

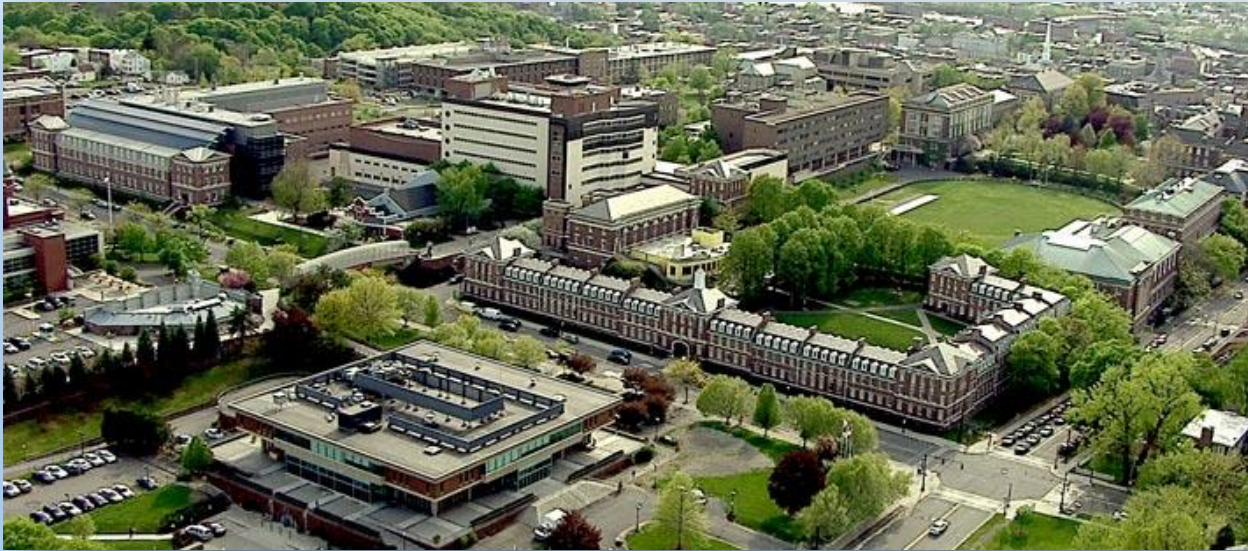
# Unresolved resonance region cross section measurements and validation

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SG51, WPEC, May 15, 2024, NEA, 46, quai Alphonse Le Gallo, 92100 Boulogne-Billancourt, France

# Outline

- The ENDF format and its implication for experimental data
- The effect of self-shielding on measured cross sections
  - Some call this effect transmission enhancement (to be explained)
- From experiment to evaluation
- Processing “high energy resolution” transmission for URR evaluation
- Examples
  - Ta-181 transmission, capture, and validation
  - U-238 transmission and self-indication

# Unresolved resonance region experiments

- The URR energy range starts where resonances can no longer be experimentally resolved
- In the URR:
  - Observed resonances overlap
  - Some resonances are small and not observable (higher  $l$ -values)
- Quantities of interest for evaluation (more on this later)
  - Average cross section (we measure neutron transmission and capture/fission yield)
  - Angular distributions
- Issues
  - Measured average transmission or yield have some amount of self-shielding
  - Intermediate structure results in broad fluctuations (valid for some nuclides)
  - Difficult to handle more than one isotope in a measured sample
  - Information from partly resolved cross sections is lost (need smooth data)
    - Improvement might be possible but is it needed?
  - URR has a relatively small effect in most applications, thus approximations are often used.



# URR options in ENDF

- **LSSF=0**
  - File 2 – URR parameters
    - Used to generate probability tables (or self-shielding factors)
  - File 3 – background cross section (usually zero)
  - Cross section = (File 2 generated cross section) + (File 3 background)
- **LSSF=1**
  - File 2 – URR parameters
    - Used to calculate self-shielding factor  $SF = \langle \sigma \text{ from F2 shielded} \rangle / \langle \sigma \text{ from F2 not shielded} \rangle$
  - File 3 – Gross structure infinite dilution cross section (with covariance in file 33)
  - Cross section = SF (File 2) x cross section (File 3) (fixes the energy grid)
- Probability tables and self-shielding factors are generated in application codes like NJOY
- URR average cross section is generally represented as **smooth function** of average URR parameters in a certain energy range around energy  $E_s$ .

$$\langle \sigma_x(E) \rangle = f(E_s, \bar{D}^{J,l}, \bar{\Gamma}_n^{J,l}, \bar{\Gamma}_\gamma^{J,l}, \bar{\Gamma}_f^{J,l}, \text{others})$$

# URR transmission enhancement example

- Example calculating neutron transmission through a 6 mm Ta sample
- If the cross-section was known in high energy resolution, the “true” transmission (what we measure):

$$\langle T \rangle = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n \cdot \sigma_t(E)} dE = 0.59$$

- If we use only the average cross-section :

$$\bar{T} = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n \cdot \langle \sigma_t \rangle} dE = e^{-n \cdot \langle \sigma_t \rangle} = 0.51$$

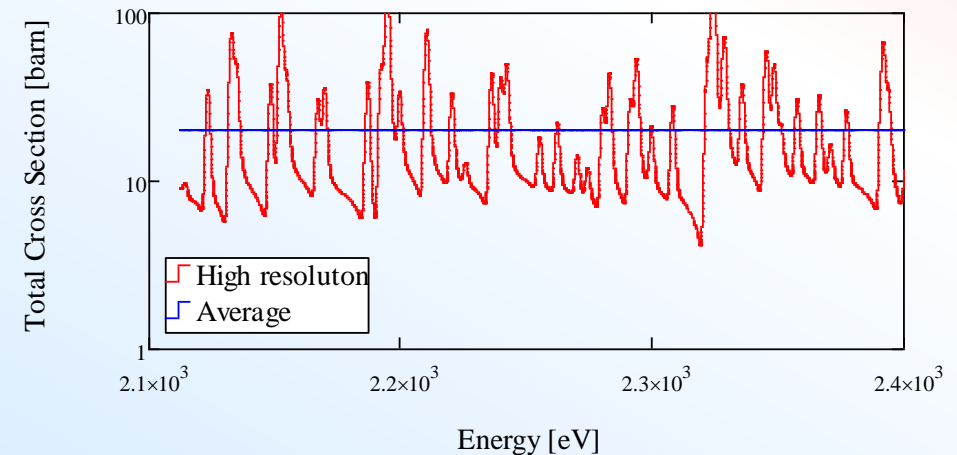
- Fluctuations enhance (increase) transmission and thus reduce the effective cross section (relative to the average) hence the term cross section self-shielding
- When measuring the total cross section with a “thick” sample, a correction for the self shielding is needed.

- Can use two sample thicknesses →

Froehner, et al, “Cross-section fluctuations and self-shielding effects in the unresolved resonance region “, International Evaluation Co-operation volume 15 (NEA-WPEC--15), Nuclear Energy Agency of the OECD, NEA, (1995).

- Can use a model-based approach → SESH

Froehner, Fritz, and Brown, Jesse, *SESH*, Computer Software. <https://github.com/brownjm1968/sesh>. 01 Oct. 2021. Web. doi:10.11578/dc.20211001.4.



# More on URR transmission enhancement

Neutron transmission through a sample:  $T(E) = e^{-n\sigma_t(E)}$

The “true” average transmission from energy  $E_1$  to  $E_2$

$$\langle T \rangle = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n\sigma_t(E)} dE = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n[\sigma_t(E) + \langle \sigma_t \rangle - \langle \sigma_t \rangle]} dE$$

Enhancement due to  $\sigma_t(E)$  fluctuations

$$\langle T \rangle = e^{-n\langle \sigma_t \rangle} \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n[\sigma_t(E) - \langle \sigma_t \rangle]} dE$$

Note: positive and negative contributions

$$T_{sf}(E) = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n[\sigma_t(E) - \langle \sigma_t \rangle]} dE$$

$$\bar{T} = e^{-n\langle \sigma_t \rangle}$$

Self-shielded

True average

$$\langle T \rangle = \bar{T} * T_{sf}(E) \quad \text{where } T_{sf}(E) > 1, \rightarrow \langle T \rangle > \bar{T} \rightarrow \sigma_{measured} = \frac{-1}{n} \ln(\langle T \rangle) < \langle \sigma_t \rangle$$

Evaluation procedure must correctly preserve the observed fluctuations of  $\sigma_t(E)$

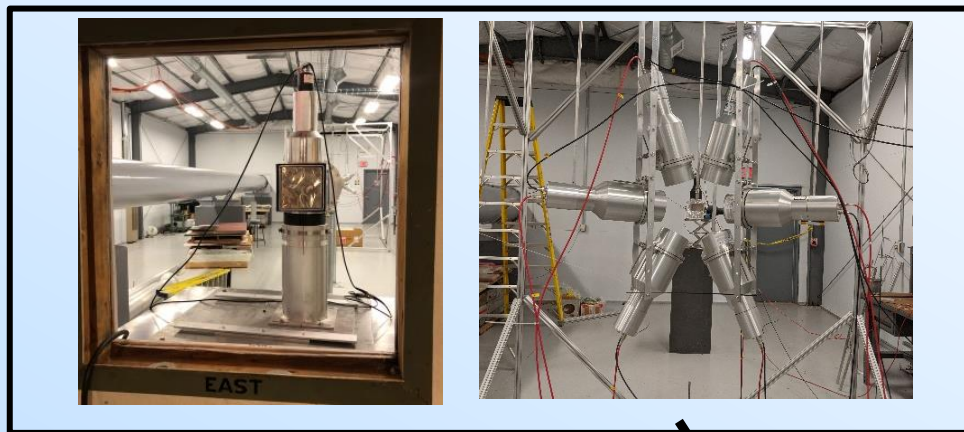
# What is measured?

- To generate a measurement suitable for evaluation, it is sufficient to measure cross sections in wide energy bins such that there are statistically enough resonances in each bin.
  - Practically we measure with much better energy resolution than needed because the data can be averaged later.
- A high energy resolution measurement will be averaged for evaluations.
- We measure neutron transmission and yield not cross sections

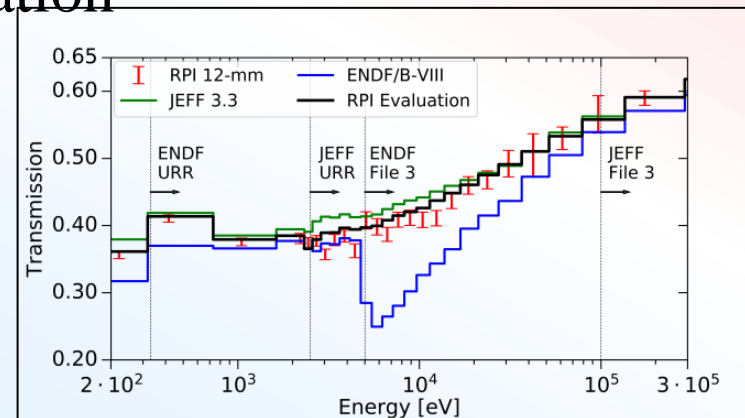
$$\langle T \rangle = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n \cdot \sigma_t(E)} dE \quad \langle Y_x \rangle = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} \left(1 - e^{-n \cdot \sigma_t(E)}\right) \frac{\sigma_x(E)}{\sigma_t(E)} + Y_{ms} dE$$

