easiest and most accurate method for quantifying the concentration of any element is to include with the irradiation of the sample a standard of known weight of the element being determined. Then the weight of the

element being determined is obtained as follows:

$$\frac{\text{Weight of element in unknown}}{\text{Weight of element in standard}} = \frac{\text{Activity of element in unknown}}{\text{Activity of element in standard}}$$

This method obviates the need for careful neutron flux monitoring and counting instrument calibrations, since the only measurements that are required are the relative counting rates of the neutron activation products of the element of interest in the unknown and in the standard. This comparative method requires that the sample and standards be both irradiated and counted under the exact same conditions.

The neutron activation method of analysis does not distinguish between various physicochemical forms of an element in a particular sample, but provides a measure of its "total" concentration. However, if different physicochemical species of the same element are separated prior to the neutron irradiation, NAA can provide a measure of each specific form.

In practice, the fundamental steps in a typical NAA are:

- 1) Irradiate weighed quantities of the sample and standard in suitable containers for a sufficient time to give adequate radioactivity for the element to be determined.
- 2) After the irradiation, INAA is accomplished by simply counting the samples and standards on Ge(Li) and/or NaI(Tl) gamma-ray spectrometers at optimum times following the irradiation to directly measure elements producing short, intermediate and long-lived neutron activation products.
- 3) Compare the peak areas of samples and standards under identical counting conditions, making decay corrections, Compton corrections and background corrections when necessary.
- 4) If INAA does not provide the desired sensitivity, radiochemically separate the activation product(s) of interest from the interfering radionuclides and count the separates in the same geometries as the standards.

- 5) To insure that the gamma-ray being measured is due solely to the radionuclide of interest, check the half-life of the activity to be certain it is decaying at the proper rate. Further confirmation may be possible when the radionuclide emits more than one gamma-ray per disintegration, in which case both the energies and relative intensities can be used for verification.

For more detailed discussions of the basic theory, fundamentals and applications of NAA, the readers are referred to the following references: Anonymous (1971); Barbier (1969); DeVoe and LaFleur (1969); Guinn and Lukens (1965); Kruger (1971); Lenihan and Thompson (1969); Lutz et al. (1971) Rakevic (1970); Ryan (1973) and Smales (1967).

### III. NEUTRON IRRADIATION FACILITIES AND PROCEDURES

The most generally applicable neutron source for trace level activation analysis is the high flux nuclear reactor. Most of these reactors normally operate at power levels in the range of 10 to 1000 kilowatts and produce thermal neutron fluxes of  $10^{11}$  to  $10^{13}$  n/cm<sup>2</sup>/sec. Reactors used in such work are generally of the research type and are becoming increasingly available for use by the scientific community.

The neutron flux in such reactors varies spatially about the reactor core and inhomogeneities in the flux must be taken into account. Such inhomogeneities can be minimized by rotating the samples in the flux field in various types of specimen racks designed for that purpose. In a typical irradiation, up to 50 samples and standards can be irradiated in one rotator simultaneously. Although the horizontal neutron flux gradients in each rotator are minimized by revolving the samples, there exists a small vertical flux gradient which must be accounted for by placing a standard at each tier of samples if they are stacked vertically.

"Rabbit" irradiations are used for activating samples for periods of seconds to minutes, where the induced activity of interest is a short-lived isotope, one having a half-life in

the range of seconds to minutes. Samples are activated and counted one at a time (usually purely instrumentally), with the reactor operating steadily and a standard sample of the element of interest similarly activated and counted at intervals.

#### IV. SAMPLE PREPARATION

##### A. Materials and Manipulations

A tremendous advantage of using INAA is that sample handling and manipulations are reduced to a minimum, thereby greatly reducing the probability of contaminating a sample. Contamination problems are particularly severe when dealing with natural waters. The problems of contamination in trace element analysis have been thoroughly dealt with in the literature (Robertson, 1968; 1972; Thiers, 1957 a, b).

Solid samples, such as sediments, biological tissue, air filters and sea and fresh water salts are normally weighed and heat sealed in cleaned polyethylene, polypropylene or high purity quartz vials ranging in size from 1 cc to 10 cc. Sample size can range from a few milligrams to tens of grams depending upon the sensitivity required and sample size available. Desiccation of sediments and biological tissue prior to encapsulation is accomplished by either freeze-drying or oven-drying at 105°C. Special precautions are necessary when drying samples for trace Hg analyses to avoid losses by volatilization. These precautions are discussed in the section dealing with mercury analysis by NAA.

Natural waters, especially seawater, must usually be evaporated to dryness if the samples are to be irradiated for more than several minutes, since gaseous radiolysis products ( $H_2$ ,  $O_2$  and other gases) are generated and produce high pressures which rupture the plastic irradiation vials. Problems due to pressure build-up are dependent upon the length of the irradiation, and almost any kind of sample can be irradiated for a few minutes. Desiccation can be accomplished by either

freeze drying or slowly evaporating the water under an infrared heat lamp. Again, great care must be taken not to volatilize Hg and other volatile elements during these drying operations.

Water samples can be neutron irradiated directly, without evaporation, if the water is sealed in quartz vials. However, in the case of seawater, the radiolysis products build up dangerously high pressures which have even shattered sealed quartz vials and this method is not recommended for routine use. In addition, quartz contains much higher levels of impurities than polyethylene or polypropylene and is a potential source of contamination. Tanner et al. (1972), and Brune and Landstrom (1966) have described a method for neutron irradiating 30 to 80 ml samples of fresh water in a frozen state in polyethylene containers as a means of preserving the samples prior to and during the neutron irradiation.

It is important that samples in a given set be encapsulated in reproducible, identical geometries so that all samples will be neutron irradiated in a known orientation.

B. Elemental Standards

In multielement INAA, it is usually not practical to use separate individual elemental standards, and composite standards consisting of compatible mixtures of elements are preferred. Standard solutions containing 5 to 10 elements at optimum concentrations to provide good counting statistics and minimal interelement interferences can be conveniently prepared. Thus, only a few standard capsules need to be included in a large set of samples for neutron activation. It is important that standards for each element being measured be included in each irradiation. The use of flux monitors for intercomparison of irradiations made at different times and reactor fuel configurations should be discouraged, since some activation products are produced to a large degree by resonance capture of fast neutrons. The production of such activation products depends not only upon the neutron flux, but also upon the neutron energy spectra.

Elemental standards should be encapsulated and irradiated identically as the samples. Standards should be prepared in a matrix as close to the sample matrix as practically possible in order to minimize problems such as neutron self-shielding, irreproducible activation geometries and interfering nuclear reactions. Perlman and Asaro (1969) have discussed in some detail the necessity of carefully preparing standards in the same geometries and matrices which match, as closely as possible, that of the samples. For example, when neutron activating large quantities of sea salts, the elemental standards should be added to an equivalent amount of NaCl encapsulated in the same geometry as the sample. However, if sample sizes are kept very small, and the samples do not contain high concentrations of large cross section elements, exquisite matrix matching is not a critical requirement for the types of samples normally encountered in the marine environment.

The use of standard reference materials of known elemental composition as activation standards is a convenient method of simulating the sample matrix. Several reference materials are available for use as biological, environmental, and geological standards. The National Bureau of Standards (NBS) supplies orchard leaf, tuna meal and bovine liver as standard reference materials for analysis of biological material. Elemental concentrations in these materials are gradually being certified by NBS and they may become useful activation standards once their elemental concentrations are established. The Environmental Protection Agency (EPA) in cooperation with NBS has made up four environmental standards: fly ash, coal, oil and gasoline. Concentrations of a number of elements (especially those of prime environmental concern) will be established during 1973. The U.S. Geological Survey supplies a wide variety of geological standard reference materials which have reasonably well documented elemental concentrations (Flanagan, 1969; Fleischer, 1965). These rock samples are somewhat non-uniform for some elements, but represent the best available reference materials for analysis of marine sediments. Also, EPA has provided three freshwater sediments which many laboratories have analyzed for mercury. Two have mercury levels of  $\sim 45$  and  $\sim 105$  ppm Hg, respectively, which are much higher concentrations than normally found in natural

sediments. The third has ~ 0.06 ppm and is at the lower end of most mercury values in sediments. These sediments are available in limited amounts from J. H. Finger, Environmental Protection Agency, Surveillance and Analyses Division, Athens, Georgia. Frequently, however, individual elemental standards must be prepared and included with the irradiations when the concentrations of those particular elements are not sufficiently high in the standard reference materials to be accurately measured.

To date, no standard reference natural water samples are available and analysts are required to prepare their own elemental standards. However, the EPA has made available six water reference samples which when diluted as prescribed will give different concentrations of As, Cd, Cr, Cu, Pb, Se and Zn in the ppb concentration range. These samples are available from J. A. Winter, Methods Performance Evaluation Activity, National Environmental Research Center, EPA, Cincinnati, Ohio, 45268.

C. Preconcentration and Postirradiation Separations

In most cases the only matrices that require preconcentration of some trace elements prior to neutron activation are fresh and sea waters, where the natural elemental concentrations are near or below the parts per billion range, thus requiring large volumes of sample to achieve the desired sensitivity. Atmospheric particulates are naturally preconcentrated from large volumes of air during their collection on various filter media. Generally, the trace metal levels in marine organisms, sediments and atmospheric particulates are sufficiently high so that convenient sized samples can be neutron activated and postirradiation radiochemical separations be performed.

Postirradiation separations are usually the most desirable, because once the radioactive species are produced, any amounts or types of stable reagents and labware can be used in the chemical manipulations without contaminating the samples with the elements being measured. However, in the case of sea salts, if trace elements with short-lived (< 1 to 2 days) neutron activation

products are to be measured, the trace elements must be separated from the NaCl matrix before neutron irradiation, since high radiation doses result from the large concentrations of  $^{24}\text{Na}$  and  $^{38}\text{Cl}$  which are produced when sea salts are neutron activated.

Preconcentration schemes for natural waters can be very simple, involving perhaps only the removal of much of the major activity component, such as  $^{24}\text{Na}$ . Alternatively, the separations can be made groupwise, and both such procedures are usually followed by gamma-ray spectrometry to obtain final specific measurement of each individual neutron activation product. Occasionally, and especially where the highest possible sensitivity is required, very thorough preconcentration or postirradiation separations are necessary. In the following sections chemical separation schemes are given for those elements of interest which cannot be measured in the various matrices by purely INAA.

V. GAMMA-RAY SPECTROMETRY

Since all of the elements of interest in this work, except Pb and Sn, form neutron activation products with easily measurable gamma-rays, this section will be concerned only with gamma-ray spectrometry involving solid state Ge(Li) diode detectors and NaI(Tl) scintillation crystals.

A. Ge(Li) Diode Spectrometry

Lithium drifted germanium detectors have revolutionized INAA because of their excellent energy resolving capability (about 2 to 3 keV FWHM above about 200 keV). Even better resolution can be attained at energies below about 200 keV utilizing low energy photon detectors (LEP detectors). These detectors will allow increasing use of low energy gamma-rays and X-rays for INAA. Heretofore, single NaI(Tl) crystals were capable of resolving only a few of the major constituents in complex mixtures of radionuclides. Ge(Li) detectors, however, can resolve the gamma rays of nearly all neutron activation products formed in marine environmental matrices, if their concentrations are sufficiently high to be detected. Gamma-ray energies can be determined to within  $\pm 0.3$  keV, and the energies of nearly all neutron activation products are now

known to  $\pm$  0.1 keV. A good example of this improved resolution is shown in Figure 7, which compares the gamma-ray spectra of neutron activated sea salts counted on both a NaI(Tl) crystal and a Ge(Li) diode detector. The main disadvantage of Ge(Li) detectors is their relatively low efficiency, compared to NaI(Tl) crystals. However, for most neutron activated samples the low efficiency of the Ge(Li) detectors is not a serious problem, since sufficiently high concentrations of the neutron activation products of the elements of interest can normally be produced in high flux nuclear reactors.

The basic principles and applications of Ge(Li) diode detectors have been adequately described in the literature (Bertolini and Coche, 1968; Brown et al, 1969; Camp, 1967; Cooper, 1973; Dearnaley and Northrop, 1966; Heath, R. L., 1969; Hollander, 1966; Keil and Bernt, 1972; Lutz et al, 1971). Briefly, the hardware required for a basic Ge(Li) gamma-ray spectrometer is the Ge(Li) detector, a preamplifier, an amplifier, and a 2000 or 4000 channel pulse height analyzer.

While Ge(Li) detector systems of greater versatility and sophistication have been constructed (for example, Cooper and Perkins, 1972) ordinary Ge(Li) detector systems are adequate for measuring a large group of neutron activation products in environmental samples.

In practice, the neutron activated samples and standards are transferred from the irradiation containers into standard counting geometries, which may consist of polyethylene vials of various sizes or some other suitable containers. Care must be taken to assure that the samples and the standards are encapsulated in identical counting geometries. Relatively small differences in orientation of the samples and standards within the containers can lead to serious errors if the containers are counted on or close to the Ge(Li) detector. This is especially important for such unusually shaped samples as filters and resins. One must not be seduced into placing the samples on the detector to increase counting efficiency without experimentally evaluating the geometry factors. The samples and standards are then positioned on or near the

Ge(Li) detector in a fixed geometry and counted for a sufficient length of time to achieve the necessary counting statistics. Differences in geometries in a set of samples can be minimized by counting the samples several cm away from the Ge(Li) detector if sufficient activity is present.

**B. NaI(Tl) Spectrometry**

NaI(Tl) detector systems are most useful when sensitivity is the main concern, since NaI(Tl) detectors are approximately 10 to 100 times more efficient than Ge(Li) diode detectors. NaI(Tl) crystals of various geometries are commercially available but the most commonly employed are well crystals or solid, cylindrical crystals of various sizes normally ranging from about 3 to 13 inches in diameter and about 3 to 9 inches thick. The background and Compton response of these crystals can be substantially lowered by anticoincidence shielding with large plastic phosphors or annular NaI(Tl) crystals (Perkins and Robertson, 1965; Wogman et al, 1967, for examples). Two large anticoincidence-shielded NaI(Tl) detectors can be designed to count as dual coincidence gamma-ray spectrometers, thereby greatly increasing the sensitivity and selectivity of NaI(Tl) gamma-ray spectrometry (Perkins and Robertson, 1965).

These large crystal NaI(Tl) systems are particularly applicable for measuring traces of neutron activation products which have been radiochemically separated from an interfering matrix. Their high efficiencies permit very low levels of radioactivity to be measured in relatively short counting intervals.

**C. Data Handling**

Interpretation of gamma-ray spectra obtained from NaI(Tl) and Ge(Li) detectors typically involves location of peaks in the spectra, determination of peak energies, measurement of peak areas and calculation of concentrations of the elements from which the gamma-ray emitters were formed. In NaI(Tl) spectrometry, if the radionuclide mixture is quite simple, the peak areas can be obtained by manual subtraction of background and Compton contributions. However, if the radionuclide mixture

contains more than 3 or 4 gamma emitting radionuclides, the spectrum stripping is most conveniently and accurately accomplished by using a computer program designed for that purpose. Because peak areas in Ge(Li) diode spectra are normally well resolved, determination of peak intensities by manual subtraction of the background and Compton contribution can be readily accomplished in even very complex mixtures of radionuclides. Normally, an equal number of channels on each side of the peak area are selected to represent the contribution due to background and Compton interferences and this contribution is manually subtracted from the peak area. However, if many routine analyses are required, the manual method can become tedious and time consuming and the most rapid and economical way to process the data is with a computer.

The development and application of computer programs for the analysis of gamma-ray spectra is, by no means, an elementary task. For computer analysis to be workable, spectrometer systems must be very electronically stable, precisely adjusted before counting and the spectra must be converted to a computer compatible form. Incorrect input of data confuses a computer, resulting in termination of the analysis or computation of wrong answers. Once these problems have been overcome, a computer analysis of the gamma-ray spectra is capable of rapidly converting the spectra directly into concentrations of individual constituents. However, unless one is tooling up for the analysis of a large number of routine samples, it is often more trouble than it is worth to develop a computer program designed to quantify INAA. It is a good practice to occasionally check computer programs by hand calculations to ensure that they are operating as designed.

For more detailed discussions of the use of computers for data reduction the readers are referred to the following material: DeVoe and LaFleur, 1969; Salmon and Creevy, 1970; O'Kelley, 1962.

VI. ELEMENTAL ANALYSES

A. Fresh Waters

1. Instrumental Neutron Activation Analysis

Nearly all of the potential pollutant elements of interest can be measured in fresh waters (atmospheric precipitation, rivers, lakes, tap water, etc) by INAA utilizing a very simple radiochemical group separation to remove interfering neutron activation products. Figure 1 illustrates the relative abundances of the major and some of the minor radio-nuclides formed during neutron activation of Columbia River water and their change in concentration with time. It is obvious that the high activity due to  $^{24}\text{Na}$  (Columbia River water contains about 2 mg Na/l) will mask the gamma-ray spectra of all of the trace neutron activation products of the elements of interest, except  $^{38}\text{Cl}$ ,  $^{42}\text{K}$ ,  $^{56}\text{Mn}$  and possibly  $^{87\text{m}}\text{Sr}$ . Therefore, to measure the short-lived neutron activation products of As, Hg, Cu, Zn, Ba and La, as well as several other elements of geochemical interest, it is necessary to remove the interfering  $^{24}\text{Na}$ .

