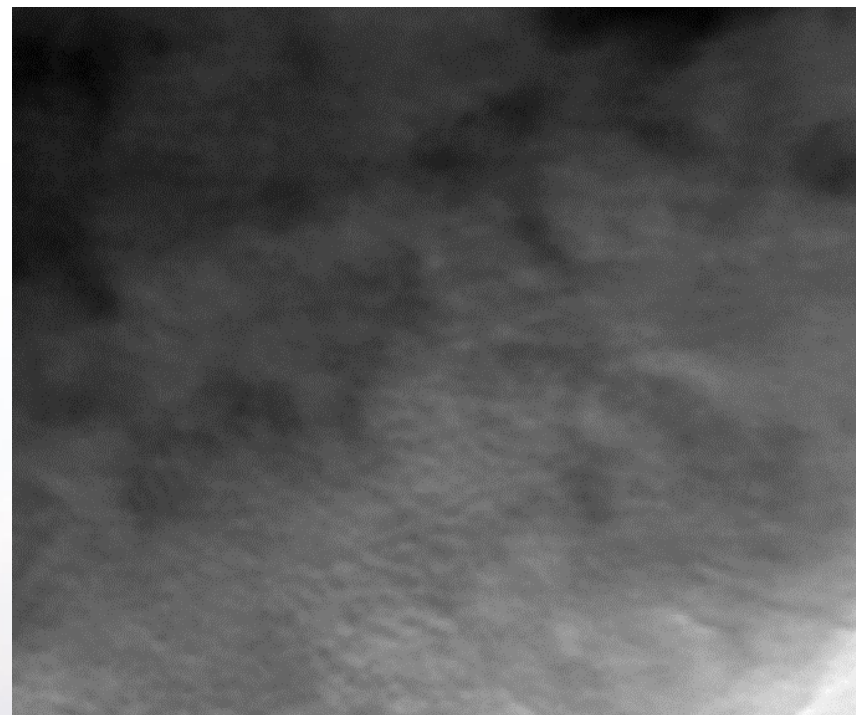
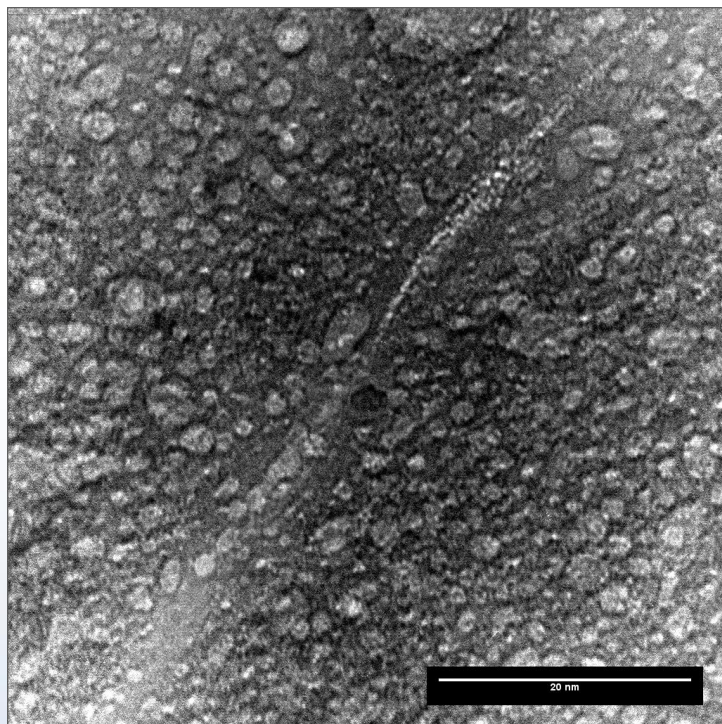


In-situ Investigation of Amorphization and Microstructural Changes in LiAlO_2

N. Madden, R. Schoell, E.J. Lang, K. Hattar

Sandia National Laboratories

September 2022



Utilizing *In situ* TEM microscopy to deconvolute governing environments and elucidate the underlying mechanisms.



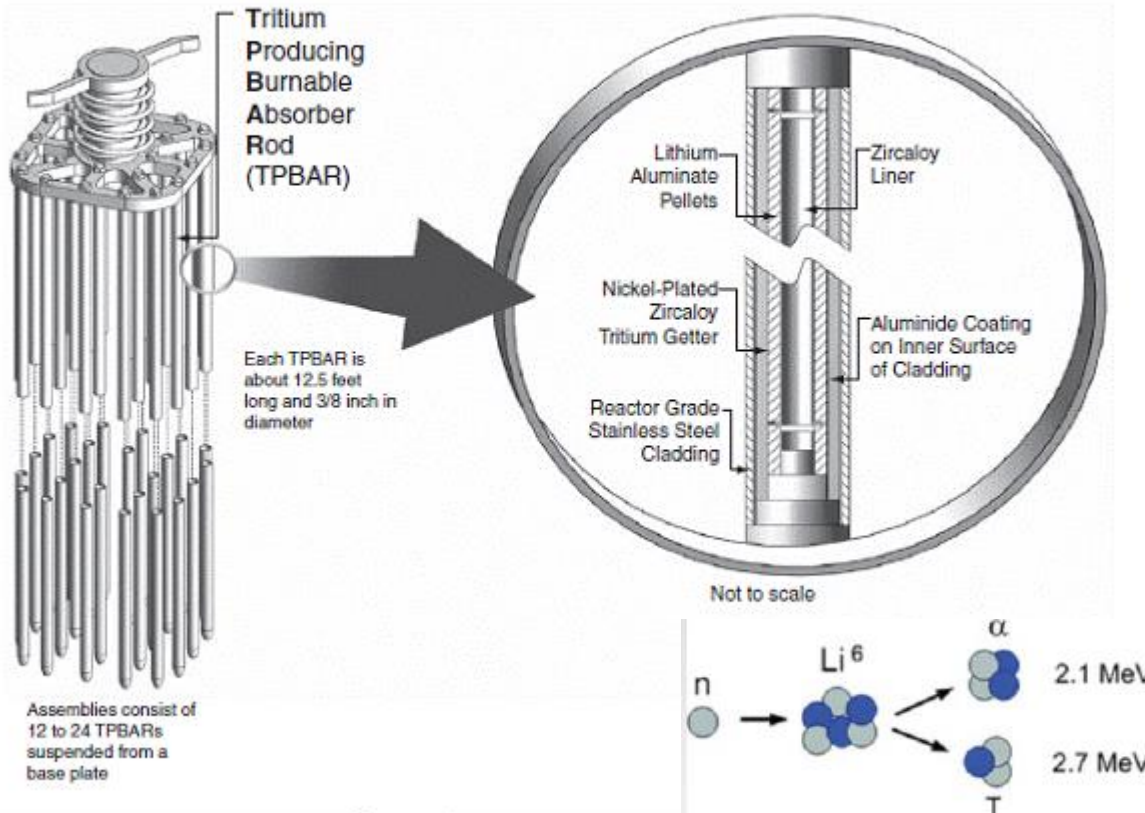
Sandia National Laboratories is a multi-mission laboratory managed and operated by National Technology & Engineering Solutions of Sandia, LLC., a wholly owned subsidiary of Honeywell International Inc., for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-NA0003525.



Sandia National Laboratories

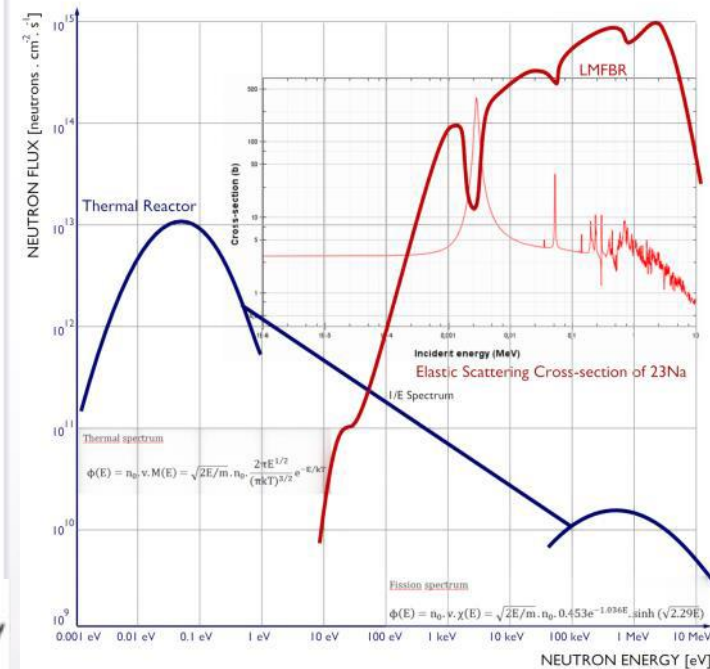
Sandia National Laboratories is a multi-mission laboratory managed and operated by National Technology & Engineering Solutions of Sandia, LLC., a wholly owned subsidiary of Honeywell International Inc., for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-NA0003525.

TPBAR Design & Reactor Environment



Source: f4h05A

Tritium Producing Burnable Absorber Rod (TPBAR)

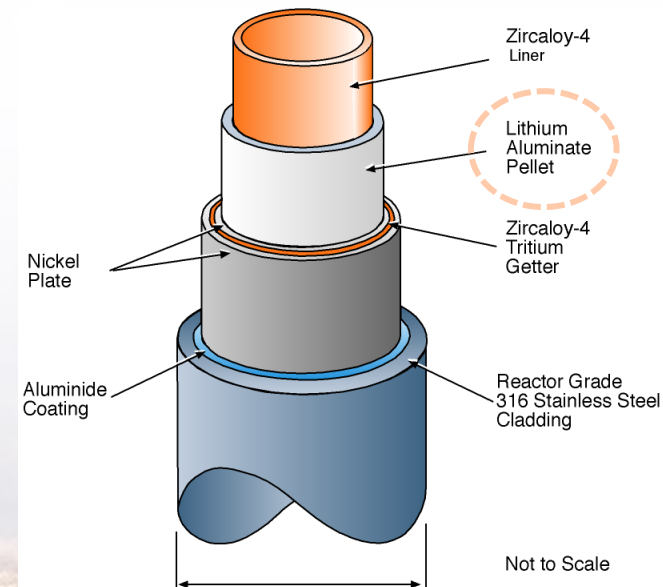
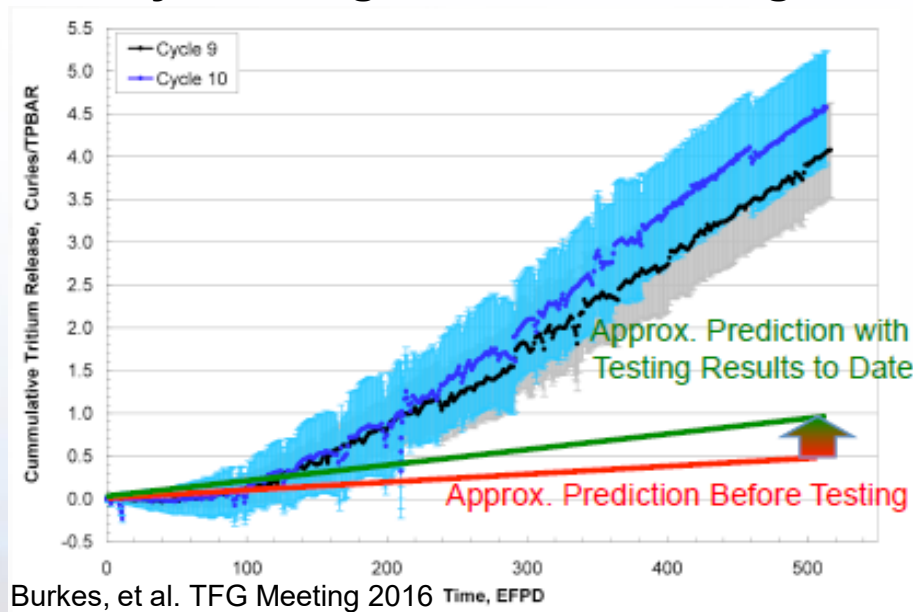


Extremely Complex Environment: Displacement Damage, Tritium Production, Helium Production, Mechanical Stress & Elevated Temperatures



Understanding Tritium Permeation in TPBAR

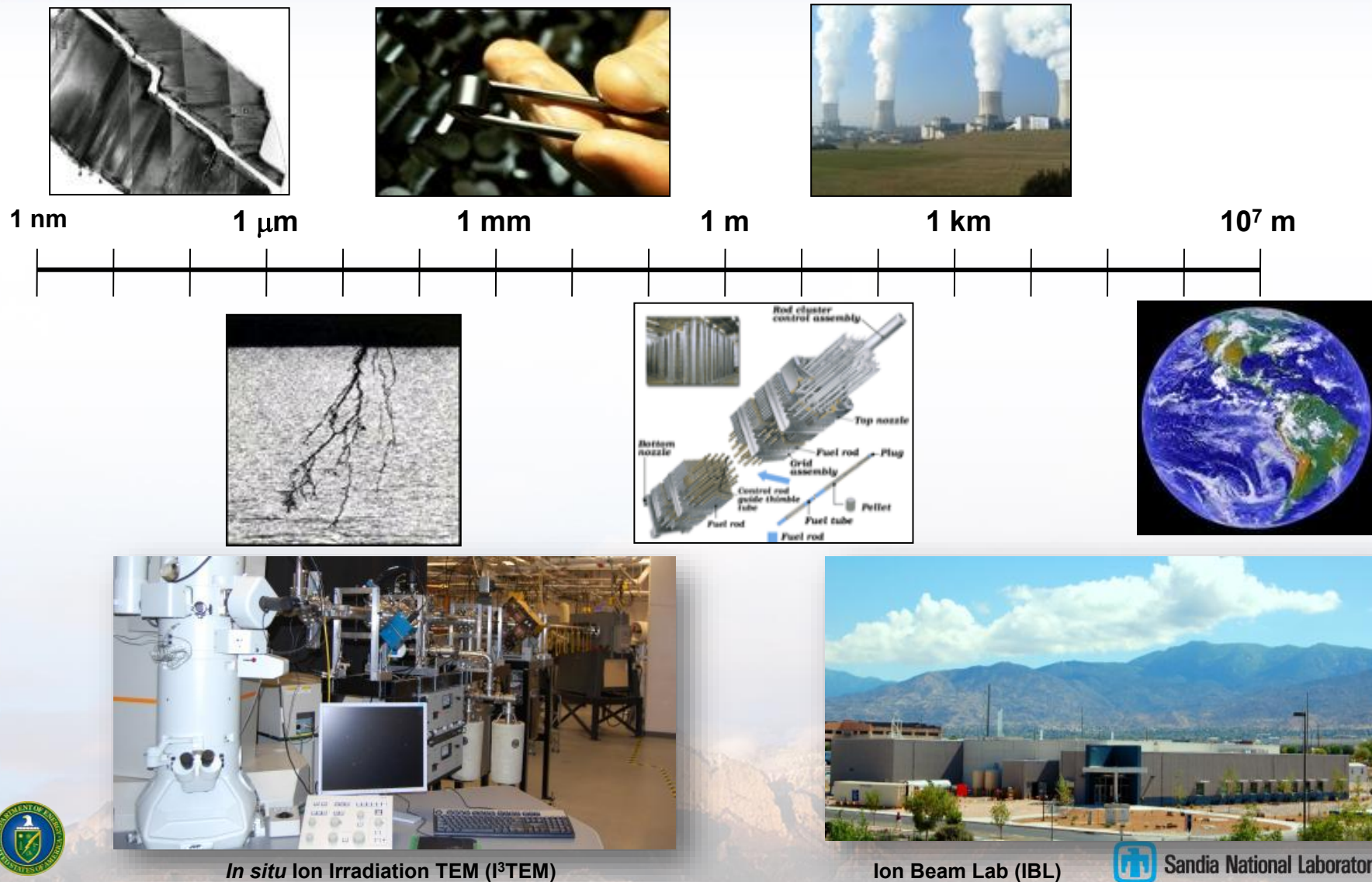
- TPBAR ^3H permeation is higher than predictive performance models
 - In 2004, during Cycle 6, the predicted levels were ~ 0.5 Ci/TPBAR/cycle and actual levels were ~ 4 Ci/TPBAR/cycle (0.04% of total ^3H produced)
- Mechanisms responsible for differences between predictions and observations are not well understood
- Currently building an understanding of fundamental ^3H -He-defect interactions