# The big picture

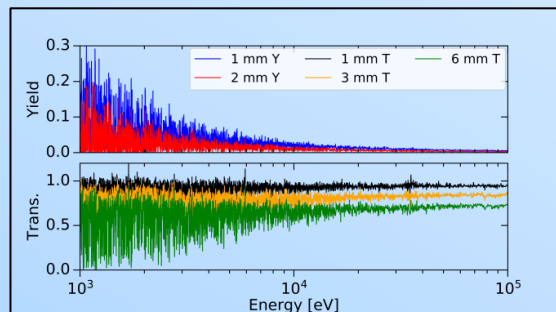
## Measurements



## Validation



## Evaluation



Grouped  
T and or Y

URR  
Parameters

SESH / SAMMY / FITACS

Updated URR  
parameters

MCNP 6.x

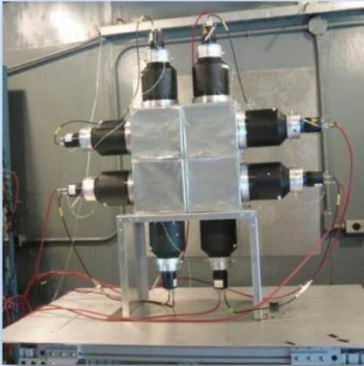
NJOY 2016

*Use Thick-Sample  
Transmission Measurement:  
Validating the URR self-  
shielding*



# Detectors for cross section measurements

MELINDA (100m)



## Transmission

- High energy resolution
- Fast timing
- Large active detector area
- Data-processing well understood

$^6\text{Li}$  glass (35m)



## Transmission

- Relatively good energy resolution
- Fast timing
- Shorter flight-path enables greater count rate
- Better count rate allows freedom of neutron targets

$\text{C}_6\text{D}_6$  Detector (45m)

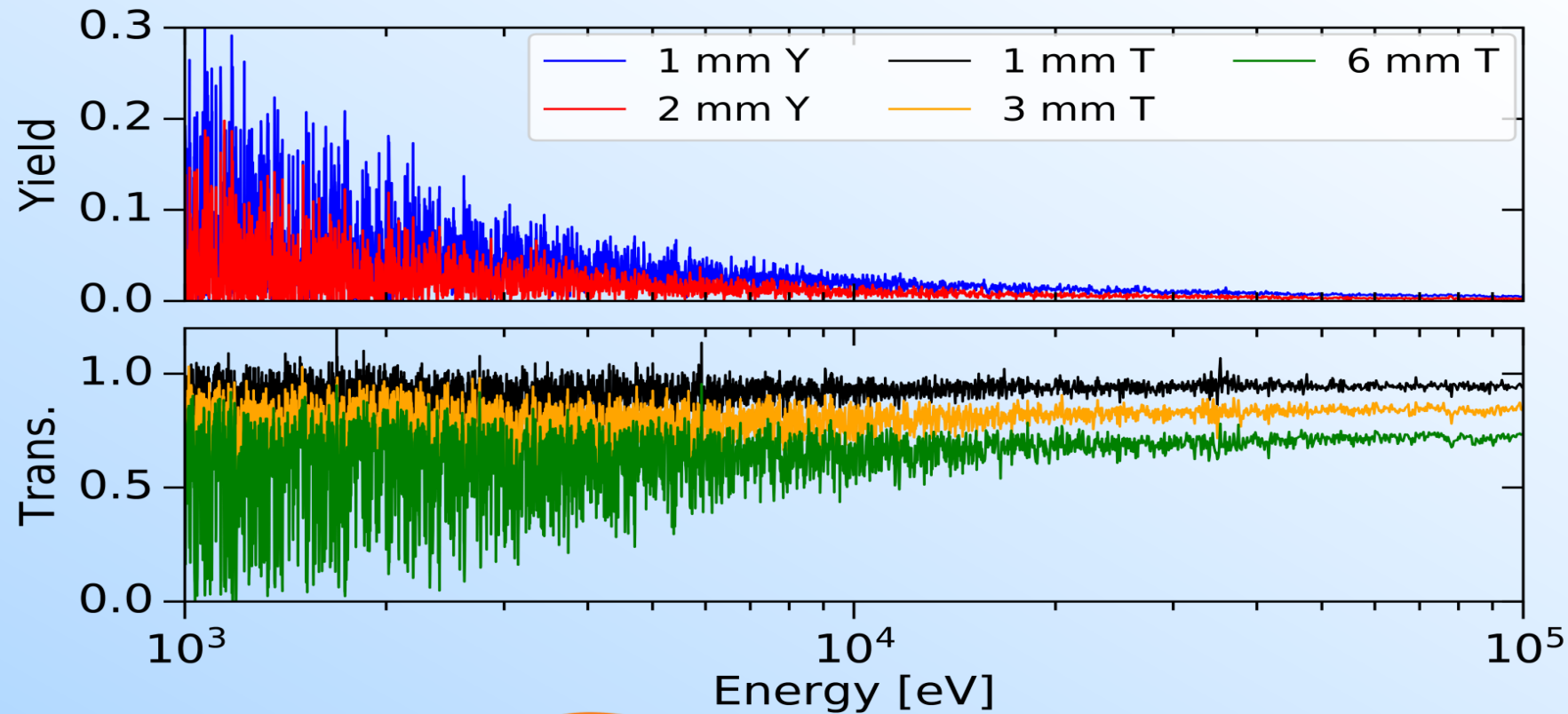


## Capture

- Highest energy resolution for capture at RPI
- Low neutron sensitivity
- Designed with digital acquisition system

# Example $^{181}\text{Ta}$

- $^{181}\text{Ta}$  Evaluation Datasets:
  - Capture Yield: 1 and 2 mm
  - Transmission: 1, 3, and 6 mm
  - URR in ENDF/B-8.1 is for  $E > 2200$  eV and structure is clearly visible in the measurements.



# How to process experimental data in the URR

## Transmission example

Current

A path with less self shielding correction

Measure T in high or low resolution

Group T (in TOF)

Correct T for self shielding using SESH

Convert T to  $\sigma_t$

- To use the current (green) route
  - Need average transmission
  - If EXFOR entry has uncorrected high resolution cross section
    - Convert to transmission, group, self shielding correct, and converted back to cross section
    - Or use another method to self-shielding correct grouped cross section

Measure T in high resolution

Convert T to  $\sigma_t$

Group  $\sigma_t$

Correct  $\sigma_t$  for self shielding

- Self Shielding correction of the blue path:
  - With more than one sample use the Froehner method:
 
$$\langle \sigma \rangle = \frac{n_2 \bar{\sigma}_1 - n_1 \bar{\sigma}_2}{n_2 - n_1}$$
  - For single sample **need to know the experimental energy resolution**

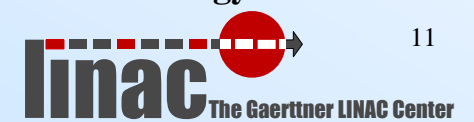
Fit averaged cross section

Fit  $\sigma_t$  with SAMMY

Recent updates to SAMMY will make thing simpler (discussed later)

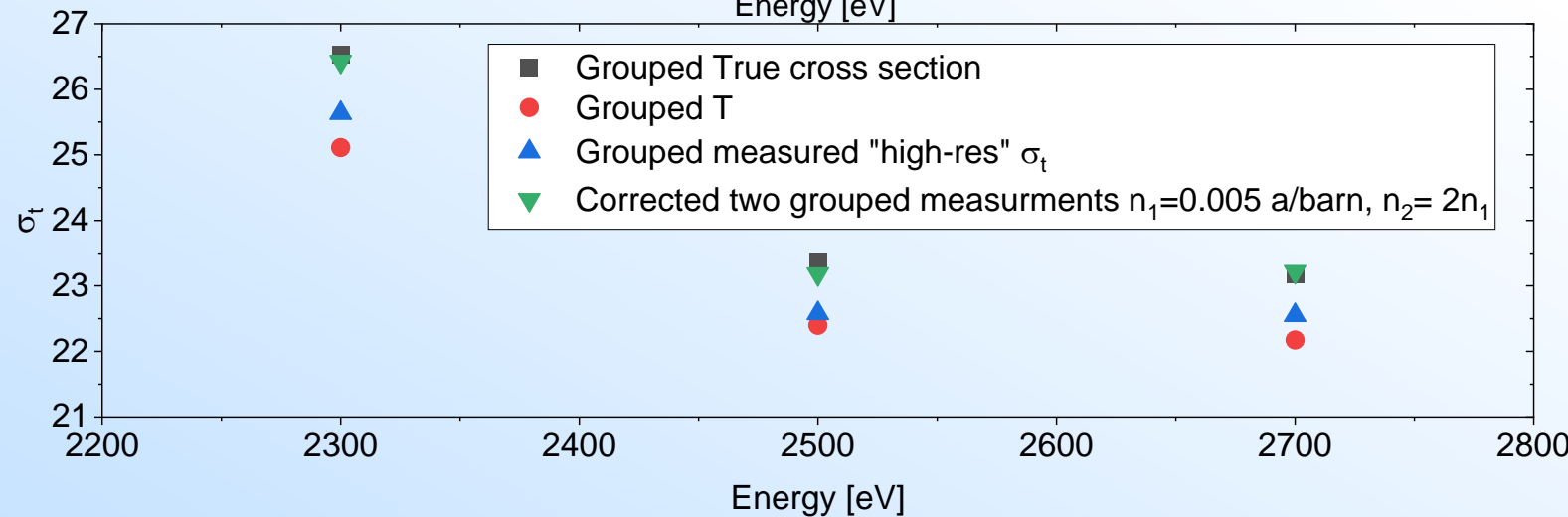
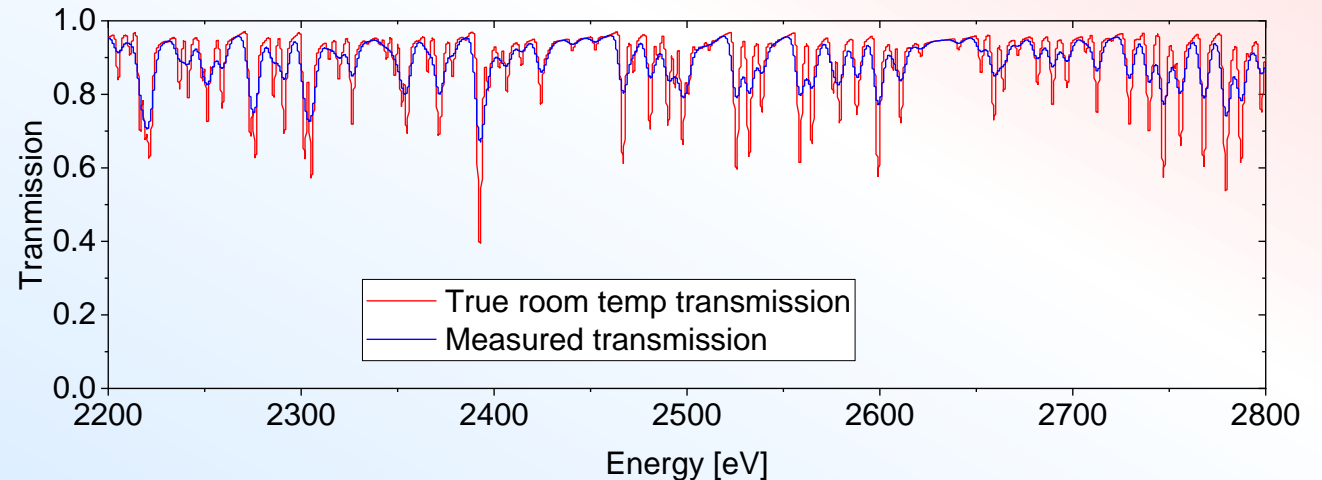