Tanner et al. (1972) have described a simple two-step separation scheme which eventually permits the semi-instrumental measurement of about 19 trace elements (see Figure 2). The method simply consists of passing a neutron activated water sample, containing carriers of 0.1 mg each of Na, K, Rb and Cs, through a column of Dowex 1x2 anion exchange resin which has been converted to  $\text{OH}^-$ ,  $\text{S}^=$  and  $\text{CO}_3^{=2}$  forms by contacting with 1M solutions of  $\text{NaOH}$ ,  $\text{Na}_2\text{S}$  and  $\text{Na}_2\text{CO}_3$ , respectively. All trace activation products, except the alkali metals and the alkaline earths, are retained on the resins by precipitation-ion exchange mechanisms. The combined resin is then rinsed with distilled water, transferred into a plastic counting container and counted directly on a Ge(Li) detector. The column effluent, containing the radionuclides  $^{24}\text{Na}$ ,  $^{42}\text{K}$ ,  $^{86}\text{Rb}$ ,  $^{134}\text{Cs}$  and traces of neutron activation products of the alkaline earths Ca, Ba and Sr, is treated with an excess of 4% sodium tetraphenylboron to

FIGURE 1

RELATIVE CONCENTRATIONS OF INDUCED RADIONUCLIDES IN COLUMBIA RIVER WATER FOLLOWING AN INTEGRAL NEUTRON IRRADIATION OF  $2 \times 10^{16} \text{ n/cm}^2$ , EMPHASIZING THE PROBLEM OF MEASURING SHORT-LIVED ACTIVATION PRODUCTS BY INAA IN THE PRESENCE OF HIGH CONCENTRATIONS OF SODIUM-24

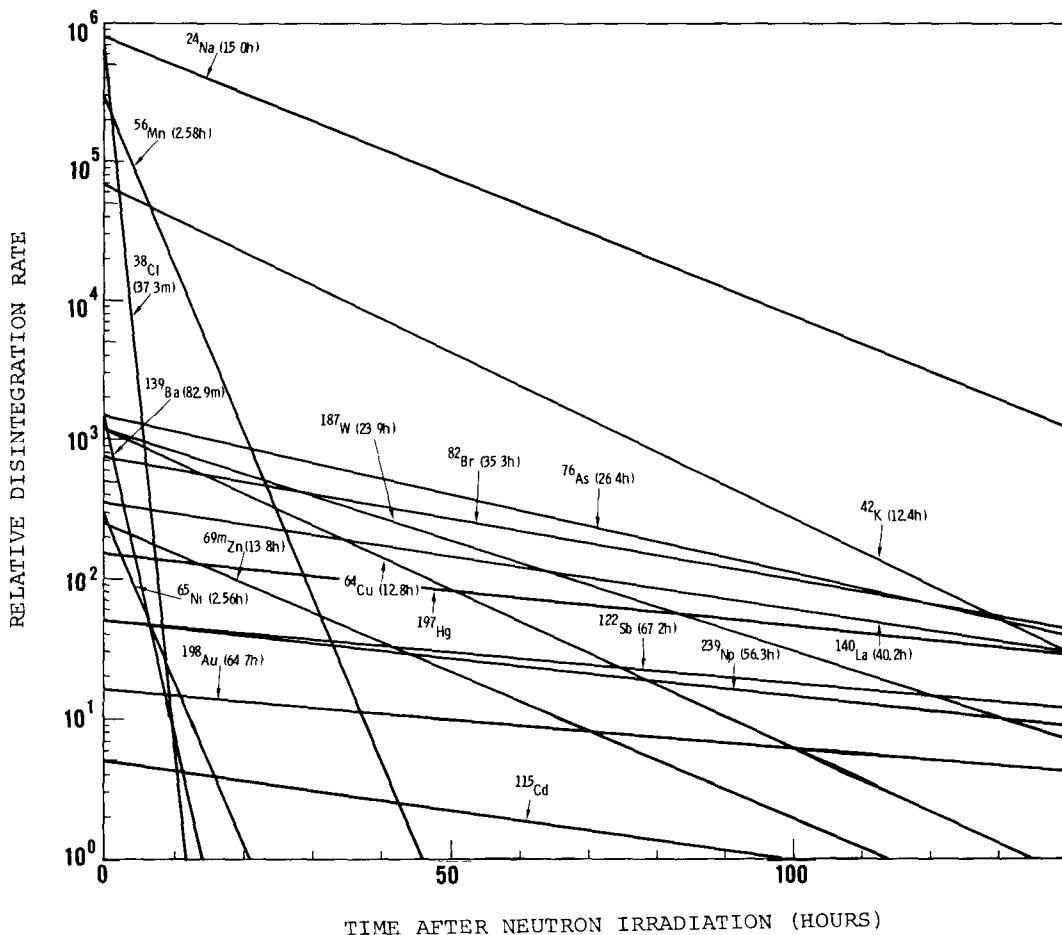
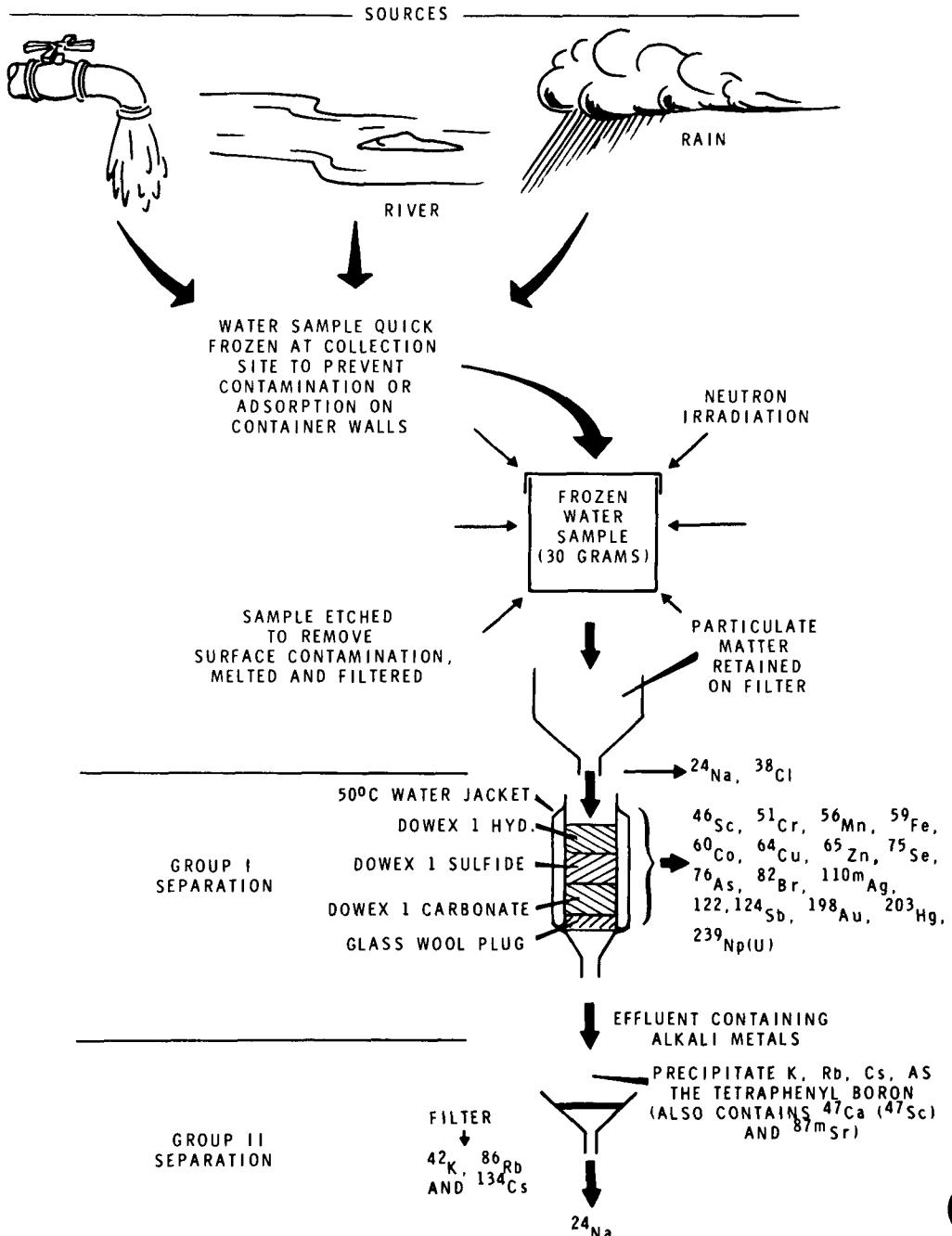


FIGURE 2

MULTIELEMENT ANALYSIS OF TRACE CONSTITUENTS IN NATURAL WATER  
BY NEUTRON ACTIVATION, TWO-GROUP CHEMICAL SEPARATIONS AND  
GAMMA-RAY ANALYSIS

(after Tanner, et al, 1972)



precipitate Cs, K and Rb. The precipitate is washed, filtered and counted directly on a Ge(Li) detector. Standard solutions of the elements of interest are sealed in quartz vials, irradiated, transferred to plastic counting containers and counted in the same geometries as the samples to quantify the measurements by the comparator method. Figure 3 illustrates the gamma-ray spectra of the short-lived radionuclides measured in neutron activated Columbia River water before and after the separation scheme. Figure 4 illustrates the gamma-ray spectra of the long-lived neutron activation products contained on the resins and tetraphenylboron precipitate by counting 23 and 24 days after the irradiation. Eleven additional neutron activation products can be detected from these counts.

In Table 1 the elements which can normally be measured in fresh waters by this semi-INAA technique are indicated, together with the detection limits of many of the elements of interest. Without any separations at all it would still be possible to measure Ag, Br, Cl, Co, Cr, Cs, Fe, Mn, Na, Rb, Sb, Sc, Se, U, Zn, Ag and occasionally Hg, simply by counting the neutron activated river water samples at several optimum times following the irradiation (approximately 4 hours, 3 days and 20 days). The radiochemical separations permit the measurement of As, Cu, La and K.

Table 1 also lists the approximate limits of detection for 20 elements in river water utilizing the method of Tanner et al. (1972), and compares these limits with maximum allowable concentrations of some of the more toxic heavy metals in drinking water, as established by the U. S. Public Health Service. As indicated in Table 1, this semi-INAA technique can be used to detect most heavy metal pollutants in fresh waters at concentrations well below maximum permissible limits. Of the most toxic heavy metals, only Cd, Pb, Sn and occasionally Hg cannot be measured by this method.

FIGURE 3 GAMMA-RAY SPECTRUM OF NEUTRON ACTIVATED RIVER WATER  
(after Tanner, et al, 1972)

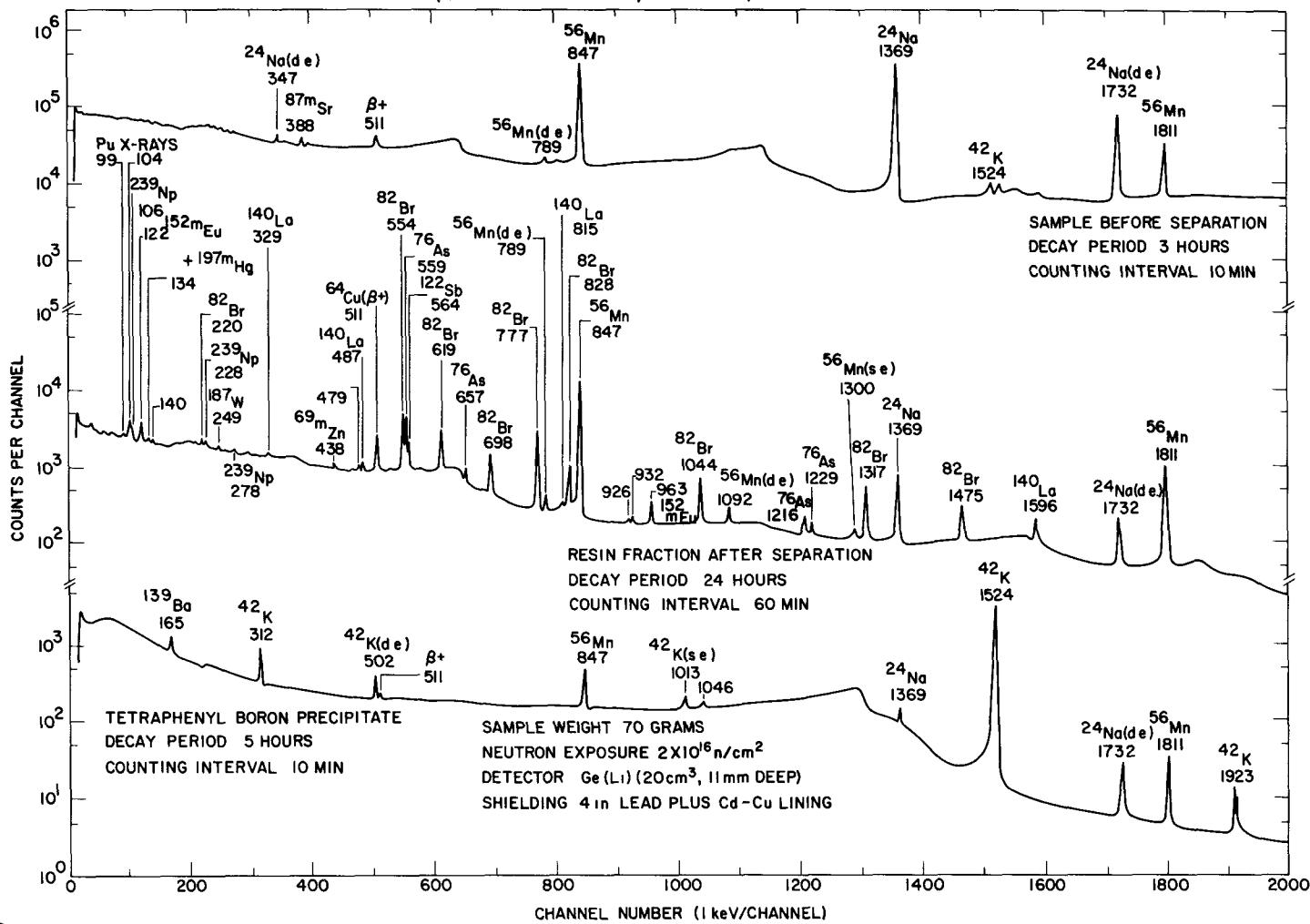


FIGURE 4 GAMMA-RAY SPECTRUM OF NEUTRON ACTIVATED RIVER WATER  
(after Tanner, et al, 1972)

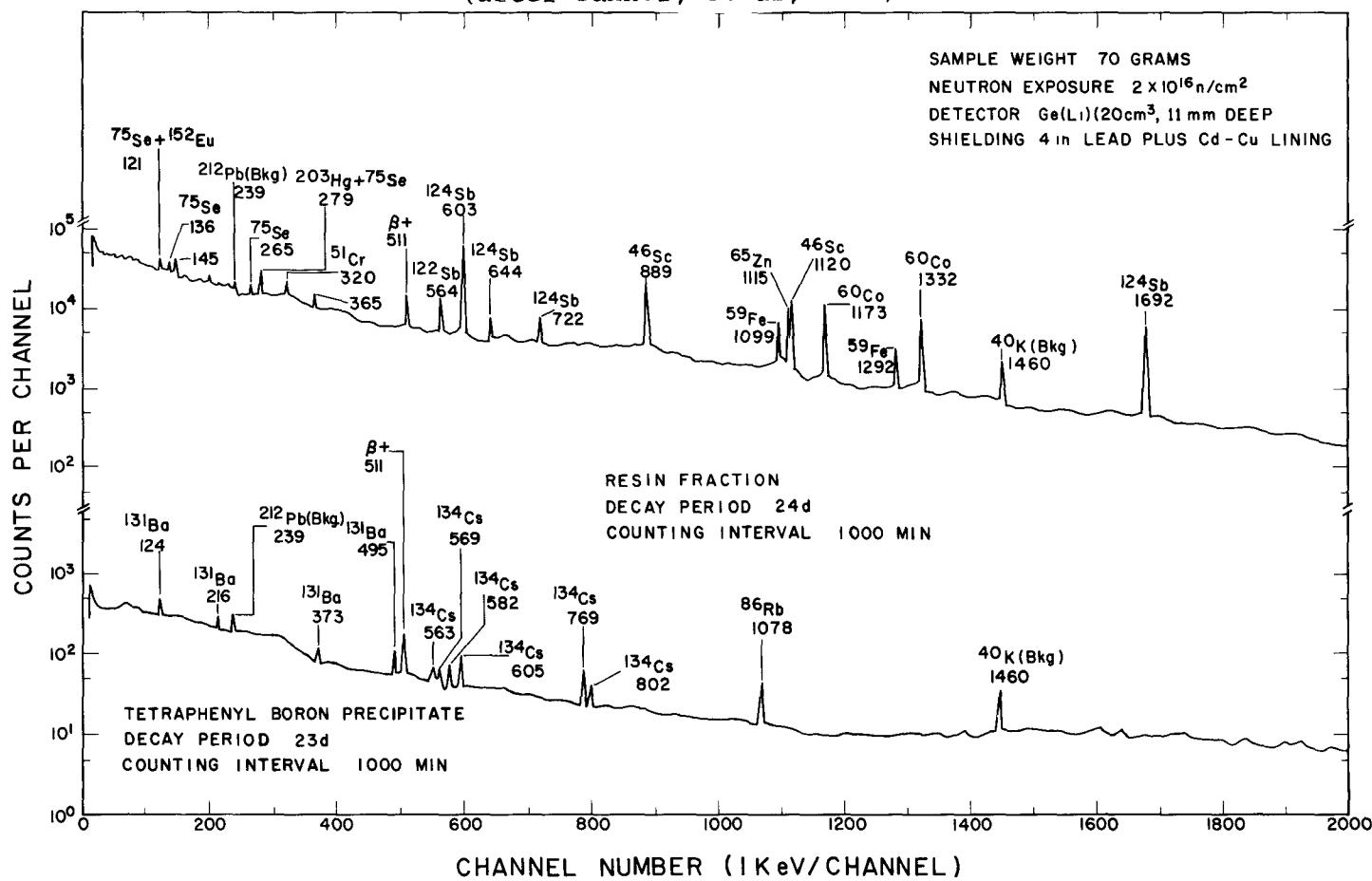


TABLE 1

APPROXIMATE LIMITS OF DETECTION OF VARIOUS TRACE METALS  
IN RIVER WATER BY NEUTRON ACTIVATION AND GROUP SEPARATIONS

Element	Approx. Conc. in River Waters ( $\mu\text{g/l}$ ) (a)	Detection Limits by NAA After Group Separation ( $\mu\text{g/l}$ ) (b)	Allowable Limits ( $\mu\text{g/l}$ ) (c)
Ag*	0.3	0.01	50
As	2	0.1	50
Ba	10	5	1000
Br*	20	0.01	
Cd	--	50	10
Cl*	7800	100	
Co*	0.2	0.01	
Cr*	1	0.1	50 ( $\text{Cr}^{+6}$ )
Cs*	0.02	0.01	
Cu	7	0.1	1000
Fe*	--	10	300
Hg	0.07	0.1	(10)
K*	2300	10	
Mn*	7	0.01	50
Na*	6300	10	
Rb*	1	0.1	
Sb*	0.3	0.01	
Sc*	0.004	0.001	
Se*	0.2	0.1	10
Zn*	20	1	5000

\* Elements normally detected and measured by INAA

(a) after Turekian (1969)

(b) after method of Tanner et al. (1972)

Thirty to eighty ml water samples; integrated thermal neutron exposure  $2 \times 10^{16}$  n/cm<sup>2</sup>; count for 10 min., 60 min., and 1000 min. after decay times of 3 hrs, 24 hrs and 23 days, respectively; 20 cc Ge(Li) detector.

(c) U. S. Public Health Service (1962).

Edgington and Lucas (1971) and Girardi and Sabbioni (1968) have developed a method of removing  $^{24}\text{Na}$ , the major interference in NAA of environmental samples, based on selective retention of the  $^{24}\text{Na}$  on hydrated antimony pentoxide (HAP). All neutron activation products, except  $^{24}\text{Na}$ ,  $^{182}\text{Ta}$  and  $^{233}\text{Pa}$ , reportedly pass through the HAP column and are measured by Ge(Li) gamma-ray spectrometry. The method of Edgington and Lucas (1971) provides a breakdown into further group separations by selective elution of the effluent from the HAP separation from Dowex 1x8 anion exchange resin.

