

AEC  
RESEARCH  
and  
DEVELOPMENT  
REPORT



BNWL-372

3-

**A "POISON-FREE" LOADING OF  $UO_2$  AND PU-A1  
IN THE PRCF WITH  $D_2O$ -MODERATOR**

**V. O. UOTINEN**

**MAY, 1967**

SEARCHED	INDEXED	FILED	FILED	ROUTE DATE



**BATTELLE-NORTHWEST**  
BATTELLE MEMORIAL INSTITUTE / PACIFIC NORTHWEST LABORATORY

## LEGAL NOTICE

This report was prepared as an account of Government sponsored work. Neither the United States, nor the Commission, nor any person acting on behalf of the Commission:

- A. Makes any warranty or representation, expressed or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this report, or that the use of any information, apparatus, method, or process disclosed in this report may not infringe privately owned rights; or
- B. Assumes any liabilities with respect to the use of, or for damages resulting from the use of any information, apparatus, method, or process disclosed in this report.

As used in the above, "person acting on behalf of the Commission" includes any employee or contractor of the Commission, or employee of such contractor, to the extent that such employee or contractor of the Commission, or employee of such contractor prepares, disseminates, or provides access to, any information pursuant to his employment or contract with the Commission, or his employment with such contractor.

## PACIFIC NORTHWEST LABORATORY

RICHLAND, WASHINGTON

operated by

BATTELLE MEMORIAL INSTITUTE

for the

UNITED STATES ATOMIC ENERGY COMMISSION UNDER CONTRACT AT(45-1)-1830

PRINTED BY/FOR THE U. S. ATOMIC ENERGY COMMISSION

3 3679 00060 5305

BNWL-372  
UC-80, Reactor Technology

A "POISON-FREE" LOADING OF  $UO_2$  AND Pu-Al  
IN THE PRCF WITH  $D_2O$ -MODERATOR

By

V. O. Uotinen

Nuclear Experiments and Analysis Section  
Reactor Physics Department

May, 1967

FIRST UNRESTRICTED  
DISTRIBUTION MADE **MAY 25 '67**

PACIFIC NORTHWEST LABORATORY  
RICHLAND, WASHINGTON

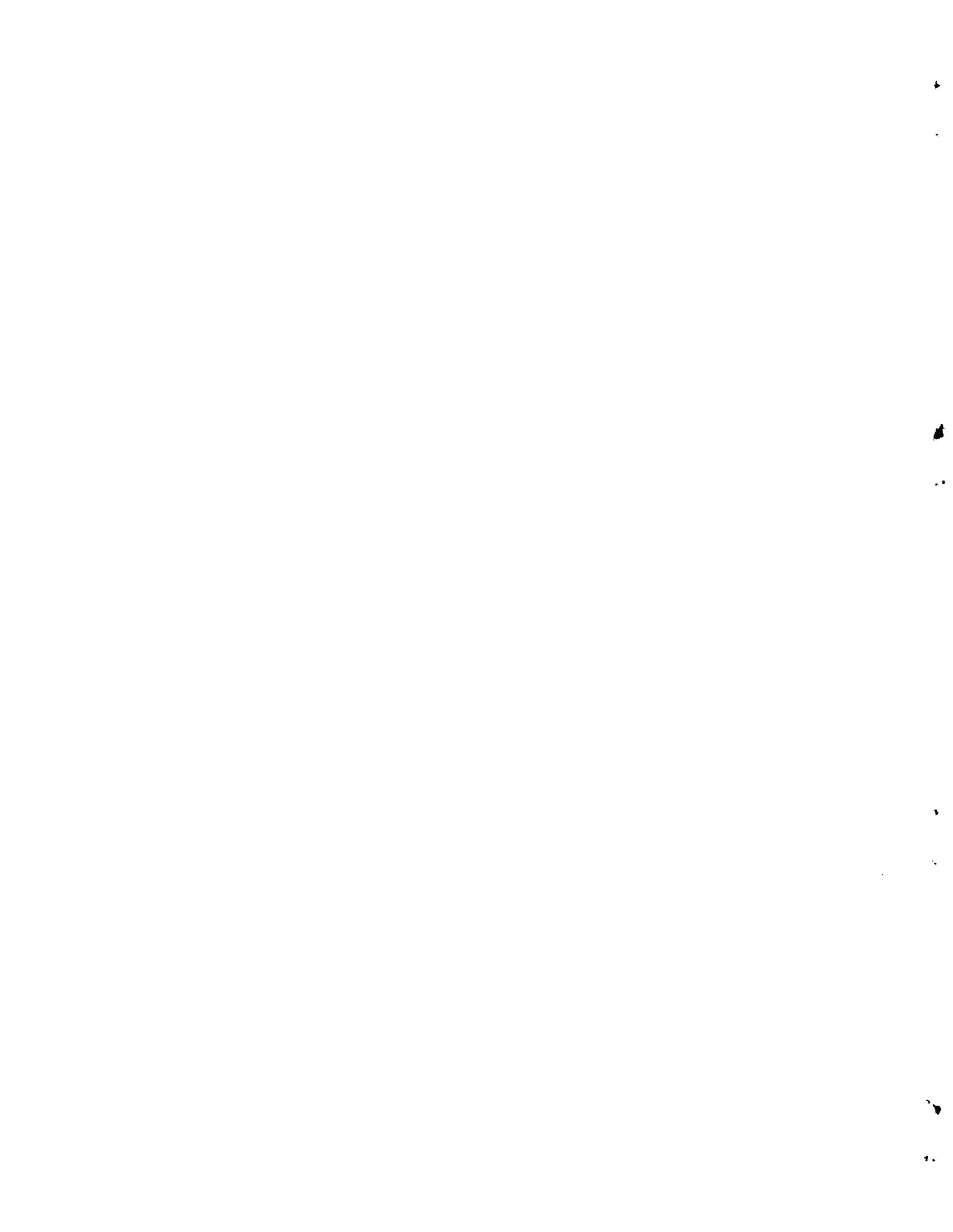
Printed in the United States of America  
Available from  
Clearinghouse for Federal Scientific and Technical Information  
National Bureau of Standards, U.S. Department of Commerce  
Springfield, Virginia 22151  
Price: Printed Copy \$3.00; Microfiche \$0.65

TABLE OF CONTENTS

INTRODUCTION . . . . .	1
SUMMARY . . . . .	1
DESCRIPTION OF FUEL . . . . .	1
DESCRIPTION OF CONTROL ROD . . . . .	1
EXPERIMENTAL RESULTS . . . . .	2
Critical Configurations . . . . .	2
Reaction Rates of Gold Foils. . . . .	4
Reactivity Worth of Control Rods. . . . .	6
CALCULATIONS . . . . .	7
CONCLUSIONS . . . . .	10
REFERENCES . . . . .	11
DISTRIBUTION . . . . .	13