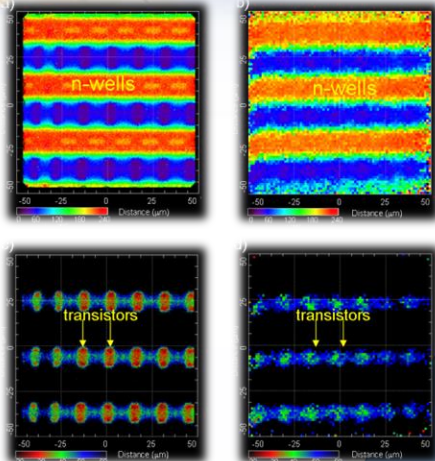
Simulating neutron irradiation in a reactor is complicated, and TPBAR adds the complication of ^3H production



Investigating the **nm** Scale to Understand the **km** Scale



Sandia's Ion Beam Laboratory



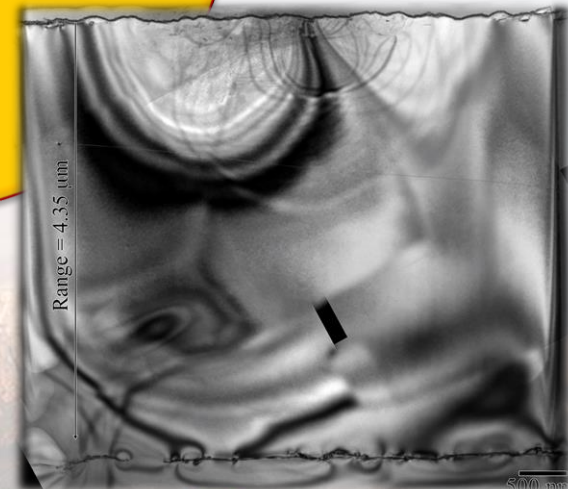
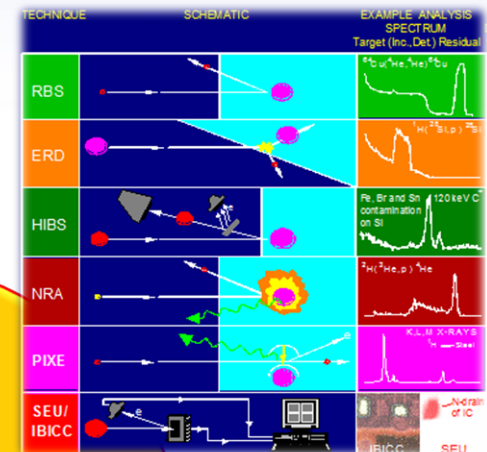
Ion Beam Analysis (IBA)

Radiation Effects
Microscopy (REM)



Ion Beam
Modification (IBM)

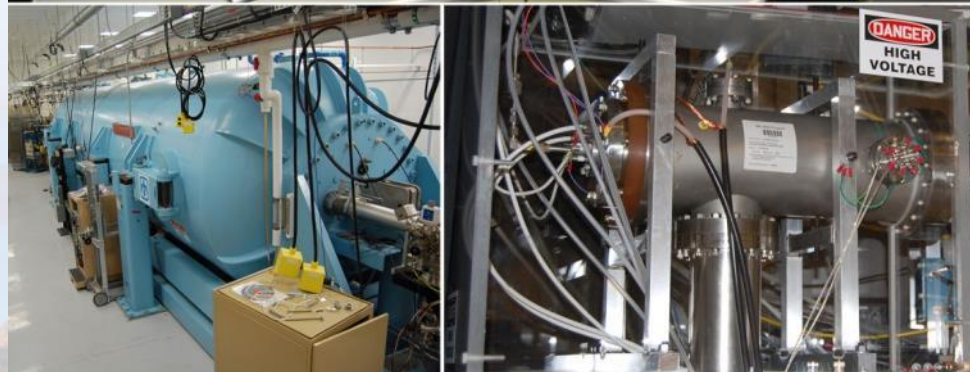
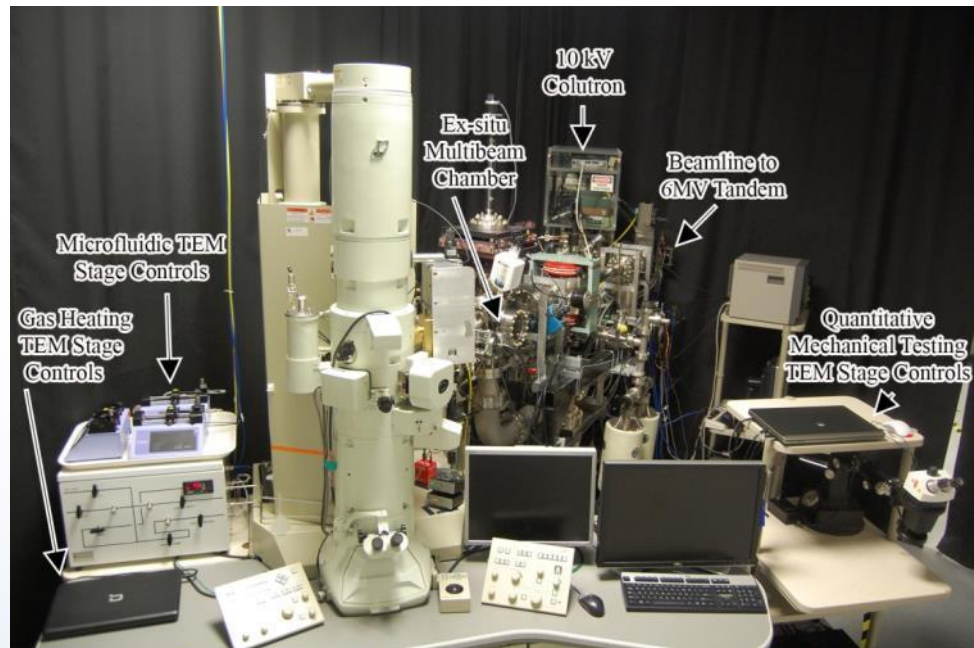
In situ Ion Irradiation
Transmission Electron
Microscopy
(I³TEM)



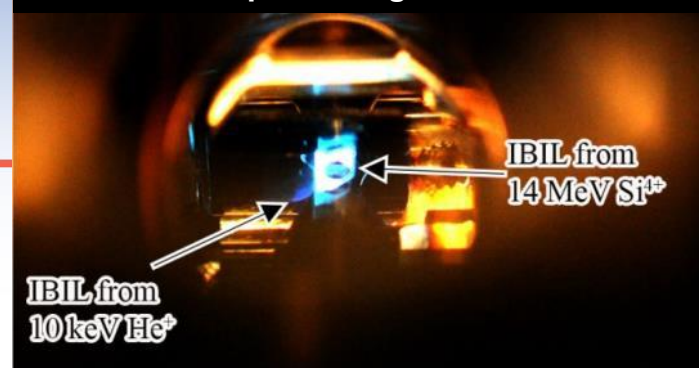
Sandia's Concurrent *In situ* Ion Irradiation TEM Facility

Collaborator: D.L. Buller

10 kV Colutron - 200 kV TEM - 6 MV Tandem

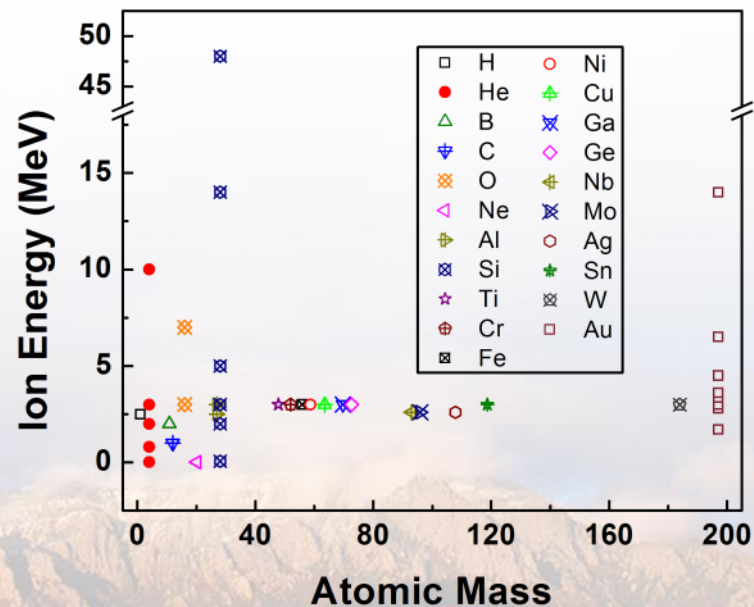


IBIL from a quartz stage inside the TEM



Direct real time observation of ion irradiation, ion implantation, or both with nanometer resolution

Ion species & energy introduced into the TEM



Benefits & Limitations of *in situ* TEM

Benefits

1. Real-time nanoscale resolution observations of microstructural dynamics

Limitations

1. Predominantly limited to microstructural characterization
 - Some work in thermal, optical, and mechanical properties
2. Limited to electron transparent films
 - Can often prefer surface mechanisms to bulk mechanisms
 - Local stresses state in the sample is difficult to predict
3. Electron beam effects
 - Radiolysis and Knock-on Damage
4. Vacuum conditions
 - 10^{-7} Torr limits gas and liquid experiments feasibility
5. Local probing
 - Portions of the world study is small

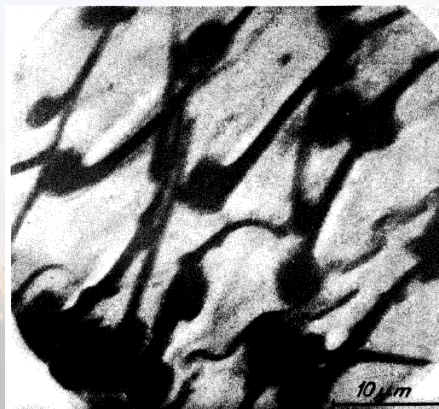


Fig. 6: Wing surface of the house fly.
(First internal photograph, $U = 60$ kV, $M_s = 2200$)
(Dietel, E. and Müller, H.O.: Z. Wiss. Mikroskopie 52, 53-57 (1955))

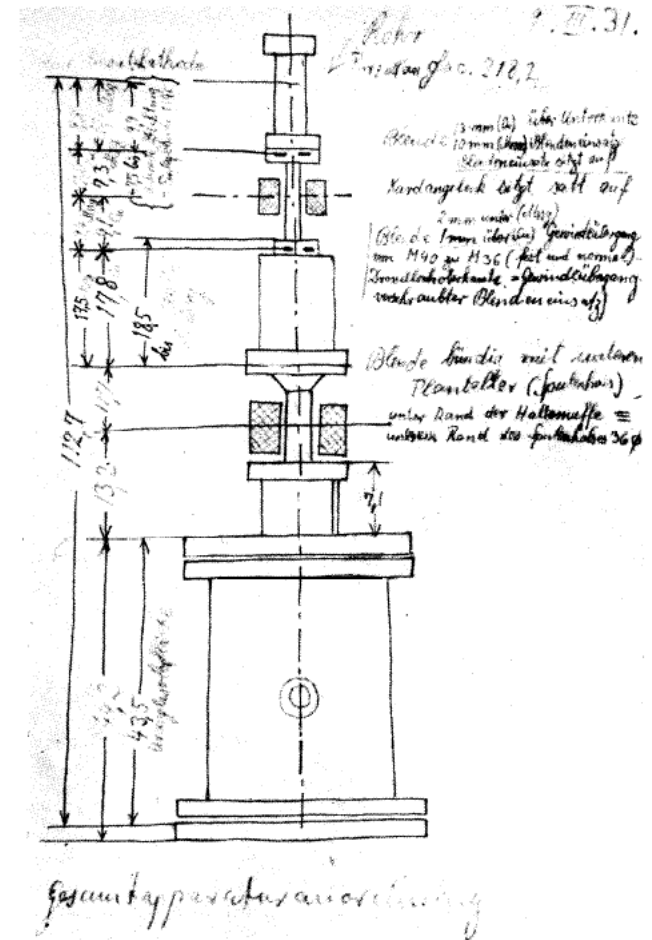
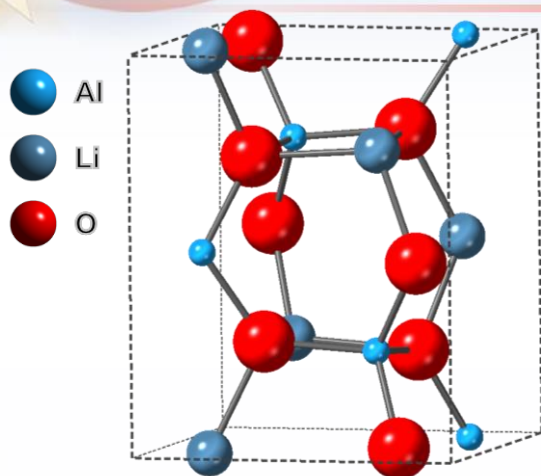


Fig. 2: Sketch by the author (9 March 1931) of the cathode ray tube for testing one-stage and two-stage electron-optical imaging by means of two magnetic electron lenses (electron microscope) [8].