Rensselaer



# Example when high resolution transmission was measured

- Assume we know the true cross section
- Include some experimental resolution
- Process in 3 ways
  1. Average the true cross section to get the true average
  2. Average transmission and convert to cross section
  3. Convert transmission to cross section and average
- No. 3 above results in close answer to the true cross section
  - However, the correction requires knowledge of the experimental energy resolution, or can be accomplished with the Froehner method if two sample thicknesses were measured
  - Correction of the average transmission requires URR parameters and a code like SESH or MCNP and can work when only one sample is available



Evaluators need to be aware of which quantity they use and how to correct SF



# SAMMY Evaluation

## RRR

Input:

- Resonance Parameters
- Data Reduction Parameters
- Experimental Conditions
- Experimental Data: **Transmission and Capture Yield**

**SAMMY:  
R-matrix Bayesian Fitting  
Program**

New Resonance  
Parameters

## URR

Input:

- Average Resonance Parameters
- Experimental Conditions
- Experimental Data: **Total and Capture cross section**

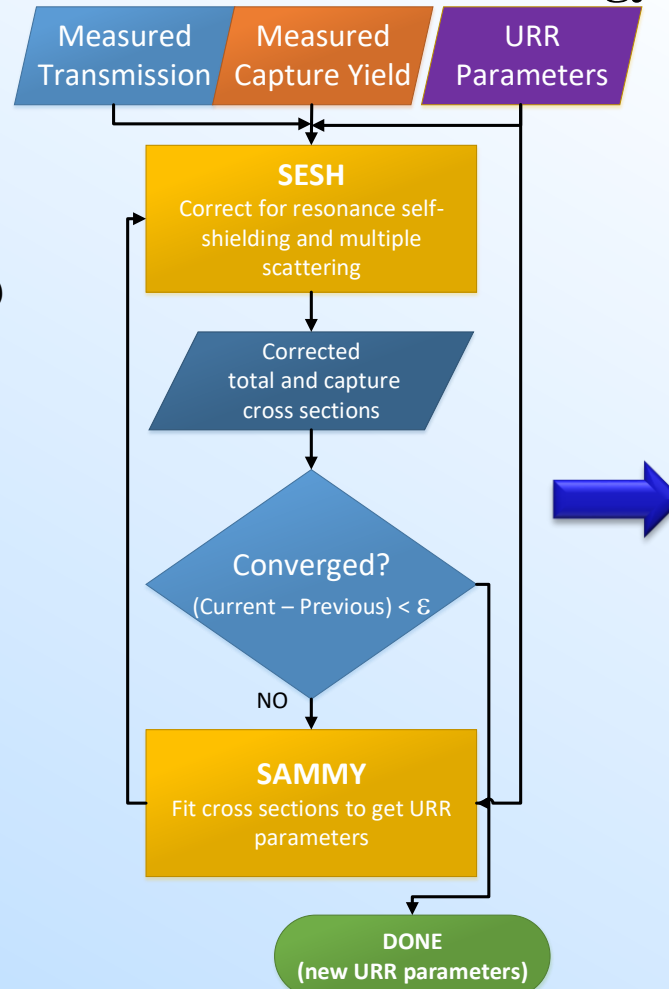
**SAMMY/FITACS & SESH:**  
▪ Hauser Feshbach Bayesian  
Fitting Program  
▪ MC Self-Shielding Code

New Average  
Resonance  
Parameters

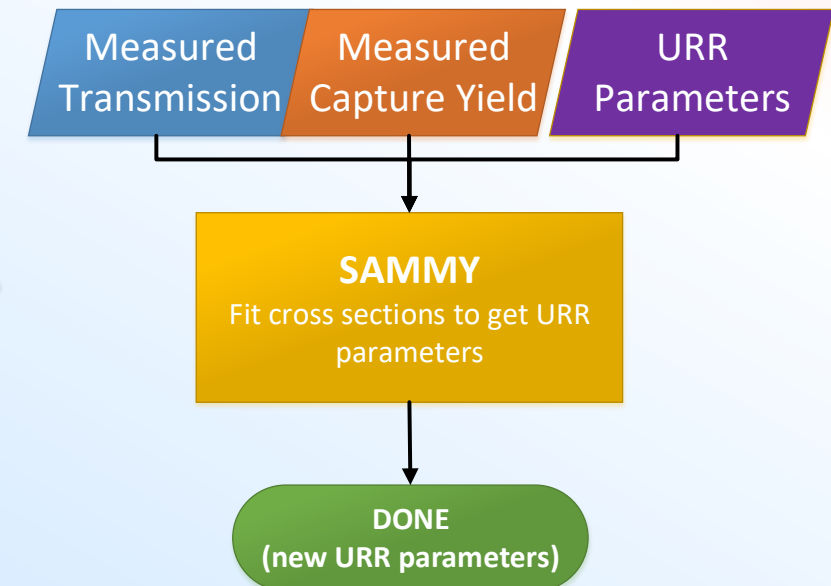
# SAMMY SESH Integration

- PhD student thesis
- Collaboration with ORNL/NNL
- Help modernize SAMMY URR analysis
- Integration of a SESH like code into SAMMY
- Eliminate external iterations
- Directly fit transmission (T) and capture yields (Y) like SAMMY can do in the RRR
- SAMMY capabilities like the RRR in the URR
  - Include samples with multiple isotopes and multiple samples
  - Mix transmission and capture yield in one fit
- **Completed**
  - Transmission
  - Capture
  - Multiple samples
- **In-progress**
  - Multiple isotopes in a sample

## Current URR methodology



## New SAMMY



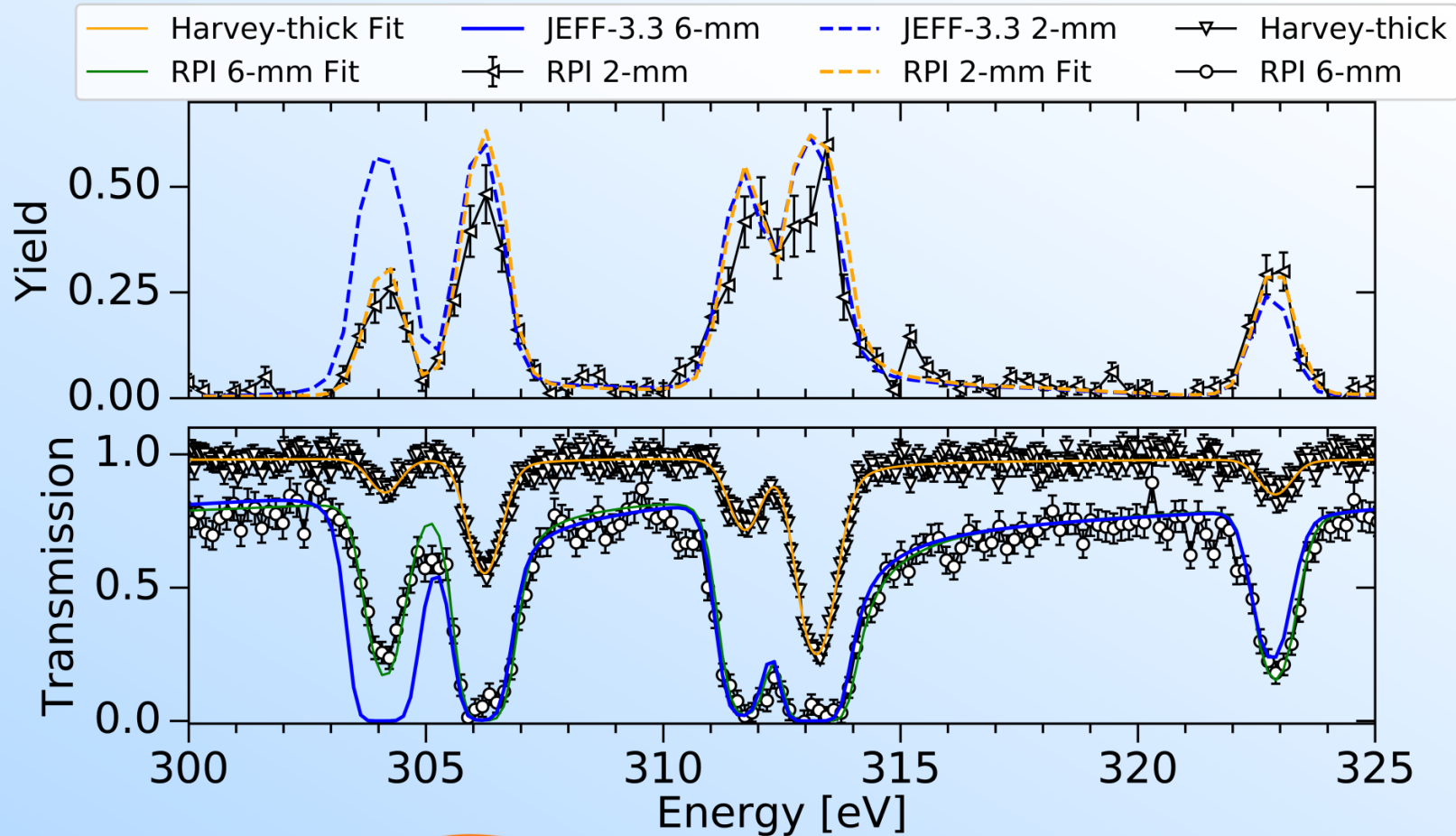
# Examples from previous work



# $^{181}\text{Ta}$ Data to Evaluate: RRR (one example)

End of ENDF/B-VIII.0 RRR:

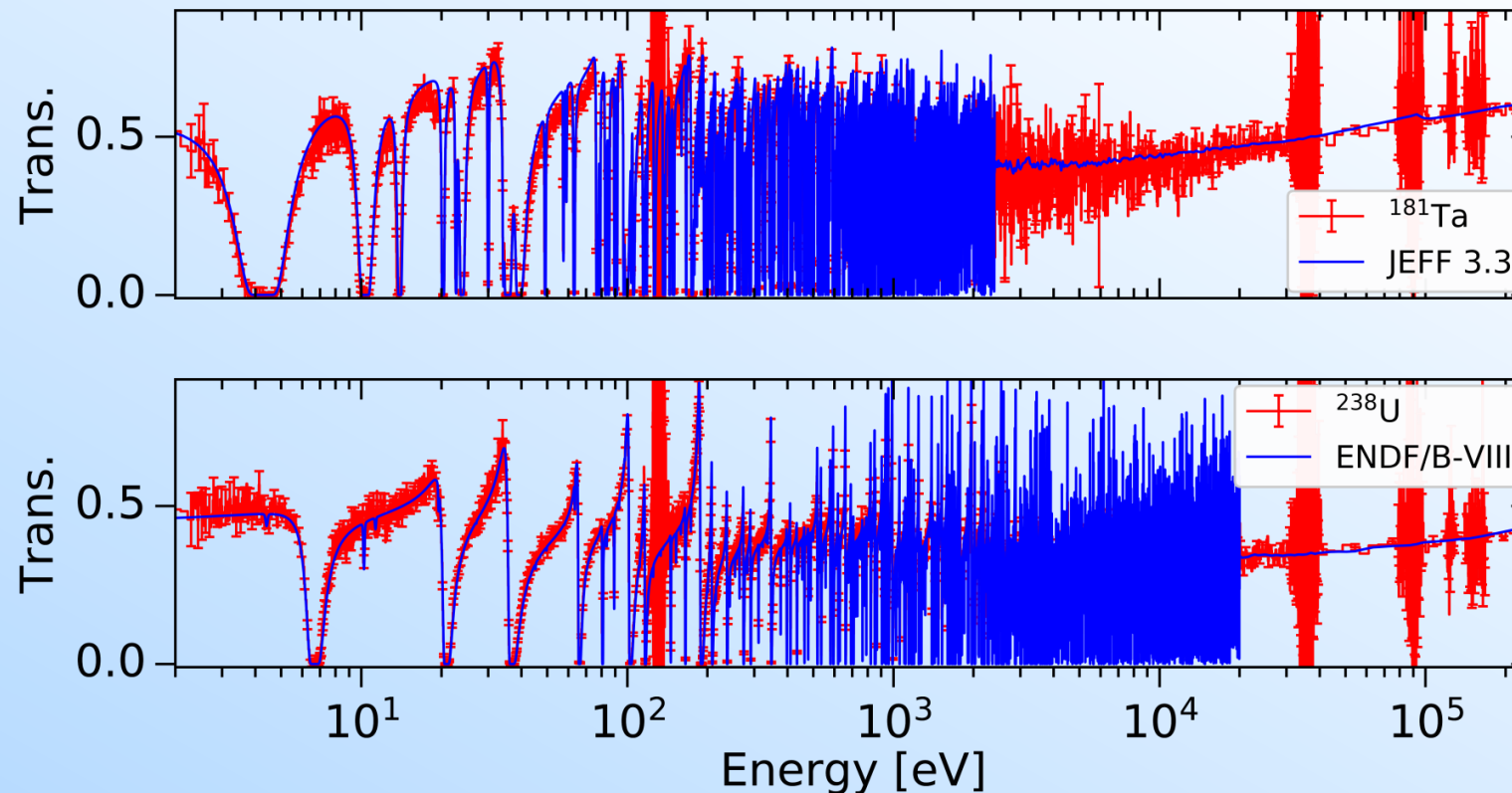
- 304 eV resonance updated
- Transmission and capture yield are well resolved