Neutron activation analysis is not a sensitive method for Pb and Sn, but Cd can be measured by preconcentration from 500-1000 ml of fresh water. Since Hg and Ag are normally present in uncontaminated fresh waters at concentrations near or below their detection limits for INAA, preconcentration or post irradiation separations are described below for these elements, as well as for Cd.

#### Suspended Particulate Matter

Particulate matter suspended in natural waters carry significant percentages of the total amounts of several elements in the waters. To determine the partitioning of the elements between the two phases, it is desirable to isolate the suspended particulate material, by either centrifugation or membrane filtration. The preferred method is by membrane filtration, and the usual filter pore size used is about 0.3 to  $0.5\mu$ . Both Millipore  $^{\text{R}}$  and Nuclepore  $^{\text{R}}$  filters have commonly been used for this purpose, and both filters contain roughly comparable traces of most impurities. It is recommended that as large a water sample as practically possible be filtered to obtain enough suspended matter for elemental analysis and to reduce the probability of contamination of the filtered water during the filtering process. The first several hundred ml of filtrate should be discarded because of contamination which may be leached from the filters.

Several types of all-plastic filtering devices are commercially available (Millipore Corporation; General Electric Co., for examples) which are suitable for filtering natural waters. The usual procedure for INAA of suspended particulates is to collect the material on tared filters (Millipore  $\textcircled{R}$ , Nuclepore  $\textcircled{R}$ , etc.), dessicate to constant weight and reweigh the filters to obtain a gravimetric analysis. The weighed filters are then encapsuled in plastic irradiation vials and neutron activated, together with appropriate standards of the elements of interest. After the irradiation, the samples are then counted on a Ge(Li) detector at optimum decay times to measure up to 24 trace elements. Since the suspended particulate matter in natural waters is usually a combination of both geological and biological material, the discussion of which elements can be measured and the detection sensitivities which can be achieved will be covered in the following sections concerned with analyses of biological and geological material.

Several problems may arise during this procedure. The filters themselves may lose weight when large volumes of water are filtered through them. The concentrations in the filters of many elements of interest vary considerably from filter to filter and from batch to batch, creating a large uncertainty in the blank than one would like. The filters may become friable due to irradiation damage under long irradiations at high neutron fluxes, leading to problems in subsequent manipulations with the filters. Weighing of membrane filters on an analytical balance may present some problems due to the electrostatic charge which accumulates on the filters. This can be overcome by inserting a special  $\alpha$ -emitting Po source (Watson Brothers, Northern California Division, Scientific Instruments, San Mateo, California) inside the balance case. The ionization of the air produced by the

alpha particles neutralizes the electrostatic charge on the membrane filter. Nuclepore <sup>®</sup> filters appear to offer the most desirable taring characteristics because they absorb less moisture than other types of membrane filters.

2. Analysis of Elements Requiring Preconcentration or Postirradiation Separations

The trace metals of pollution concern which frequently need to achieve the necessary sensitivity for NAA are Hg, Cd, Ag, As and Se. Procedures are given below which accomplish these separations.

Under certain conditions, mercury losses by diffusion through polyethylene irradiation containers during neutron activation of fresh waters have been reported (Bate, 1971). Individual investigators should evaluate their particular methods of encapsulating and irradiating natural waters to insure that such losses are not significant. One method of minimizing such losses is to irradiate the water sample in a frozen state, as indicated in the following procedure.

Mercury Procedure I

Landstrom and Wenner (1965) have reported a simple procedure for the isolation of <sup>203</sup>Hg from neutron activated natural water samples. A 30 ml water sample, neutron irradiated in a frozen state for 12 hours at a flux of about  $2 \times 10^{12}$  n/cm<sup>2</sup>/sec, is transferred to a distillation flask and acidified with H<sub>2</sub>SO<sub>4</sub>. Hydrogen peroxide and HBr are added and the Hg is distilled as the bromide. The distillate is made 3.5 M in H<sub>2</sub>SO<sub>4</sub> and 0.1 M HCl, H<sub>2</sub>O<sub>2</sub> is added, Br is driven off and the solution is passed through a small column of Dowex 2x10 anion exchange resin in the sulfate form. The column, containing the sorbed Hg, is rinsed with 0.1 N HCl to elute Sb, As, and Se and the anion resin is then counted to determine <sup>197-203</sup>Hg activity. Mercury standards, irradiated with the samples, are counted in the same geometry to quantify the measurements. Recovery is reported to be 96%.

Mercury Procedure II

Ljunggren et al. (1971), have reported a method for separating radiomercury from neutron activated fresh waters. Water samples 0.3 to 10 ml in volume are sealed in cleaned quartz vials and irradiated for 2 to 3 days in a thermal neutron flux of about  $3-4 \times 10^{12}$  n/cm<sup>2</sup>/sec. After a short cooling period, the samples and 20 mg of Hg carrier are digested in the Bethge apparatus with 15 ml of 10:1 mixture of concentrated nitric acid and sulfuric acid. To the digested solution is added a mixture of 5 ml of 70% perchloric acid and 0.5 gm of glycine in 5 ml of water. The Hg is distilled from the solution until the temperature reaches 250°C. The distillate (containing the Hg) is diluted to about 300 ml with distilled water and Hg is electrodeposited on a tared thin gold foil. A platinum foil (2 x 3 cm) is used as an anode and a thin gold foil (1 x 3 cm) as the cathode, and these electrodes are immersed in the electrolyte about 6 cm apart. About 95% of the Hg is electrodeposited after 15 hours of electrolysis at a voltage of 4 to 6 volts and a current of 0.3 to 0.5 ampere. When the electrolysis is completed, the electrolyte is neutralized with NH<sub>4</sub>OH, using phenol red as an indicator. Before the current is interrupted, the gold foil is removed from the solution, and rinsed with distilled water and alcohol. The electrical contacts are then removed and the gold foil is dried and reweighed to obtain the chemical yield. The foil is sealed in a plastic bag and the <sup>197-203</sup>Hg activity is measured by gamma-ray spectrometry. The use of a sample of 10 ml places the detection limit at 0.01 ng/ml.

Mercury Procedure III

Brune and Jirlow (1967) have developed a method for separating radiomercury from neutron activated fresh water samples based on the isotopic exchange technique reported by Kim and Silverman (1965). Water samples are neutron irradiated in polyethylene containers in a frozen state for 3 to 13 hours at a flux of about  $2 \times 10^{12}$  n/cm<sup>2</sup>/sec. After the irradiation, the water samples are transferred to a distilling flask equipped with a reflux condenser. A 3:1 mixture of nitric and

sulfuric acid is added and the sample refluxed for about 20-40 minutes. The solution is cooled and then diluted with an equal volume of water, and the pH is adjusted to 1 to 2 by addition of  $\text{NH}_4\text{OH}$ . The solution is transferred to an appropriate sized bottle and 0.5 gm of  $\text{NH}_4\text{Br}$  is added, followed by 0.2 ml of elemental inactive Hg. The solution is vigorously stirred for 40 minutes to allow the radiomercury to exchange with the inactive Hg. The Hg is then separated on a sintered glass filter disk, washed with water and acetone, and then dissolved in 5 ml of concentrated nitric acid. The nitric acid is transferred to an appropriate container and  $^{197-203}\text{Hg}$  is measured by gamma-ray spectrometry.

#### Mercury Procedure IV

Thatcher and Johnson (1971) have developed a precipitation method for measuring Hg in fresh waters by NAA. Water samples are placed in 20 ml quartz vials which are immediately sealed. The vial is neutron irradiated for 4 hours in a flux of  $5 \times 10^{12} \text{ n/cm}^2/\text{sec}$ . One day after the irradiation the vial is opened, carrier Hg in nitric acid is added and the sample is allowed to stand one hour to desorb any radioactive Hg from the quartz. The sample is transferred to a polyethylene centrifuge tube and ammonium persulfate is added to oxidize organo-mercurials. Sodium bisulfite is added to reduce excess persulfate and to dilute  $^{24}\text{Na}$  and  $^{35}\text{S}$  activities. Stannous chloride is next added to precipitate metallic mercury. The precipitate is centrifuged and counted for  $^{197}\text{Hg}$ . Sensitivity is 0.05  $\mu\text{g/l}$  but could be improved by simply activating the samples for a longer period of time.

The above four methods are capable of measuring only "total" mercury (inorganic plus organic forms), since the post-irradiation digestion procedures should destroy any organically bound Hg. Although these four procedures have been developed especially for neutron activation analysis of Hg in fresh waters, there are numerous postirradiation radio-

chemical separation schemes developed for other matrices which can be slightly modified to meet the needs of fresh water analyses. These procedures are contained in an excellent monogram by Roesmer (1970).

If Hg concentrations in fresh waters are too low to measure in 1 to 30 ml volumes as described in the above procedures, preconcentration from large volumes (0.2 to 2 liters) of water is necessary. Lai and Weiss (1962) have described a very simple preconcentration procedure, which has subsequently been used at Battelle-Northwest Laboratory (Robertson, unpublished work, 1972) for preconcentration of Hg from seawater and should be applicable to fresh waters. A water sample (0.2 to 2 liters) is adjusted to a pH of 3.5 to 8, warmed to about 60°C and 20 ml of a 1% solution of thionalid in acetone is added. The water samples are then chilled in a refrigerator and the thionalid crystals (containing quantitatively cocrystallized Hg) are collected on membrane filters, washed with high purity water, and encapsulated in polyethylene vials for neutron activation analysis.  $^{203}\text{Hg}$  can be directly measured in the neutron activated thionalid by Ge(Li) gamma-ray spectrometry several weeks after the irradiation.

Weiss and Crozier (1972) have described a method of preconcentrating Hg from seawater, which should be equally applicable to fresh waters. Mercury is separated from 500 ml of water by adding 1 mg of Cu carrier and coprecipitating the Hg with the Cu as sulfides by bubbling  $\text{H}_2\text{S}$  gas into the water for 30 second. The CuS precipitate is collected by membrane filtration, washed with high purity water and encapsulated in a polyethylene vial for neutron activation. After the irradiation the  $^{197-203}\text{Hg}$  is radiochemically separated and measured by gamma-ray spectrometry.

Minczewski (1967), Mizuike (1965) and Joyner (1967) have described a number of group preconcentration procedures which should be adaptable for separating Hg from large volumes of water.

### Cadmium Procedures

Landstrom and Wenner (1965) have outlined a postirradiation radiochemical separation scheme for the measurement of Cd by neutron activation analysis, based on its anion exchange properties. However, the method is not generally applicable, because it lacks the sensitivity to measure environmental levels of Cd in only 10 to 30 ml of fresh waters. Therefore, it is necessary to preconcentrate the Cd from large volumes (0.2 to 2 liters) of water prior to the neutron activation step. Riley and Taylor (1968) and Goya and Lai (1967) have described a preconcentration method for Cd in seawater, based on adsorption onto Chelex-100 chelating resin, and this method should be adaptable to fresh waters. Other preconcentration schemes, as outlined by Minczewski (1967) and Mizuike (1965) should be rather easily adaptable for preconcentrating Cd from large volumes of fresh waters.

After the preconcentrates have been neutron activated, appropriate radiochemical separations as outlined by Devoe (1960) may be used to further isolate the radiocadmium.

### Silver Procedures

Kharkar, et al. (1968) have measured Ag in river waters by NAA. Their procedure is to irradiate freeze-dried river salts, having added 25 mg of spec-pure  $\text{Na}_2\text{CO}_3$  per 100 ml of river water to provide enough salts with which to conveniently work. Several weeks after the irradiation the salts are dissolved in 20 ml of 1  $\text{N}$  HCl and silver carrier is added. Silver chloride is filtered off, rinsed with 0.1  $\text{N}$  HCl, dissolved in a small volume of  $\text{HNO}_3$  and then converted to an  $\text{NH}_4\text{OH}$  solution. The solution is scavenged with  $\text{Fe}(\text{OH})_3$  to remove interferences and the Ag is then precipitated with  $\text{H}_2\text{S}$ . The precipitate is dissolved in concentrated  $\text{HNO}_3$ , diluted with water and the Ag is reprecipitated in dilute HCl. The  $\text{AgCl}$  precipitate is washed, dried and counted for  $^{110\text{m}}\text{Ag}$ .

The same thionalid preconcentration procedure (Lai and Weiss, 1962) described earlier for Hg can be used to isolate Ag from

large volumes of fresh waters, to achieve the desired sensitivity. Silver can then be directly measured by counting the neutron activated thionalid crystals (containing quantitatively cocrystallized Ag) on a Ge(Li) diode detector or dual coincidence gamma-ray spectrometer. If further radiochemical purity is required, the method of Kharkar et al. (1968), can be used.

#### Arsenic Procedures

Arsenic can usually be measured in river or lake waters by the semi-INAA method of Tanner et al. (1972). However, in some natural waters, particularly precipitation, the As must be preconcentrated from large volumes of water. The As procedures described in the following section for seawater analyses are completely applicable for the determination of As in fresh waters.

#### Selenium Procedure I

For some natural waters the Se must be preconcentrated to achieve the necessary sensitivity. Kharkar et al. (1968) have developed a separation scheme for measuring Se in fresh waters. One hundred ml of freeze-dried river salts (containing 25 mg of spec-pure  $\text{Na}_2\text{CO}_3$ /100 ml of river water as a salt carrier) is neutron activated. Several weeks after the irradiation the sample is dissolved in 1  $\text{N}$  HCl, carriers are added,  $\text{AgCl}$  is filtered off and the solution is made 6  $\text{N}$  in HCl and  $\text{H}_2\text{S}$  added to precipitate Mo and Se sulfides. The Mo sulfides are dissolved away in dilute  $\text{HNO}_3$  and the Se sulfide is purified by dissolving in concentrated  $\text{HNO}_3$ , precipitating Se with  $\text{SO}_2$ , distilling off Se as the bromide and finally reducing Se as the metal with  $\text{NaHSO}_3$ . The Se metal is filtered, washed, dried and counted to measure  $^{75}\text{Se}$  activity.

### Selenium Procedure II

Weiss, et al. (1971) have measured Se in melted samples of the Greenland ice sheet by NAA. One to 4 liter samples are acidified with  $\text{HNO}_3$  and evaporated to 1 ml and neutron activated. After the irradiation the Se is radiochemically separated by repeated precipitations of the Se metal carrier. The final Se precipitate is filtered and beta-counted in a gas-flow proportional counter periodically to measure  $^{81}\text{Se}$  and check its half-life. Yield is checked by counting carrier-free  $^{75}\text{Se}$  added at the start of the procedure. Selenium concentrations as low as 5 ng/kg were measured.

### Chromium Procedures

Chromium is frequently below the detection limit of INAA and must be preconcentrated from large volumes of fresh waters. The methods given for Cr measurements in the following section on seawater analysis are directly applicable for fresh waters.

### Suspended Particulate Matter

The pollutant elements of concern contained on separated suspended particulate matter can be analyzed by post-irradiation radiochemical separations once the material has been properly digested. Refluxing in  $\text{HNO}_3\text{-H}_2\text{SO}_4$  is normally a sufficient digestion procedure if siliceous materials are not present. If large amounts of silica are present, HF must be added to the acid digest to volatilize the silica. Care must be exercised to insure that volatile elements such as Hg are not lost during the digestion procedure.

## B. Seawater

### 1. Instrumental Analysis

Because of the tremendous amounts of  $^{24}\text{Na}$  (15.4 hrs) which are produced during neutron irradiation of seawater, INAA of seawater is applicable only for those elements in

seawater which have relatively long half-life activation products. From Figure 5, it is obvious that none of the short-lived neutron activation products of interest could possibly be measured instrumentally, since the  $^{24}\text{Na}$ ,  $^{38}\text{Cl}$ ,  $^{42}\text{K}$  and  $^{82}\text{Br}$  are orders of magnitude more abundant, and completely mask the gamma-ray contributions of the activation products of interest. For the INAA of sea salts, the neutron irradiated samples are stored for about one month to allow these relatively short-lived interfering radionuclides to decay to tolerable concentrations.

Figure 6 shows that after about 24 days out of the reactor, the  $^{24}\text{Na}$  has decayed to insignificant concentrations and  $^{82}\text{Br}$  (36 hrs) is now the major gamma-emitting interfering radionuclide. Bromine-82 decays with the emission of a large number of high energy gamma-rays which interfere with the measurement of the trace activation products of interest, and it is necessary to store the samples for another two weeks to allow the  $^{82}\text{Br}$  to decay to tolerable levels. High concentrations of  $^{35}\text{S}$  and  $^{32}\text{P}$  remain in the samples, but they are pure beta emitters and can be tolerated in the gamma-ray spectra. The  $^{35}\text{S}$  and  $^{32}\text{P}$  are both produced by fast neutron reactions with chlorine-35, as well as by the usual thermal neutron ( $n,\gamma$ ) reactions on sulfur and phosphorus. The month-old activated sea salts are then transferred into a constant counting geometry (small polyethylene vials or stainless steel planchets) and counted directly on a  $\text{Ge}(\text{Li})$  diode detector or a dual coincidence gamma-ray spectrometer (Robertson, et al. 1968). The activated sea salt samples are encased in a thin (0.7 mm thick) lead container to attenuate the low energy bremsstrahlung radiation from the beta-emitters  $^{35}\text{S}$  and  $^{32}\text{P}$ .

Figure 7 illustrates a typical gamma-ray spectrum of neutron activated sea salts counted on a  $\text{Ge}(\text{Li})$  detector 54 days after the neutron irradiation, and demonstrates the tremendous resolution advantages of  $\text{Ge}(\text{Li})$  detectors compared to  $\text{NaI}(\text{Tl})$  crystals. It is apparent that the eight trace elements, Sr, Rb, Cs, U, Sb, Zn, Fe, and Co, can be measured in sea salts

FIGURE 5

RELATIVE CONCENTRATIONS OF INDUCED RADIONUCLIDES IN SEA WATER FOLLOWING AN INTEGRATED THERMAL NEUTRON EXPOSURE OF  $5 \times 10^{16}$  n/cm<sup>2</sup>. EMPHASIZING THE PROBLEM OF MEASURING SHORT-LIVED ACTIVATION PRODUCTS IN THE PRESENCE OF HIGH CONCENTRATIONS OF SODIUM-24, BROMINE-82, POTASSIUM-42 AND SULFUR-35.