LIST OF FIGURES

1	PRCF Loading Diagram: Loading No. E-9	2
2	PRCF Loading Diagram: Loading No. E-12	3
3	PRCF Loading Diagram: Loading No. E-14	3
4	PRCF Loading Diagram: Loading No. E-17	4
5	Radial Traverse with Gold Foils	5
6	PRCF Loading Diagram: Loading No. E-23	7
7	Geometry of 19-Rod Fuel Cluster	8
8	Comparison Between Calculated and Observed Gold Activity	10



A "POISON-FREE" LOADING OF  $UO_2$  AND Pu-Al  
IN THE PRCF WITH  $D_2O$ -MODERATOR

V. O. Uotinen

#### INTRODUCTION

The control rods that are used in the  $D_2O$ -moderated Plutonium Recycle Critical Facility (PRCF) are of the "shutter" type so that, in their normal "withdrawn" position, there is still a large amount of cadmium in the reactor.<sup>(1)</sup> Thus, under normal operating conditions the three control rods create three large flux sinks in the reactor. This report describes an experiment that has been conducted to establish a critical configuration with all three control rods and their thimbles removed from the reactor. This experiment determined a critical loading for an essentially "poison-free" core.

The PRCF contained fuel in a two-zone loading, an inner zone of natural uranium oxide<sup>(2)</sup> and an outer zone of 1.8 wt% Pu-Al.<sup>(3)</sup> The fuel elements were 19-rod clusters and were arranged in a hexagonal lattice with an 8 in. pitch.

#### SUMMARY

The "poison-free" critical configuration contained 24  $UO_2$  fuel elements and 16 Pu-Al fuel elements. With this loading the reactor was critical at a moderator height of 98.15 in. In this control-rod-free loading, measurements were made of the radial distribution of the reaction rate of gold foils. The Westcott epithermal index,  $r$ , was determined from a cadmium ratio measurement. Also, the reactivity worth of each control rod was measured as the control rods were reinstalled, one at a time.

A calculation has been performed with the one-dimensional diffusion-theory code HFN<sup>(4)</sup> to obtain analytical values of the effective multiplication factor and the critical radius for the control-rod-free loading. The calculated effective multiplication factor is  $\sim 0.7\%$  high and the calculated critical radius is  $\sim 2\%$  low.

#### DESCRIPTION OF FUEL

The  $UO_2$  fuel elements contained natural uranium dioxide. The plutonium in the Pu-Al fuel elements contained 93.55 wt%  $Pu^{239}$ , 6.00 wt%  $Pu^{240}$ , 0.45 wt%  $Pu^{241}$ , and  $< 0.01$  wt%  $Pu^{242}$ . Each fuel element was a cluster of 19 rods. Each rod was 88 in. long and 0.500 in. in diameter and surrounded by a 0.03 in. thick Zircaloy-2 jacket. A Zircaloy-2 wire wrap, 0.072 in. diam, separated the rods of a cluster.

#### DESCRIPTION OF CONTROL ROD

Each control rod consists of two concentric aluminum tubes. The inner tube (1.000 in. OD with 0.088 in. wall) travels vertically inside the outer tube (1.315 in. OD with 0.055 in. wall). Both tubes are enclosed in an aluminum thimble with a 1.660 in. OD and a 0.140 in. wall thickness. The outer surfaces of both tubes have alternate 6 in. long regions that are coated with 0.025 in. of cadmium. The reactivity of the PRCF is increased by raising the inner cylinder so that its cadmium sections are shaded from thermal neutrons by the

cadmium on the outer cylinder. The total travel of the inner cylinder from the "shutter closed" position to the "shutter open" position is 6 in.; and the total exposed area of the cadmium sections is cut approximately in half in going from "closed" to "open."

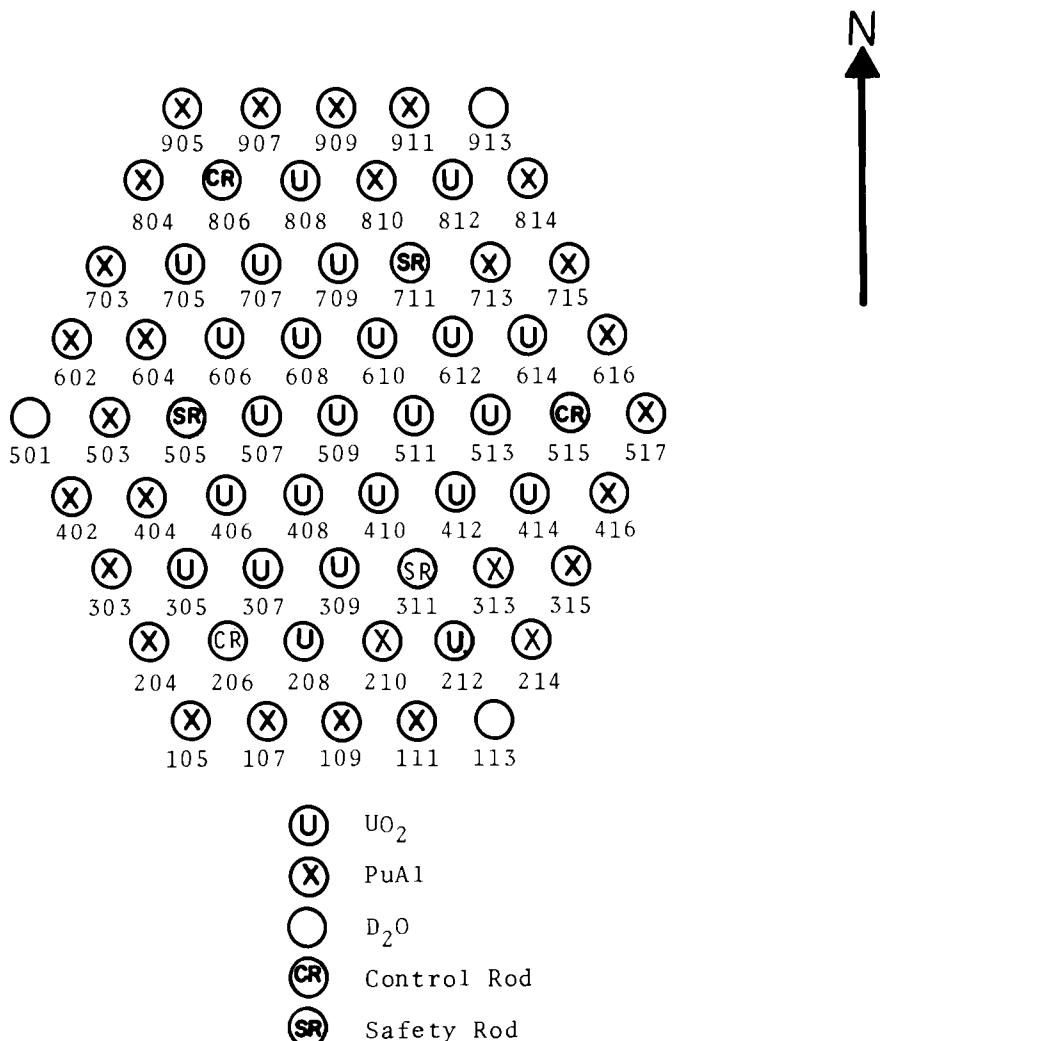
### EXPERIMENTAL RESULTS

#### CRITICAL CONFIGURATIONS

The PRCF was initially loaded to critical with the three control rods at a radius of 24 in. in the reactor

(Figure 1). The control rods were removed one at a time and the loading was adjusted at each step to achieve criticality with two, one, and finally with no control rods in the reactor (Figures 2, 3, and 4).