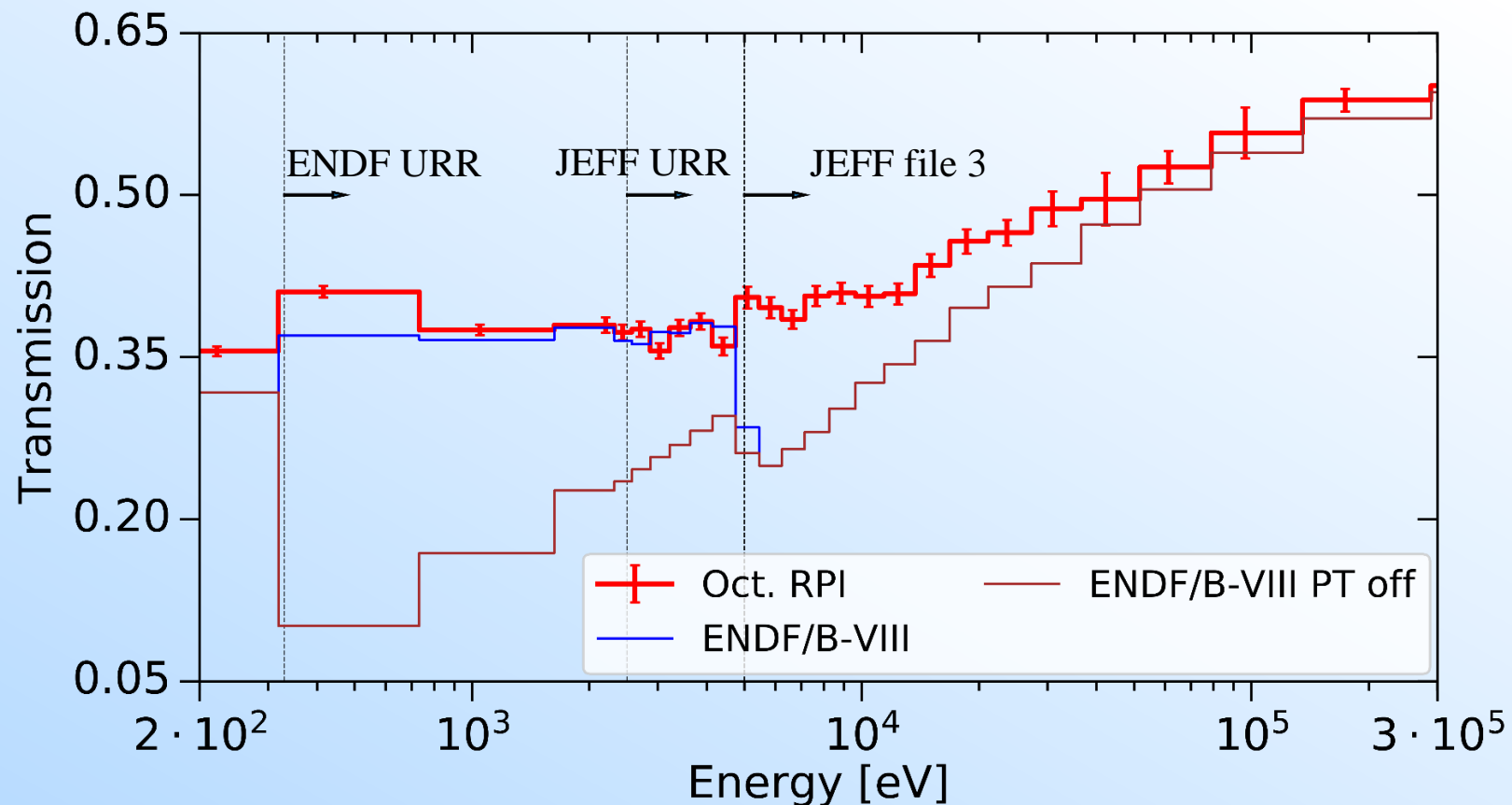
# Validation of $^{181}\text{Ta}$ transmission data using $^{238}\text{U}$

- $^{181}\text{Ta}$  Validation Dataset:
  - $^{181}\text{Ta}$ : 12 mm used as URR validation sample due to heavy self-shielding
  - $^{238}\text{U}$  verification dataset



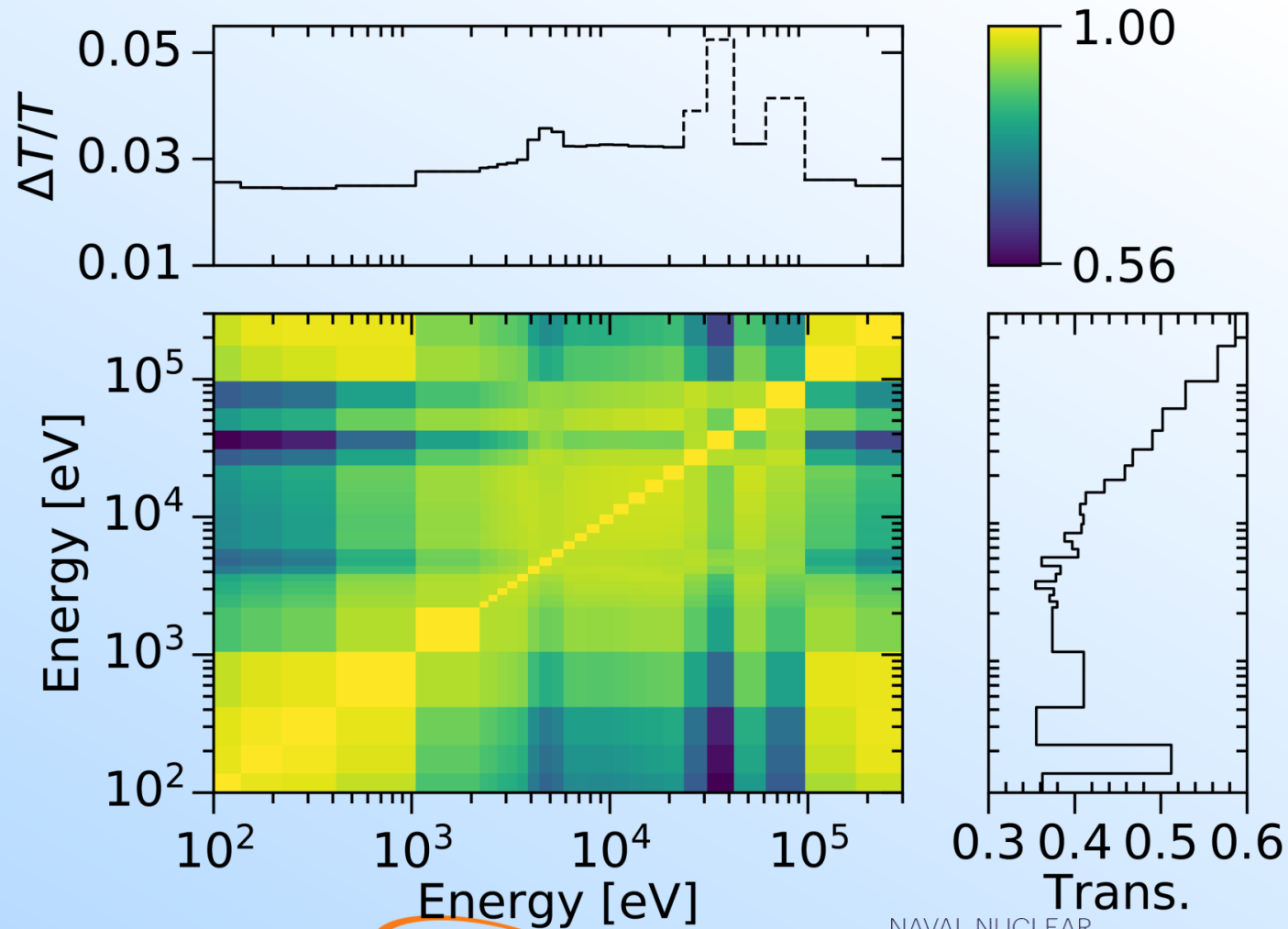
# Resonance Self-Shielding effect in Ta

- The effect of self shielding is shown by turning off the URR treatment in MCNP
- Near 400 eV self-shielding increase the transmission by a factor of about 4

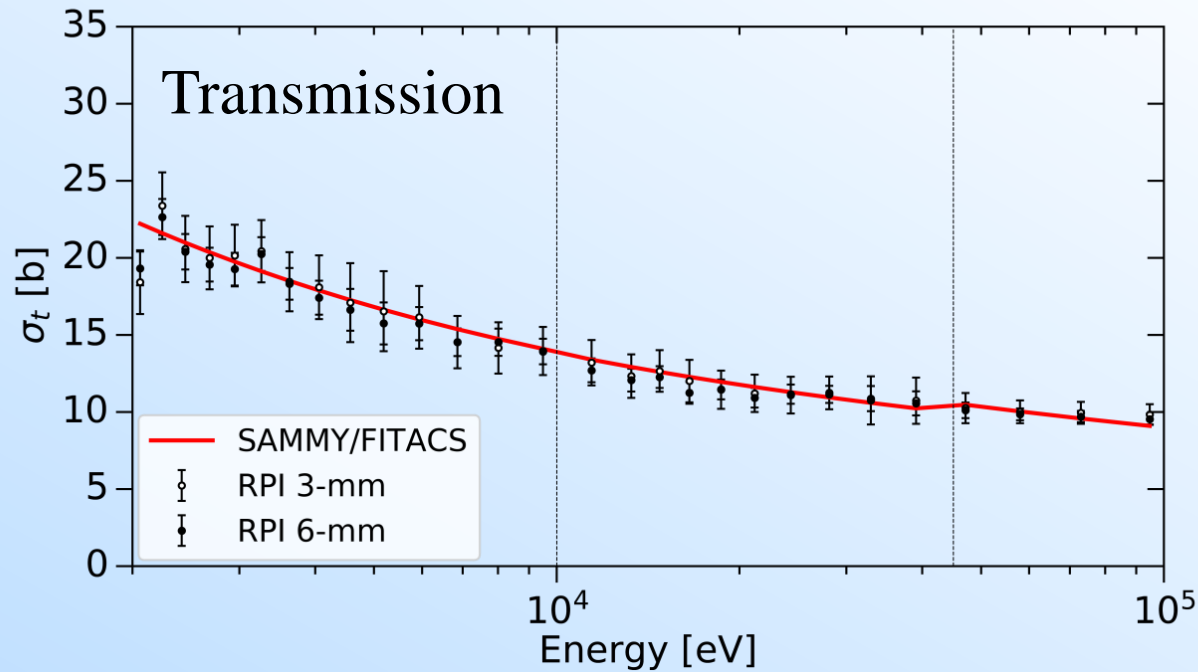


# We also generate an experimental covariance (Ta-181)

$$C_y = F_x C_x F_x^T = F_{x,\text{stat}} C_{x,\text{stat}} F_{x,\text{stat}}^T + F_{x,\text{sys}} C_{x,\text{sys}} F_{x,\text{sys}}^T$$