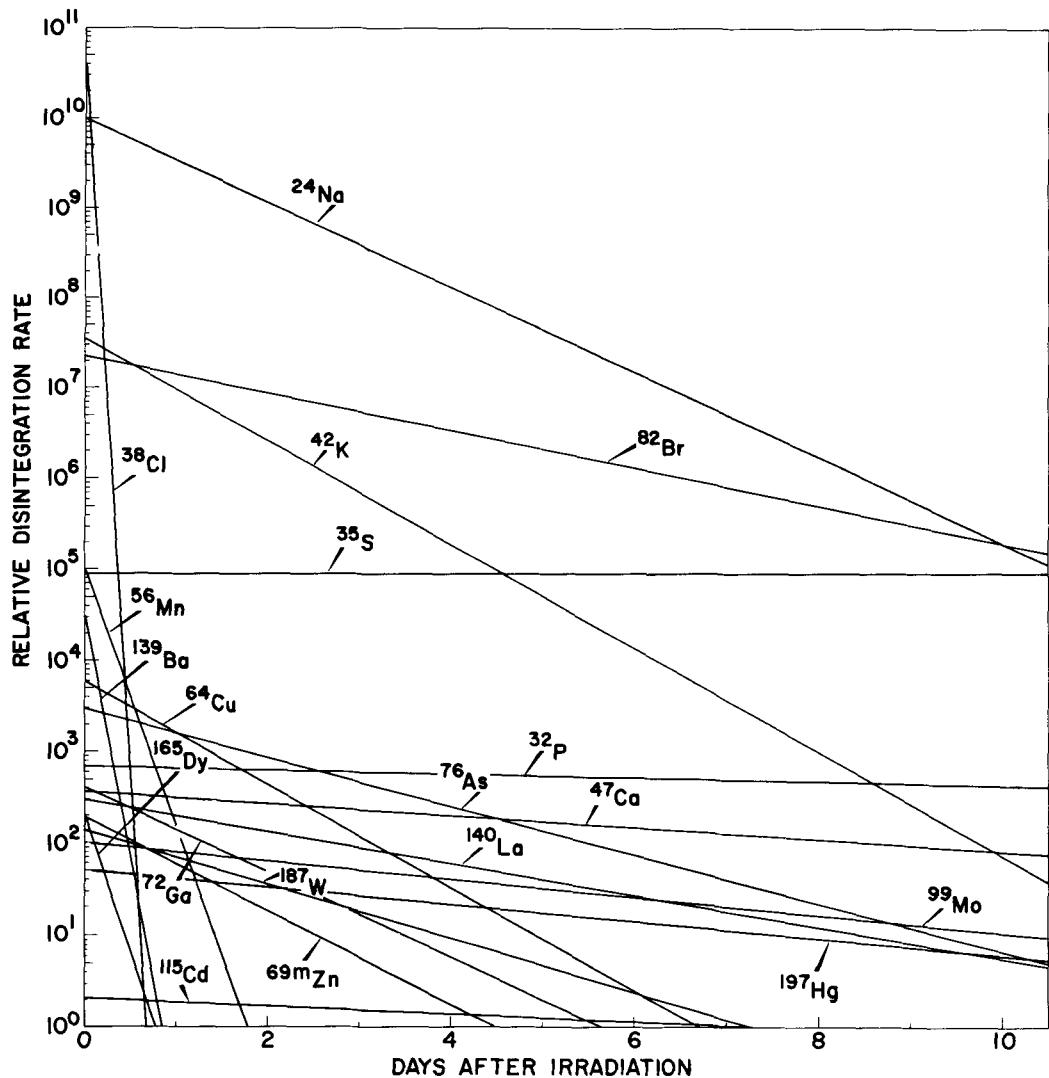


FIGURE 6

RELATIVE CONCENTRATIONS OF INDUCED RADIONUCLIDES IN SEA WATER  
FOLLOWING AN INTEGRATED THERMAL NEUTRON EXPOSURE OF  $10^{18}$  n/cm<sup>2</sup>.  
EMPHASIZING THE PROBLEM OF MEASURING INTERMEDIATE AND LONG-LIVED  
ACTIVATION PRODUCTS IN THE PRESENCE OF HIGH CONCENTRATIONS OF  
SODIUM-24, BROMINE-82, AND SULFUR-35.

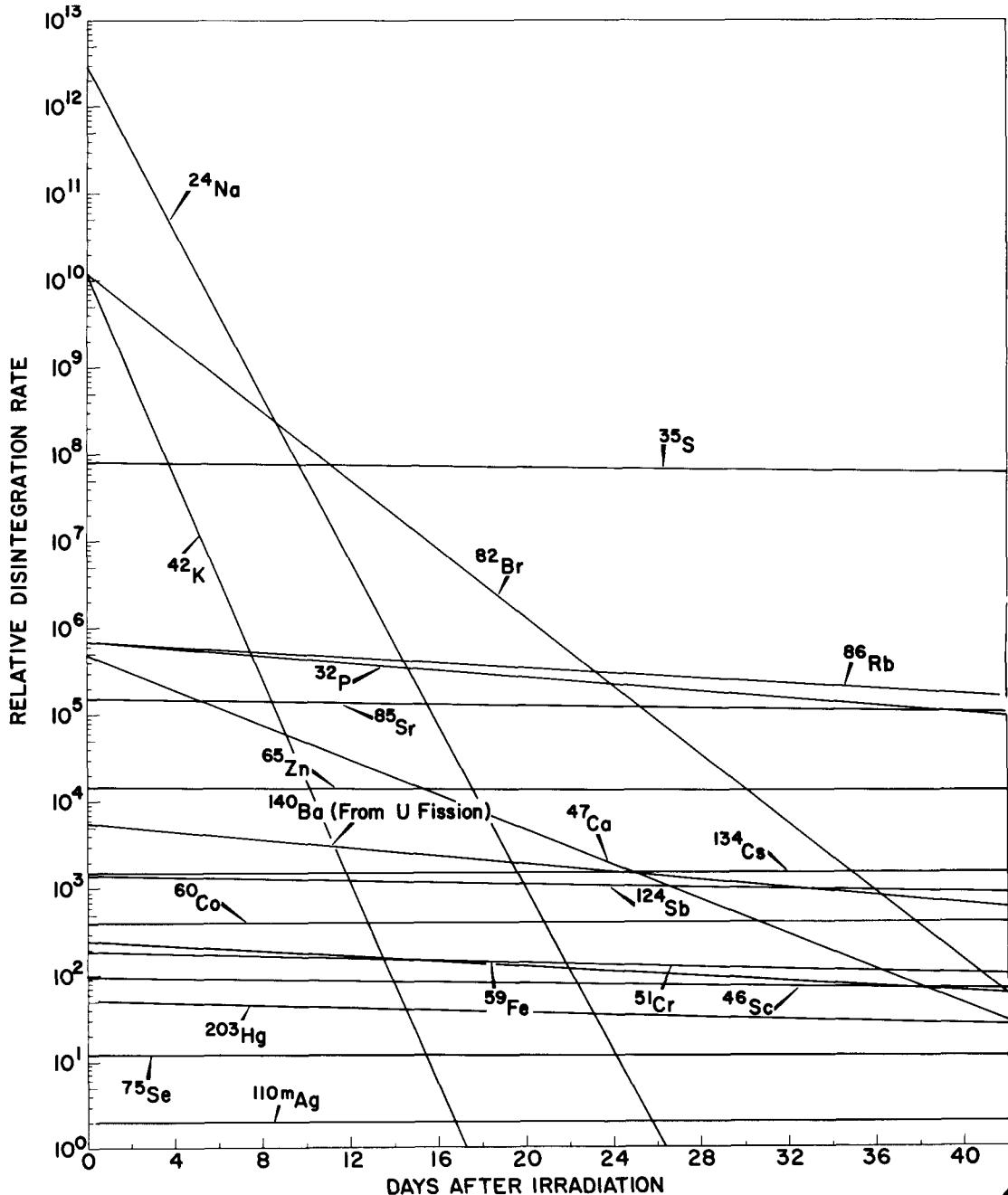
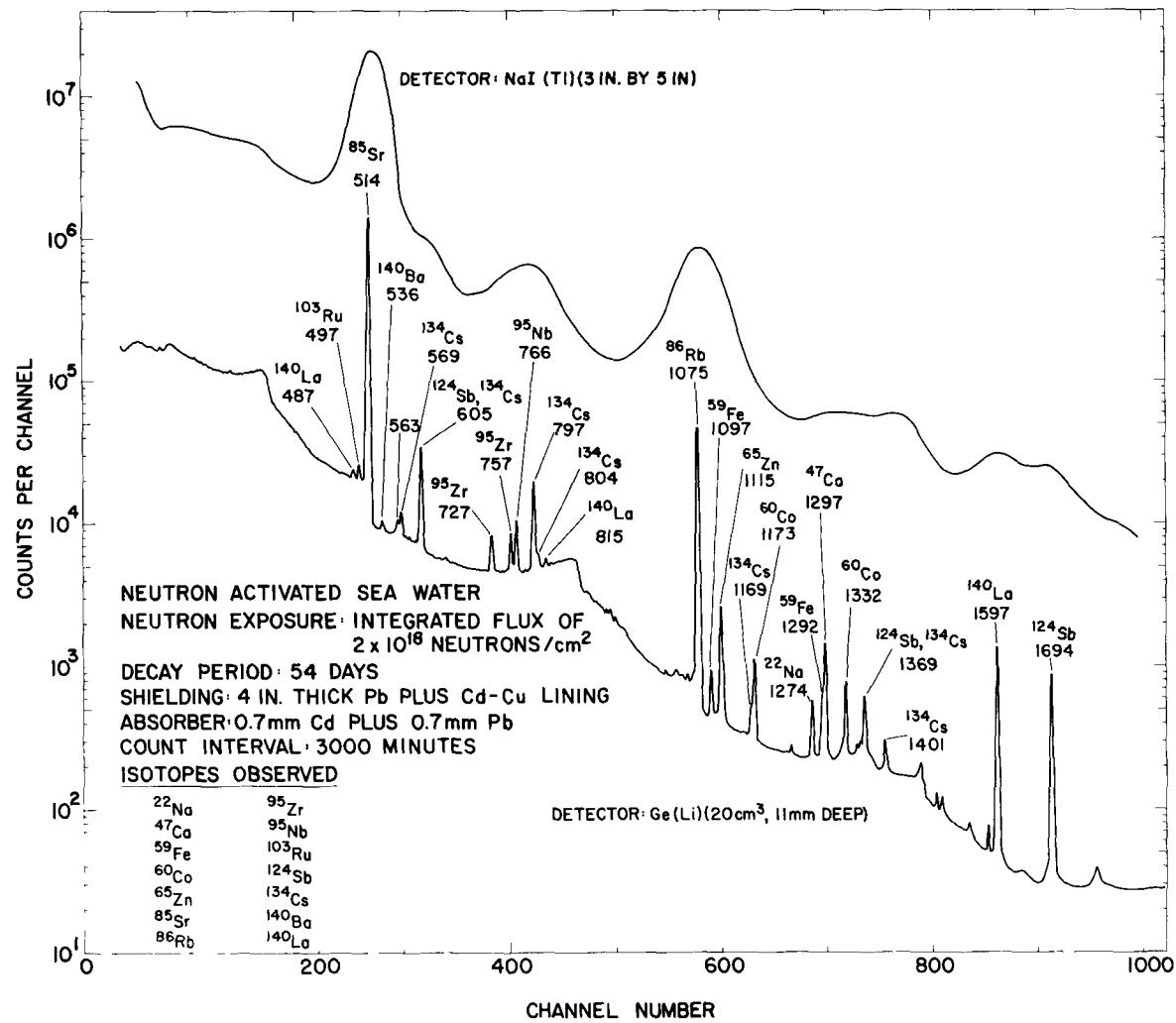


FIGURE 7 GAMMA-RAY SPECTRUM OF NEUTRON ACTIVATED SEA WATER



by this technique. Only Sb and Zn are of immediate interest from an environmental pollution concern. Table 2 lists the trace elements in seawater which can normally be measured by INAA and compares the detection limits of this method with the actual oceanic concentrations of the elements observed in seawater. It is obvious that this method is sufficiently sensitive to measure any of the naturally existing concentrations of these elements observed in the oceans.

Mercury can be measured instrumentally in neutron activated sea salts by Ge(Li) diode spectrometry if the samples are irradiated in an extremely thermalized neutron flux. If the neutron flux contains an appreciable fast neutron component, large amounts of the pure beta-emitter  $^{32}\text{P}$  are produced by a  $^{35}\text{Cl}$  ( $n, \alpha$ )  $^{32}\text{P}$  reaction. The bremsstrahlung activity from the  $^{32}\text{P}$  (maximum beta particle energy 1709 keV) interferes with the sensitive detection of the 279 keV gamma ray from the  $^{203}\text{Hg}$  neutron activation product. The Hanford plutonium production reactors, which have now all been closed down, were among the few nuclear reactors in the U.S. which possessed the necessary highly thermalized neutron fluxes. The Triga type of research nuclear reactors, now commonly available throughout the country, do not possess the required neutron flux characteristics necessary for the INAA of mercury in sea salts.

It is often desirable to remove suspended particulate matter from seawater for trace metal analyses, especially in coastal or estuarine waters containing relatively high concentrations of biological and geological material. The techniques for removal and analysis are essentially the same as those described earlier in the section dealing with fresh waters.

Of the pollutant elements of interest only Zn and Sb can be measured in seawater by INAA. However, if preconcentration or postirradiation separations are performed, neutron activation can be used to measure all of the pollutant elements of interest, except Pb and Sn. Table 3 lists the estimated minimum detectable concentrations of the potential pollutant trace

TABLE 2

ESTIMATED DETECTION LIMITS FOR INSTRUMENTAL NEUTRON  
ACTIVATION ANALYSIS OF SEAWATER\*

<u>Element</u>	<u>Typical Oceanic Concentration (<math>\mu\text{g/l}</math>)**</u>	<u>Detection Limit (<math>\mu\text{g/l}</math>)</u>
Sr	8000	100
Rb	120	5
Fe	1-20 (5)	1
Zn	0.5-10 (3)	0.2
U	3.3	0.1
Cs	0.3	0.003
Sb	0.2	0.005
Hg	0.01-0.3 ( ? )	0.05
Co	0.001-0.1 (0.03)	0.001
Sc	$1-20 \times 10^{-4}$	$1 \times 10^{-4}$

\* 25 ml seawater; 24-hour irradiation at  $10^{13} \text{ n/cm}^2/\text{sec}$ ;  
40 days decay; 1000 minute count on 20 cc Ge(Li) diode  
detector.

\*\* Estimated average concentrations in parentheses.

TABLE 3

## ESTIMATED MINIMUM DETECTABLE CONCENTRATIONS OF POLLUTANT ELEMENTS IN SEAWATER BY INAA AND BY NAA WITH SFPARATIONS

Trace Element	Typical Reported Concentrations in Open Ocean (Micrograms/Liter)	Minimum Detectable Concentrations (in micrograms/liter)	
		INAA*	NAA with Separations**
Hg	0.02-0.2	0.05	0.001
Cd	0.06-0.7	16,000	0.001
Ag	0.002-0.05	1.0	0.003
As	2-3	Not Possible	0.0001
Cu	0.5-2	Not Possible	0.002
Cr	0.02-0.6	0.3	0.003
Zn	0.5-10	0.2	0.01
Sn	0.02	Not Possible	9
Se	0.08	0.2	0.02
Sb	0.2	0.02	0.00003

\* 25 ml seawater; 24 hour irradiation at  $10^{13}$  n/cm<sup>2</sup>/sec; 40 days decay; 1000 minute count on 20 cc Ge(Li) diode detector; based on 3 $\alpha$  above Bkg-Compton contribution in peak areas.

\*\*500 ml seawater; elements chemically separated; 24 hour irradiation at  $10^{13}$  n/cm<sup>2</sup>/sec; 3 days decay; 500 minute count on a 20 cc Ge(Li) diode detector; based on twice Bkg contribution in peak areas.

metals of interest in seawater as measured by both INAA and NAA with separations from interfering radionuclides. These detection sensitivities are compared with the actual observed or estimated naturally occurring concentration ranges for these elements in the ocean. It is obvious that neutron activation with separations possesses the necessary sensitivity to measure all of these trace metals at naturally occurring concentrations well above their detection limits. Some of the necessary preconcentration and/or post-irradiation chemical separation schemes permitting the measurement of these elements in seawater are given below.

2. Analysis of Elements Requiring Preconcentration or Postirradiation Separations

Mercury

Before beginning any of the Hg analytical procedures summarized below, tests for both gain and loss of Hg during storage must be made.

A nearly quantitative loss of both  $Hg^{++}$  and methyl Hg chloride from unacidified seawater onto polyethylene storage containers has been observed. The  $Hg^{++}$  and methyl Hg chloride, added as  $^{203}Hg$  species, can be kept completely in solution for long storage periods by acidifying the seawater to a pH of about 1.5 with high purity HCl (Robertson, unpublished results, 1972, Battelle-Northwest).

On the other hand, leaching of stable Hg from the walls of acid washed polyethylene distilled water and seawater acidified to pH 1.5 and stored at room temperature has been observed by M. H. Bothner (unpublished results, 1972, Univ. of Washington). The Hg levels of acidified seawater stored in polyethylene at room temperature increased at a fairly steady rate over a 90-day period from .02 to .17 ppb. This effect was not observed on storage of the acidified seawater in hot acid washed pyrex flasks. It is not now known whether leaching of Hg by acidified water samples occurs with all kinds of plastic containers or even for all kinds of polyethylene containers.

Mercury Procedure I

The method of Ljunggren et al. (1971) described previously for analyses of Hg in fresh waters (see Mercury Procedure II in the section on fresh waters) has been utilized to measure Hg in seawater. The procedure is identical to that for fresh waters, except that the neutron irradiated seawater samples must be stored in appropriate shielding until the very intense gamma-radiation resulting from high concentrations of  $^{24}Na$  have decayed to workable levels. However, this procedure is not recommended, since the seawater irradiated in sealed quartz ampoules builds up tremendous pressures from radiolysis products and the quartz ampoules are dangerous to handle.

### Mercury Procedure II

Weiss and Crozier (1972) developed a preconcentration and post-irradiation separation for measuring Hg in seawater by neutron activation analysis. To 500 ml of filtered, acidified (10 ml conc. HCl per liter of seawater) seawater is added 1 mg of Cu (II) carrier and the Cu is precipitated as the sulfide by bubbling H<sub>2</sub>S gas into the seawater for 30 seconds. The Hg in the seawater is coprecipitated with the CuS. The precipitate is collected on a membrane filter, rinsed with high purity water and placed in a 15 ml polyethylene irradiation vial. The walls of the precipitation vessel were washed successively with 5, 3 and 2 ml volumes of nitric acid and the washings were added to the irradiation vial. Blanks were prepared in an identical manner, except high purity water was used instead of seawater. The samples, blanks and appropriate standards (10 ml in 15 ml irradiation vials) are neutron activated for 1 hour in a flux of  $2 \times 10^{12}$  n/cm<sup>2</sup>/sec. After irradiation, the sample or blank was quantitatively transferred to a vessel that contained 1 ml of mercury carrier. The solution was brought to pH 1-2 with ammonia liquor and 5 ml of tin (II) chloride were added. The precipitate which formed was isolated by centrifugation. The supernate was discarded and the precipitate was dissolved in 5-10 ml of aqua regia. Copper carrier (1 ml) was added to the solution, the pH was adjusted as above and precipitation with tin (II) was repeated. This precipitate was collected by filtration, and dissolved in aqua regia and the solution was adjusted to pH 7 with ammonia liquor. Hydrogen sulfide gas was bubbled through the solution for 30 seconds. The sulfide precipitate was collected on a Millipore filter and counted.