LiAlO₂ Background

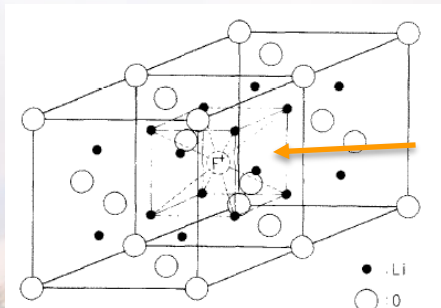


Previous Work

- Structural defects
 - Luo *et al* JNM 372 (2008) 53-58
- Volume swelling
 - Noda JNM 179-181 (1991) 37-41
- ³H detrapping
 - Oyaidzu *et al* JNM 375 (2008) 1-7
- Gas diffusion and release
 - Raffray *et al* JNM 210 (1994) 143-160

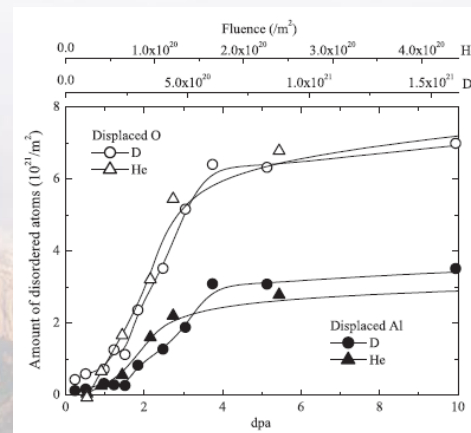
γ -LiAlO₂ is tetragonal (SG: P 41 21 2)

- H isotopes are thought to trap in oxygen vacancies
 - ²H release occurs at the same temperature as defect annealing in implanted LiAlO₂



Can be determined with luminescence

Noda JNM 179-181 (1991) 37-41



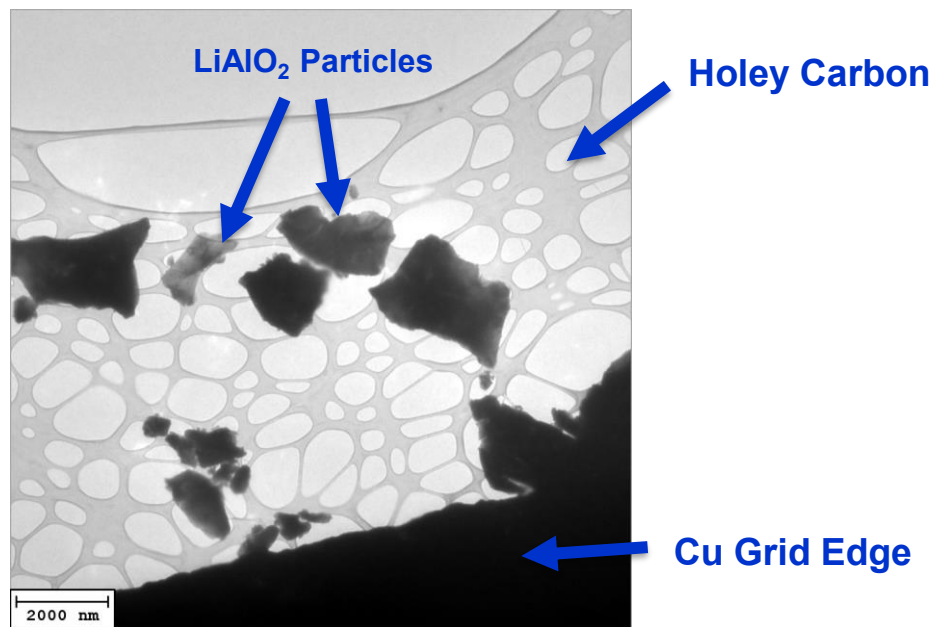
Katsui *et al.*
NIMB 268 (2010)
2735-2739

National Laboratories



Previous work investigated LiAlO_2 powders and In-situ Ion Irradiation

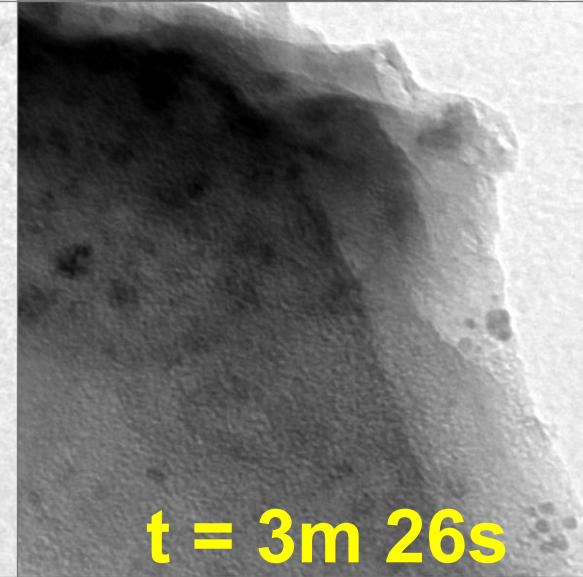
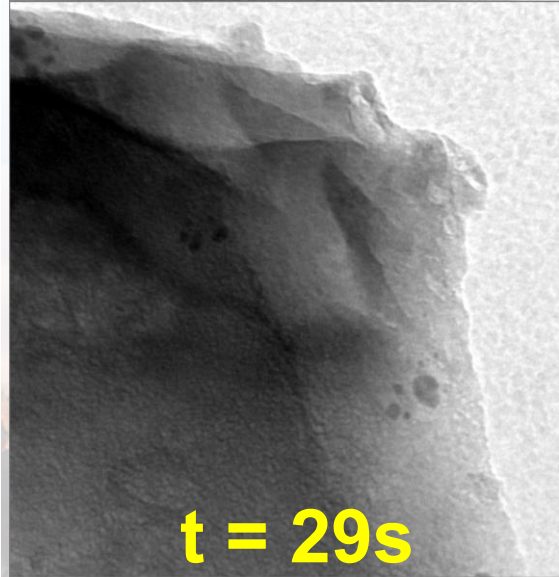
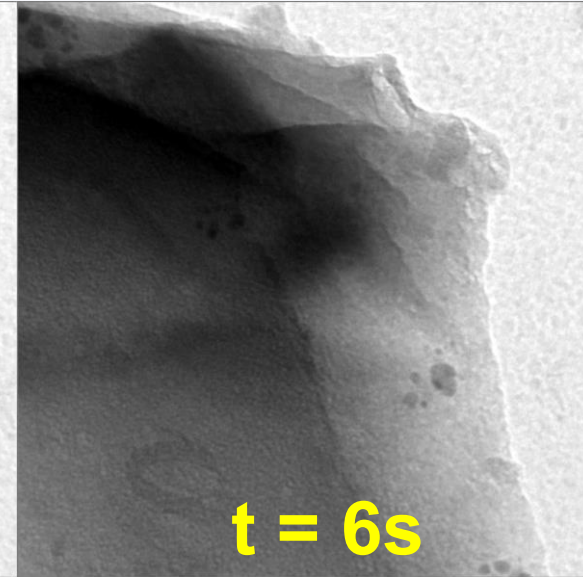
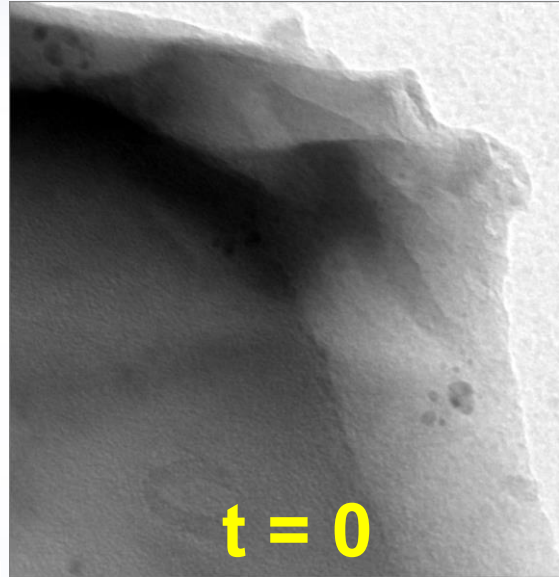
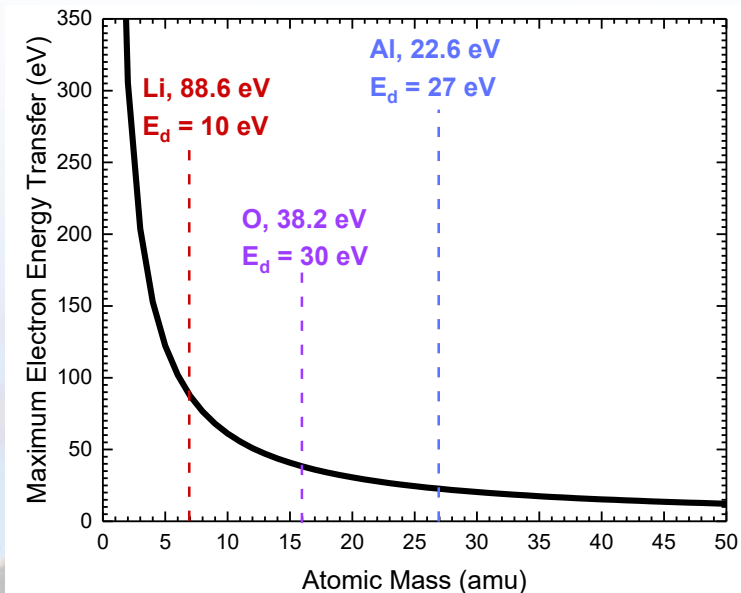
- Powders were drop-cast onto TEM grids



- Samples were heated to **310°C** using Hummingbird HT stage
- Four sets of irradiations:
 - 200 keV electrons → simulates Gamma (γ) irradiation
 - 200 keV e^- + 10 keV He → simulates γ and He accumulation from ^6Li transmutation and ^3H decay
 - 200 keV e^- + 10 keV He + 5 keV D → simulates γ , He, and ^3H interaction
 - 200 keV e^- + 10 keV He + 5 keV D + **1.7 MeV Au** → simulates γ , gas build-up + cascades

Prior Work Showed Electrons Alone Induced Void Growth

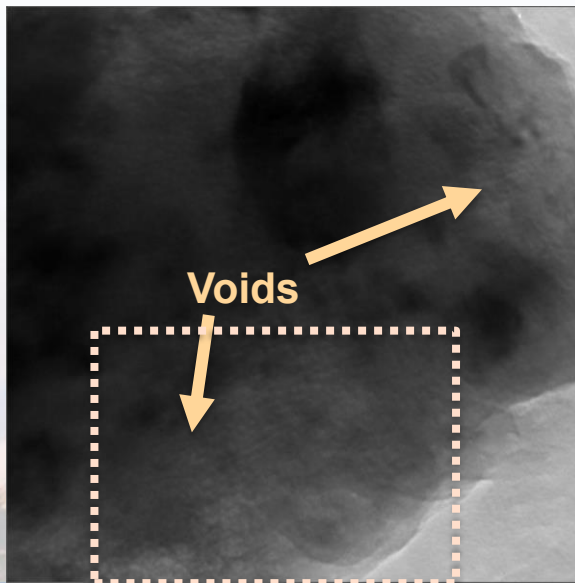
- Voids were observed to form under the electron beam in several particles
- Rate of void formation is not consistent between particles
- Possibly due to electron beam displacing Li and O atoms



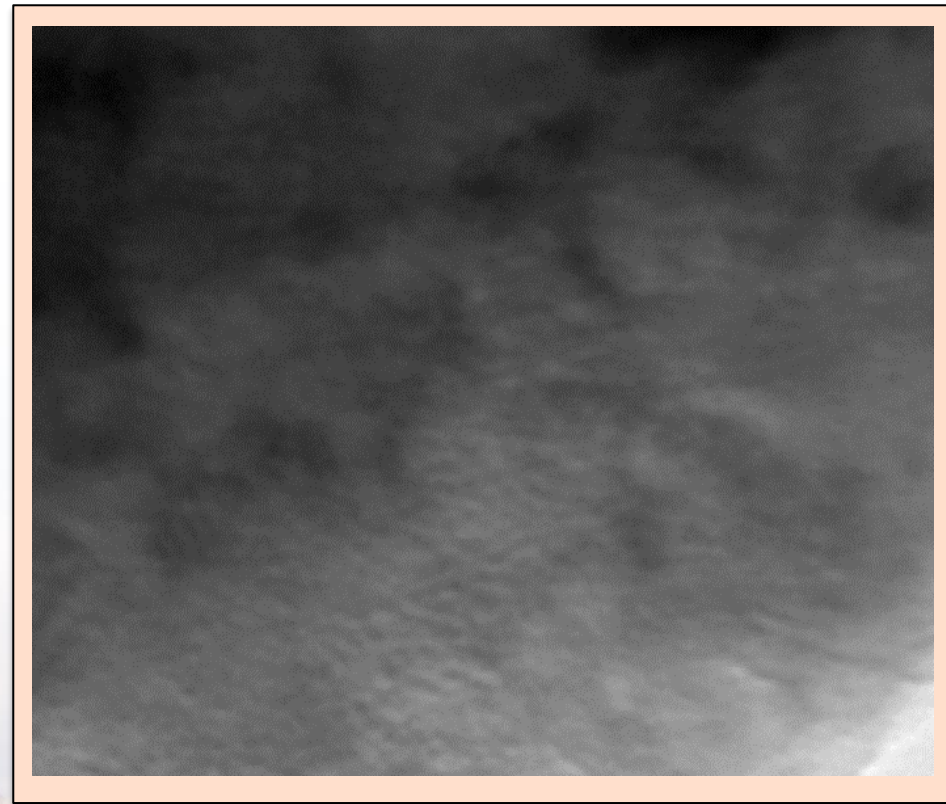
In-situ TEM He implantation at 310°C

- Each frame = 1 min of irradiation
- Because the voids are difficult to see in powders, I paused the video at a few points to show overfocus images
- Electron beam on for most of experiment
- Bubbles formed after ~13 min (1.5×10^{17} He/cm²)

After Irradiation



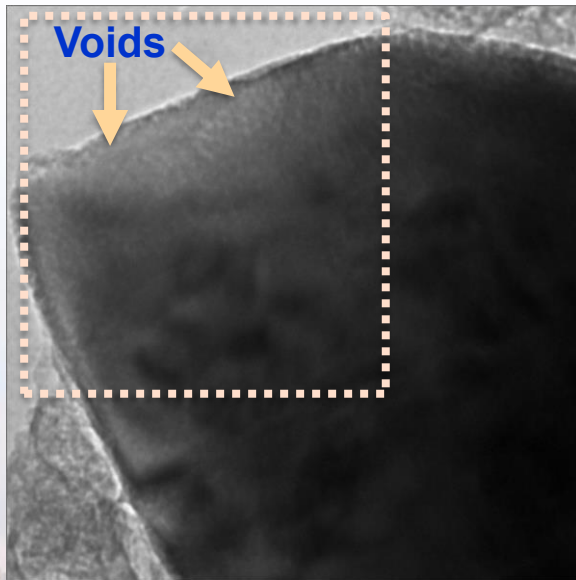
In-situ Video



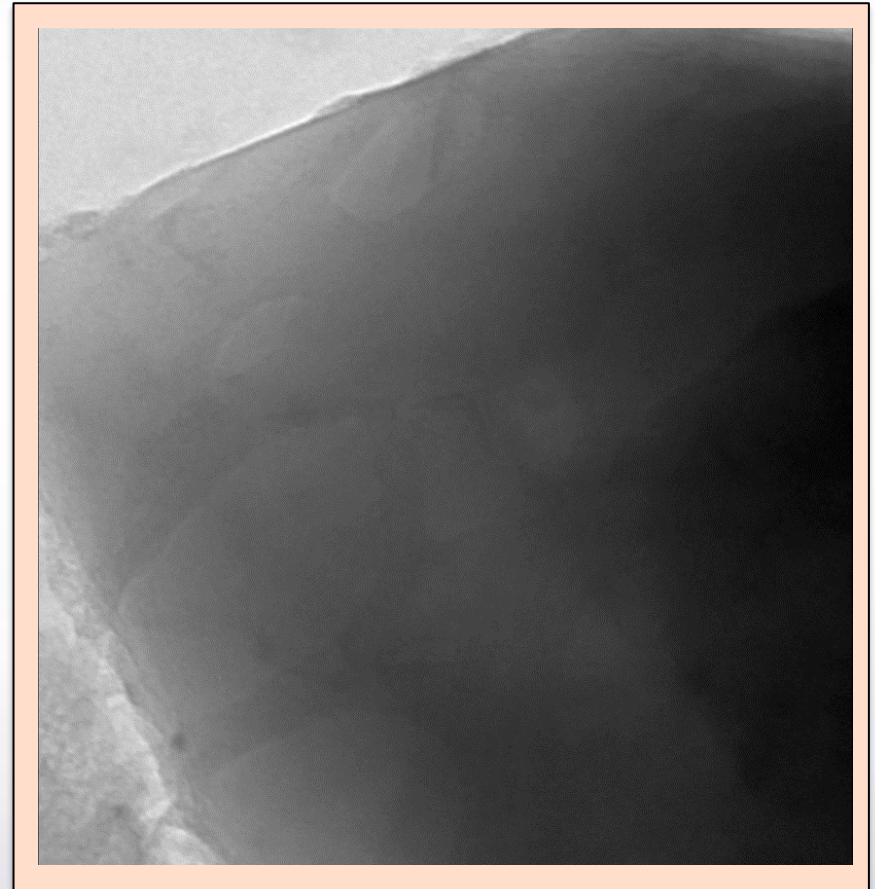
In-situ TEM He & D irradiation at 310°C

- Each frame = 5 min of irradiation
- All underfocus images
- Electron beam was off except for imaging
- Bubbles formed after ~60 min (1.7×10^{17} He/cm², 3.4×10^{17} D/cm²)