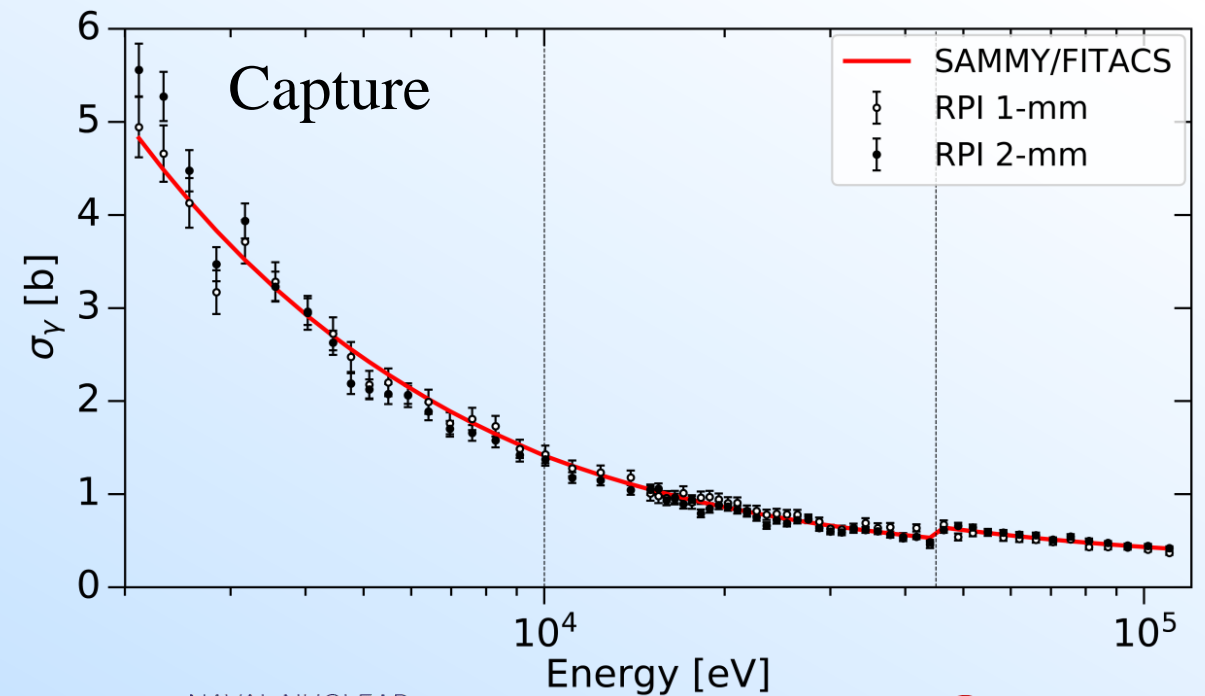


# Multi-Region URR (Ta-181)



## Fit options considered

1. One region URR
2. Multi Region URR
3. Extended RRR

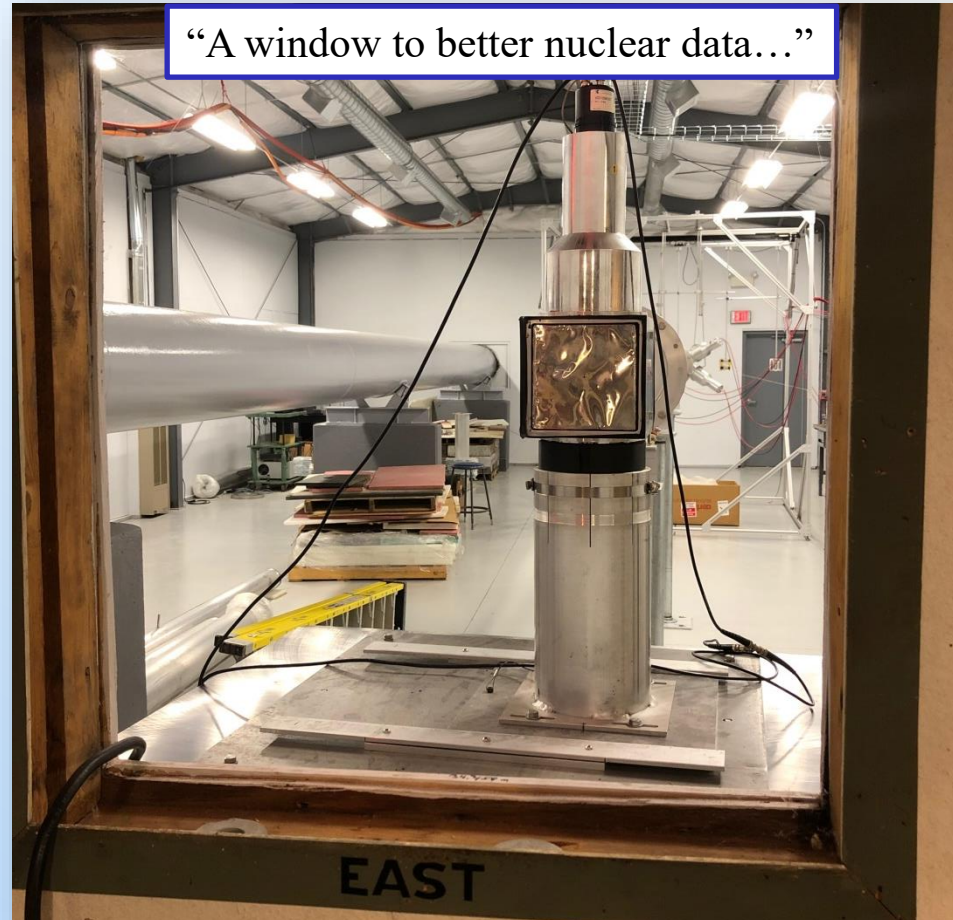




# Thick sample transmission for validation

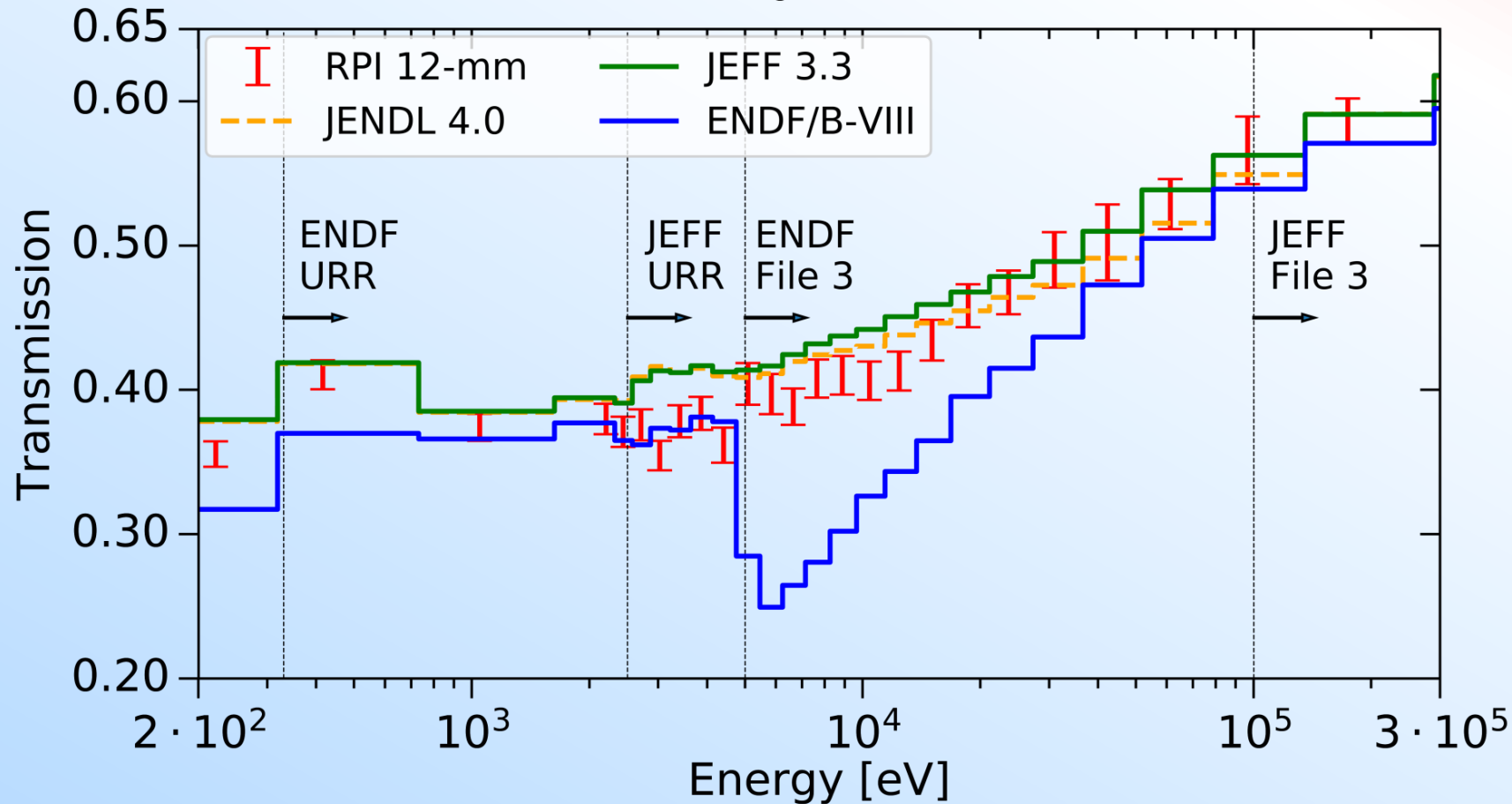
## $^6\text{Li}$ doped scintillating glass detector