The critical configurations are described in Table I. Listed for each loading are the positions of control rods, the height of the moderator, the temperature of the moderator, and the excess reactivity. A value of  $4.631 \times 10^{-3}$  was used for the effective fraction



*FIGURE 1. PRCF Loading Diagram:  
Loading No. E-9*

of delayed neutrons,  $\beta_{\text{eff}}$ , which was used to obtain values of reactivity in terms of  $\Delta k/k$ . The uncertainties quoted in the last column of Table I are due only to errors in measuring doubling times.

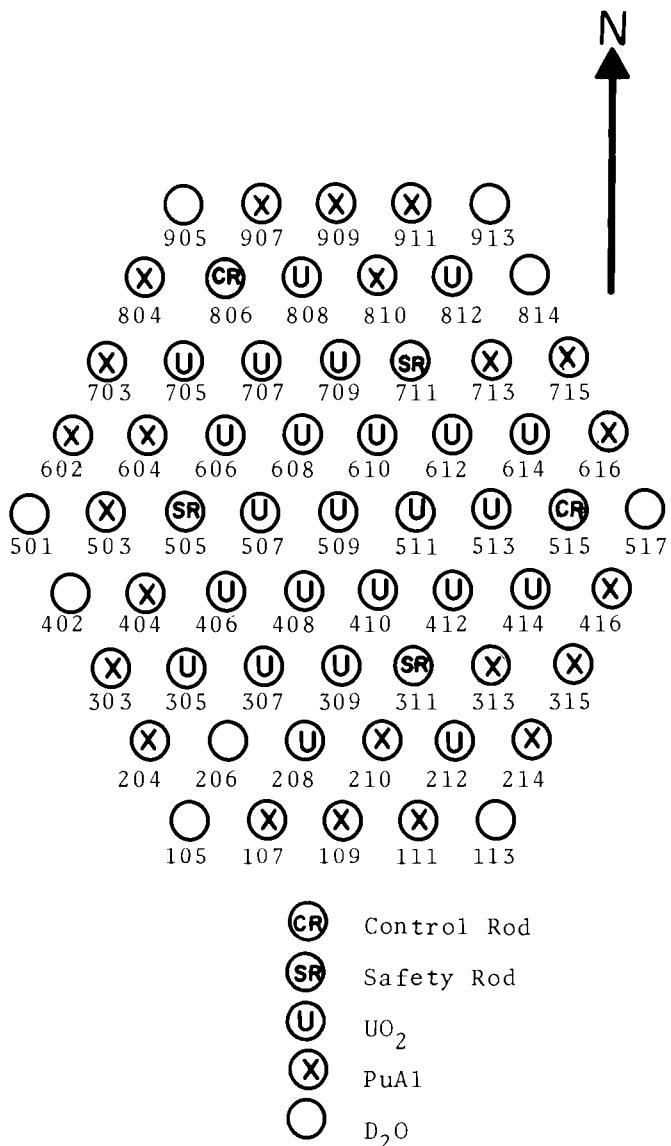


FIGURE 2. PRCF Loading Diagram:  
Loading No. E-12

Three safety rod channels, which consist of 3.50 in. OD, 3.115 in. ID aluminum thimbles containing air, were in all of the loadings described in this report. Diffusion-theory calculations predict a worth of 8.8 mk for replacing these three channels with  $\text{D}_2\text{O}$ .