After Irradiation



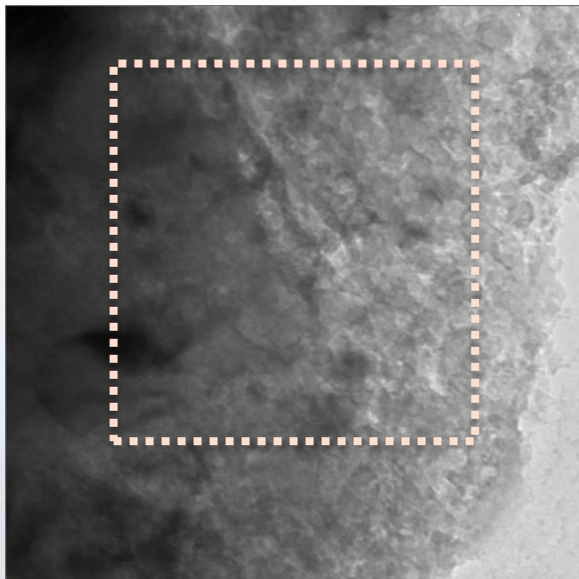
In-situ Video



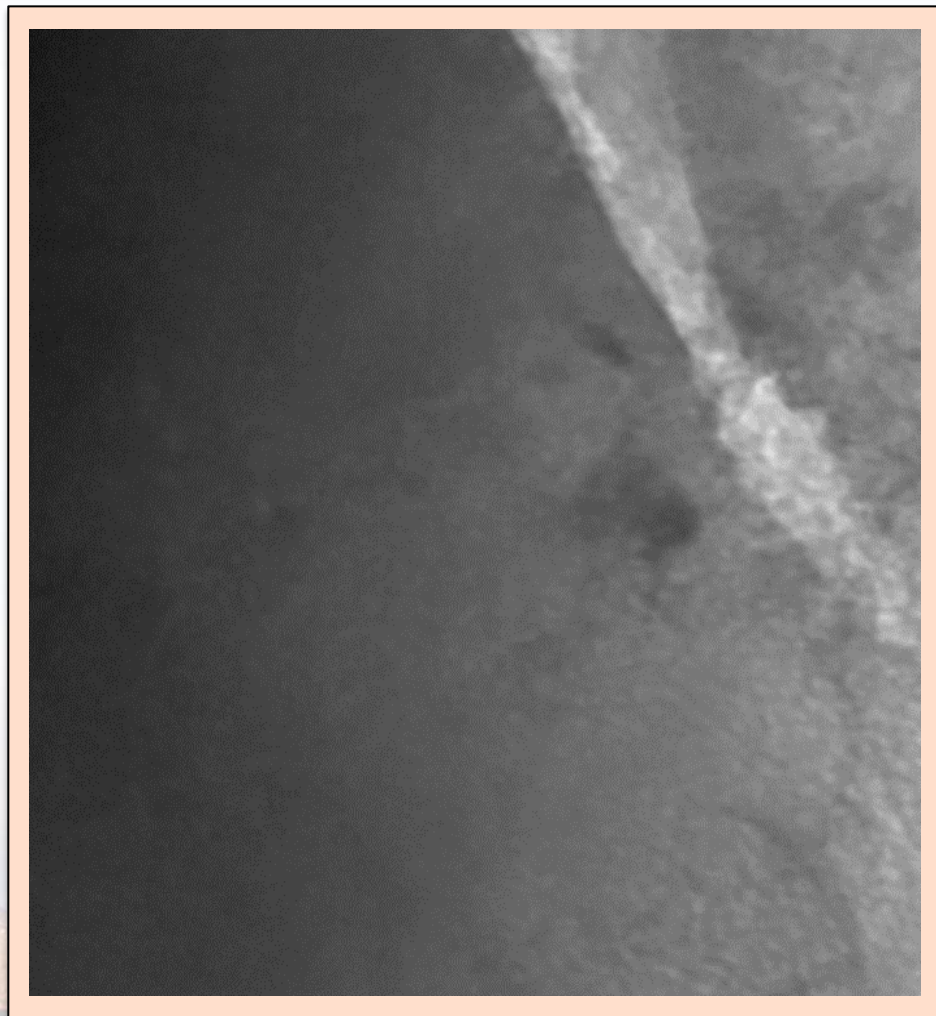
In-situ TEM He, D, & Au at 310°C

- Each frame = 5 min of irradiation
- Pre-existing voids could have an effect on this final microstructure
- Electron beam was on for most of the experiment

After Irradiation

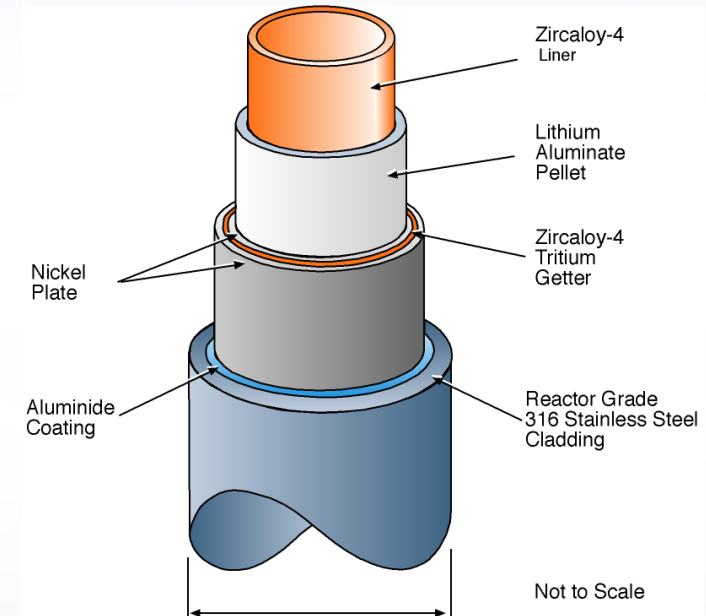


In-situ Video



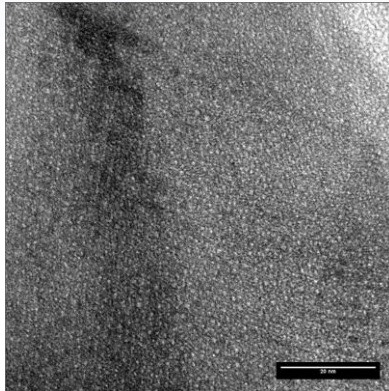
Prior work performed on powders: what affect does bulk material have?

- Effect of crystal orientation and grain boundaries
- TEM lamella of single-crystal and polycrystalline lithium aluminate prepared
- Lamella irradiated ex-situ with He and H
 - Room temperature
 - 10 keV H/He
 - Single Crystal: 10 keV He to a-b) 1×10^{17} and c-d) 5×10^{18}
 - ♦ Post-implantation annealing to 200 ° C and 450 ° C
 - Polycrystalline: 10 keV H to a-d) and 10 keV He to a-b) 1×10^{17} and c-d) 5×10^{18}
 - ♦ Post-implantation annealing to 200 ° C and 450 ° C

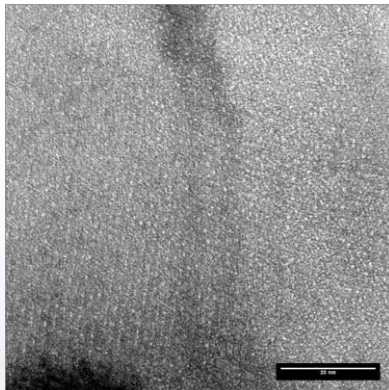


He implantation dose plays a significant role in He bubble formation

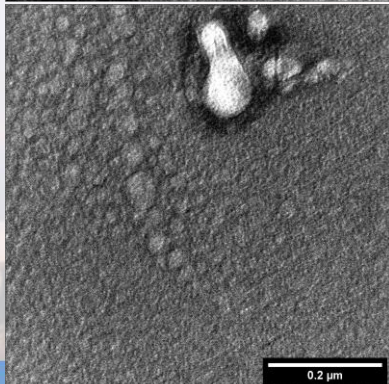
1×10^{17} He



As-implanted



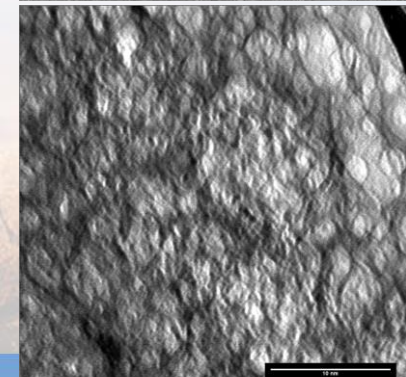
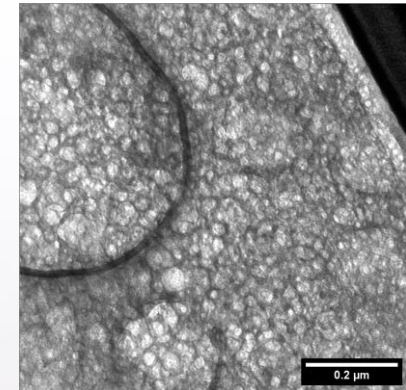
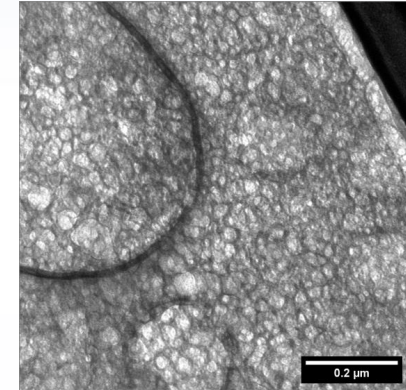
200 C



450 C



5×10^{18} He



Low dose: He bubbles grow significantly

High Dose: Less obvious bubble growth

Annealing of single crystal lamella results in bubble growth at low fluence

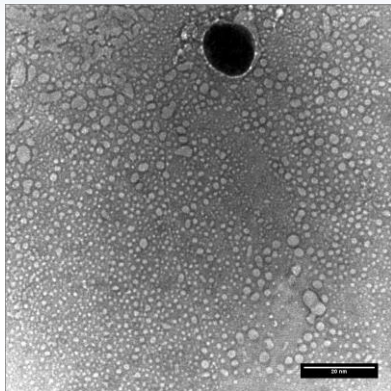
	Pre-Anneal (nm)	Post Anneal
1×10^{17} - 200 C	2.86 nm	1.21 nm
1×10^{17} - 450 C	2.53 nm	4.98 nm
5×10^{18} - 200 C	14.0 nm	14.8 nm
5×10^{18} - 450 C	3.61 nm	4.99 nm

But: lithium aluminate in TPBAR applications is not single crystal!

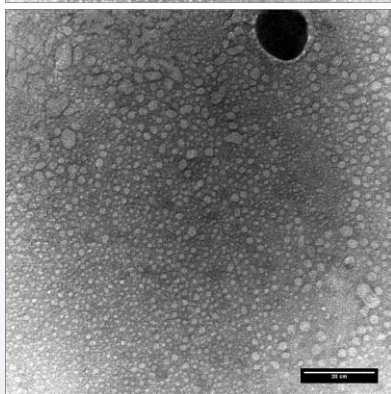


Polycrystalline annealing of lithium aluminate

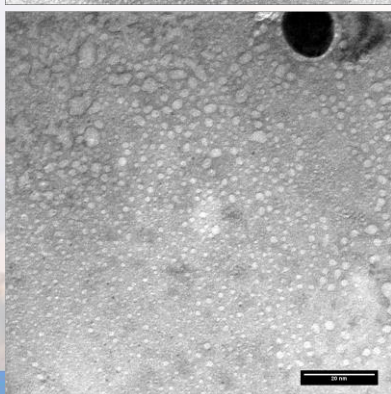
$1 \times 10^{17} \text{ H} + 1 \times 10^{17} \text{ He}$



As-implanted

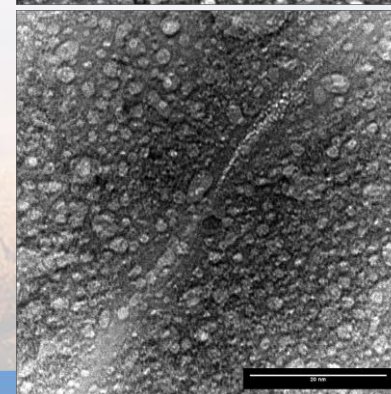
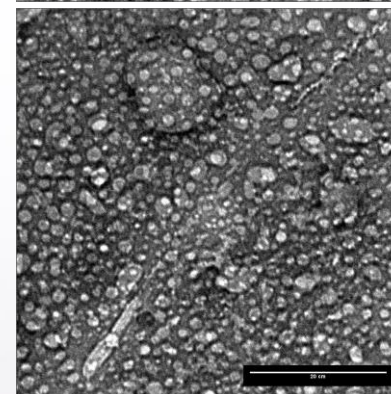
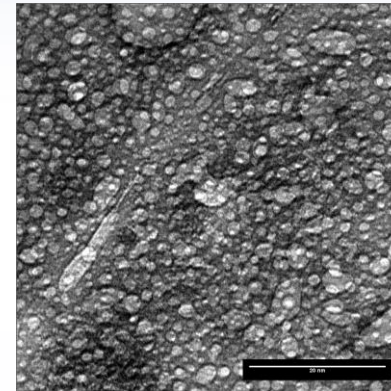


200 C



450 C

$1 \times 10^{18} \text{ H} + 5 \times 10^{18} \text{ He}$



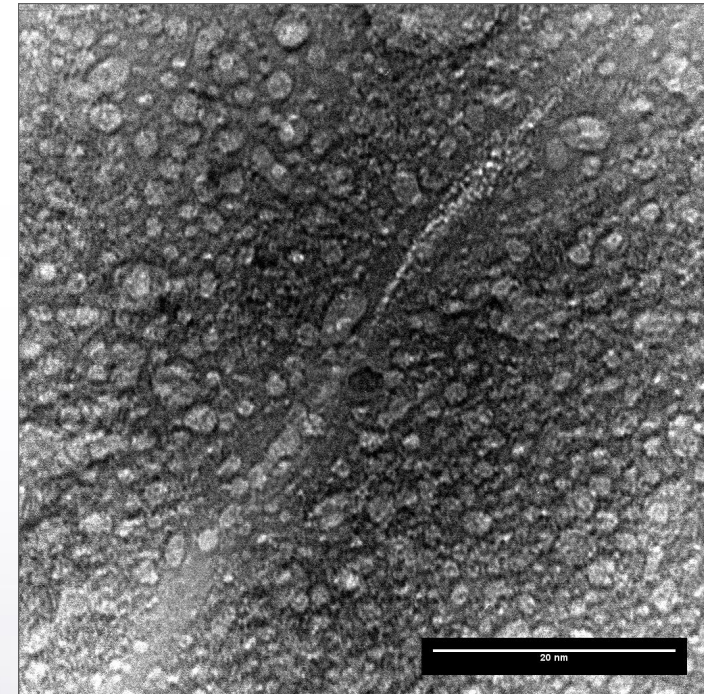
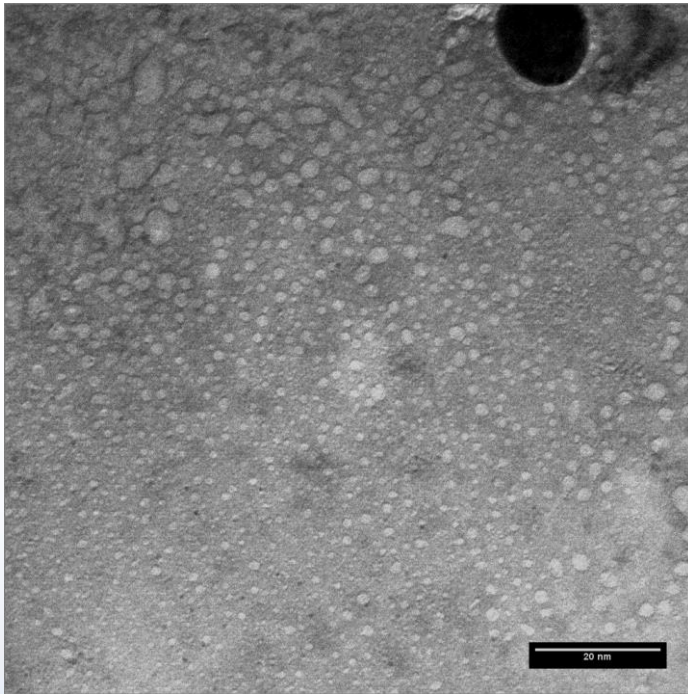
Little evidence of bubble growth

Grain boundaries arresting bubble growth?