- 2 PMT's viewing a light tight aluminum case



# Validation Transmission and evaluations

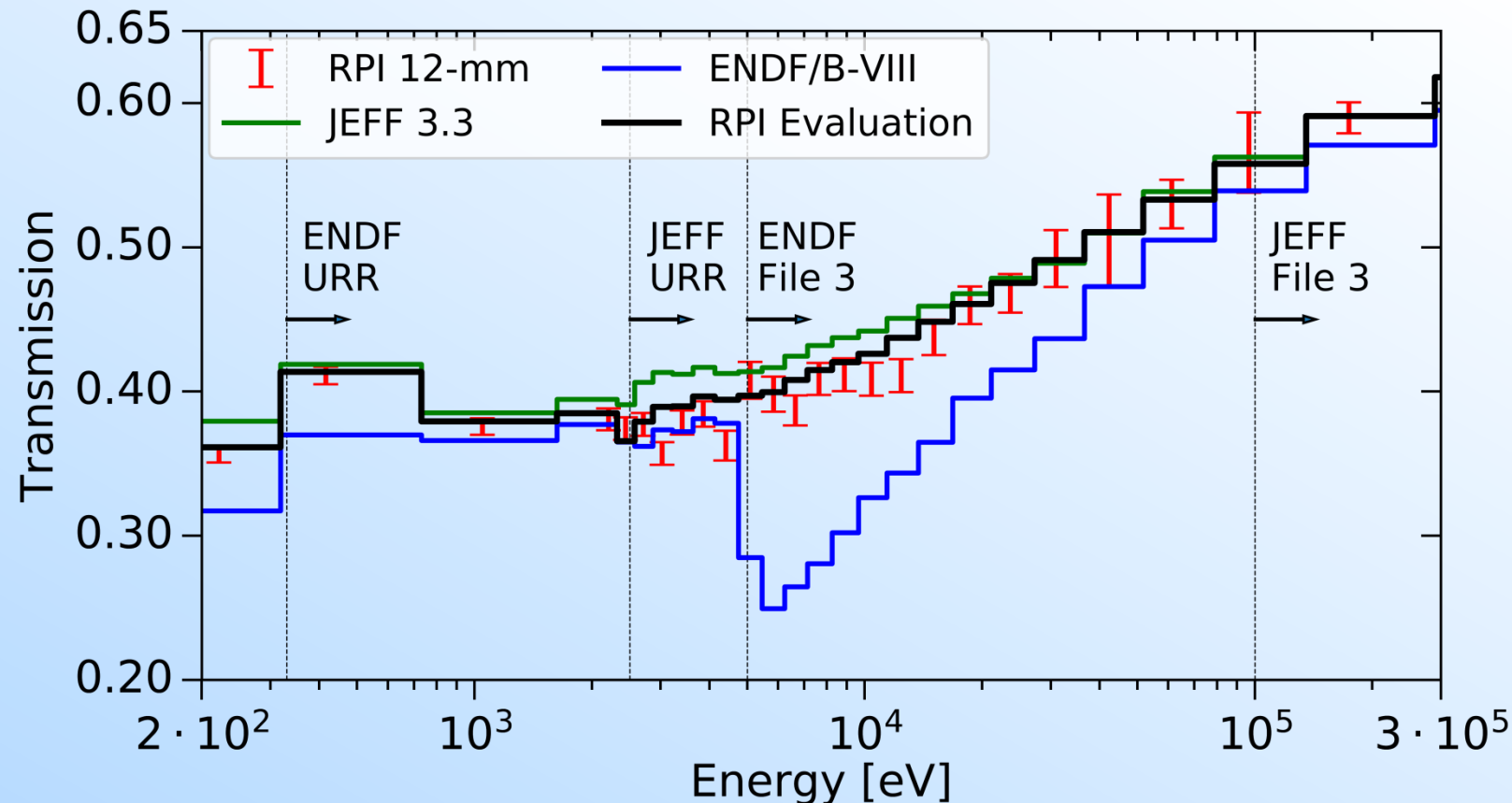
- Transmission for 12 mm sample grouped to have about 50 resonances per bin
- Observe the limitations of the URR treatment using JEFF-3.3, JENDL-4.0, and ENDF/B-VIII.0



Jesse M. Brown, R. C. Block, A. Youmans, H. Choun, A. Ney, E. Blain, D. P. Barry, M. J. Rapp and Y. Danon, "Validation of Unresolved Neutron Resonance Parameters Using a Thick-Sample Transmission Measurement", Nuclear Science and Engineering, vol. 194, no. 3, pp. 221-231, 2019, DOI:10.1080/00295639.2019.1688087

# RPI Evaluation: Updated JEFF-3.3

- Updated RRR and URR parameters
- Very sensitive to  $a_c$ ,  $D$  and other  $\langle Pars \rangle$
- **Using the RPI evaluation** we can improve agreement with measured data



# Validation of URR evaluations using transmission and self indication for $^{238}\text{U}$ and Ta

- Revisit the data in the PhD thesis of Byoun
  - Tae Y. Byoun, Experimental investigation of the resonance self-shielding and Doppler effect in uranium and tantalum, PhD thesis, Rensselaer Polytechnic Institute, Troy, N. Y. May 1973
- Data on EXFOR was recently updated and some errors were removed.
- Includes transmission and self-indication measurements for depleted U and Ta
- Temperature dependent measurements at 77, 295, and ~1000K

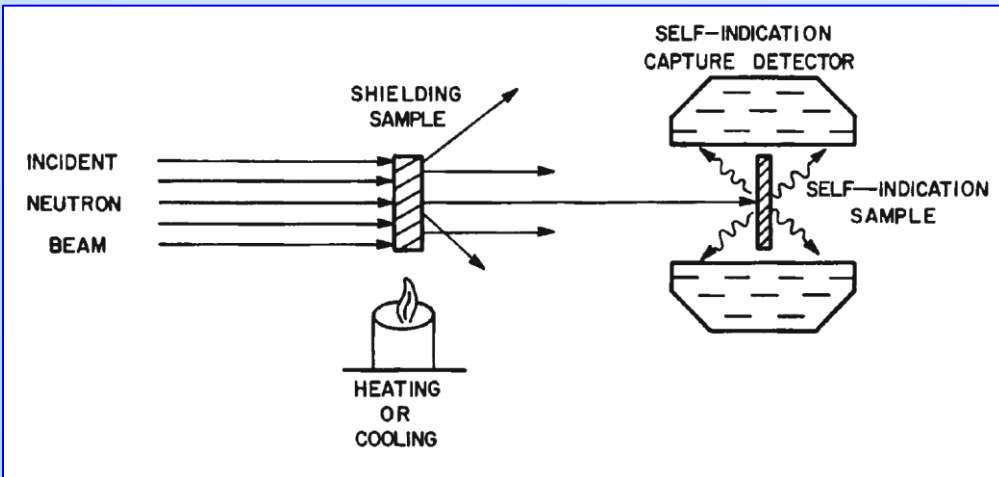


# What is self indication

- Self-indication is a transmission experiment where a capture detector with the sample material is used to detect neutrons
- Provides information on the self shielding effect in the sample material

## Transmission

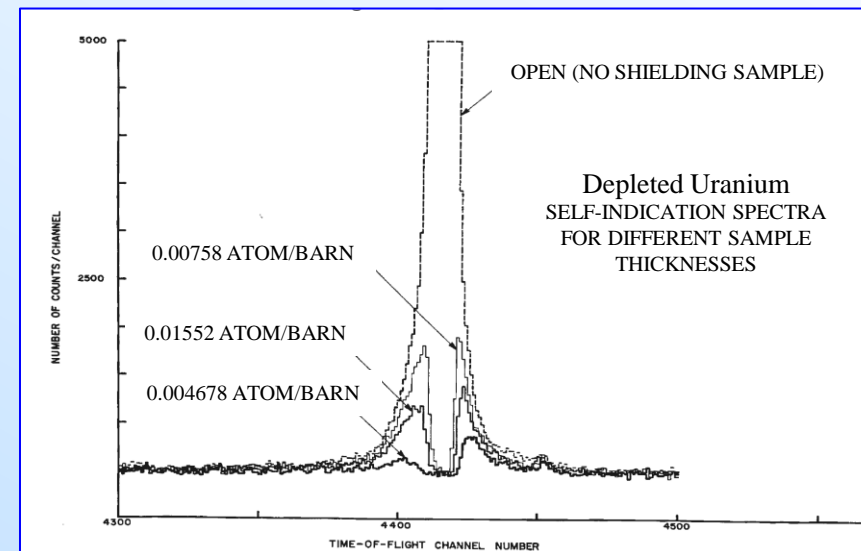
$$T = \int_{E_1}^{E_2} e^{-n\sigma_t(E)} dE$$



Figures from Byoun's PhD thesis

## Self-shielded capture yield

$$Y_{si} = \int_{E_1}^{E_2} \left(1 - e^{-n\sigma_t(E)}\right) \frac{\sigma_\gamma(E)}{\sigma_t(E)} e^{-n\sigma_t(E)} dE$$



# Self indication ratio (SIR)

Theory (1<sup>st</sup> order)

Experiment

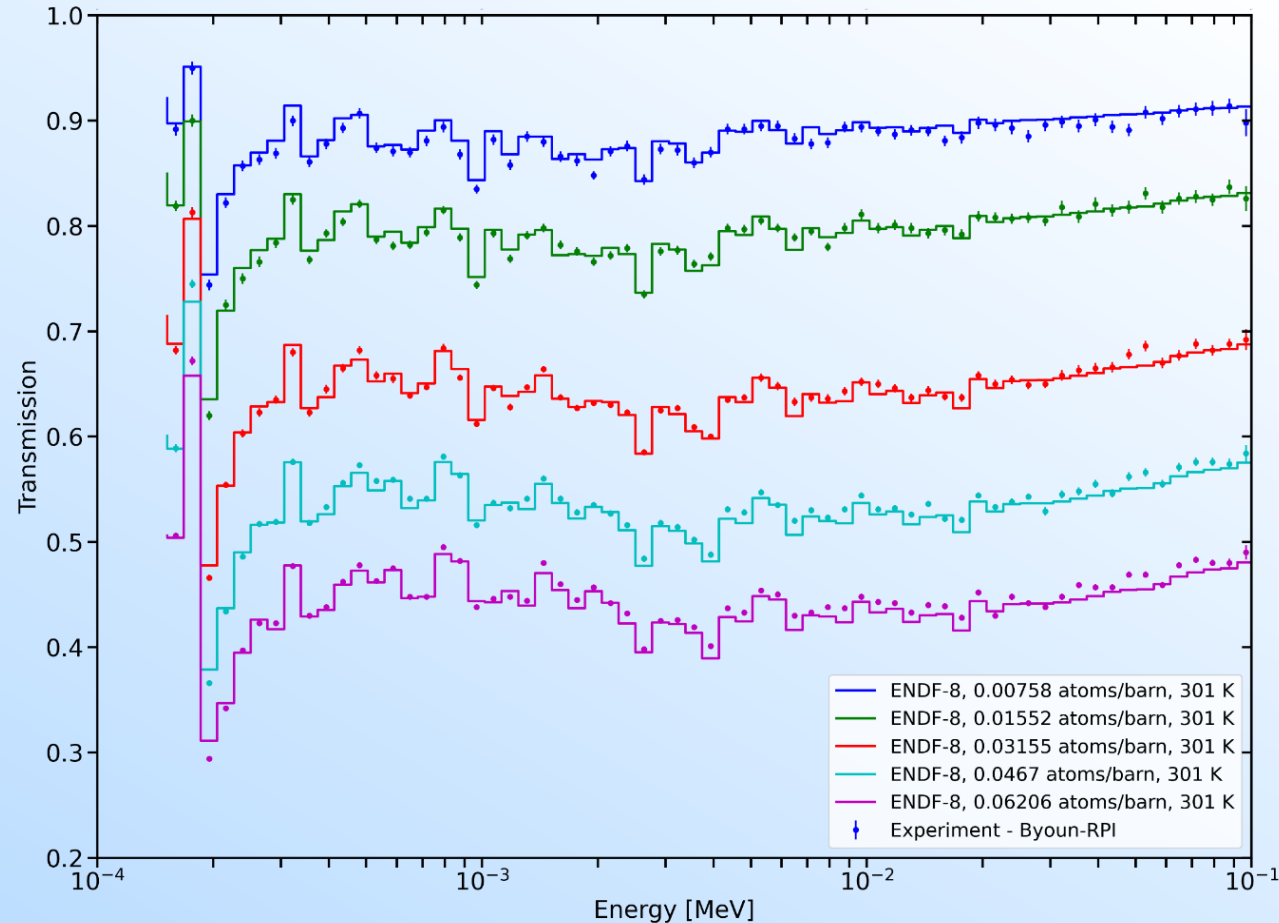
$$SIR = \frac{\int_{E_1}^{E_2} \left(1 - e^{-n\sigma_t(E)}\right) \frac{\sigma_\gamma(E)}{\sigma_t(E)} e^{-n\sigma_t(E)} dE}{\int_{E_1}^{E_2} \left(1 - e^{-n\sigma_t(E)}\right) \frac{\sigma_\gamma(E)}{\sigma_t(E)} dE}$$

$$SIR = \frac{\sum_{i=E_1}^{E_2} (C_i^s - B_i^s)}{\sum_{i=E_1}^{E_2} (C_i^0 - B_i^0)}$$