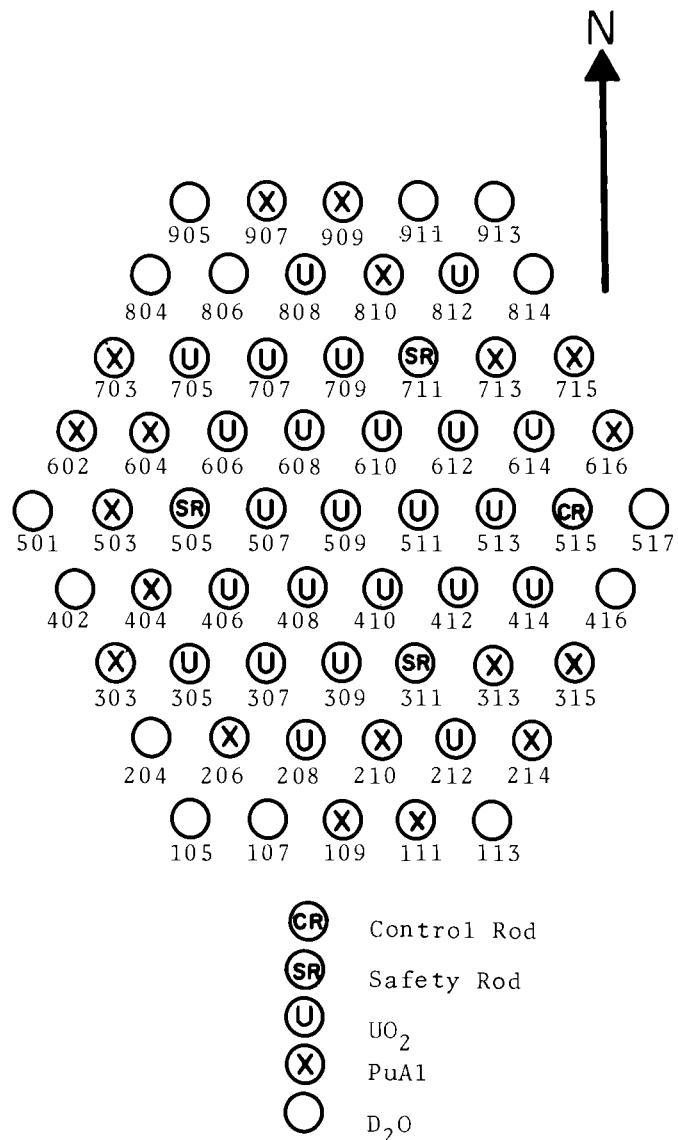


FIGURE 3. PRCF Loading Diagram:  
Loading No. E-14

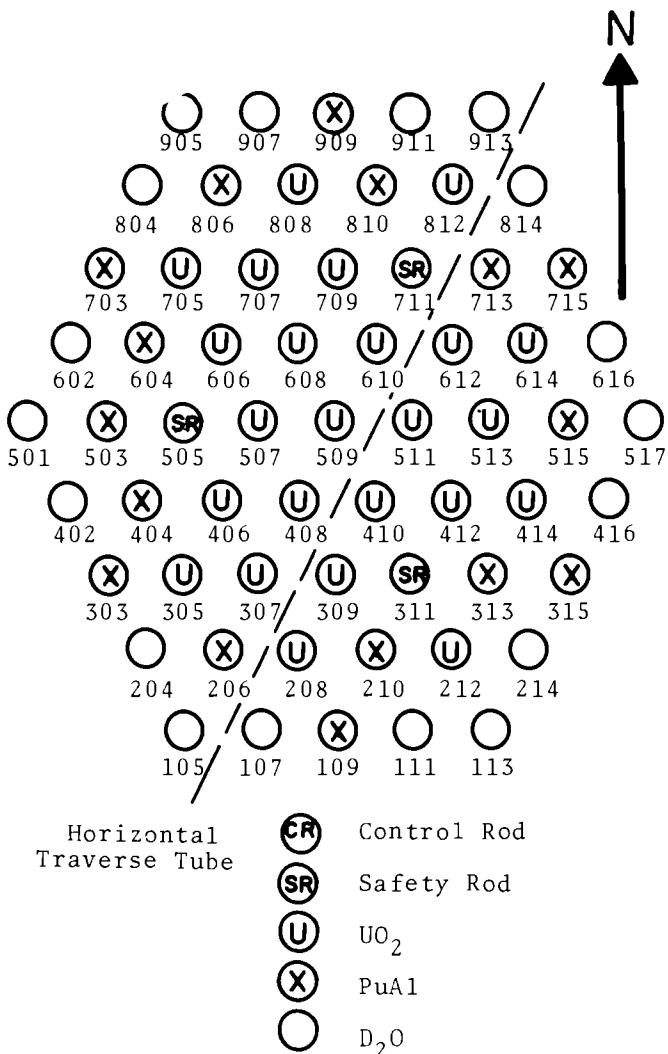


FIGURE 4. PRCF Loading Diagram:  
Loading No. E-17

#### REACTION RATES OF GOLD FOILS

The activation of gold foils was determined as a function of radial position in the control-rod-free reactor. This was done by irradiating gold foils, 0.500 in. in diameter and 0.005 in. thick, in a 3/4 in. OD aluminum tube which spans the reactor horizontally at a distance of 3 ft. 10 in. below the top of the top grid plate and crosses within 3.5 in. of the center of the reactor. Two of the foils were covered with 40 mils of cadmium.

The results of the traverse are compared in Figure 5 with a traverse that was taken in a loading that contained three control rods.<sup>(1)</sup> The two curves have been normalized at a point 4 in. SSW of center. The curve that was reproduced from Reference 1 shows a greater depression at the center which is due to an aluminum guide tube in the central cell, and a hump on the SSW side which is due to the presence of a safety rod void in a lattice position adjacent to the foils. In the present experiment this hump does not appear, because the safety rod positions have been rotated 60°. Instead there is

TABLE I. Critical Configurations

Loading Number	Positions of Control Rods			Moderator Height, in.	Moderator <sup>(b)</sup> Temp., °C	Reactivity, milli-k
	No. 1	No. 2	No. 3			
E-9	open	open	closed	105.25	24.40	0.762 ± .001
E-12	removed	open	4.5 in. open	105.25	24.50	1.029 ± .002
E-14	removed	removed	open	105.25	25.08	0.831 ± .001
E-17	removed	removed	removed	100.00 <sup>(b)</sup>	25.35	1.077 ± .004

(a) Raising the moderator from 100.00 to 105.25 in. increases the reactivity by  $1.62 \pm .06$  mk.

(b) The moderator temperature reactivity coefficient is  $-0.246 \pm .005$  mk/°C

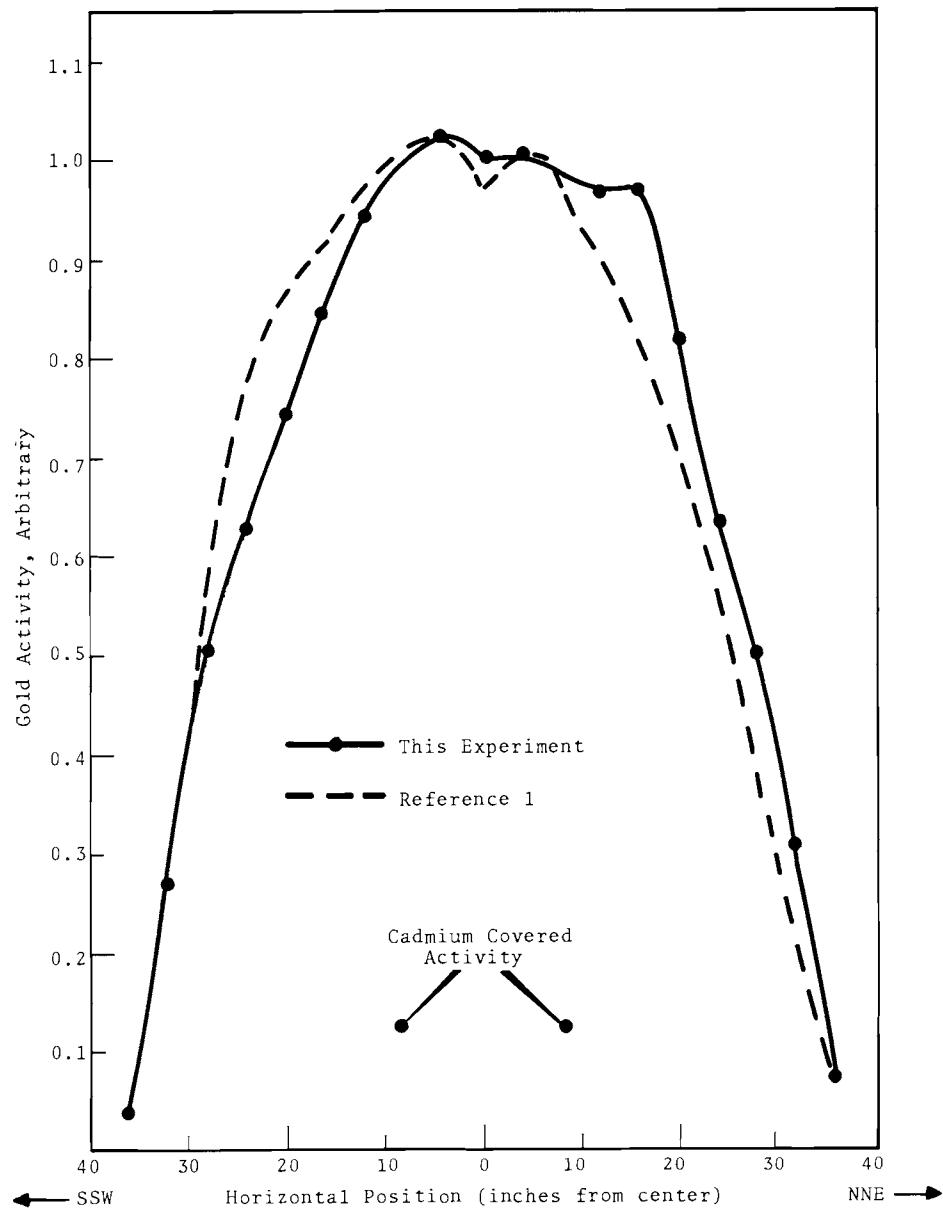


FIGURE 5. Radial Traverse with Gold Foils. The two curves have been normalized at a position 4 in. SSW of center.

now a hump on the NNE side, which now contains a safety rod void. Also, the hump on the NNE side is enhanced because a control rod has been removed from the NNE side.