The irradiated standard Hg solutions after addition of 1 ml of mercury carrier, were precipitated directly as the sulfide and counted.

The 77 keV  $\gamma$ -ray of <sup>197</sup>Hg was measured one week after the irradiation with a NaI(Tl) detector coupled to a pulse-height analyzer.

After the radioactivity measurement the mercury sulfide was dissolved in 2 ml of aqua regia and diluted to 25 ml with water. Atomic absorption measurements of this solution were compared to that of a mercury standard to obtain the carrier yield. The yield is  $81.1 \pm 9.8\%$  for 12 determinations and the sensitivity is 4 nanograms of Hg.

#### Mercury Procedure III

Robertson (unpublished data, 1972) has developed a very simple neutron activation procedure for measuring mercury in seawater. Seawater samples (stored at pH  $\approx$  1.5 to prevent Hg losses by adsorption onto container walls) of 200 to 1000 ml are adjusted to pH 3.5 to 8 with  $\text{NH}_4\text{OH}$ , warmed to about 60°C and 20 ml of 1% thionalid is added. The seawater is chilled in a refrigerator and the thionalid crystals (containing co-crystallized Hg) are collected on a membrane filter, washed with high purity water and encapsulated in polyethylene irradiation vials. Blanks are prepared in an identical manner, using high purity water instead of seawater. The samples, blanks and standards are neutron activated for 5 to 8 hours in a flux of about  $5 \times 10^{12}$  n/cm<sup>2</sup>/sec. Several weeks after the irradiation the samples, blanks and standards are transferred into new vials and counted in the same geometries on a Ge(Li) diode detector to measure <sup>203</sup>Hg. The recovery of Hg is quantitative and the sensitivity is about 5 nanograms.

#### Silver Procedure I

The same thionalid co-crystallization procedure described above (Mercury Procedure III) is used for determining Ag in seawater, since Ag is also quantitatively co-crystallized from seawater by thionalid. Sensitivity can be improved by directly counting the samples on a dual coincidence gamma-ray spectrometer described in Section V - Gamma-Ray Spectrometry, to measure the multiple coincidence gamma-rays from <sup>110m</sup>Ag. The sensitivity is about 0.001  $\mu\text{g}$  Ag/l.

### Silver Procedure II

Schutz and Turekian (1965a) have determined Ag in seawater by neutron activating freeze-dried sea salts (from 75 ml of seawater) and appropriate Ag standards for two days in a flux greater than  $10^{13}$  n/cm<sup>2</sup>/sec. After several weeks the irradiated salts are dissolved in 25 ml of water and 1 ml of concentrated HCl, silver carrier is added and the solution heated to coagulate the AgCl precipitate. The precipitate is centrifuged, washed with dilute nitric acid and then treated with concentrated nitric acid, diluted with water and then dissolved in ammonium hydroxide. Interfering radionuclides in the solution were scavenged twice by precipitation with ferric hydroxide, and silver was finally precipitated by the addition of nitric acid. It was filtered, dried and weighed as silver chloride. The precipitate was re-dissolved and <sup>110m</sup>Ag measured by gamma-ray spectrometry.

### Arsenic Procedure I

Ray and Johnson (1972) report that serious losses (up to 70% in 1 week) occur by adsorption of As from seawater onto polyethylene storage containers, even when the seawater is acidified with 9 ml of concentrated HCl/1 of seawater. However, Robertson (1972 unpublished results), using radio-arsenate and arsenite tracers, observed no losses of arsenic from seawater onto polyethylene container surfaces in both acidified and unacidified samples. E. A. Crecelius (1972 unpublished results, U. of Washington) has studied the loss of arsenic added to seawater and lake water as As-74 arsenate. Tests were made with (1) the waters acidified to a pH of 1 with HCl and stored at 20°C, (2) acidified and frozen, (3) unacidified and frozen, and (4) stored unacidified at 20°C. Losses of only 1-14% were found with polyethylene and pyrex containers over a two month period. Portmann and Riley (1964) found polyethylene, pyrex and soda glass absorb 6, 6 and 16%, respectively of arsenate added to unacidified seawater.

Robertson (1972 unpublished work) has measured arsenic in seawater by an  $\text{Fe(OH)}_3$  coprecipitation technique. To 35 ml of seawater in 50 ml plastic conical centrifuge tubes is added 5 mg of Fe(III) carrier, followed by enough dilute  $\text{NH}_4\text{OH}$  to just precipitate the  $\text{Fe(OH)}_3$ . The precipitate is centrifuged and thoroughly washed twice with two 15 ml portions of high purity water to remove Na and then transferred into small polyethylene irradiation vials and slowly evaporated to dryness. Blanks are prepared in the same manner using 35 ml of high purity water instead of seawater. The samples, blanks and standards are neutron irradiated for 5 to 8 hours in a flux of about  $5 \times 10^{12} \text{ n/cm}^2/\text{sec}$ . Two or three days after the irradiation the  $\text{Fe(OH)}_3$  is dissolved in a 10:1 mixture of nitric-hydrochloric acids and transferred into new counting vials and counted for 10-20 minutes each on a Ge(Li) diode detector to directly measure the  $^{76}\text{As}$  559 keV gamma ray. Some  $^{82}\text{Br}$  (554 keV) and  $^{122}\text{Sb}$  (564 keV) are present, but are easily resolved by the Ge(Li) detector. The orchard leaf standards are counted on the Ge(Li) detector in an identical geometry to quantify the measurements. The yield is quantitative and the sensitivity is about 0.08  $\mu\text{g As/l}$ .

#### Arsenic Procedure II

Ray and Johnson (1972) have used the thionalide preconcentration technique to separate As from seawater prior to neutron activation. To 250 ml of seawater is added 1 ml of 5% ascorbic acid and the sample is heated just to boiling. The sample is allowed to cool for 10 minutes and 0.5 ml of ascorbic acid is added and cooled to room temperature. Ten ml of 5N  $\text{H}_2\text{SO}_4$  is added followed by 2 ml of 2% thionalid in acetone. Stir 5 minutes, wait 10 minutes and then bring solution just to boiling and boil very slowly for 30 minutes to remove acetone. Remove from heat and cool overnight. Collect thionalide crystals (containing co-crystallized As) on membrane filters, rinse with 300 ml of deionized water and encapsule wet filter in a polyethylene vial. Blanks are prepared by using deionized water instead of seawater. The samples, blanks and standards are neutron irradiated for

7 hours in a flux of  $10^{12}$  n/cm<sup>2</sup>/sec. One day after the ir-radiation count the samples, blanks and standards on a Ge(Li) diode detector to measure the <sup>76</sup>As 559 keV gamma ray.

Copper Procedure I

Slowey and Hood (1971) have modified the procedure of Rona et al. (1962), for determining inorganic and organic forms of Cu in seawater. The seawater samples are oxidized for 5 minutes by reflux digestion with 10 ml each of 0.1N potassium persulfate and 0.4N H<sub>2</sub>SO<sub>4</sub>. The solution is cooled, adjusted to a pH of 5.5 to 6.5 and 2 ml of 2% sodium diethyldithiocarbamate in water is added and mixed. Twenty ml of chloroform is added and the solution shaken for 15 minutes to extract Cu. Three extractions are made and the combined chloroform is scrubbed by shaking with high purity water. The Cu carbamate salt is back-extracted from chloroform with three 15 ml portions of 1N HNO<sub>3</sub>. The acid solution is evaporated to a small volume, transferred to 1.5 ml polyethylene vials, evaporated to dryness, heat-sealed and neutron activated with appropriate standards for one hour in a flux of  $2 \times 10^{12}$  n/cm<sup>2</sup>/sec. After neutron activation the copper is separated by sorption on Dowex 1x8 (200 mesh) anion exchange resin from 12N HCl. Interfering radionuclides are eluted from the resin with 12N HCl and then <sup>64</sup>Cu and <sup>56</sup>Mn are eluted together using 2.5N HCl. The <sup>64</sup>Cu and <sup>56</sup>Mn in the sample and standards are measured by gamma-ray spectrometry. The method is nearly quantitative and the sensitivity for Cu is  $3 \times 10^{-3}$   $\mu$ g.

No other method for measuring Cu in seawater by neutron activation analysis have been reported in the literature, but a convenient method for concentrating Cu from large volumes of seawater is by sorption on Chelex-100 chelating resin (Riley and Taylor, 1968). Copper is quantitatively retained on Chelex-100 at a pH of 7.6 and eluted with 2N HNO<sub>3</sub>, and it may be possible to measure Cu without any further separation. If further radiochemical purity is required, a postirradiation separation employing a suitable solvent extraction procedure could be readily accomplished.

#### Selenium Procedure

The only method for measuring Se in seawater by neutron activation analysis is that reported by Schutz and Turekian (1965b). Seventy-five ml samples of seawater are freeze-dried and the salts and appropriate standards are neutron activated for 2 days in a flux greater than  $10^{13}$  n/cm<sup>2</sup>/sec. Several weeks after the irradiation the salts are dissolved in dilute HCl and Se carrier is added. The Se is then precipitated as the sulfide by bubbling H<sub>2</sub>S gas into the solution. The precipitate is centrifuged, washed with distilled water, dissolved in HNO<sub>3</sub>, and reprecipitated as Se metal by bubbling SO<sub>2</sub> into the solution. The Se is oxidized to SeO<sub>2</sub> with HNO<sub>3</sub>, dried and weighed. The SeO<sub>2</sub> is then dissolved in water and brought to a standard volume for counting by gamma-ray spectrometry. Selenium concentrations of 0.05 to 0.1  $\mu$ g/l were reported using this method.

This method is rather tedious and time consuming and perhaps a more convenient method could be easily developed. Chau and Riley (1965) found that Se(IV) could be coprecipitated from seawater with Fe(OH)<sub>3</sub> with  $\approx$  95% efficiency in pH 4-6 range. Any other forms of Se present in seawater must be converted to Se(IV) for quantitative coprecipitation with the Fe(OH)<sub>3</sub>.

#### Chromium Procedures

Schutz and Turekian (1965b) have developed a method for measuring Cr in seawater by neutron activation analysis but the method is tedious and is not recommended as a routine method. Stanford (1971) reported a method based on extraction of Cr(VI) by methyl isobutyl ketone after acidification with hydrochloric acid. For analysis of total Cr, any Cr(III) present is oxidized to Cr(VI) with potassium peroxydisulfate and a copper (II) catalyst. Fukai (1967) and Chuecas and Riley (1966) reported that Cr(III) could be preconcentrated from seawater by coprecipitation with Fe(OH)<sub>3</sub>. Any Cr(VI) present must be reduced to Cr(III) with sodium sulfite in acid medium to measure total Cr concentrations. The methods

of Stanfield (1971) and Fukai (1967) have both used the diverse chemistries of Cr(III) and Cr(VI) in their methods as a measure of the chemical species naturally present in seawater. After Cr has been preconcentrated from a large volume (0.5 to 2 liters) of seawater, the concentrate can be neutron activated and post-irradiation separations as outlined by Pijck (1964) can be performed.

C. Marine Organisms

1. Instrumental Analysis

INAA is particularly applicable for the measurement of trace metals in marine organisms. Approximately 18 trace elements can be measured in most marine organisms by INAA, including the potential pollutant elements Hg, Ag, As, Sb, Zn, and Se (Haller et al., 1969; Morrison and Potter, 1972; Nadkarni and Ehman, 1970; Ranticelli et al., 1969; Robertson et al., 1972). Cadmium and chromium are present in most biological tissues at concentrations just under their INAA detection limits, but can be measured in some specimens when their concentrations exceed about 1 and 0.1  $\mu\text{g/gm}$  dry weight, respectively. The ultimate sensitivities achieved by INAA for many elements are limited by the high concentrations of the pure beta-emitter  $^{32}\text{P}$  (14 d) formed in the activated samples. The bremsstrahlung radiation produced from the  $^{32}\text{P}$  creates an instrumental dead time problem when biological samples receive too great a neutron exposure.

In practice, the INAA of biological tissue samples consists of encapsulating a 10-1000 mg sample of dried tissue (fresh to dry weight ratios should be obtained) in a cleaned plastic irradiation vial and neutron irradiating the sample and appropriate standards to an integral thermal neutron exposure of about  $10^{17} \text{ n/cm}^2/\text{sec}$ . Sampling and handling procedures for biological tissues during trace element analysis by NAA have been evaluated by Maletskos et al., (1970). After the irradiation, the samples and standards are transferred into standard counting geometries and counted on a Ge(Li) detector at optimum times following the irradiation to measure both short- and long-lived neutron activated products.

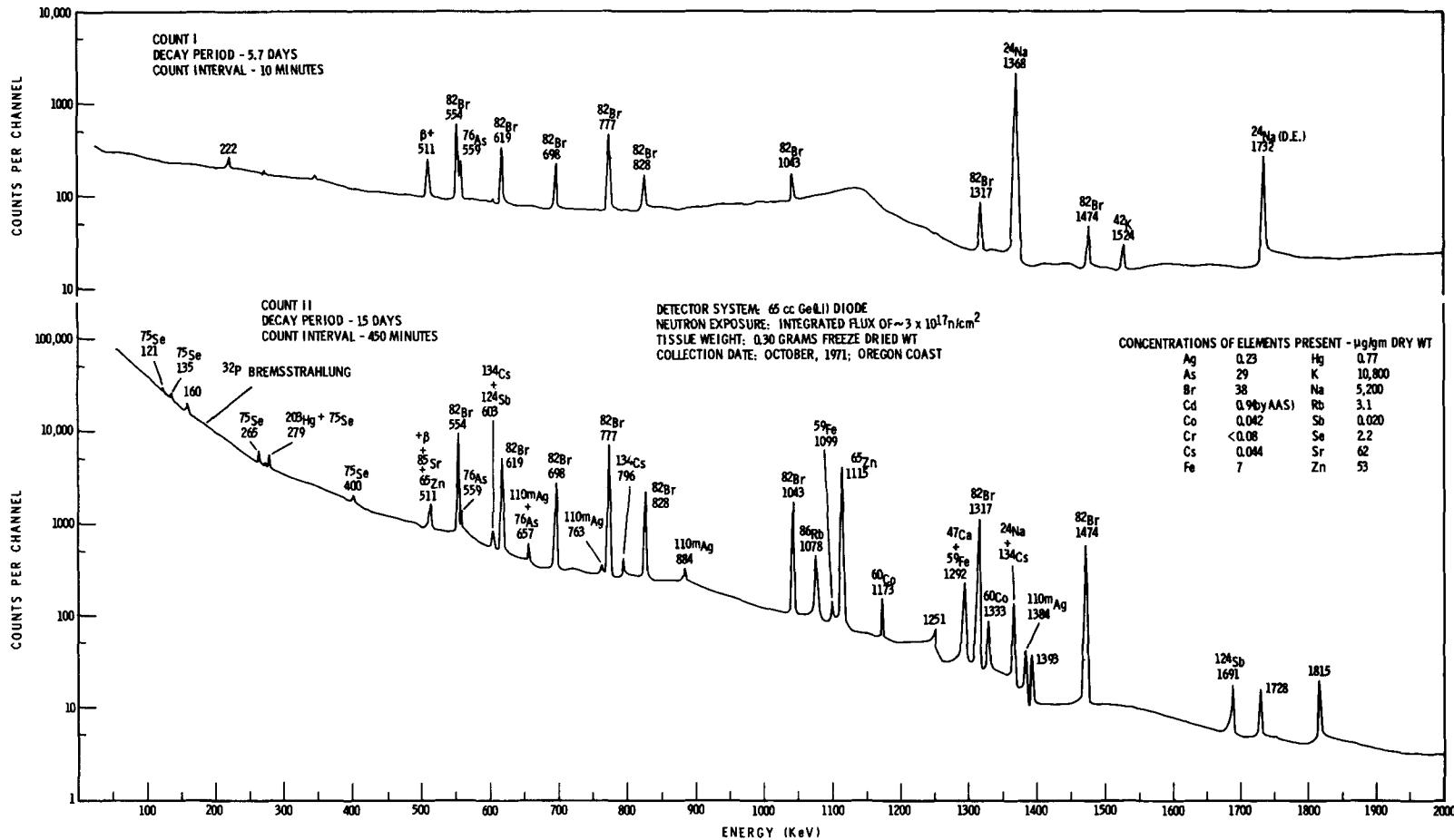
Muscle tissue from two different marine organisms were neutron irradiated and the resulting gamma-ray spectra are shown in Figures 8 and 9. The figures illustrate the neutron activation products which can be measured after both a short and long storage period between the irradiation and counting. The two gamma-ray spectra are similar in many respects, except that Ag, Sr and Sb can readily be detected in the shrimp muscle tissue but not in the sole muscle. The accumulation of Ag, Sr and Sb in these shrimp collected off the Oregon and Washington coast is believed to be due to natural processes and not the result of pollution.

As in the case for natural waters, two to four days out of the reactor the major neutron activation products are  $^{24}\text{Na}$  and  $^{82}\text{Br}$ , but normally high enough concentrations of  $^{42}\text{K}$  and  $^{76}\text{As}$  are present for accurate measurements. Some  $^{64}\text{Cu}$  is also present, but since some short-lived radionuclides in activated biological tissue are also positron emitters, it is safer to radiochemically separate the  $^{64}\text{Cu}$ . When cadmium concentrations in the biological tissue are over about 1  $\mu\text{g/gm}$  dry weight,  $^{115}\text{Cd}$  (54 hr) can be instrumentally measured from its 528 keV gamma-ray or the 325 keV gamma-ray from the  $^{115m}\text{In}$  isomer.

After the samples have been out of the reactor for about two weeks, most of the  $^{24}\text{Na}$  and much of the  $^{82}\text{Br}$  have decayed to tolerable levels and a new suite of long-lived neutron activation products can be instrumentally measured, including Rb, Cs, Fe, Zn, Ag, Co, Cr, Hg, Se, Sb and Sc. Table 4 lists the elements which can usually be measured by INAA, and compares their detection limits by this method with the typical concentration ranges normally found in marine biological tissue. As shown in Table 4 the INAA detection limits for most of these elements are sufficiently low to permit their measurement in almost any type of biological matrix. Cadmium, Ag, Cr, and Sb are frequently present in organisms at concentrations near or just below their INAA detection limits, and if necessary, postirradiation radiochemical separations of their neutron activation products can be performed by rather simple group chemical separations.