Also, have the added consideration of H irradiation



Polycrystalline samples show larger He bubbles and accumulation at grain boundaries



But: No evident growth via Migration and Coalescence or Oswald Ripening



Summary



LiAlO₂

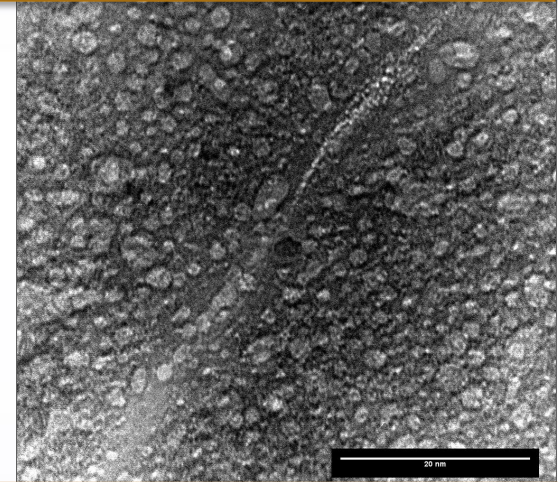
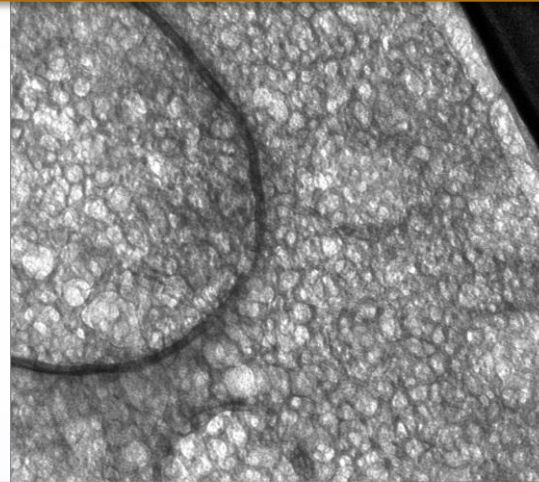
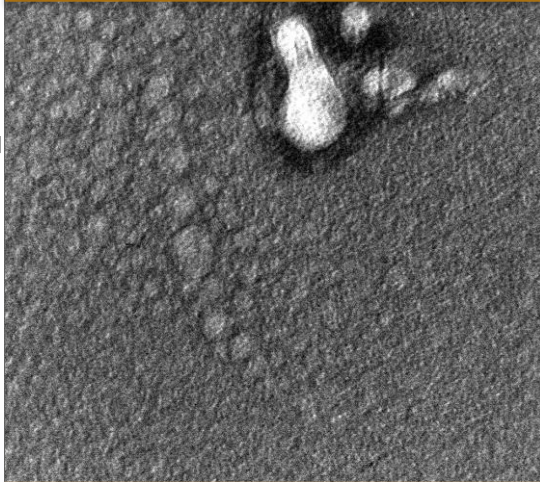
Single Crystal He



High Dose



Polycrystalline He+H



He accumulation

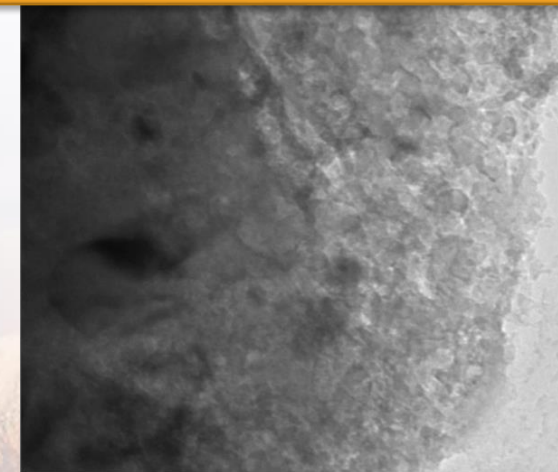
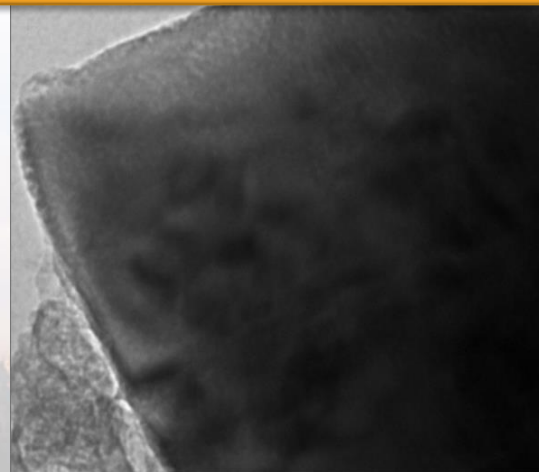
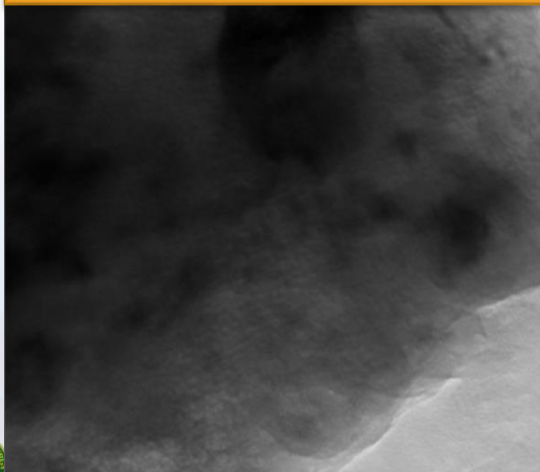


³H + He



Damage + He + ³H

LiAlO₂ Powder



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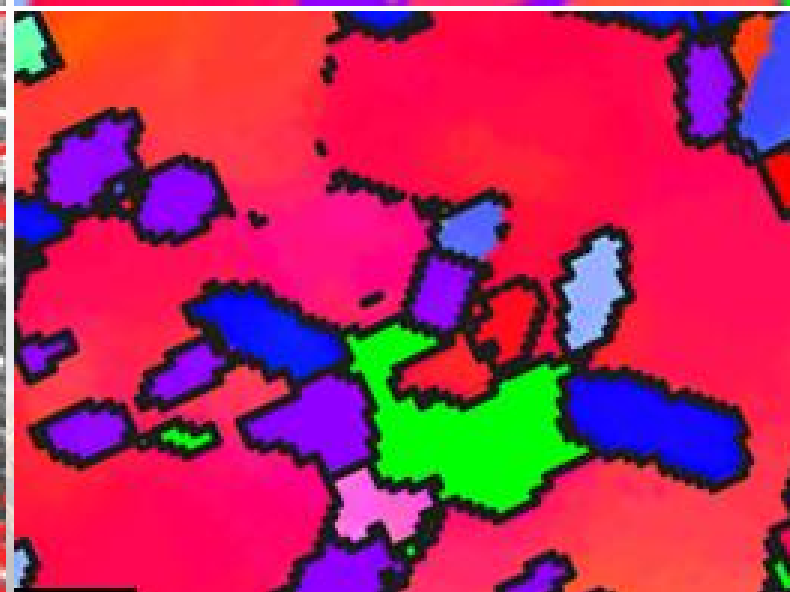
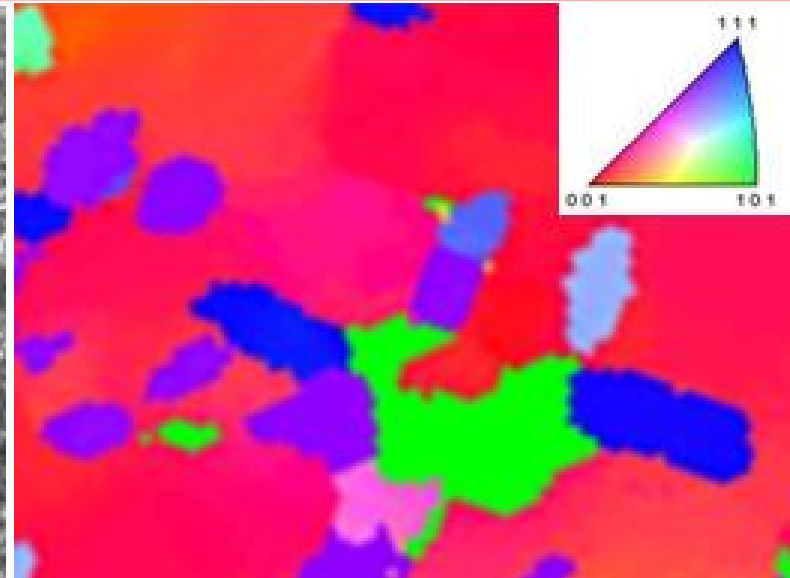
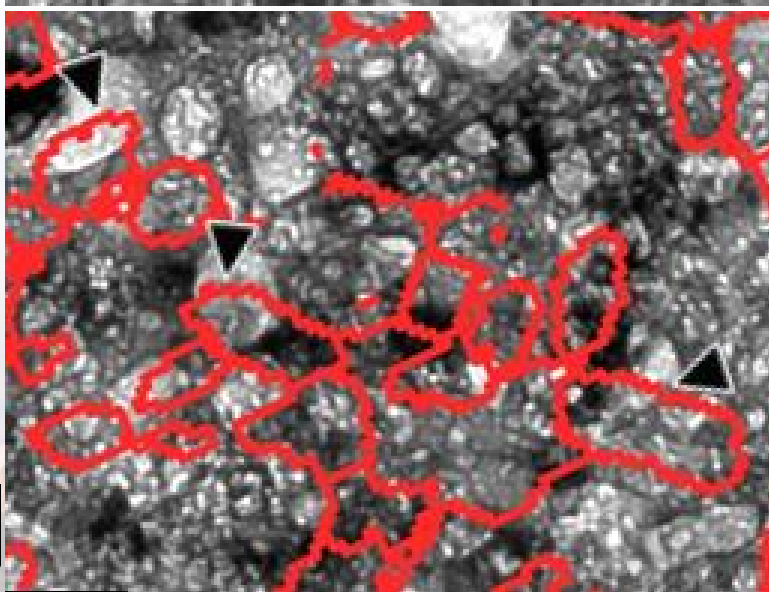
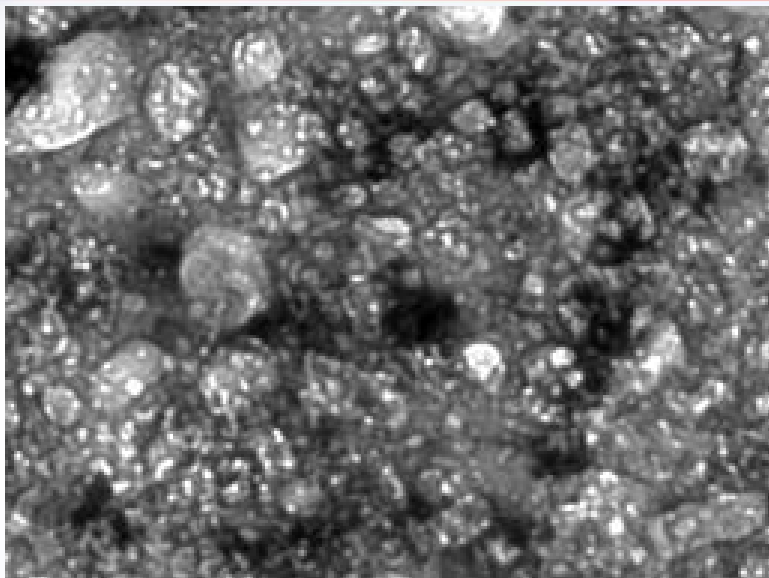
Sandia National Laboratories

Precession Electron Diffraction Post He Implantation and Annealing Reveals Hidden Grain Structure

Cavities in helium implanted, self-ion irradiated, nc nickel film annealed to 400 °C

Cavities span multiple grains at identified grain boundaries

Same to be done with lithium aluminate



100 nm

Grain structure of lithium aluminate impacts the He accumulation

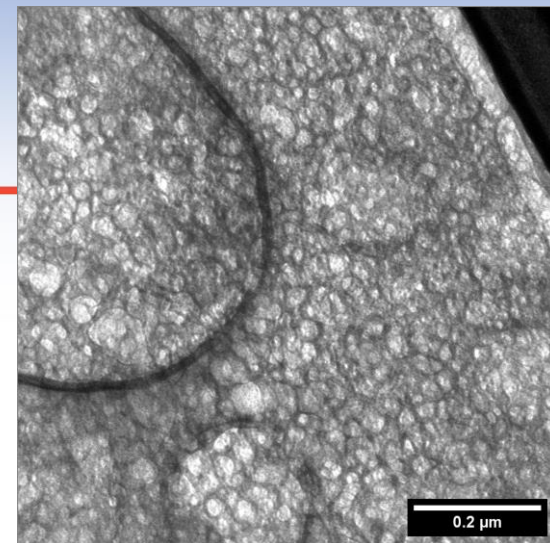
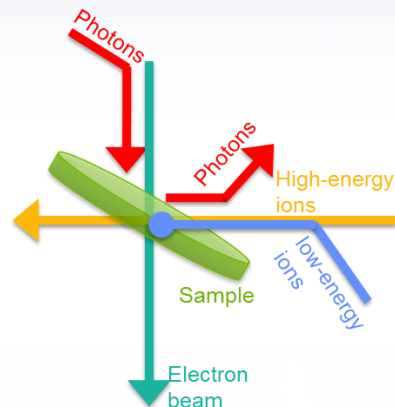
- Bulk lithium aluminate used as neutron absorber in TPBAR applications
- Prior in-situ TEM work only performed on powders – not representative of bulk
- In-situ and ex-situ H/He irradiation of bulk single crystal and polycrystalline lithium aluminate performed
- Grain boundaries accumulate He, grain orientation affects He bubble size, shape, and density



Summary

Sandia's I³TEM capabilities:

- *In situ* high energy ion irradiation from H to Au
- *In situ* gas implantation
- Heating up to 1,000 °C
- Quantitative and bulk straining
- Two-port microfluidic cell
- Gas flow/heating stage
- Electron tomography
- Precession Electron Diffraction
- Laser heating with 20W 1064nm laser
- To be installed: in-situ Raman and PL



Sandia's I³TEM provides a wealth of interesting initial observations and harsh environments with in-situ irradiation and annealing

Currently applying the current I³TEM capabilities to various material systems in combined and harsh environmental conditions

Collaborators:

- IBL: **D.C. Bufford, D. Buller, C. Chisholm, B.G. Clark, J. Villone, B.L. Doyle, S. H. Pratt, M. Steckbeck & M.T. Marshall**
- Sandia: **B. Boyce, T.J. Boyle, P.J. Cappillino, J.A. Scott, B.W. Jacobs, M.A. Hekmaty, D.B. Robinson, J.A. Sharon, W.M. Mook, F. Abdeljawad, & S.M. Foiles**
- External: **A. Minor, L.R. Parent, I. Arslan, H. Bei, E.P. George, P. Hosemann, D. Gross, J. Kacher, & I.M. Robertson**



This work was partially supported by the US Department of Energy, Office of Basic Energy Sciences. Sandia National Laboratories is a multi-mission laboratory managed and operated by National Technology & Engineering Solutions of Sandia, LLC., a wholly owned subsidiary of Honeywell International, Inc., for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-NA0003525.