The measured cadmium ratio for the 0.005 in. thick gold foils is  $8.11 \pm 0.08$  for the point on the SSW side of

center and  $8.16 \pm 0.08$  for the point on the NNE side of center.

The Westcott epithermal index,  $r$ , was determined from the formula<sup>(4)</sup>

$$CdR(0) = \frac{g + rs}{r} \left[ s + \frac{1}{K} \sqrt{\frac{T}{T_0}} \right]^{-1} \quad (1)$$

Here  $g$ ,  $r$ , and  $s$  are the quantities that

define the effective cross section  $\hat{\sigma}$  in the Westcott notation, i.e.,

$$\hat{\sigma} = \sigma_0(g + rs)$$

where  $\sigma_0$  is the cross section for neutrons with a velocity of 2200 m/sec. The K in Equation 1 is a cadmium transmission factor for thermal neutrons, T is the neutron temperature, and  $T_0$  is a reference temperature (293.6 °C). As obtained from Reference 4, the quantities g, s, and K are 1.005,  $17.28\sqrt{\frac{T}{T_0}}$  and 2.2931 respectively. The CdR(0) in Equation 1 is the cadmium ratio for an infinitely dilute foil.

The CdR(0) was determined from the measured cadmium ratio CdR using the formula

$$\left[ \frac{\text{CdR}(0)}{F} - 1 \right] = Q \left[ \frac{\text{CdR}}{F} - 1 \right]. \quad (3)$$

The Q-factor for a 5 mil gold foil was obtained from measurements by Jacks<sup>(5)</sup> and is 0.2288. The F is a correction factor which accounts for epithermal neutron absorption by the cadmium. Using a value of 1.02 for F and a value of  $8.14 \pm 0.08$  for the measured cadmium ratio, we obtain  $2.65 \pm 0.02$  for CdR(0). This, in turn, results in a value of  $0.0338 \pm 0.0004$  for the quantity  $r\sqrt{T/T_0}$  by using Equation 1.

A previous measurement of a gold cadmium ratio in a similar loading in the PRCF has been reported in Reference 1. The cadmium ratio of a 6 mil gold foil (40 mils cadmium) was found to be  $8.2 \pm 0.2$  at a radius of 11.7 in. This results in a value of  $0.0354 \pm 0.0009$  for the quantity  $r\sqrt{T/T_0}$ .

The quantity  $r\sqrt{T/T_0}$  has also been deduced from lutetium activation data which were acquired in an experiment in the PRTR.<sup>(6)</sup> The PRTR data result in a value of  $0.045 \pm 0.004$  for  $r\sqrt{T/T_0}$  at a point in the moderator. This point is between  $\text{UO}_2$  clusters and is as far as possible from all surrounding clusters.

The PRTR measurement results in a larger value of r than the PRCF measurements. The PRTR result is characteristic of the spectrum in the moderator; whereas, the PRCF result is characteristic of the spectrum inside the horizontal aluminum tube in which the foils were placed. Another difference between the two cases is that each fuel cluster in the PRTR is surrounded by an aluminum tube; whereas, the clusters in the PRCF are not. The volume ratio of  $\text{D}_2\text{O}$ -to- $\text{UO}_2$  is 11.8 in the PRTR and 13.6 in the PRCF.

#### REACTIVITY WORTH OF CONTROL RODS

Reactivity worths of incremental movements of control rod No. 3 were measured in loadings E-9, E-12, and E-14. (Figures 1 through 3 and Table I). Assuming that the shape of the integral worth curve for this control rod remains unchanged as the core size is varied, we can determine the worth of the rod (from shutter-closed to shutter-open) in each of these loadings. Calibrations of the PRCF control rods have been reported in Reference 1, where both differential and integral worth curves are presented. When the present measurements are compared with the results reported in Reference 1, we

find that the worth of control rod No. 3 from the shutter-closed position to the shutter-open position is 1.76, 1.53, and 1.31  $\text{mk}$  in loadings E-9, E-12, and E-14 respectively.

After the control-rod-free configuration had been established, the control rods and their thimbles were reinstalled, one at a time, into the PRCF; and measurements were made to determine the change in reactivity produced by each successive addition. The control rods were in their "shutters open" condition.

The results are shown in Table II. Loadings E-25 and E-27 are identical to loadings E-12 and E-14, respectively, (Figures 2 and 3); and loading E-23 is shown in Figure 6. The negative worth of a control rod increased with each successive addition because the environment into which the control rod was placed was different for each loading. The subcritical method of determining reactivity changes from changes in multiplication is inherently less accurate than the supercritical method which involves measuring positive periods. This fact is reflected by the relatively large uncertainties in the control rod worths that were determined subcritically.

### CALCULATIONS

The effective multiplication factor and the critical radius have been calculated by using the one-dimensional

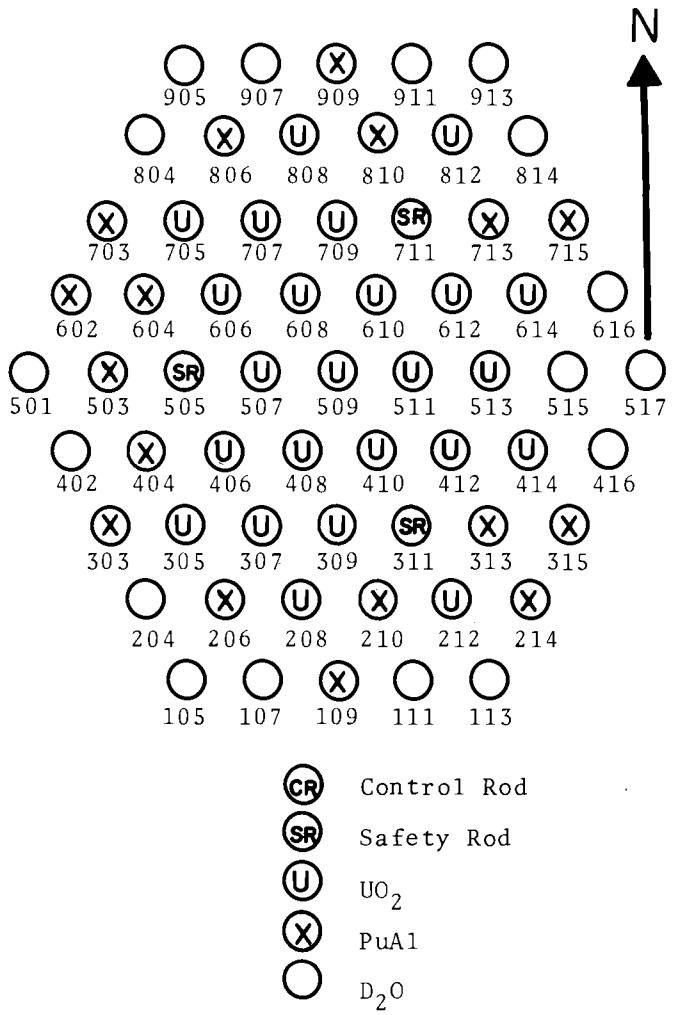


FIGURE 6. PRCF Loading Diagram:  
Loading No. E-23

TABLE II. Reactivities of PRCF Control Rods (Including Thimbles)