FIGURE 8

GAMMA-RAY SPECTRUM OF NEUTRON ACTIVATED PINK SHRIMP TAIL MUSCLE TISSUE (DOE 63)



**FIGURE 9 GAMMA-RAY SPECTRUM OF NEUTRON ACTIVATED LEMON SOLE MUSCLE TISSUE (IDOE 59)**

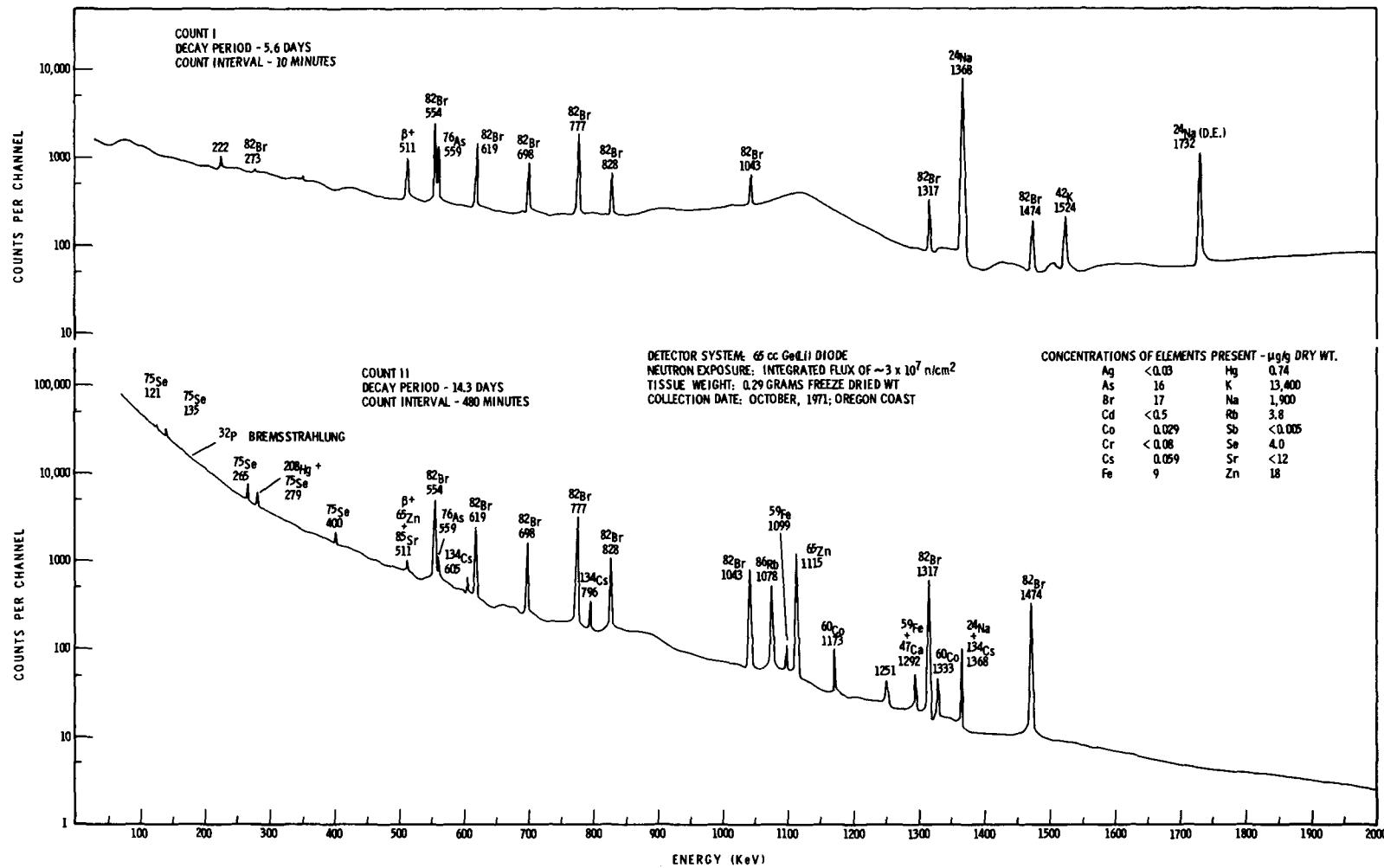


TABLE 4

ESTIMATED DETECTION LIMITS OF INSTRUMENTAL  
NEUTRON ACTIVATION ANALYSIS OF BIOLOGICAL  
MATERIAL ( $\mu\text{g/g}$  DRY TISSUE)

ELEMENT	*TYPICAL CONCENTRATION IN MARINE ORGANISMS	DETECTION LIMIT **
Na	500-1500	0.05
K	1000-30,000	20
Rb	0.5-8	1
Cs	0.02-0.4	0.0001 <sup>†</sup>
Fe	1-500	2
Zn	10-200	0.5
Br	10-200	0.05
As	<1-50	0.3
Cd	<0.1-10	1
Ag	<0.001-5	0.001 <sup>†</sup>
Co	0.001-0.5	0.001 <sup>†</sup>
Cr	<0.02-1	0.02
Hg	0.05-5	0.02
Se	0.5-50	0.05
Sb	0.0001-0.05	0.0001 <sup>†</sup>
Sc	0.00001-0.002	0.00005 <sup>†</sup>

\* Typical Ranges in various tissues, with the low ranges usually associated with muscle and high ranges with liver.

\*\* 300 mg of freeze-dried tissue; integral thermal neutron exposure of  $3 \times 10^{17}$  n/cm<sup>2</sup>; 20 min. and 1000 min. counts after decay periods of 3 to 5 days and 20 to 30 days, respectively; 20 to 40 cc Ge(Li) diode detection, except where specified.

† Detection limits obtained by dual, coincidence NaI(Tl) spectrometry. Detection limits obtained by Ge(Li) spectrometry for these five elements are approximately 5- to 10-fold higher.

2. Analysis of Elements Requiring Preconcentration or Post-irradiation Separations

The only elements which cannot routinely be measured in marine biological tissue by INAA are Cd, Cr, Cu and frequently Ag and Sb. However, these elements can be easily measured by neutron activation followed by radiochemical separations.

Cadmium Procedures

Many procedures for determining Cd in biological tissue have been reported: Lieberman and Kramer (1970); Das and de Vries (1971); Livingston et al. (1967); Ljunggren et al. (1971); Edgington and Lucas (1970, 1971); Samsahl et al. (1968); Morrison and Potter (1972).

Biological tissues have been prepared for neutron irradiation in various ways. Some investigators irradiated fresh material, others freeze-dried or ashed samples. Sample decomposition after irradiation has generally been by digestion with  $H_2SO_4$ , with  $K_2S_2O_8$  or  $HNO_3$  added to speed dissolution. The samples ashed prior to irradiation are dissolved in concentrated HCl.

One Cd separation scheme uses the extraction with dithizone/chloroform solution as the key separation step (Lieberman and Kramer, 1970). Following  $H_2SO_4/K_2S_2O_8$  digestion of the samples, the pH is adjusted to 1 to 2 and gross radioactive interferences are extracted into the dithizone layer while Cd stays in the aqueous layer. Subsequently, the pH is adjusted to 13 to 14 and Cd is then extracted into the dithizone layer. Yields are determined by adding  $^{109}Cd$  spike to each acid digestion sample.

Four other separation schemes (Morrison and Potter, 1972; Livingston et al., 1967; Edgington and Lucas, 1970, 1971; Samsahl et al., 1968) use anion exchange as the key to Cd separation. Cadmium forms very strong chloro-complexes which persist until very low HCl concentrations and the complexes are firmly bound to Dowex 1 or 2, or Amberlite IRA-400 anion exchange resins in the Cl form. The great majority of other

elements and radioactivities can be eluted from the columns before the Cd. The Cd activity has been measured both on the resin directly and following elution and subsequent precipitation. Samsahl et al. (1968) and Bowen (1967) use several precipitation and dissolution steps involving CdS to remove Cd from interfering isotopes.

#### Copper Procedure I

Although the 511 keV positron annihilation radiation from  $^{64}\text{Cu}$  can usually be detected instrumentally in neutron activation biological tissue, other short-lived radionuclides, including  $^{24}\text{Na}$ , are present which also contribute to positron annihilation radiation. Therefore, the  $^{64}\text{Cu}$  should be radiochemically separated from interfering radionuclides, principally  $^{24}\text{Na}$ . Four general separation schemes have been devised for Cu based on the insolubility of its sulfide, the solvent extractability of its organic chelates, its cation exchange behavior and its electrodeposition properties.

A rapid and simple method developed by Hahn et al. (1968), consists of digesting the biological tissue in nitric acid, dissolving the residue in 4 ml of 1N nitric acid, transferring the solution to a separatory funnel containing 2 ml of 0.1N HCl and extracting  $^{64}\text{Cu}$  with 5 ml of 0.01% dithizone in  $\text{CCl}_4$  shaking for 2 minutes. The  $\text{CCl}_4$  is transferred to a polyethylene counting vial and  $^{64}\text{Cu}$  determined by gamma-ray spectrometry.

#### Copper Procedure II

Bird et al. (1969), have measured Cu in biological tissue by NAA after removal of  $^{24}\text{Na}$  by retention on hydrated antimony pentoxide (HAP). The neutron activated tissue sample is digested in fuming  $\text{HNO}_3\text{-H}_2\text{SO}_4$ , 0.1 mg of Cu carrier is added and the solution is cooled and passed through a 0.5 gm column of HAP that had been preconditioned with 9N  $\text{HNO}_3$ . The column

effluent (and 9N HNO<sub>3</sub> rinses) is collected and counted to measure <sup>64</sup>Cu. Some <sup>54</sup>Mn is present, but can be manually stripped from the gamma-ray spectra.

Copper Procedure III

Merlini et al. (1971), measure Cu in biological tissue by NAA after separating <sup>64</sup>Cu on columns of cuprous chloride or cupric sulfide. The irradiated tissue sample is digested in hot HNO<sub>3</sub>-HF and brought to 6M HClO<sub>4</sub> or 1M H<sub>2</sub>SO<sub>4</sub> and the solution is passed through a 7 mm diameter by 30 mm long column of cuprous chloride or cupric sulfide. The column is rinsed with 15 ml of the same acid and counted directly by gamma-ray spectrometry. Copper is retained on the column, while most interfering radionuclides are eluted. Only Au, Hg, Sb, Se, Br, W and I are wholly or partially retained on the column, but do not prevent the measurement of the 511 keV <sup>64</sup>Cu activity. The counting geometry problems discussed earlier must be considered when counting unusually shaped samples such as these columns.

Copper Procedure IV

Battistone and Lofberg (1970) quantitatively determine Cu in biological tissues by NAA utilizing an electrodeposition technique. The activated samples are digested in HNO<sub>3</sub> in the presence of Cu carrier, and the digest taken just to dryness. The residue is dissolved in 1 ml of concentrated HNO<sub>3</sub> and rinsed into a 25 ml polyethylene beaker with 3 to 5 ml portions of the electroplating solution (10% sodium tartarate, 1% hydrazine hydrochloride, 1% sodium hydroxide, and 0.5% urea, prepared weekly). The Cu is electrodeposited onto cylindrical platinum electrodes at 2.5 volts and an initial current of 2.75 amperes for 45 minutes while stirring with a Teflon-coated magnetic stirring bar. The electrodes are removed from solution while the current is still on and rinsed with 95% ethyl alcohol. The electrodes are then directly counted by gamma-ray spectrometry, or the <sup>64</sup>Cu is dissolved off with several ml of 8N HNO<sub>3</sub> which is then counted.

### Silver Procedures

Silver can be conveniently separated from neutron activated biological tissue by the method of Merlini et al. (1971). After digestion of the irradiated tissue in hot  $\text{HNO}_3$ -HF, the solution is brought to  $6\text{M}$   $\text{HClO}_4$  and passed through a 7 mm diameter by 30 mm long column of cuprous chloride. Over 99% of the Ag is retained on the column, while interfering radio-nuclides are eluted with 15 ml of the same acid. The column can then be counted directly to measure the  $^{110m}\text{Ag}$  by gamma-ray spectrometry. Only Cu, Au, Se, W, I, Hg, Pa, Sn and Sb are wholly or partially retained on the column, but do not interfere with the measurement of  $^{110m}\text{Ag}$ . Bromine-82, which is also retained on the column, would normally interfere, but the digestion step with  $\text{HNO}_3$  should volatilize most of the Br.

Once the biological tissue sample has been adequately digested by acid dissolution, the same Ag separations as used for sea-water can also be employed here.

### Antimony Procedure

Antimony can be determined by INAA in most biological tissue by measuring the 1696 keV gamma-ray from  $^{124}\text{Sb}$ . The sensitivity for measuring Sb is not limited due to interferences from other neutron activation products, but is limited by the dead-time to the Ge(Li) gamma-ray spectrometer caused by the high levels of the  $^{32}\text{P}$  bremsstrahlung. Greater sensitivity could be achieved for Sb if a larger tissue sample were neutron irradiated for a longer time, and the activated tissue sample was encased in some material, such as 0.7 mm thick Pb, to attenuate the  $^{32}\text{P}$  bremsstrahlung.

An alternative approach would be to use the rapid separation scheme of Merlini et al. (1970), whereby  $^{122-124}\text{Sb}$  is retained on a cuprous chloride column from  $1\text{M}$   $\text{H}_2\text{SO}_4$ .

### Chromium Procedures

After digestion of neutron activated biological tissue samples,

Cr may be separated by the solvent extraction procedure described in the section on seawater analysis.

Bowen (1964) has also developed a separation scheme for measuring Cr in biological tissue by NAA. After digestion of the irradiated samples in  $\text{HNO}_3\text{-H}_2\text{SO}_4$  with Cr carrier added, and subsequently with  $\text{HClO}_4$ , the Cr is distilled into 10 ml of water from a  $\text{HClO}_4\text{-H}_2\text{SO}_4$  solution to which 0.1 gm of NaCl had been added. The distillation with 2 more portions of NaCl is repeated. The distillate is next transferred to a separatory funnel and shaken with an equal volume of methyl isobutyl ketone, a few drops of concentrated HCl being added to improve the extraction. The aqueous layer is discarded and the organic layer is washed with 2N HCl, and then shaken twice with 10 ml portions of  $\text{NH}_4\text{Cl}$ . The  $\text{NH}_4\text{Cl}$  solutions are scavenged with  $\text{Fe(OH)}_3$ . The supernate is made acidic with acetic acid and treated with 2 ml of barium acetate. The  $\text{BaCrO}_4$  precipitate is coagulated by boiling, centrifuged and dissolved in hot HCl and reprecipitated at pH 6. The precipitate is washed twice with water, dried, weighed to determine yield and  $^{51}\text{Cr}$  is measured by gamma-ray spectrometry.

The above procedure is quite lengthy and Meloni et al. (1969), have measured Cr in biological tissue by removing interfering  $^{24}\text{Na}$  and  $^{32}\text{P}$ . Tissue samples are digested in  $\text{HNO}_3$  and the solution made 1N in  $\text{HNO}_3$  and passed through a column of hydrated antimony pentoxide (HAP) to retain  $^{24}\text{Na}$ . The effluent (and 1N  $\text{HNO}_3$  rinse) is evaporated to about 10 ml and is then passed through a small column of  $\text{SnO}_2$  which retains the  $^{32}\text{P}$ . The effluent (and 1N  $\text{HNO}_3$  rinse) is treated with  $\text{KClO}_3$  to oxidize Cr(VI) and the solution is passed through a small column of  $\text{MnO}_2$  to retain Cr. The  $\text{MnO}_2$  is filtered, dried and counted directly to measure  $^{51}\text{Cr}$  by gamma-ray spectrometry.

#### D. Sediments

##### 1. Instrumental Analysis

Of the various analytical methods available for the determination of elemental concentrations in geological

materials, neutron activation analysis is one of the most sensitive, selective and reliable. INAA, utilizing high resolution Ge(Li) detectors, permits the nondestructive measurement of 25 to 30 elemental constituents in geological material, including marine and fresh water sediments (Brunfelt and Steinnes, 1966; Gordon et al., 1968; Robertson et al., 1972). However, most of the elements which can be measured in sediments by INAA are mainly of a geochemical interest and, of the pollutant metals of concern, only As, Sb and Cr can be measured routinely. Mercury and zinc can be measured instrumentally utilizing a special coincidence-anticoincidence Ge(Li) detector system (Cooper and Perkins, 1972). Utilizing Ge(Li) diode spectrometry and group radiochemical separations, the pollutant elements of interest can be measured (Filby et al., 1970; Morrison et al., 1969; Peterson et al., 1969).