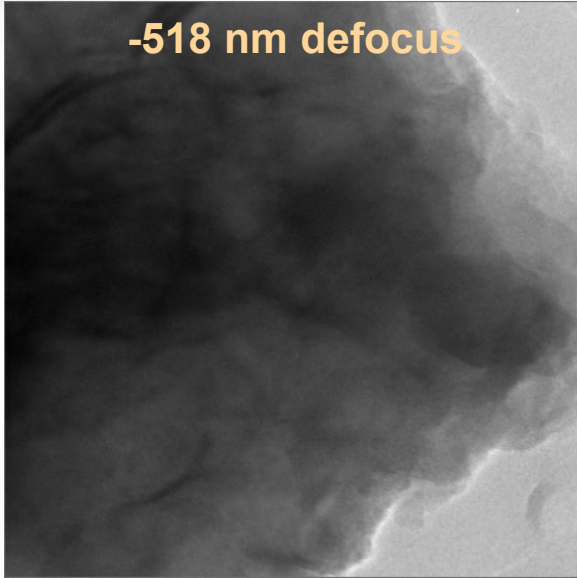
Back-up Slides



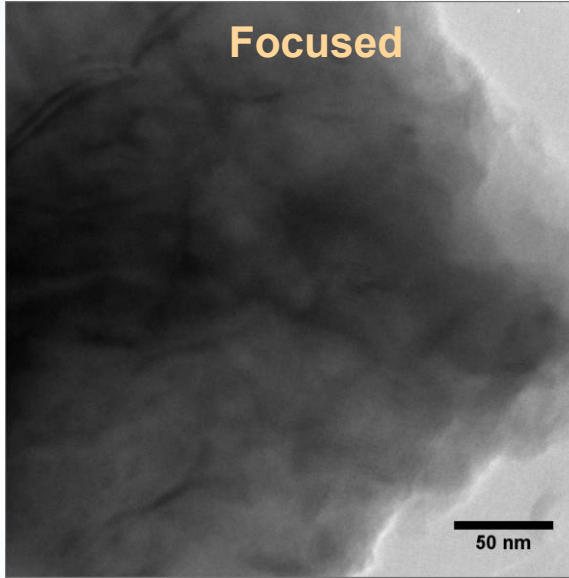
In-situ TEM He implantation at 310°C

Before

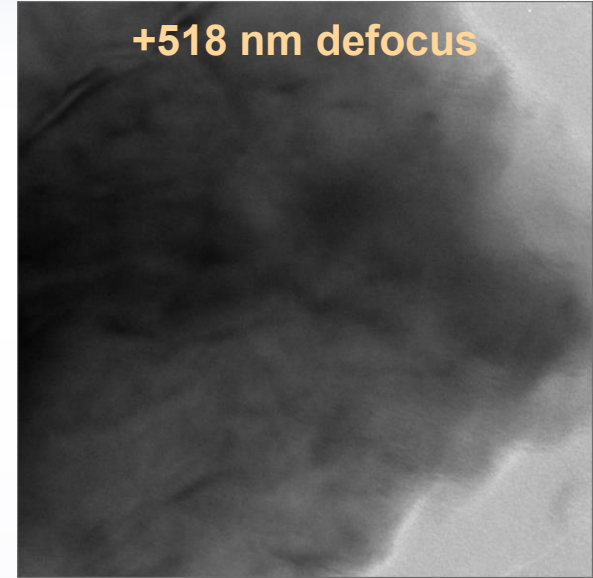
-518 nm defocus



Focused

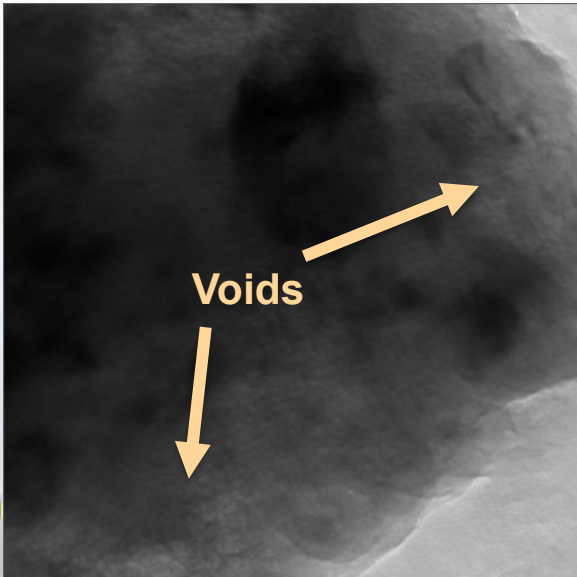


+518 nm defocus

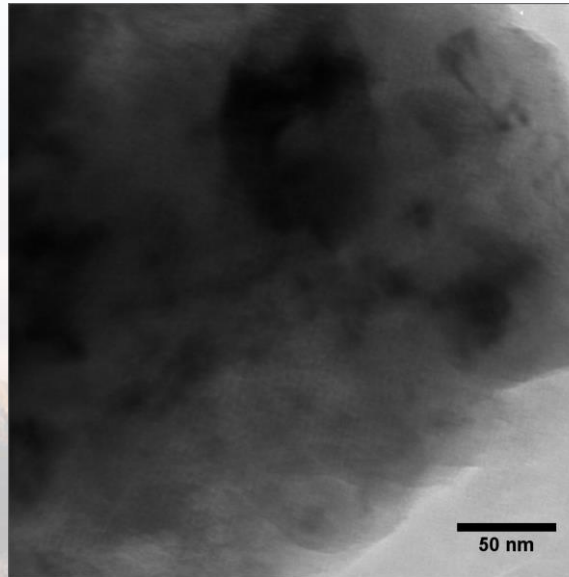


After

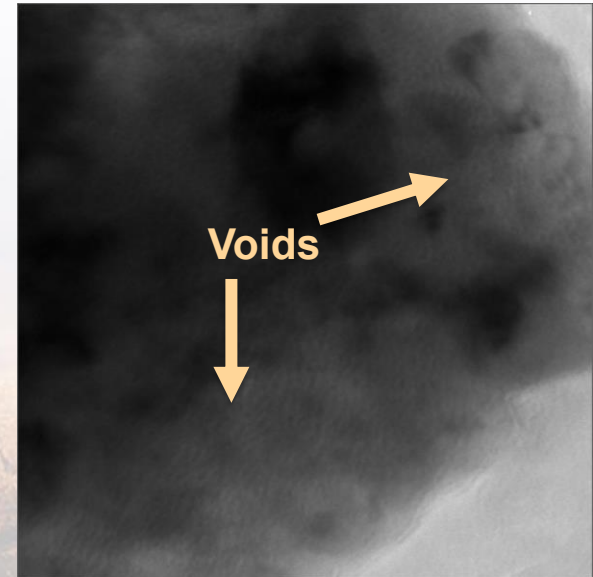
Voids



50 nm

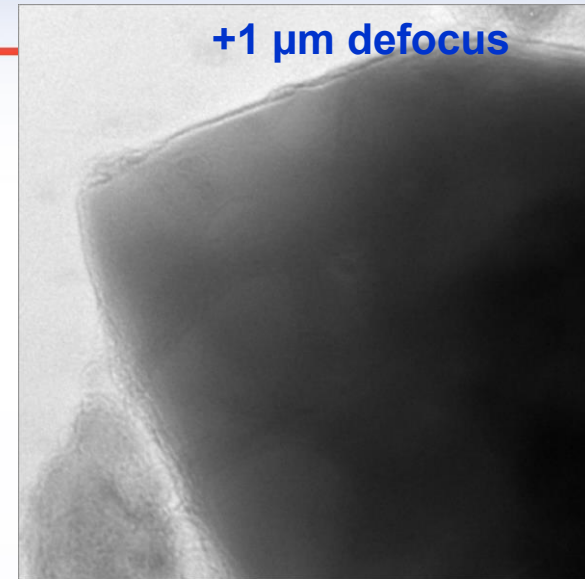
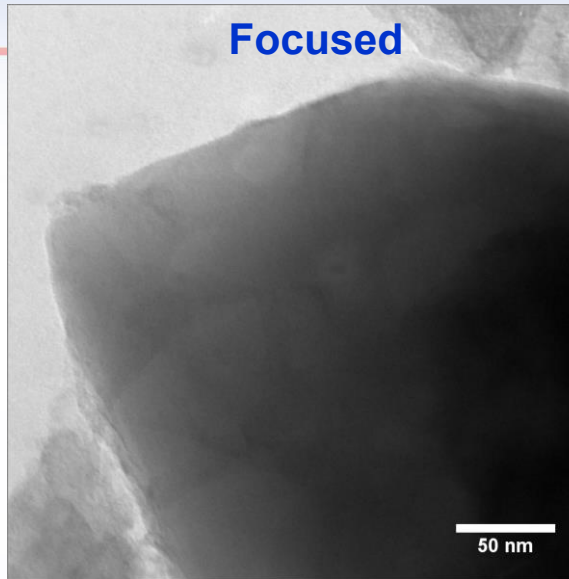
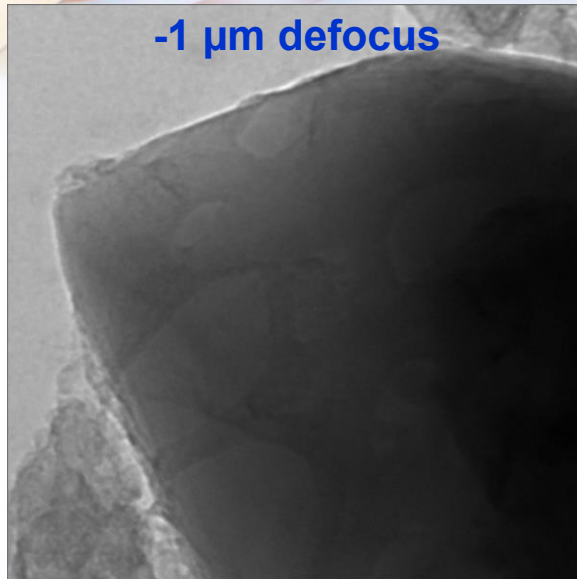


Voids

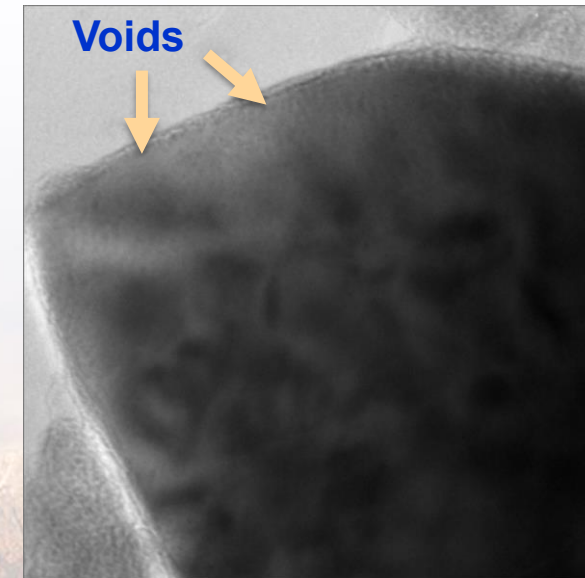
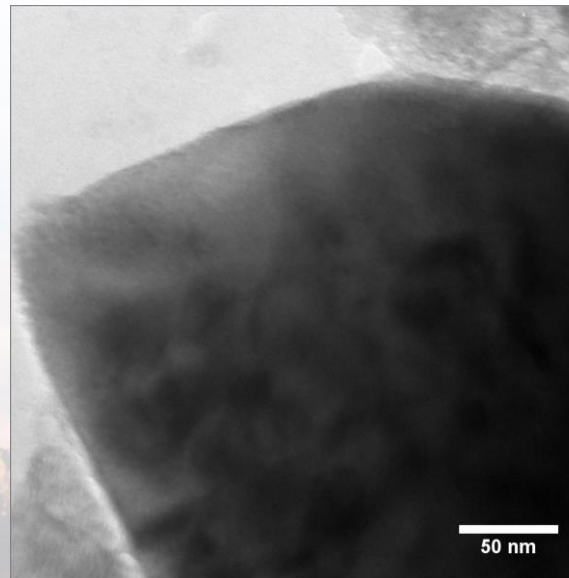
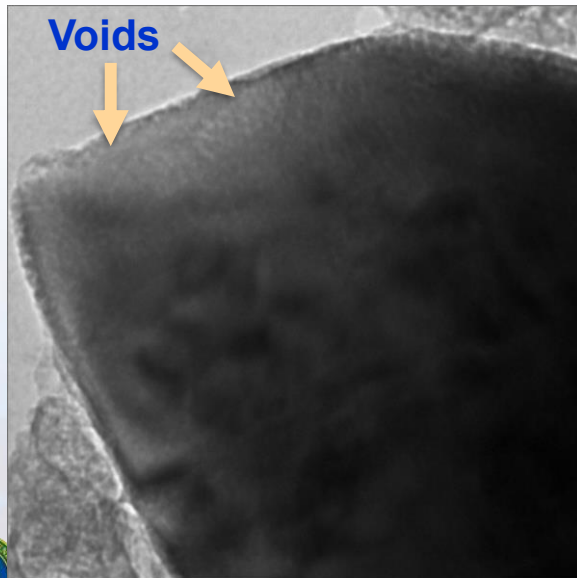