Initial Loading	Lattice Position (a)	Method of Measurement	Negative Worth, Milli-k
E-23	515	Positive Period	3.7 $\pm$ 0.1
E-25	806	Subcritical Multiplication	6.2 $\pm$ 0.6
E-27	206	Subcritical Multiplication	7.1 $\pm$ 0.5

(a) This is the lattice position into which the control rod was placed.

diffusion-theory code HFN<sup>(7)</sup> and assuming four groups of neutron energies as described in Table III. Cell average cross sections for the HFN calculations were generated with the codes THERMOS,<sup>(8)</sup> TEMPEST,<sup>(9)</sup> and HRG.<sup>(10)</sup>

For THERMOS calculations involving clustered fuel elements, the fuel was assumed to be in three concentric rings which were separated by regions of cladding and D<sub>2</sub>O (Figure 7). The scattering model assumed for D<sub>2</sub>O was a modified Nelkin model<sup>(11)</sup> with isotropic scattering in the laboratory coordinate system. The technique used for spatial correction in the slowing down calculation was the same as described in Reference 12. Thermal cross sections

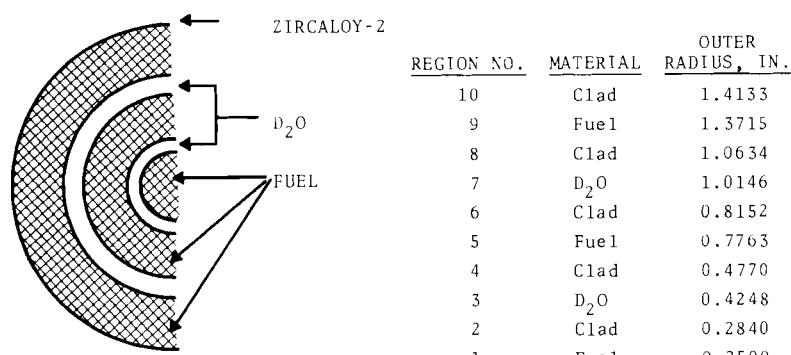
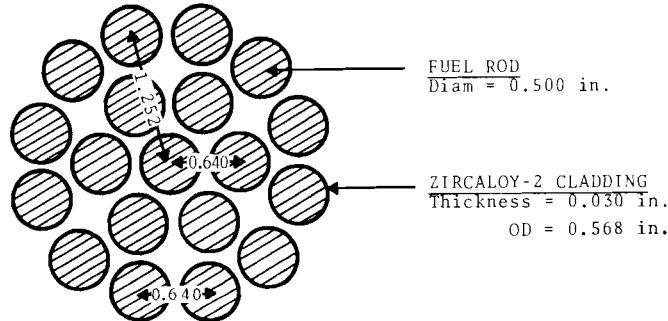
TABLE III. *Group Boundary Energies*

Group	Lower Energy	Upper Energy
1	11.7 keV	10 MeV
2	2.38 eV	11.7 keV
3	0.683 eV	2.38 eV
4	0	0.683 eV

for a safety rod cell with cadmium removed were obtained from a calculation<sup>(13)</sup> using the code TEMPEST. The volume fractions used in the calculations were 0.8267, 0.0358, and 0.1375, respectively, for D<sub>2</sub>O, aluminum, and void. The regions used in the diffusion theory calculation are listed in Table IV.

The calculated effective multiplication factor  $k_{\text{eff}}$  was 1.0068 for the

ACTUAL GEOMETRY OF 19-ROD FUEL CLUSTER



MODEL OF 19-ROD FUEL CLUSTER

FIGURE 7. *Geometry of 19-Rod Fuel Cluster*

TABLE IV. Regions Used in Diffusion-Theory Calculation

Material	Outer Radius, cm	No. of Elementary Cells in Region
UO <sub>2</sub> Fuel	42.676	16
Safety Void	46.505	3
UO <sub>2</sub> Fuel	55.438	8
Pu-Al fuel	69.961	16
D <sub>2</sub> O	92.075	(reflector)
Al + Cd	92.126	(reactor vessel wall)

just critical condition of the loading represented by Figure 4. The calculated critical radius was 68.78 cm; whereas, the "measured" critical radius was 69.96 cm. The "measured" critical radius, R, was defined as  $R = \sqrt{\frac{NA}{\pi}}$ , where N is the number of lattice positions occupied by fuel or safety rod voids, and A is the area of a unit cell. The calculations were performed assuming a temperature of 20 °C and a moderator height of 105.25 in. The calculated results have been corrected to agree with the experimental temperature (25.35 °C) and moderator height (98.15 in.). A temperature coefficient of -0.246 mk/°C<sup>(14)</sup> was used to correct for the difference in temperature, and published moderator reactivity worth curves<sup>(15)</sup> were used to correct for the difference in moderator height.

Relative reaction rates of bare gold foils were obtained from the calculated fluxes. The relative reaction rate, R(r), was defined as

$$R(r) = \frac{\phi_{th,c}(r) + k\phi_{F,c}(r)}{\phi_{th,c}(0) + k\phi_{F,c}(0)} \quad (4)$$

where

$$k = C \left\{ \frac{D_{th}}{\frac{\sqrt{\pi}}{2} \sigma_o f} \left( RI_{ec} + \frac{RI_{es}}{D_{es}} \right) \right\} \quad (5)$$

The  $\phi_{th,c}$  is the thermal flux (0 to 0.683 eV) as calculated with the code HFN, and  $\phi_{F,c}$  is the sum of the fluxes of the nonthermal groups (groups 1, 2 and 3 in Table III) as calculated with HFN. The other parameters are defined as follows:

$\sigma_o$  = microscopic activation cross section at 2200 m/sec

f = non-1/v correction factor

$D_{th}$  = average foil disadvantage factor for thermal Maxwellian neutrons

$D_{es}$  = average foil disadvantage factor for epithermal-subcadmium neutrons

$RI_{ec}$  = effective epicalcium resonance integral

$RI_{es}$  = epithermal-subcadmium resonance integral.