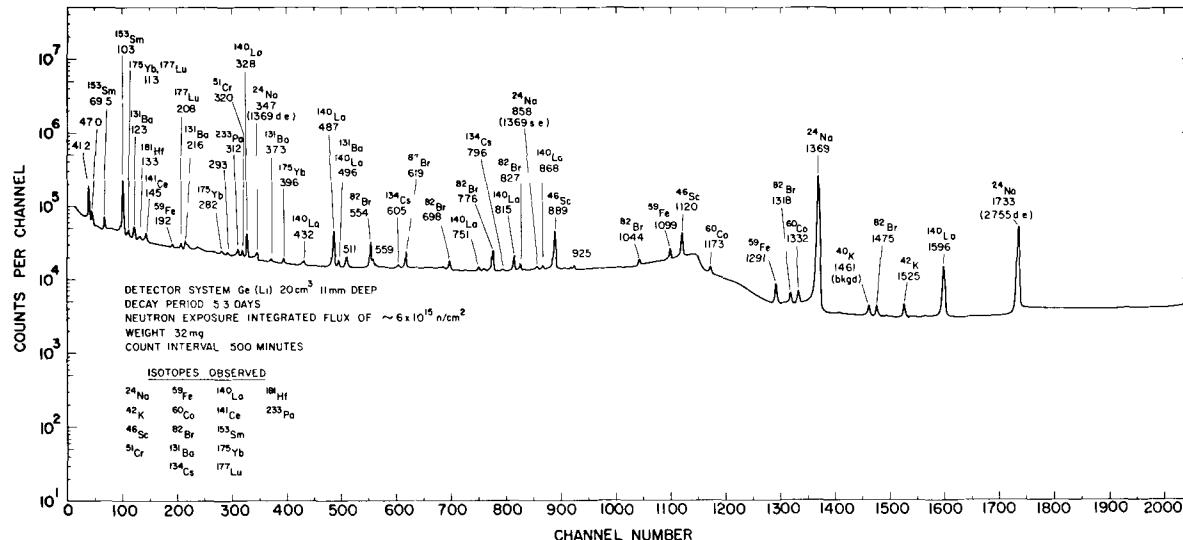
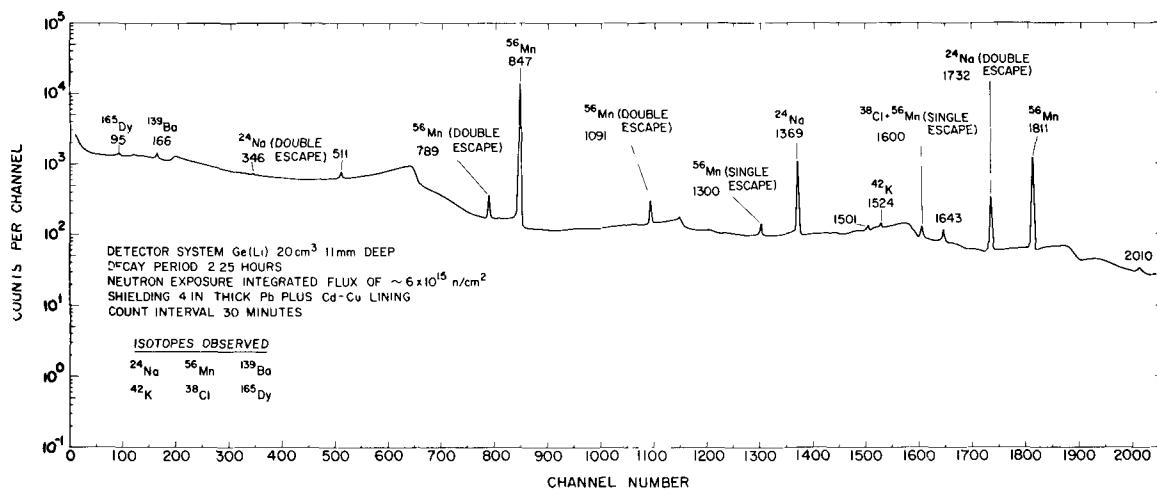
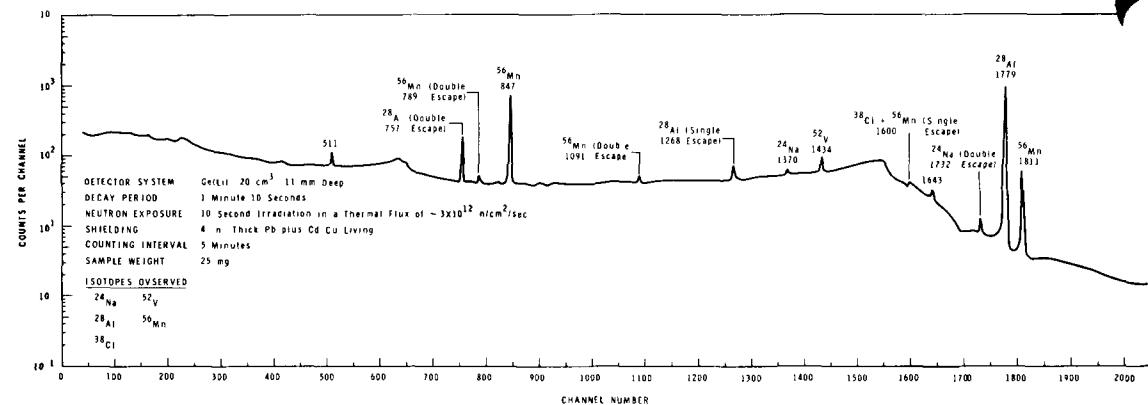
The general scheme for measuring trace elements in sediments by INAA is to weigh dried sediments (10 to 1000 mg) into cleaned plastic irradiation vials, heat-seal the vials and neutron irradiate the samples, together with appropriate elemental standards encapsulated in an identical geometry as the samples. Elements with short-lived neutron activation products (2.8 minutes to several hours) are measured by irradiating the samples in a rabbit facility for several minutes at a thermal neutron flux of 1 to  $50 \times 10^{12}$  n/cm<sup>2</sup>/sec, and then counting the samples directly on a Ge(Li) diode detector for 5 to 10 minutes. Elemental standards are identically activated and counted to quantify the concentrations of the elements of interest in the samples. To determine those elements with intermediate half-life activation products (several hours to several days), the identical sediment sample used for the rabbit irradiation, or an aliquot of the same sample, plus elemental standards are neutron irradiated to an integral thermal neutron exposure of about  $10^{17}$  n/cm<sup>2</sup>/sec. The samples and standards are then transferred into new counting containers and counted for 10 to 30 minutes on a Ge(Li) diode detector after 2 to 24 hours decay time. For the measurement of elements

with long-lived activation products (several days to several years) the samples and standards are recounted 5 to 15 days later for 60 to 200 minutes.

Some typical gamma-ray spectra of neutron activated deep sea red clay obtained by this method are shown in Figure 10. From the rabbit irradiations and short count it is possible to measure the elements Al, V, Mn, Na and Cl. Two to three hours after the long irradiation, the short-lived  $^{28}\text{Al}$  and  $^{52}\text{V}$  have decayed, and it is possible to measure Na, K, Mn, Cl, Ba and Dy. After a decay period of about 5 days the elements which can be measured are Na, K, Sc, Cr, Fe, Co, As, Br, Ba, Cs, La, Ce, Sm, Yb, Lu, Hf and Th (from  $^{233}\text{Pa}$ ).

The detection sensitivities for these elements which are obtained by INAA are dependent upon the composition of the sediment sample. For example, evaporite types of sediments having high Na and Br concentrations would create interferences which would limit the detection sensitivities of a number of elements. On the other hand, very pure calcareous sediments would permit more sensitive trace element analysis because of lower concentrations of interfering elements.

In Table 5, estimates of the detection limits for measuring trace elements in typical coastal or deep sea marine sediments by INAA are shown. The detection limits for most of the elements shown in Table 5 were derived by extrapolating from the INAA detection limits for these elements in the standard geological reference materials W-1 and G-2 (Gordon et al., 1968). These standard rocks have trace element concentration ranges somewhat similar to those observed in marine sediments, and this comparison should give detection limits for marine sediments that are reliable to within an order of magnitude. As indicated in Table 5, INAA possesses the necessary sensitivity to instrumentally measure all of the pollutant elements of interest, except Ag, Cd, Cu, Se, Zn and frequently, Hg. The following section outlines methods for the postirradiation separations of the activation products



## Gamma-Ray Spectra of Neutron Irradiated Deep Sea Red Clay Sediment after One Minute, Two Hours and Five Days Decay.

TABLE 5

ESTIMATED DETECTION LIMITS FOR THE INAA OF TRACE ELEMENTS  
IN MARINE SEDIMENTS

<u>Element</u>	<u>Typical Concentration Ranges in Marine Sediments (<math>\mu\text{g/gm}</math>)</u>	<u>INAA Sensitivity* (<math>\mu\text{g/gm}</math>)</u>
Ag	0.01-0.5	0.1
Al	10,000-90,000	10
As	2-20	1
Ba	60-8100	100
Ce	40-70	5
Cd	205	10
Co	1-200	0.1
Cr	10-200	0.8
Cs	0.3-15	0.2
Cu	10-700	1
Dy	0.2	0.1
Eu	0.2-10	0.05
Fe	20,000-60,000	200
Hf	0.1-18	0.1
Hg	0.05-3	0.5
K	3000-30,000	5000
La	2-60	2
Lu	0.2	0.1
Mn	100-10,000	10
Na	2000-40,000	100
Pb	10-200	---
Rb	1-100	1**
Sb	0.5-15	0.1
Sc	0.2-30	0.02
Se	0.1-1	2
Sm	0.5-30	0.3
Sn	0.5-15	500
Sr	200-2000	100**
Ta	0.03-3	0.01
Tb	0.1-7	0.1
Th	0.3-10	0.08
V	10-500	10
Yb	1	0.3
Zn	5-4000	5**
Zr	100-400	70

\* 100 to 800 mg of dried sediment; samples irradiated at optimum intervals ranging from 1 minute to 6 hours in a flux of  $10^{11}$  to  $10^{13}$  n/cm<sup>2</sup>/sec and counted at optimum intervals after the irradiation for 1 minute to 200 minutes; Ge(Li) diode detectors, 20 cc to 60 cc volumes.

\*\* Determined by counting on a coincidence-anticoincidence shielded Ge(Li) gamma-ray spectrometer (Cooper and Perkins, 1972).

of these elements to give the desired sensitivity for their measurements.

2. Analysis of Elements Requiring Preconcentration or Postirradiation Separations

The only elements of pollution concern which cannot normally be measured in sediments at natural background levels by INAA include Ag, Cd, Cu, Se, Zn and sometimes Hg. Radiochemical separation schemes are given below for these elements.

Mercury Procedures

One important point regardless of the type of post irradiation treatment is that Hg may be lost in drying the sediments--either by freeze drying or oven drying. Preferably, the Hg analysis should be made on the wet sediment, and the wet-dry weight ratio determined on a separate aliquot of the sediment.

Bothner (1972 unpublished data, Univ. of Washington) has used nondestructive INAA to determine Hg levels of 0.5-17 ppm in Bellingham Bay sediments, based on measuring the 279 keV  $^{203}\text{Hg}$  gamma ray. However, most sediments will have Hg levels at or below the lower limit of detection by the INAA  $^{203}\text{Hg}$  method. Increased sensitivity is achieved by counting the 77 keV gamma rays from  $^{197}\text{Hg}$ . However, because of the relatively high concentrations of  $^{24}\text{Na}$  activity produced by irradiating marine sediments, the  $^{197}\text{Hg}$  must be chemically separated following the irradiation.

The main problem of postirradiation separations of Hg from neutron activated sediments is the dissolution of the samples. Ehmann and Lovering (1967) and Landstrom et al. (1969), point out that fusion of the sediments with  $\text{Na}_2\text{O}_2$  or  $\text{Na}_2\text{CO}_3$  is to be avoided, because of loss of Hg before equilibration with the carrier is achieved. The most recommended digestion procedure is with  $\text{HNO}_3\text{-H}_2\text{SO}_4$  in a Bethge apparatus or similar glass container, closed to prevent loss of Hg. However, this method will not dissolve siliceous sediments and digestion

with HF must be performed in addition to acid leaching, care being taken not to volatilize Hg during the process.

Following the digestion there is no uniform procedure for the Hg separation but Ehmann and Lovering (1967), Kennedy et al. (1971), and Landstrom et al. (1969), use a key ion exchange step, taking advantage of the strong affinity of Dowex 1 or 2 for chloro-complexes of Hg to bind Hg to the resin while eluting most interfering isotopes. The Hg was subsequently eluted and precipitated as HgS for counting (Ehmann and Lovering, 1967), or electroplated onto Au or Pt foils Pillay et al. (1971), Landstrom et al. (1969), and Ljunggren et al. (1970).

An extremely useful summary of extraction, coprecipitation and ion exchange behavior of Hg in different acid solutions is in the monograph by Roesmer (1970).

#### Cadmium Separations

Once the sediment sample is dissolved by appropriate acid digestion, the general Cd separation procedures outlined in the section of analysis of marine organisms can be used.

Landstrom et al. (1969), Morrison et al. (1969), and Peterson et al. (1969), have outlined sequential group chemical separation schemes for measuring a large number of elements, including Cd, in geological material by NAA. The methods of Morrison et al. (1969), and Peterson et al. (1969), are the same. After dissolution of the sample by  $H_2SO_4$ -HF digestions the residue is dissolved in  $8N$  HCl and passed through a column of hydrated antimony pent-oxide (HAP) to remove  $^{24}Na$ . The effluent (and  $8N$  HCl rinse) is passed through a column of Dowex 1x8 anion exchange resin which is then washed with  $8N$  HCl. The resin, containing sorbed radiocadmium and radionuclides of Zn, Sb, Re, Ir and Au is counted directly on a Ge(Li) detector to measure  $^{115}Cd$  (54 hr).

Landstrom et al. (1969), treats the irradiated sediment with  $H_2SO_4$ -HBr to distill off the volatile bromides of Hg, Sn, Sb, As and Se. The distillate is then digested with HF to complete the dissolution of siliceous material, and the residue is finally dissolved in HCl and passed through two columns of Dowex 2 anion exchange resins in the sulfate and hydroxide forms under various acid concentrations. The Cd is subsequently sorbed onto a column of Dowex 2 anion exchange resin in the sulfate form for counting of the  $^{115}Cd$  by gamma-ray spectrometry.

Copper and Silver Separations

After dissolution of the neutron irradiated sediment by appropriate acid digestion, the Cu and Ag can be separated by the procedures outlined in the sections on analyses of marine organisms and natural waters.

Selenium Separations

Case et al. (1969), described a group chemical separation method for determining Se in geological material by NAA. The irradiated sample is dissolved by  $H_2SO_4$ -HF digestion in the presence of carriers and made 6N in HCl and 18N in  $H_2SO_4$ . The Se is distilled by dropwise addition of HBr at 220°C, using  $N_2$  carrier gas. The distilled Se is precipitated as the sulfide, which is then dissolved in  $HCl-HNO_3$  and counted by gamma-ray spectrometry to measure  $^{75}Se$ . If further radiochemical purity is required, the procedures given in the sections on natural water analyses could be employed.

Akaiwa (1966), Greenland (1967), and Kiesl (1969) have reported radiochemical separation schemes for measuring Se in meteorites and Wiersma and Lee (1971), and Chau and Riley (1965) described nonradioactive determinations of Se in fresh water and marine sediments. Any of the procedures could be readily adapted to the measurement of Se in marine sediments by NAA.

### Zinc Procedures

Zinc cannot routinely be measured in marine sediments, because the 1115 keV gamma-ray from its neutron activation product  $^{65}\text{Zn}$  is swamped by the high concentrations of  $^{46}\text{Sc}$ , which has a major gamma ray of 1120 keV. Therefore, to measure  $^{65}\text{Zn}$  in neutron activated sediments, it needs only to be separated from  $^{46}\text{Sc}$ . The sequential separation schemes developed by Morrison et al. (1969), and Peterson et al. (1969), can be used for this purpose. The procedure is exactly the same as that described earlier for measuring Cd in sediments.

### E. Atmospheric Particulates

#### 1. Instrumental Analysis

The contamination of the oceans and aquatic environments by deposition of anthropogenic aerosols from polluted atmospheres is of real concern (see Duce et al. 1972, and Robertson et al. 1972, for examples). It is essential that the atmospheric concentrations of metal pollutants be monitored especially at coastal locations so that estimates of the air-to-sea transfer rates of these aerosols can be made. For the multielement analysis of atmospheric particulates collected on filter media, no analytical method can match INAA for sensitivity and selectivity. Up to 33 elements can be measured in urban air by this nondestructive technique of selecting optimum neutron irradiation and counting times for the detection and measurement of neutron activation products ranging in half-life from 2.3 minutes to 5.3 years (Dams et al. 1970; Dudey et al. 1969; Gordon, 1971; Pillay and Thomas, 1970; Tuttle, 1971; Zoller and Gordon, 1970).

The collection and preparation of aerosol samples for INAA, especially at remote areas, requires extreme care to avoid contaminating the sample during handling and pumping operations. Duce et al. (1973) have given practical recommendations for avoiding contamination errors in air sampling.

Atmospheric particulate material is collected by pumping

air through suitable filter media or cascade impactors. Air volumes are measured with recording or integrating flow meters. The most popular filter media presently in use are Whatman 41<sup>®</sup> cellulose filters and Millipore<sup>®</sup> membrane filters. Dams et al. (1972) have published an excellent paper evaluating 10 various filter media for suitability for atmospheric particulate sampling and elemental analysis by nondestructive INAA using Ge(Li) gamma-ray spectrometry. They conclude that, of the commercially available filters tested, Whatman 41<sup>®</sup> cellulose is optimum from the standpoint of low blanks, particle retentivity and ease of handling. However, these filters do have a tendency to clog during prolonged sampling of dirty air. Stafford and Ettinger (1971) have carefully evaluated the retention efficiencies of Whatman 41<sup>®</sup> filters as a function of particle size and air sampling velocity, and these data are very useful for quantifying air sampling with these filters. A relatively new filter media, Nuclepore<sup>®</sup> filters (General Electric Co.) have recently been evaluated for aerosol collection, and their properties have been compared with a variety of other membrane and cellulose filters (Spurny et al., 1969a,b). Yoshida and Ikegawa (1968) have published a very useful survey of the characteristics of many types of filter media for air sampling. It is difficult to recommend any one particular filter as a panacea for all air sampling applications. The investigator should be aware of the characteristics of all of the products that are commercially available, and then determine which filter media best suits his needs.

In air pollution studies it is critical that as much as possible be known about the past history of the air he has sampled. Duce et al. (1973) recommend that sampling pumps be controlled by wind direction. Equipment for this purpose is commercially available (e.g. Weathermeasure Corp.), and permits one set of pumps to operate when the wind is blowing from any predetermined sector of the compass, and another

set to operate when the wind is outside the sector. This type of control is extremely important in attempting to determine atmospheric fluxes of pollutants into or out of a particular area, or for controlling local contamination of the samples. Permanent records of wind speed and direction at sampling sites should be maintained, as well as other important meteorological parameters, such as precipitation and synoptic conditions.

Cascade impactors (Andersen, 1965; Cohen and Montan, 1966; Lundgren, 1967, as examples) consist of several funnel-like chambers of decreasing orifice size through which a stream of air is pumped. Particles of various size ranges are collected on baffles downstream from each orifice. Typically, particles of radius  $\geq 8\mu$  are collected in the first stage, with smaller particles down to about  $0.2\mu$  collected on succeeding stages. Particles collected on high-purity polyethylene or mylar sheets placed on the baffles of the various stages can be analyzed by INAA. Lundgren impactors are preferred by some investigators because of their higher flow rates, continuous incremental sample collection and ease of sample handling. After collection, the filters or impaction media are weighed to determine the total weight of solids collected, and the filter or a known fraction of it is removed and encapsulated in a polyethylene irradiation vial. The filters and appropriate standards are then neutron activated for predetermined time intervals for the measurement of short- and long-lived neutron activation products.

A typical procedure for the analysis of elements forming short-lived activation products is to irradiate the filter sample for 5 minutes in a rabbit facility at a thermal neutron flux of about  $1$  to  $50 \times 10^{12}$  n/cm<sup>2</sup>/sec, wait 2 to 10 minutes after the irradiation and then count the sample for 5 to 10 minutes on a Ge(Li) diode detector. For the measurement of elements forming intermediate and long-lived radionuclides, the same sample or a portion of the same filter, plus elemental standards, are irradiated for 5 to

8 hours at a thermal neutron flux of 1 to  $50 \times 10^{12}$  n/cm<sup>2</sup>/sec, and the samples and standards are transferred to new counting containers and counted on a Ge(Li) detector for 10 to 30 minutes when 1 to 3 days out of the reactor, and for 60 to 200 minutes when 10 to 30 days out of the reactor.