- $C_i^s$  and  $B_i^s$  are measurements with the transmission sample,  $C_i^0$  and  $B_i^0$  without it
  - Use of a thin capture sample can simplify the math
- Increased sensitivity to resonance wings and capture interactions
- Unlike capture measurements, normalization is not needed
- The temperature of the transmission sample can be changed

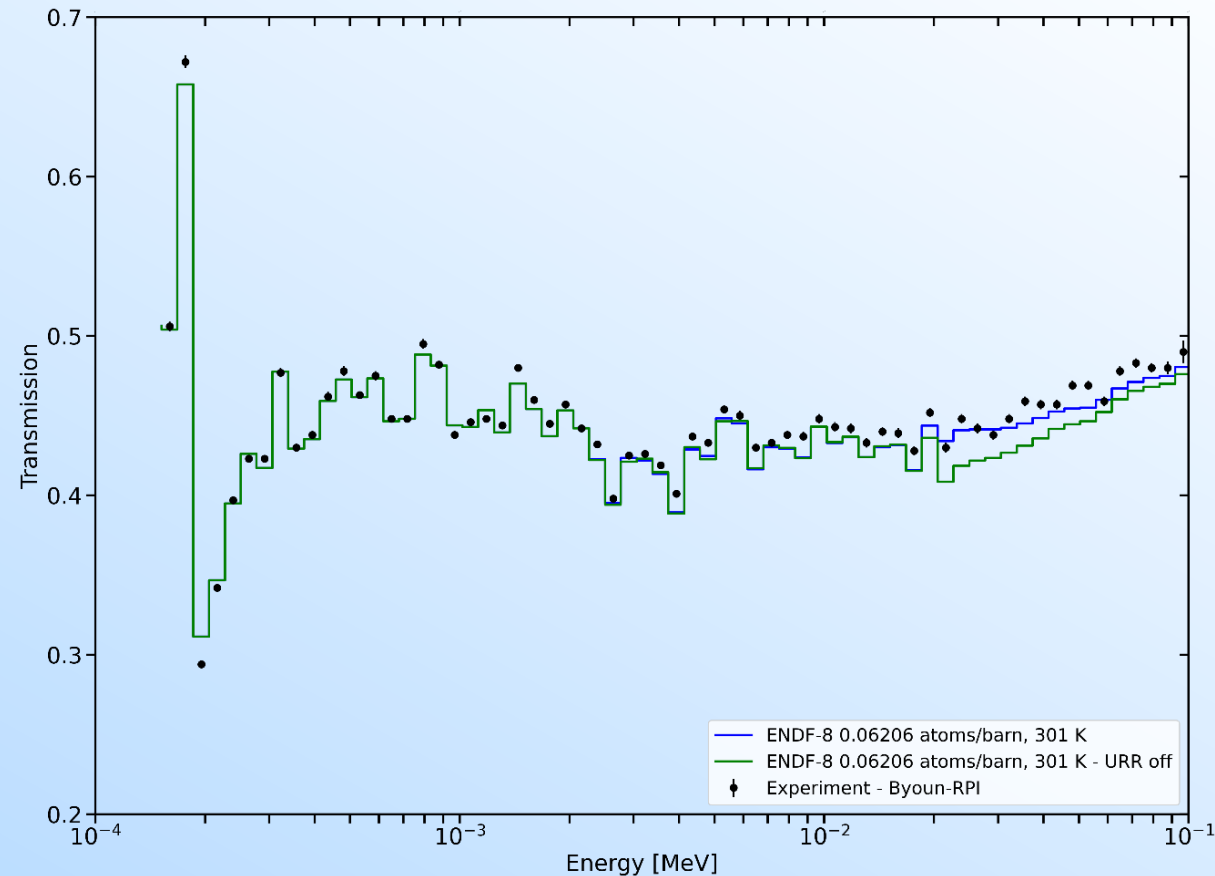
# $^{238}\text{U}$ transmission at room temp

- Overall good agreement between experiments and calculations including URR ( $E > 20$  keV)



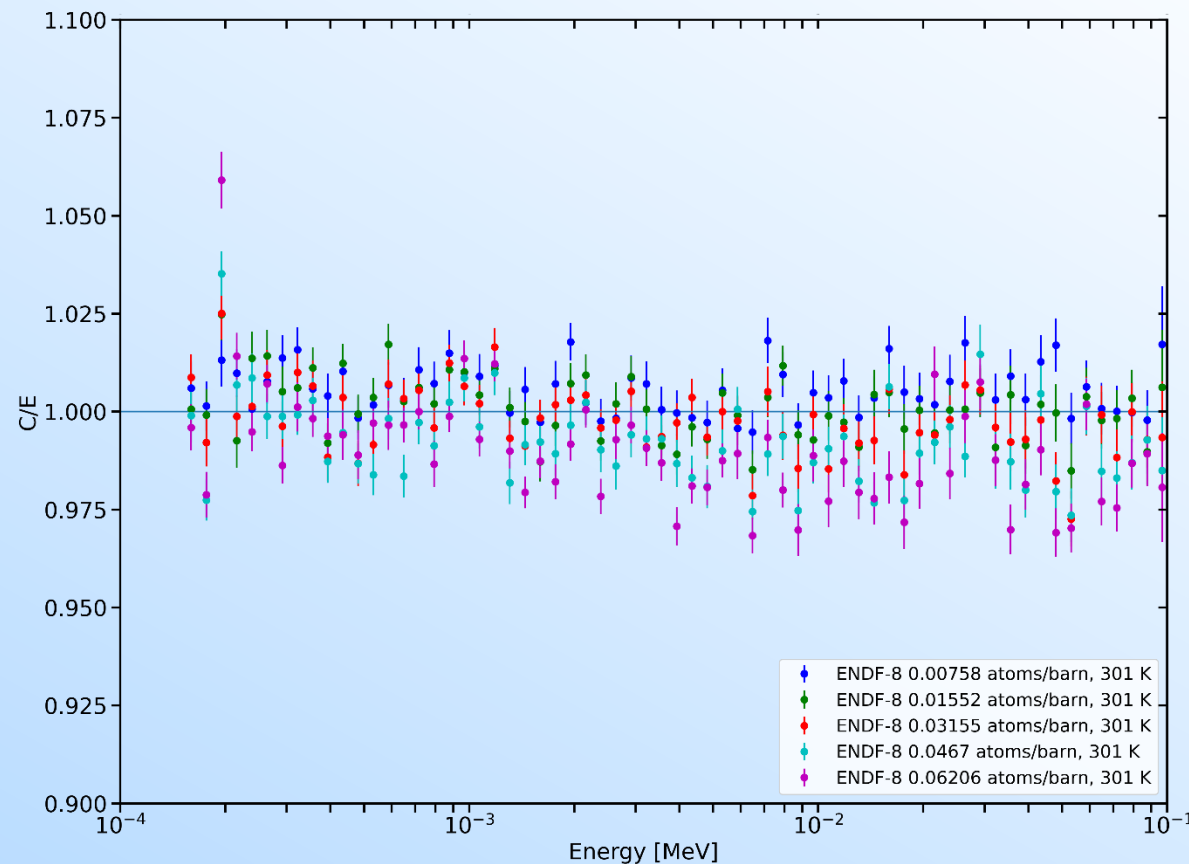
# $^{238}\text{U}$ effect of probability tables

- Probability tables are necessary to better match the experimental transmission URR
- Sample thickness and energy bin size affect the amount of resonance self shielding



# C/E for transmission

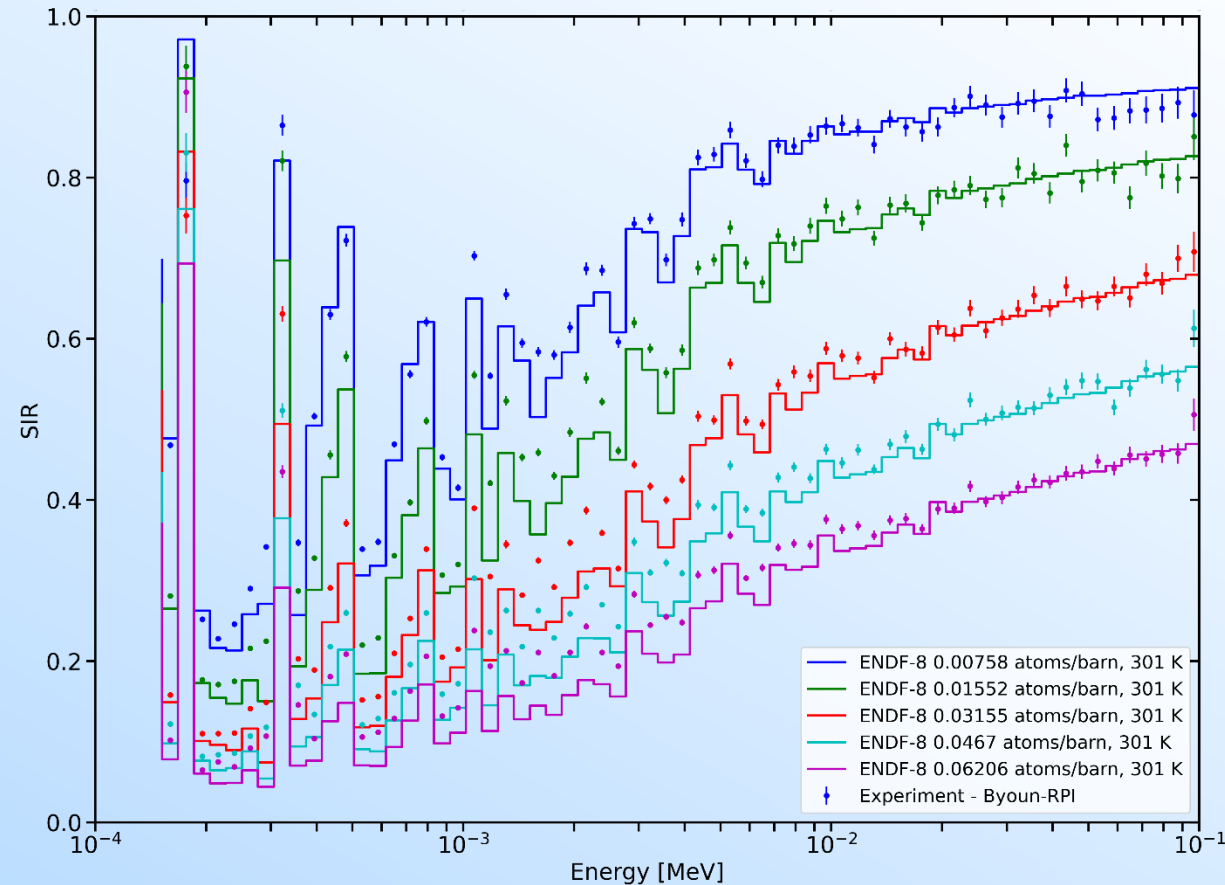
- Difference between evaluation and experiments is less than 2.5%
- The evaluation seems to get a bit lower with increasing thickness