The constant C was determined experimentally, and it relates the ratio,  $\phi_{th}/\phi_F$ , of the actual thermal and epithermal fluxes to the ratio calculated with HFN. The calculated ratio,  $\phi_{th,c}/\phi_{F,c}$ , is 1.624 at r = 8.0 in. Thus, the constant C is given by

$$C = 1.624 \frac{\phi_{th}(8.0)}{\phi_F(8.0)} \quad (6)$$

or

$$C = 1.624 \frac{\frac{\sqrt{\pi}}{2} \sigma_o f}{RI_{ec} D_{th}} \left[ \frac{CdR}{1.02} - 1 \right]^{-1} \quad (7)$$

where CdR is the measured gold cadmium ratio at r = 8.0 in.

Inserting Equation (7) into Equation (5), and using the measured cadmium ratio, 8.14, we have

$$k = 0.233 \left[ 1 + \frac{RI_{es}}{D_{es} RI_{ec}} \right] = 0.265 \quad (8)$$

The values used for  $D_{es}$ <sup>(16)</sup>,  $RI_{es}$ <sup>(16)</sup>, and  $RI_{ec}$ <sup>(17)</sup> are 1.0412, 52 barns,

and 354 barns, respectively. The relative reaction rate is, thus, given by

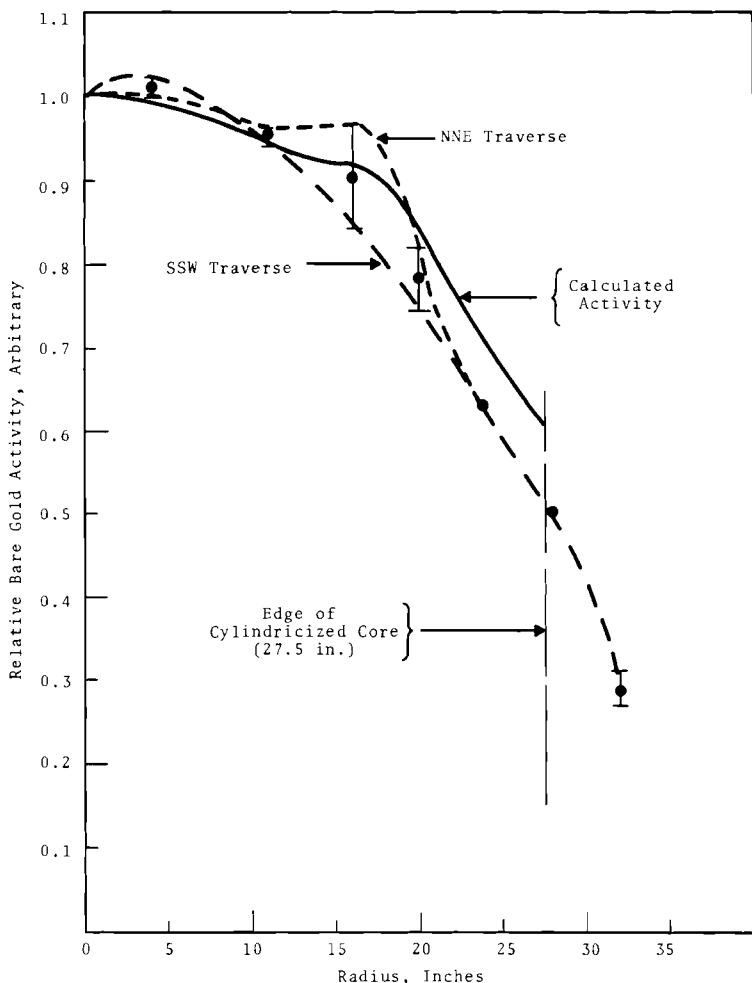
$$R(r) = \frac{\phi_{th,c}(r) + 0.265 \phi_{F,c}(r)}{\phi_{th,c}(0) + 0.265 \phi_{F,c}(0)} \quad (9)$$

The  $R(r)$  is shown as the solid curve in Figure 8, where it is compared with the observed gold activation distribution. The calculated distribution agrees more closely with the NNE traverse than with the SSW traverse because the geometry used in the calculation more closely resembles the geometry on the NNE side.

### CONCLUSIONS

It is difficult to accurately represent discrete control rods in a one-dimensional diffusion theory calculation. To assist in the evaluation of calculational methods a control-rod-free critical configuration was assembled in the  $D_2O$ -moderated PRCF.

A calculation with the diffusion theory code HFN, using cell-average cross sections generated with the codes THERMOS, TEMPEST, AND HRG, predicts the effective multiplication factor to within 0.7% of the measured value for the control-rod-free loading of  $UO_2$  and Pu-Al clusters.



*FIGURE 8. Comparison Between Calculated and Observed Gold Activity (The Tips of the Error Flags Define the NNE and SSW Experimental Traverses. The Solid Line is the Calculated Activity from HFN. The Curves Have Been Normalized at the Center.)*

The calculational scheme of THERMOS, HRG, TEMPEST, and HFN has been used to calculate the effective multiplication factor of a Pu-Al-D<sub>2</sub>O loading (18) the calculated  $k_{\text{eff}}$  is ~4% high. The same calculational method has been used to calculate infinite multiplication factors for UO<sub>2</sub>-D<sub>2</sub>O systems; (19) the calculated  $k_{\infty}$  are consistently ~3% low. Thus, it appears that the good agreement that we obtain between the calculated and measured  $k_{\text{eff}}$  for our two-zone loading may be the result of compensating errors.

The calculated gold activity is considerably greater than the measured activity at radii greater than ~20 in. This discrepancy may be caused in part by the failure of diffusion theory to adequately represent events near the core-reflector boundary. Another reason for the discrepancy may be because the calculational method consistently overestimates the resonance absorption in uranium cells. (19) Thus, the calculated neutron density in the inner (UO<sub>2</sub>) zone is expected to be low.