In-situ TEM He & D irradiation at 310°C

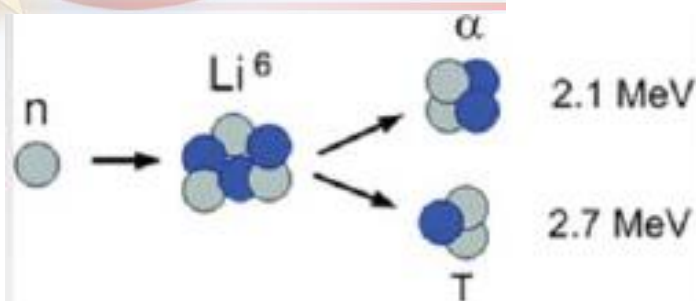
Before



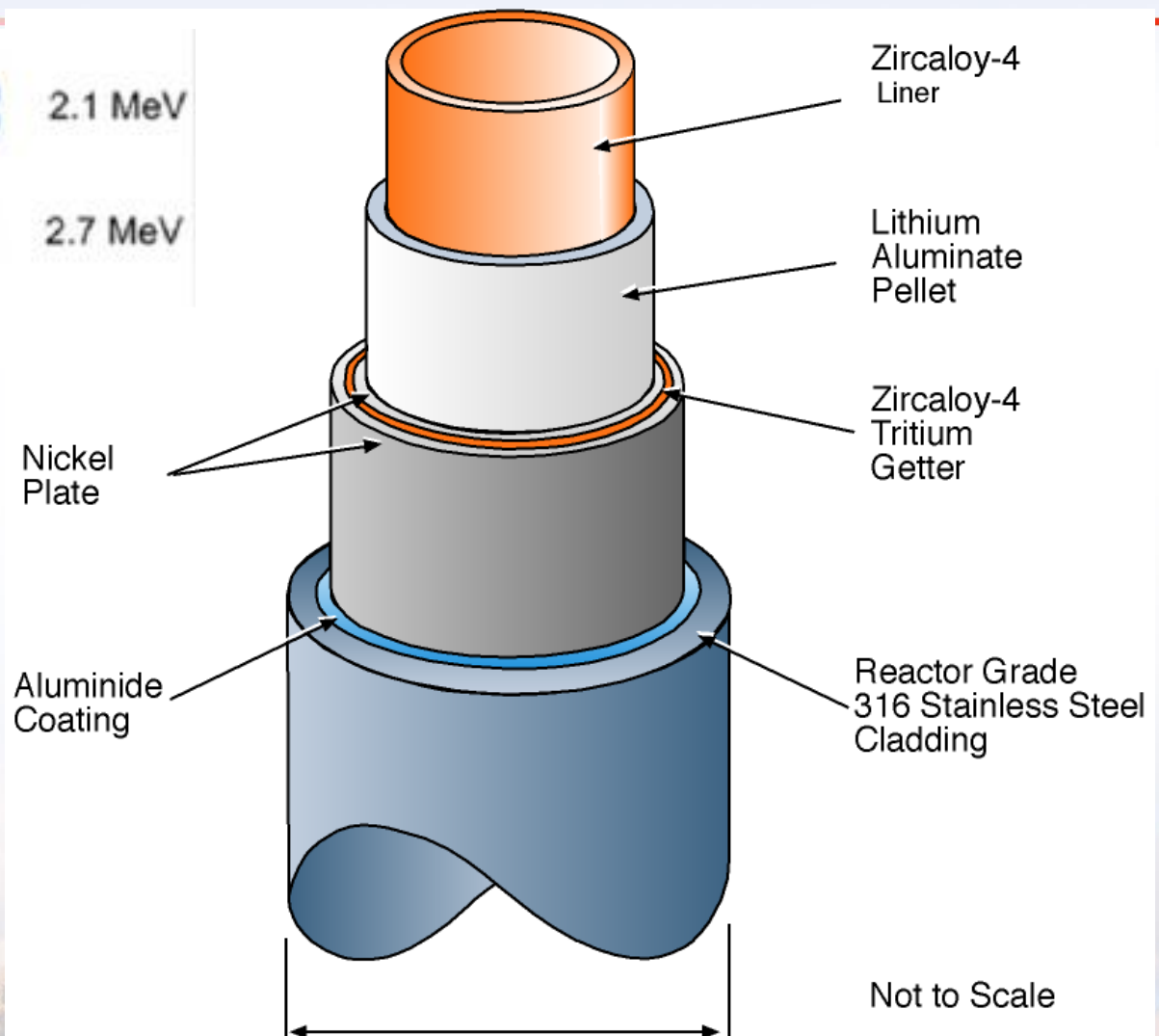
After



Tritium Producing Burnable Absorber Rod



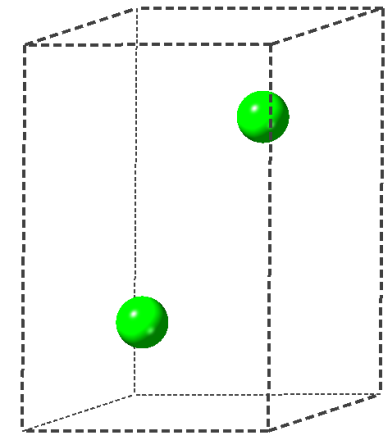
- Displacement Damage
- Helium Implantation
- Tritium Implantation
- Elevated Temperatures



Zircaloy Background

What is Zircaloy?

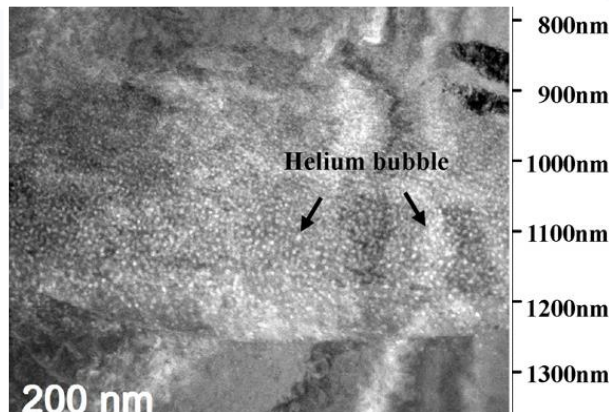
- Zircaloy-2: predominantly used as fuel cladding for BWRs
 - α -Zr, 1.5% Sn, 0.15% Fe, 0.1% Cr, 0.05% Ni
- Zircaloy-4: Removed the Ni and increased Fe content for less H uptake in certain reactor conditions
 - α -Zr, 1.5% Sn, 0.2% Fe, 0.1% Cr
- Zr-Nb alloys (e.g. Zirlo) are also common
- α -Zr has a **hexagonal close-packed (HCP)** crystal structure up to 810°C



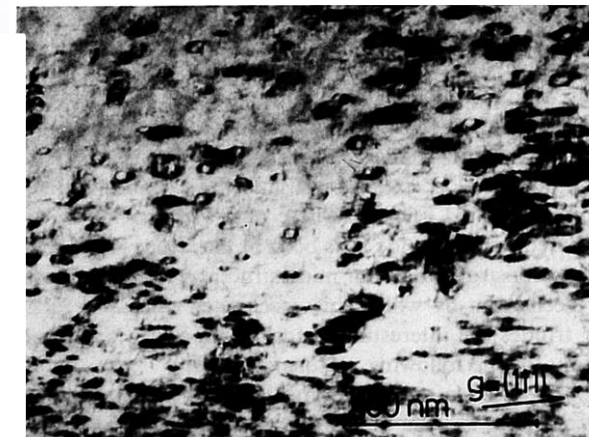
Crystal Structure of α -Zr (HCP)

Gas and defect behavior in Zr/Zr alloys

- ^3H , H , and He diffusion and release
- Bubble formation
- Irradiation induced metallic precipitate formation



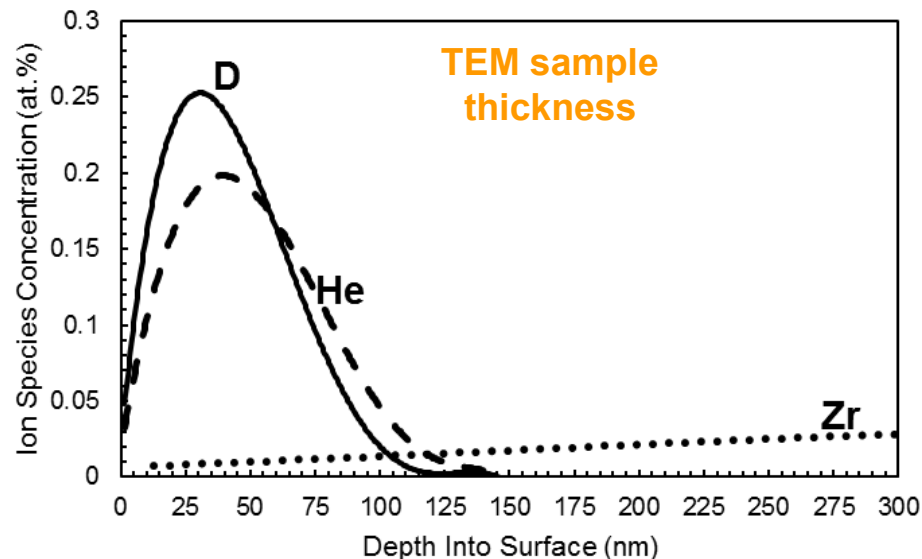
He bubbles in Zr-Nb alloy
Shen et al Mat Char 107 (2015) 309-316



TEM of Zr tritide after 325d
Schober et al JNM 141-143 (1986) 453-457

Zr alloy in-situ ion irradiation parameters

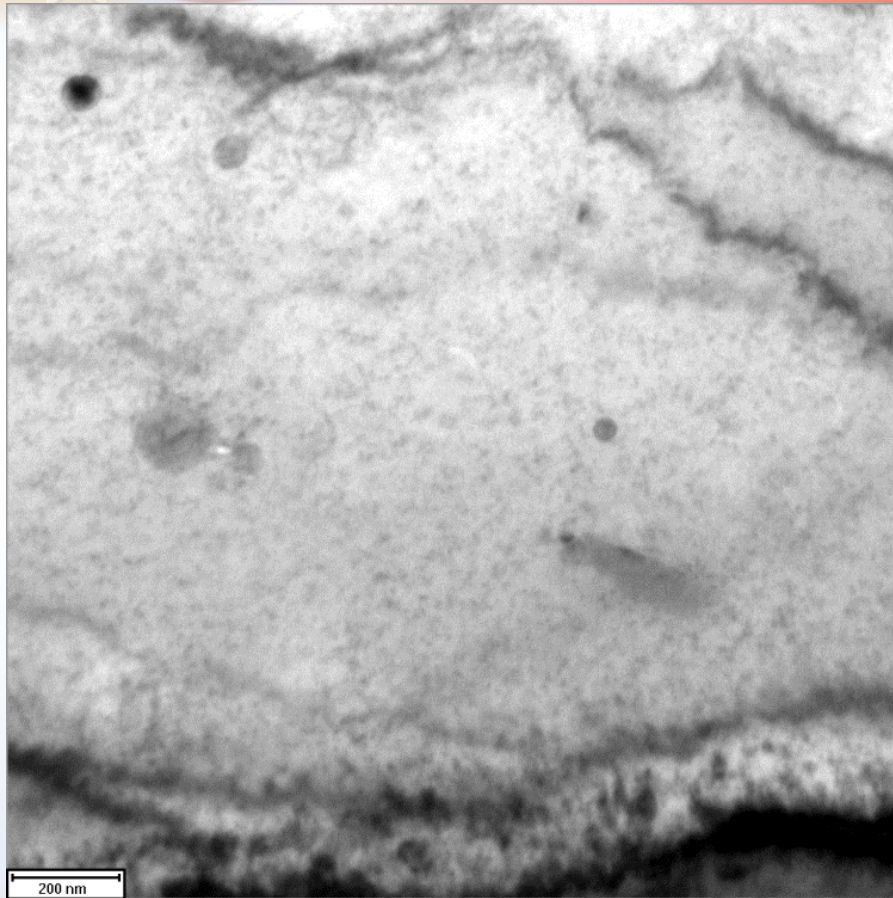
- Samples were prepared by electropolishing zirconium alloy samples (mostly ZIRLO)
- Several sets of irradiations done at 310°C, including:
 - 10 keV He → simulates He accumulation from ^6Li transmutation and ^3H decay
 - 10 keV He + 5 keV D + 3 MeV Zr → simulates gas build-up + displacement cascades
- SRIM, a Monte Carlo based program for simulating the number of displacements produced by an ion, was used to predict damage dose and concentration profiles.
- These preliminary experiments were run overnight and the exact gas concentrations/damage doses are not all known



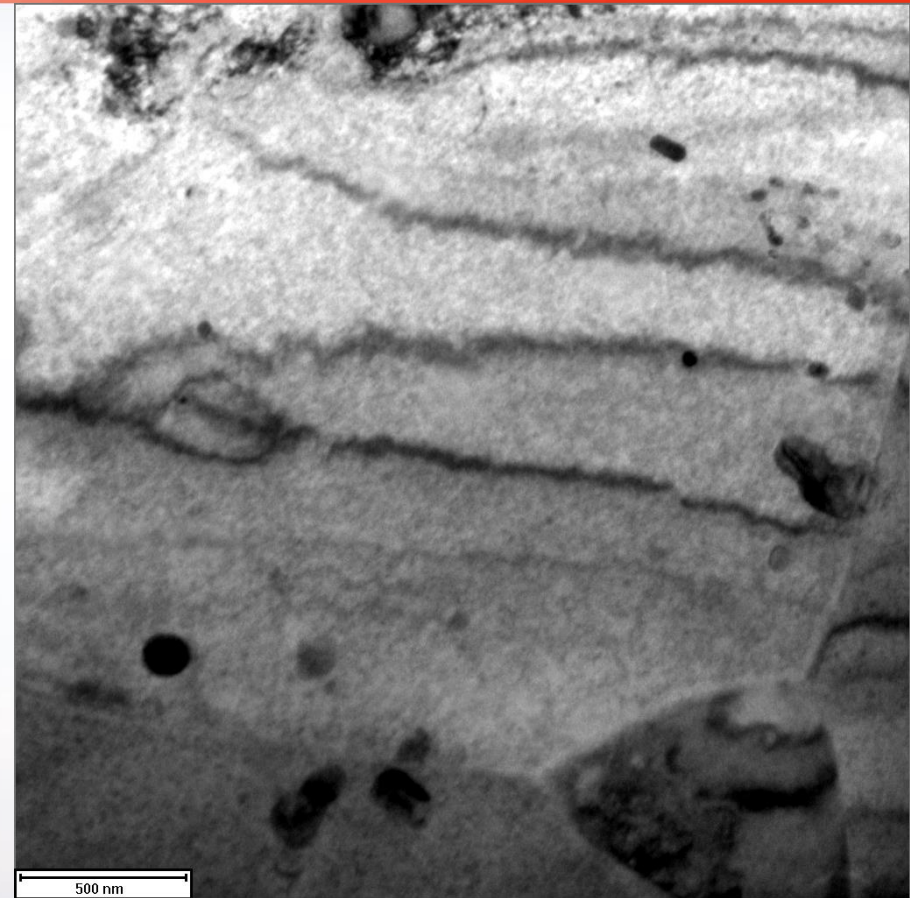
He and D profiles are implanted within the TEM sample, while most Zr passes through the sample, leaving only cascade damage.