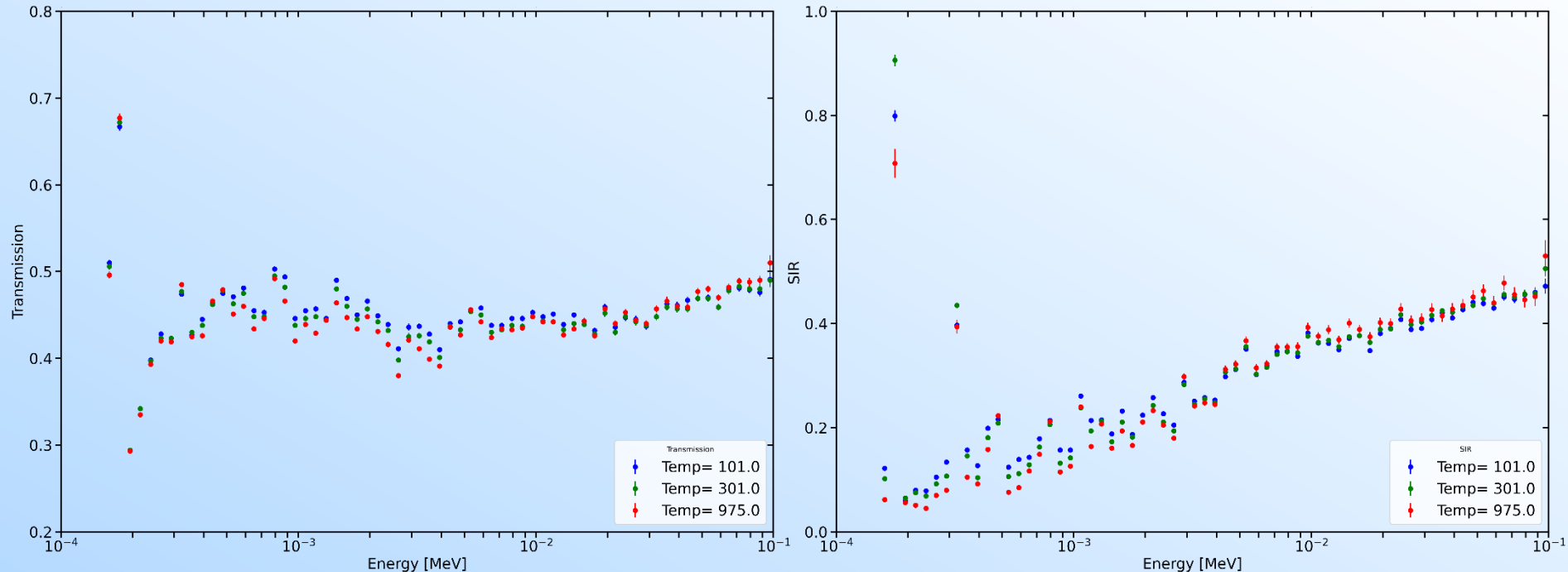
# $^{238}\text{U}$ Self indication ratio

- The evaluation and experiment seems to agree well in the URR
- Differences are visible in the RRR, possible issues with the capture cross section??



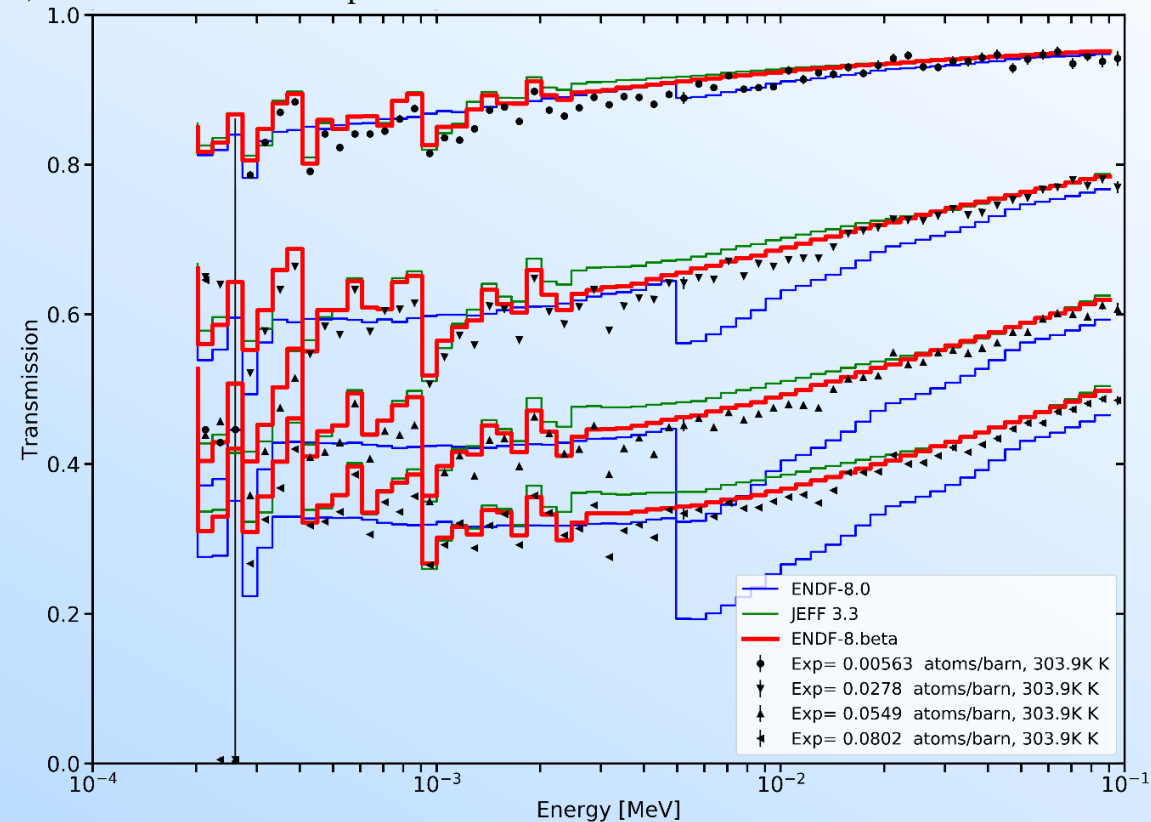
# U-238 at different temperatures

- Two effects: changes to both areal density (expansion) and self-shielding
  - Density: Temperature  $\uparrow \rightarrow N \downarrow \rightarrow$  Transmission  $\uparrow$
  - Self-shielding: Temperature  $\uparrow \rightarrow \sigma \rightarrow \sigma_{av} \rightarrow$  Transmission  $\downarrow$
- Larger changes observed in SIR, shows a clear trend with temperature with cross at about 5 keV
- Need to compare to evaluations...



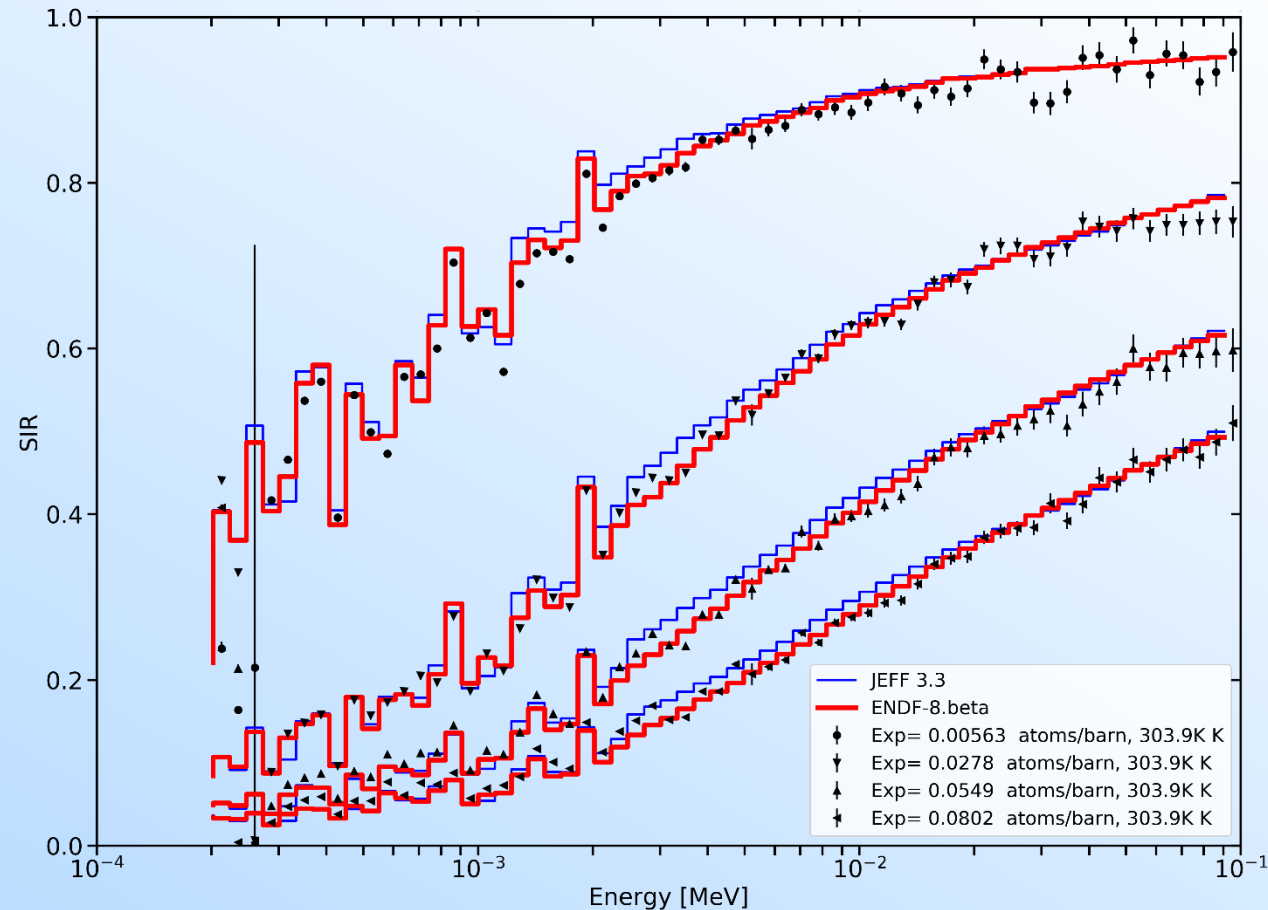
# Ta Transmission

- Known problem with ENDF-8
  - See: Jesse M. Brown, R. C. Block, A. Youmans, H. Choun, A. Ney, E. Blain, D. P. Barry, M. J. Rapp and Y. Danon, “Validation of Unresolved Neutron Resonance Parameters Using a Thick-Sample Transmission Measurement”, Nuclear Science and Engineering, vol. 194, no. 3, pp. 221-231, 2019, DOI:10.1080/00295639.2019.1688087
- JEFF-3.3 and ENDF-8.beta show good agreement in URR
- ENDF-beta seems to agree better, but there is room for improvement



# Ta- Self indication ratio

- The new ENDF evaluation shows better agreement in the URR



# Conclusions

- URR experiment provide average transmission or yield data that can be used for evaluations
- Self shielding correction of the experiment might be needed (depends mostly on sample thickness)
- High energy resolution experiments in the URR can produce a less self shielded average cross section but methodology for correction in requires more knowledge of the experiment
- Examples for  $^{181}\text{Ta}$ ,  $^{238}\text{U}$  including transmission, capture and self indication were provided.
- A method for URR cross section validation using a thick sample was demonstrated Brown et al.