Gamma-ray spectra obtained under these conditions are shown in Figure 11. In this aerosol sample, collected in a highly industrialized region of Cleveland, Ohio, one can measure from the short count the trace elements Al, Mg, Ti, V, Cu, Cl, Mn and Br. Calcium and S can often be observed by extending the energy scale to above 3 MeV to observe high-energy  $\gamma$  rays of 8.8-min <sup>49</sup>Ca and 5.1-min <sup>37</sup>S. From the intermediate count, it is possible to measure Na, K, As, Br, Sb, Sm, La, U and Cd; and from the long count the elements Cu, Fe, Co, Zn, Se, Sb, Cr, Sc, Cs, Ce, Eu, Yb, Hf, Hg, and Th. Concentrations of these elements in the filter are given in Table 6. The elemental composition of the aerosol collected on this filter is very atypical of normal urban atmospheres, and resulted from point source releases to the atmosphere from specific industries. For example, the Sb concentration is very high, and evidently originates from either a lead works or a zinc smelter. The As, Br, Co, Se and Ti concentrations are also higher than in normal urban atmospheres, and are probably released from the same sources or from nearby industries. The Na and Cl concentrations are also unusually high, and may result from the salting of Cleveland streets in the winter for snow and ice removal. The unusual nature of this sample emphasizes the fact that many sampling locations are needed to adequately characterize the trace element distributions in an urban atmosphere.

Detection sensitivities achieved by INAA for multielement analysis of aerosols are not fixed values, but are strongly dependent upon the composition of the sample. For example, the relatively high concentration of Sb on the Cleveland air filter shown in Figure 11 limited the sensitivity for elements forming neutron activation products with gamma-rays of lower energy than the main <sup>124</sup>Sb gamma rays at 603 and 1691 keV. Compton scattering from these and other <sup>124</sup>Sb gamma rays produced a relatively high "background" for the measurement of gamma rays of lower energies.

FIGURE 11 GAMMA-RAY SPECTRUM OF NEUTRON ACTIVATED CLEVELAND AIR FILTER

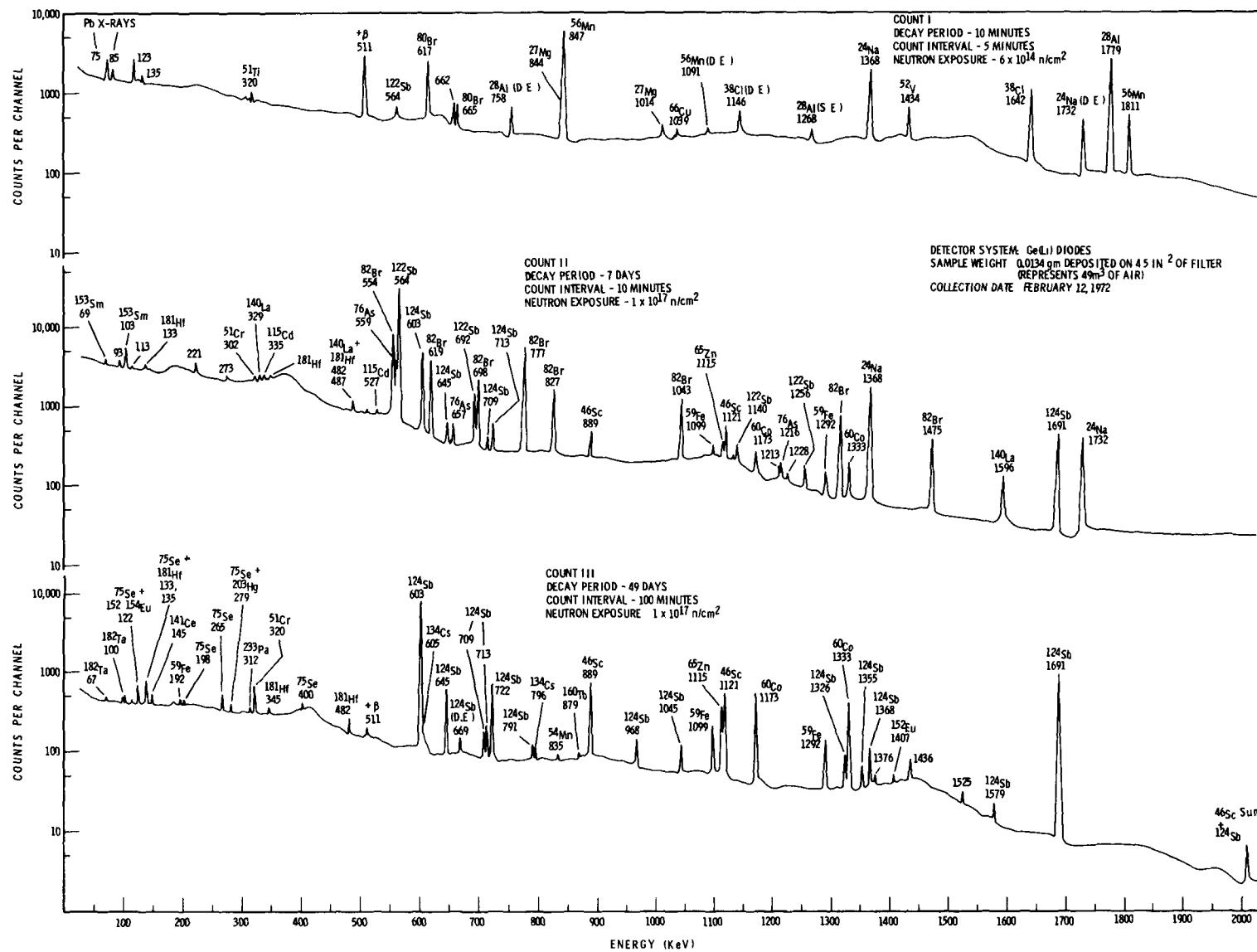


TABLE 6

ELEMENTAL CONCENTRATIONS IN CLEVELAND AIR FILTERS  
FEBRUARY 12, 1972

<u>Element</u>	<u>ng/m<sup>3</sup> Air</u>	<u>µg/gm Particulate</u>
Al	9890	36600
As	316	1170
Br	513	1900
Cl	9450	34900
Co	32.7	121
Cr	96	360
Eu	0.075	0.28
Fe	9750	36100
Hf	1.61	5.96
Hg	<0.03	<0.1
La	7.2	27
Mg	2190	8100
Mn	277	1030
Na	10460	38700
Sb	503	1860
Sc	2.41	8.92
Se	27.4	101
Sm	1.20	4.44
Ta	0.66	2.4
Tb	0.21	0.78
Th	0.72	2.7
Ti	630	2330
V	17.8	65.9
Zn	1040	3850

TABLE 7

SENSITIVITIES FOR DETERMINATION OF TRACE ELEMENTS IN AEROSOLS BY INAA  
(After Dams, et al, 1970)

Element	Decay Time Before Counting	Detection Limit * ( $\mu\text{g}$ )	Minimum Detectable Concentration in Urban Air ( $\mu\text{g}/\text{m}^3$ ), 24-Hr. Sample †
Al	3 minutes	0.04	0.008
S	"	25.0	5.0
Ca	"	1.0	0.2
Ti	"	0.2	0.04
V	"	0.001	0.002
Cu	"	0.1	0.02
Na	15 minutes	0.2	0.04
Mg	"	3.0	0.6
Cl	"	0.5	0.1
Mn	"	0.003	0.0006
Br	"	0.02	0.004
In	"	0.0002	0.00004
I	"	0.1	0.02
K	20-30 hours	0.075	0.0075
Cu	"	0.05	0.005
Zn	"	0.2	0.02
Br	"	0.025	0.0025
As	"	0.04	0.004
Ga	"	0.01	0.001
Sb	"	0.03	0.003
La	"	0.002	0.0002
Sm	"	0.00005	0.000005
Eu	"	0.0001	0.00001
W	"	0.005	0.0005
Au	"	0.001	0.0001
Sc	20-30 days	0.003	0.000004
Cr	"	0.02	0.00025
Fe	"	1.5	0.02
Co	"	0.002	0.000025
Ni	"	1.5	0.02
Zn	"	0.1	0.001
Se	"	0.01	0.0001
Ag	"	0.1	0.001
Sb	"	0.08	0.001
Ce	"	0.02	0.00025
Hg	"	0.01	0.0001
Th	"	0.003	0.00004

\* Air filters (20 x 25 cm, or aliquots thereof) neutron irradiated at optimum intervals ranging from 5 min. to 5 hrs at fluxes of  $2\text{-}15 \times 10^{12} \text{ n/cm}^2/\text{sec}$ ; Counted for 7 to 70 min at optimum intervals ranging from 3 min to 20-30 days after the irradiation; 30 cc Ge(Li) detector.

† 24 hr sampling period at air sampling rate of 12 liters/min-cm<sup>2</sup>.

Nevertheless, estimates of the detection sensitivities of the elements which can be measured by INAA are useful and give one a feel for the excellent sensitivities obtainable by this technique.

An estimation of the minimum detectable concentrations of 37 elements in urban aerosols as measured by INAA is given in Table 7. The sensitivities obtainable ( $\mu\text{g}/\text{m}^3$ ) in nonurban aerosols are even greater than those shown in Table 7, since larger volumes of cleaner air (lesser interelement interferences) can be sampled. The INAA method possesses the sensitivity to directly measure all of the metal pollutants of interest, except Cd, Pb, and sometimes Ag, Cu and Hg. Cadmium, Cu, Ag and Hg can be measured by neutron activation followed by radiochemical separation of their activation products. Although Pb and Sn are insensitive to neutron activation analysis, it is worth pointing out that another nuclear method of analysis, X-ray fluorescence spectrometry, is a convenient method for the nondestructive analysis of Pb in aerosol samples.

## 2. Analysis of Elements Requiring Preconcentration or Postirradiation Separations

Occasionally, Cd, Cu, Ag and Hg are present on air filters in concentrations too low to measure by INAA and radiochemical separations of the neutron activated filters are necessary to separate interfering radionuclides. Since the particulate material collected on aerosol filters resembles geological and/or biological matrices, the radiochemical separations for these elements from irradiated air filters is the same as those described for geological and/or biological materials. The filters and collected aerosols are dissolved by digestion in  $\text{H}_2\text{SO}_4$ -HF or  $\text{HNO}_3$ -HF. Once the samples are in solution, the previously described radiochemical separations for the elements of interest may be performed.

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APPENDIX

PERTINENT NUCLEAR DATA RELATING TO THE MEASUREMENT OF THE TRACE METALS  
IN MARINE ENVIRONMENT SAMPLES BY INSTRUMENTAL AND NON INSTRUMENTAL NEUTRON ACTIVATION ANALYSIS

ELEMENT	TARGET ISOTOPE	ISOTOPIC ABUNDANCE %	PRODUCT NUCLIDE	HALF LIFE	THERMAL NEUTRON CROSS SECTION (BARNs)	BEST GAMMA RAY FOR MEASUREMENT (keV)	NUMBER OF GAMMA DECAYS PER 1000	ASSOCIATED GAMMA RAYS KeV (y/s/1000 DISINTEG)	POSSIBLE INTERFERING RADIONUCLIDES IN MEASUREMENT OF BEST GAMMA RAY (keV)	
									KeV	KeV
Ag	Ag 107	51.82	Ag 108	2.4m	35	633	17	4334(5) 6141(2,6)	Nd 151(629) Nd 149(630)	Dy 165(633)
	Ag 109	48.18	Ag 110m	260d	3	657	960	764(230) 884(710) 937(317) 1384(211)	As 76(657)	
Al	Al 27	100	Al 28	2.3m	0.23	1779	1000	NONE	Nd 151(1776)	
As	As 75	100	As 76	26.5h	4.5	559	440	657(66)	Br 82(554) Nd 149(556) Eu 152m(562)	
Au	Au 197	100	Au 198	64.8h	98.8	411	960	676(9.7) 1088(1.94)	Eu 152(416) Ir 192(417)	
Ba	Ba 130	0.101	Ba 131	11.6d	8.8	496	450	124(200) 216(190) 373(130)	Cd 115(492) Ru 103(497)	
	Ba 138	71.66	Ba 139	83m	0.4	166	217		Th 235(163) Gd 161(165) Th 233(169)	
Br	Br 81	49.463	Br 82	35.5h	3.3	777	257	554(226) 619(133) 1044(90) 1317(87)	Mg 27(170) Nd 151(171)	
Ce	Ce 140	88.48	Ce 141	32.5d	0.6	145	490	NONE	W 187(774) As 76(775) Eu 152(779)	
Cd	Cd 114	28.86	Cd 115	2.3d	1.1	335	528	492(101) 528(264)	Ce 143(142) Fe 59(142) Yb 175(145)	
	Cd 116	7.58	Cd 117	2.5h	1.4	273	310	314(160) 345(180) 434(13)	Eu 154(146) Eu 152(148) Te 131m(150)	
Cl	Cl 37	24.471	Cl 38	37.3m	0.56	1642	380	2167(470)	Br 82(273)	
Co	Co 59	100	Co 60	5.24y	19	1332	1000	1173(1000)	NONE	
Cr	Cr 50	4.31	Cr 51	27.8d	17	320	90	NONE	Ir 192(317) Lu 177(319) Nd 147(319)	
Cs	Cs 133	100	Cs 134	2.1y	28	795	880	475(150) 569(140) 605(980)	Os 193(321) Lu 177(321)	
Cu	Cu 63	69.09	Cu 64	12.9h	4.5	511	380	345(5)	Te 131m(794)	
	Cu 65	30.91	Cu 66	5.1m	2.3	1039	93	833(2.5)	ALL POSITRON EMITTERS	
Dy	Dy 164	28.18	Dy 165	2.55h	700	362	10	2805(7) 633(16.4)	Nd 151(1042) Ce 70(1042)	
Eu	Eu 151	47.82	Eu 152m	9.3h	2800	841	111	122(50) 344(17) 963(93)	Th 233(60) Gd 159(64)	
	Eu 151	47.82	Eu 152	12.4y	5900	122	294	344(285) 964(162) 1112(154) 1408(229)	Mg 56(846) Te 129(845)	
Fe	Fe 58	0.33	Fe 59	45d	1.2	1099	560	192(25) 1292(440)	Yb 169(118) Se 75(121) Eu 154(123)	
Hf	Hf 180	35.24	Hf 181	43d	10	482	810	133(400) 136(60) 346(130)	Ba 131(124)	
Hg	Hg 196	0.146	Hg 197	65h	880	77	180	191(10)	Tb 160(1103)	
	Hg 202	29.80	Hg 203	46.6d	4	279	820	NONE	Re 188(478) Eu 154(478) W 187(480)	
In	In 113	4.28	In 114m	50d	8	190	161	558(35) 725(35)	Ir 192(485) Re 188(485) La 140(487)	
	In 115	95.72	In 116m	54m	154	1097	530	417(60) 1293(800)	Pt 197(77)	
K	K 41	6.88	K-42	12.4h	1.1	1525	180	313(1.8)	Hg 197m(279) Se 75(280) Os 193(280)	
La	La 139	99.911	La 140	40.2h	8.9	1596	970	328(210) 487(460) 816(240)	Yb 175(283) Ir 192(283)	
Lu	Lu 176	2.59	Lu 177	6.7d	2100	208	75	113(44)	Eu 154(188) Te 131m(189) Hg 197(191)	
Mg	Mg 26	11.17	Mg 27	9.5m	0.03	1014	300	844(700)	Pt 197(191) Fe 59(192)	
Mn	Mn 55	100	Mn 56	2.58h	13.3	847	990	1810(297) 2113(151)	Ga 72(156)	
Na	Na 23	100	Na 24	15.0h	0.13	1369	1000	2754(1000)	W 187(206) Lu 177m(208)	
Ni	Ni 58	67.88	Ni 58	71d	0.1	810	990	511(80)	Mo 101(102) Nd 151(1016)	
	Ni 64	1.08	Ni 65	2.56h	1.5	1482	836	366(150) 115(547)	Mg 27(844)	
Rb	Rb 85	72.15	Rb 86	18.7d	0.9	1076	88	NONE	Ca 47(807) Eu 152(810)	
Sb	Sb 121	57.25	Sb 122	2.8d	6	564	663	692(33)	Ge 77(1478) Mo 93m(1479) Nd 151(1486)	
	Sb 129	42.75	Sb 124	60.2d	3.3	1691	500	602(980) 723(98) 2091(69)	As 76(559) Cs 134(563) Eu 152(566)	
Sc	Sc-45	100	Sc-46	83.8d	13	889	1000	1121(1000)	NONE	
Se	Se 74	0.87	Se 75	120d	30	264	630	1211(76) 136(604) 279(264) 400(126)	Ag 110m(884) Ir 192(884)	
Sm	Sm 152	26.72	Sm 153	47h	210	103	270	NONE	Yb 169(261) Cd 115(263) Ta 182(264)	
Sr	Sr 84	0.56	Sr 85	65d	0.8	514	100	NONE	La 140(267)	
Ta	Ta 181	99.988	Ta 182	115d	21	1221	278	100(142) 1121(350) 1189(166)	Ta 182(100)	
								1230(117)	As 76(510) La 140(511) W 187(512)	
Tb	Tb 159	100	Tb 160	72d	46	879	318	298(263) 962(126) 966(276) 1178(166)	Ir 194(1219) As 76(1220)	
Th <sup>(a)</sup>	Th 232	100	Th 233	22.1m	7.4	312	440	NONE	W 187(879) Ce 143(880)	
Tl	Tl 50	5.34	Tl 51	5.80m	0.14	320	955	608(151) 928(45)	Nd 147(319) Ir 192(317) Cr 51G20	
U <sup>(b)</sup>	U 238	99.27	U 239	23.5m	2.7	228	120	106(230) 209(40) 278(140)	Pt 199(317) Ru 105(317) Ta 182m(318)	
			Np 239	2.35d					Nd 151(320)	
V	V 51	99.76	V 52	3.77m	4.9	1434	990	NONE	Gd 159(226) Lu 177m(228) Ta 182(229)	
Yb	Yb 176	12.73	Yb 177	1.9h	7	150	168	121(17) 1079(47) 1120(10) 1241(26)	Ce 143(232)	
Zn	Zn 64	48.89	Zn-65	243d	0.46	1116	506	NONE	Ba 139(430)	
	Zn 68	18.57	Zn 69	14h	0.1	439	950	NONE	Te 131(150) Kr 85m(151) Sr 85m(151)	
									Te 129m(112) Eu 152(112) Ta 182(113)	
									Tb 160(1115) Sc 46(1121)	
									Nd 147(440) Os 193(441)	

<sup>(a)</sup> THORIUM IS MEASURED BY COUNTING THE 27 DAY  $^{233}\text{Pa}$  DAUGHTER OF  $^{233}\text{Th}$

<sup>(b)</sup> URANIUM IS MEASURED BY COUNTING THE 2.35 DAY  $^{239}\text{Np}$  DAUGHTER OF  $^{239}\text{U}$

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