#### REFERENCES

1. R. A. Bennett and L. C. Schmid. Approach to Critical and Calibration Experiments in the Plutonium Recycle Critical Facility, HW-80206, General Electric Co., Richland, Washington. July 1964.
2. M. K. Millhollen. Specifications for Swaged UO<sub>2</sub> 19-Rod Cluster, PRT Fuel Element Mark I, HW-64359, General Electric Co., Richland, Washington. 1960.
3. M. D. Freshley. Plutonium Spike Fuel Elements for the PRT Fuel Element Mark I-G, HW-69200 PT. 1, 1961, and R. E. Sharp. The Mark I-H, HW-69200 PT. 2, General Electric Co., Richland, Washington. 1961.
4. C. H. Westcott. Effective Cross Section Values for Well-Moderated Thermal Reactor Spectra, CRRP-960, National Research Council of Canada, Atomic Energy Project, Chalk River, Ontario. November 1, 1960.
5. G. M. Jacks. A Study of Thermal and Resonance Neutron Flux Detectors, DP-608, E. I. du Pont de Nemours & Co., Aiken, South Carolina. August 1961.
6. J. R. Triplett, R. E. Dunn, V. W. Gustafson, R. E. Peterson, J. J. Regimbal, J. T. Russell, and L. C. Schmid. Plutonium Recycle Test Reactor Critical Test Results, HW-61900BA. General Electric Co., Richland, Washington. 1961.
7. J. R. Lilley. Computer Code HFN-Multigroup Multiregion Neutron Diffusion Theory in One Space Dimension, HW-71545, General Electric Co., Richland, Washington. November 17, 1961.
8. H. C. Honeck. THERMOS: A Thermalization Transport Theory Code for Reactor Lattice Calculations, BNL-5826, Brookhaven National Laboratory, Upton, N. Y.
9. R. H. Shudde and J. Dyer. NAA Program Description, "Tempest, a Neutron Thermalization Code", North American Aviation Corporation. 1960.
10. J. L. Carter, Jr. Unpublished Data, Pacific Northwest Laboratory, Richland, Washington. 1966. (HRG-Hanford Revised Gam).
11. H. C. Honeck. "An Incoherent Thermal Scattering Model for Heavy Water", Trans. Am. Nucl. Soc., vol. 5, No. 1, 1962.
12. J. R. Worden, W. L. Purcell, and L. D. Schmid. Physics Experiment-High Power Density Core of the PRT, BNWL-221, Pacific Northwest Laboratory, Richland, Washington. January 1966.
13. U. P. Jenquin. Unpublished Data, Pacific Northwest Laboratory, Richland, Washington. 1966.
14. W. P. Stinson. Unpublished Data, Pacific Northwest Laboratory, Richland, Washington. 1966.

15. *V. O. Uotinen and L. C. Schmid. The D<sub>2</sub>O Moderator Level Coefficients of the PRCF, HW-81397, General Electric Co., Richland, Washington. August 27, 1964.*
16. *R. A. Bennett. "Effective Resonance Integrals of Cu and Au," Nuclear Physics Research Quarterly Report, October, November, December, 1959, HW-63576, General Electric Co., Richland, Washington. 1959.*
17. *N. P. Bauman. Resonance Integrals and Self-Shielding Factors for Detector Foils, DP-817. E. I. du Pont de Nemours & Co., Aiken, South Carolina. January 1963.*
18. *U. P. Jenquin. Unpublished Data, Pacific Northwest Laboratory, Richland, Washington. 1966.*
19. *S. L. Fawcett, F. W. Albaugh, R. S. Paul, et al. Pacific Northwest Laboratory Monthly Activities Report for August 1966, BNWL-321, Pacific Northwest Laboratory, Richland, Washington. September 1966.*

DISTRIBUTION

<u>Number of Copies</u>	<u>Number of Copies</u>
1 A. B. Atomenergi P.O. Box 9042 Stockholm-9, Sweden Attn: B. Perohagen	1 European Atomic Energy Community 51-53 Rue Belliard (EURATOM) Brussels 4, Belgium Attn: Pierre Kruys
2 Argonne National Laboratory Attn: Robert Avery B. Toppel	1 General Atomic San Diego, California Attn: H. B. Stewart
2 Atomic Energy Commission Division of Reactor Development and Technology Reactor Physics Branch Attn: I. F. Zartman	4 General Electric Company Pleasanton, California Attn: L. P. Bupp E. A. Evans J. G. Carver M. A. Robkin
3 Atomic Energy Commission Division of Reactor Development and Technology Water Reactors Branch Attn: W. R. Voigt, Chief R. M. Grube (2)	2 General Electric Company Richland, Washington Attn: M. C. Leverett W. S. Nechodom
1 Atomic Energy Commission Reactor Evaluation Staff Attn: J. M. Vallance	4 General Electric Company San Jose, California Attn: S. Levy E. L. Zebroski D. L. Fischer R. Kanne
1 Atomic Energy Commission Division of Production Attn: F. P. Baranowski	3 Richland Operations Office Richland, Washington Attn: H. A. Fouse R. K. Sharp Tech Library Information
3 Atomic Energy Commission, Richland Office of DRDT Site Rep. P. G. Holsted L. R. Lucas W. E. Fry	1 Savannah River Laboratory Aiken, South Carolina Attn: E. J. Hennelly
2 Babcock and Wilcox Co. Lynchburg, Va. Attn: M. C. Edlund Donald Roy	2 United Kingdom Atomic Energy Authority Atomic Energy Establishment Winfrith, Dorchester, Dorset, England Attn: C. G. Campbell G. H. Kinchin
1 Belgonucleaire 35 Rue Des Colonies Brussels, Belgium Attn: E. Fossoul	2 United Nuclear Corporation White Plains, N.Y. Attn: G. A. Sofer J. R. Tomonto
287 Division of Technical Information Extension Oak Ridge, Tennessee	4 Westinghouse Electric Corporation Pittsburgh, Pa. Attn: N. R. Nelson H. W. Graves F. L. Langford L. Orr
4 Douglas United Nuclear Inc. Richland, Washington Attn: T. W. Ambrose R. Nilson C. E. Bowers G. F. Owsley	

DISTRIBUTION (contd)Number  
of Copies117 Battelle-Northwest Laboratories  
Richland, Washington

R. W. Albaugh	D. A. Kottwitz
C. L. Bennett	J. W. Kutcher
R. A. Bennett	D. D. Lanning
S. R. Bierman	J. H. Lauby
C. L. Brown	B. R. Leonard, Jr.
W. L. Bunch	W. R. Lewis
S. H. Bush	R. C. Liikala
G. J. Busselman	C. W. Lindenmeier
J. J. Cadwell	W. W. Little
J. L. Carter	R. C. Lloyd
D. E. Christensen	L. L. Maas
R. G. Clark	D. R. Marr
E. D. Clayton	R. P. Matsen
G. M. Dalen	D. D. Matsumoto
E. C. Davis	R. E. Nightingale
F. G. Dawson	T. J. Oakes
D. R. de Halas	R. S. Paul
B. H. Duane	R. E. Peterson
S. L. Engstrom	W. W. Porath
E. A. Eschbach	W. L. Purcell
S. L. Fawcett	W. A. Reardon
J. R. Fishbaugher	J. J. Regimbal
D. G. Foster	C. R. Richey
H. A. Fowler	R. E. Schenter
A. G. Gibbs	J. E. Schlosser
D. W. Glasgow	L. C. Schmid
V. W. Gustafson	G. L. Simmons
C. E. Haines	J. D. Smith
G. E. Hanson	R. B. Smith
R. W. Hardie	R. I. Smith
O. K. Harling	K. B. Stewart
R. A. Harris	W. P. Stinson
H. Harty	J. J. Stoffels
C. M. Heeb	D. H. Thomsen
R. E. Heineman	V. O. Uotinen (25)
H. L. Henry	A. D. Vaughn
N. A. Hill	M. T. Walling, Jr.
P. L. Hofmann	A. E. Walter
R. H. Holeman	J. D. White
R. M. Humes	L. D. Williams
U. P. Jenquin	J. R. Worden
R. L. Junkins	H. S. Zwibel
E. L. Kelly, Jr.	Technical Information Files (5)
	Technical Publications (2)