10 keV He⁺ Implantation at 310°C



**After Implantation.
Damage, No Cavities.**



**Still no cavities after subsequent
irradiation with 3 MeV Zr.**

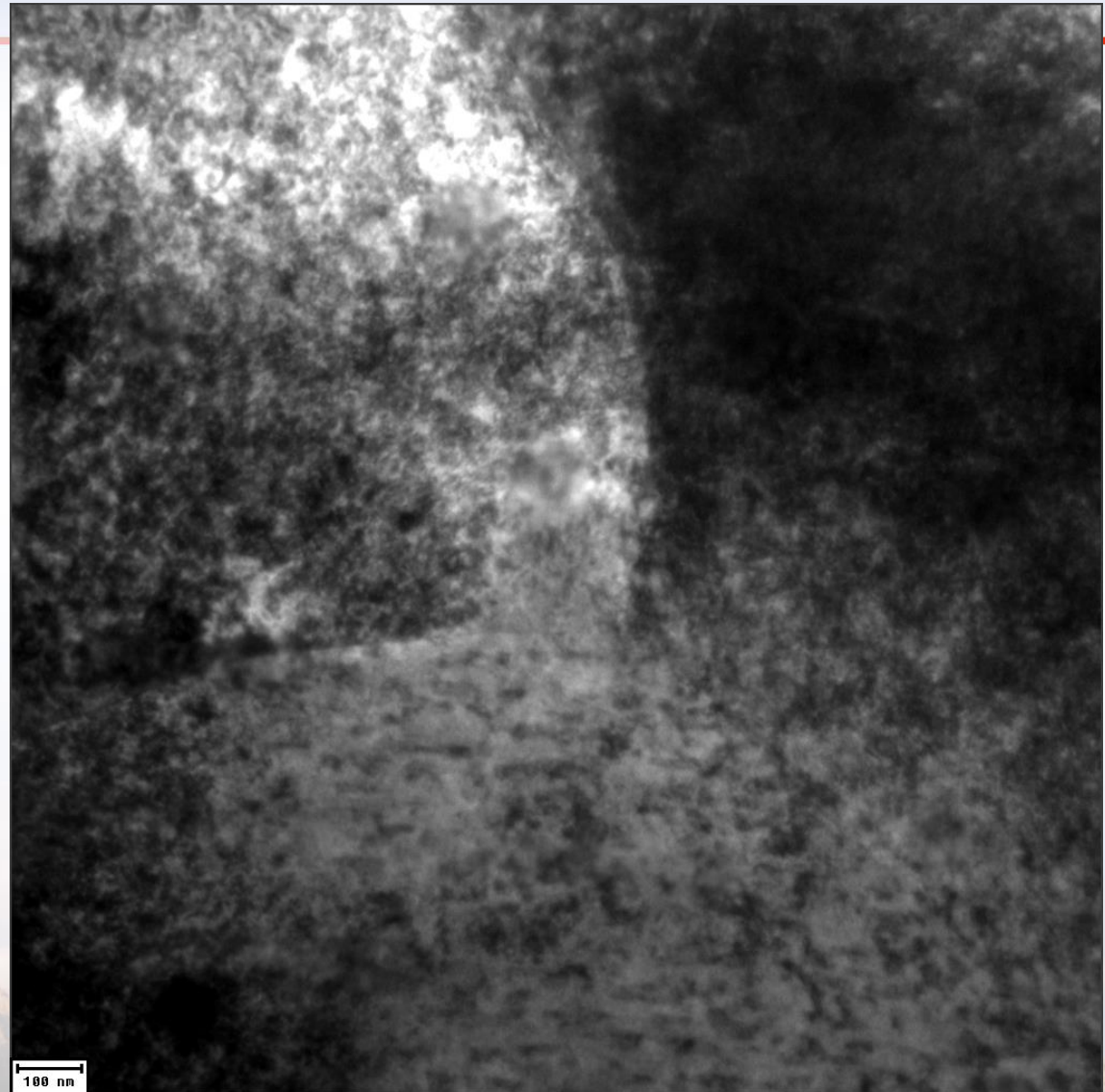
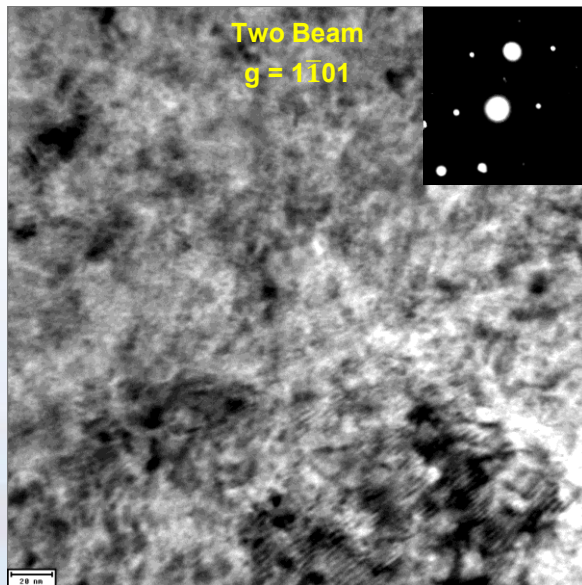


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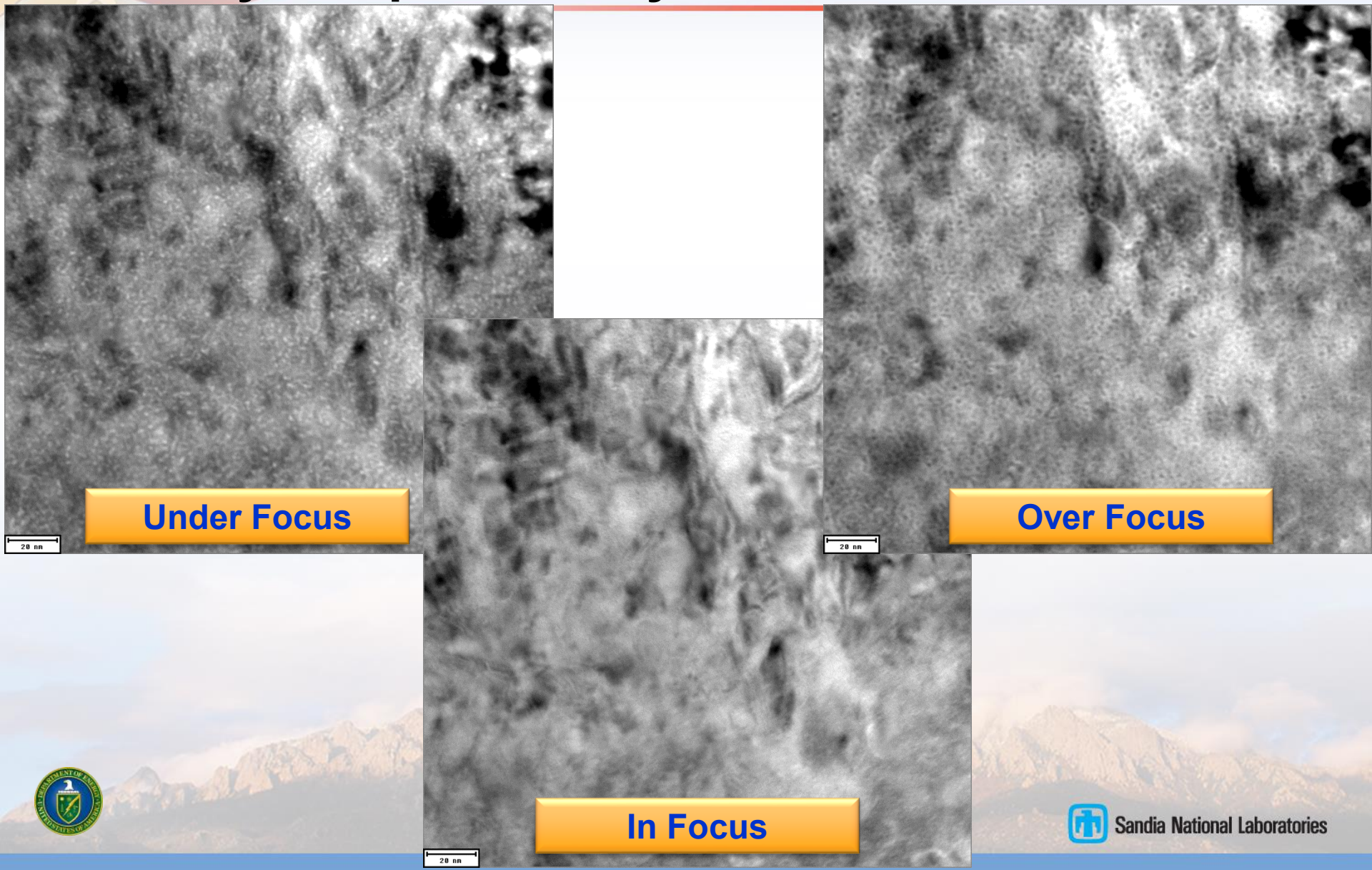
Concurrent D & He Implantation & Zr Irradiation

After triple beam irradiation

- Very dense, complex defect structure
- No visible cavities
- Fuzzy defects difficult to characterize

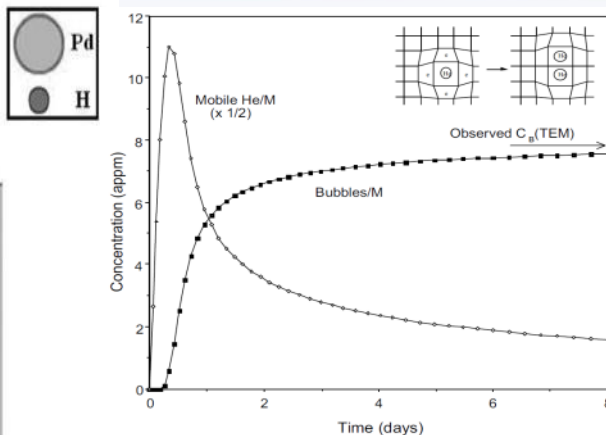
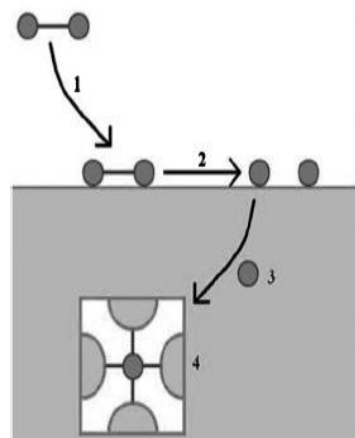


Cavities were observed in He implanted Zr alloy samples 30 days after irradiation



In situ TEM Hydrogen Exposure

Contributors: B.G. Clark, P.J. Cappillino, B.W. Jacobs, M.A. Hekmaty, D.B. Robinson, L.R. Parent, I. Arslan. & Protochips, Inc.



R. Delmelle, J., Phys. Chem. Chem. Phys. (2011) p.11412

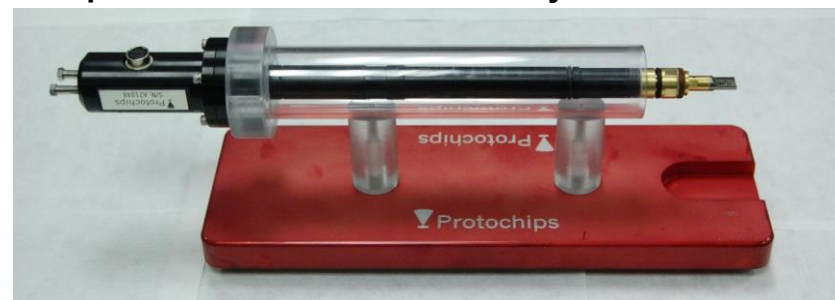
Cowgill, D., Fusion Sci. & Tech., 28 (2005) p. 539

Trinkaues, H. et al., JNM (2003) p. 229

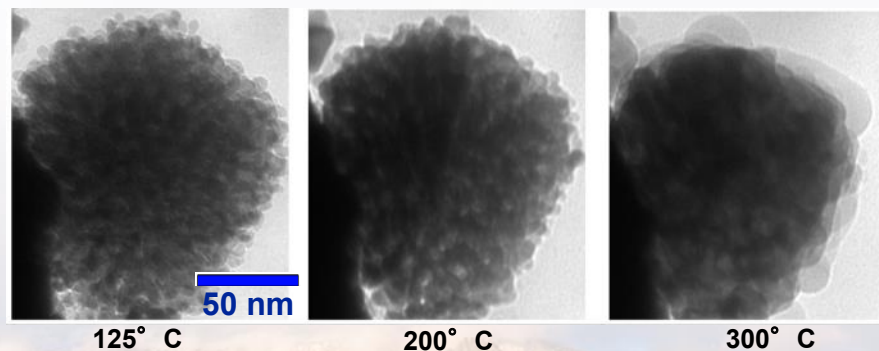
Thiebaud, S. et al. JNM (2000) p. 217

Vapor-Phase Heating TEM Stage

- Compatible with a range of gases
- *In situ* resistive heating
- Continuous observation of the reaction channel
- Chamber dimensions are controllable
- Compatible with MS and other analytical tools

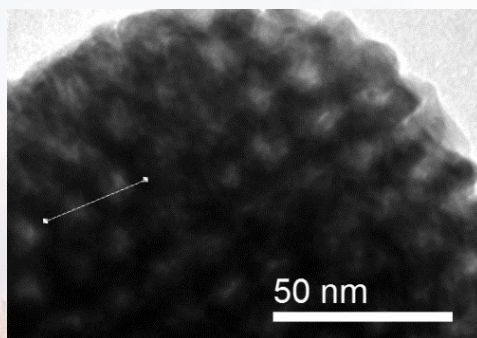
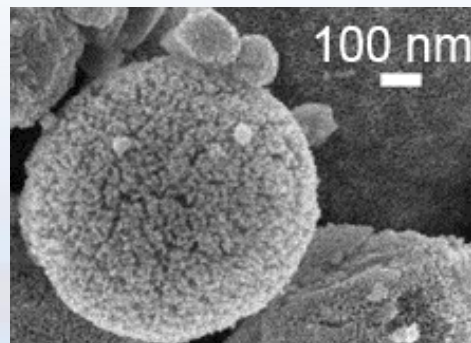


- 1 atm H₂ after several pulses to specified temp.

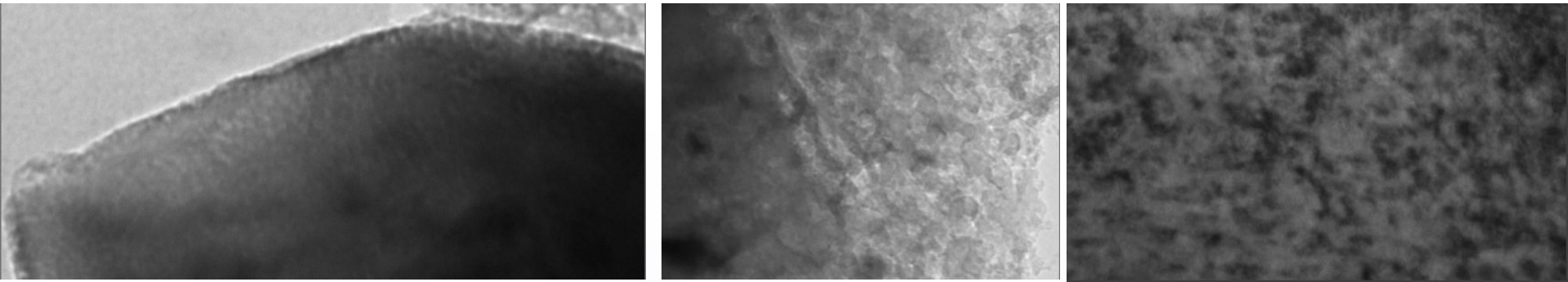


New *in situ* atmospheric heating experiments provide great insight into nanoporous Pd stability

Harmful effects may be mitigated in nanoporous Pd



Simulations of Damage and Gas Accumulation in TPBAR Materials with In-situ Triple Ion Beam Irradiation TEM



**Caitlin A. Taylor, Brittany Muntifering, David Senior, Clark Snow, and
Khalid Hattar**

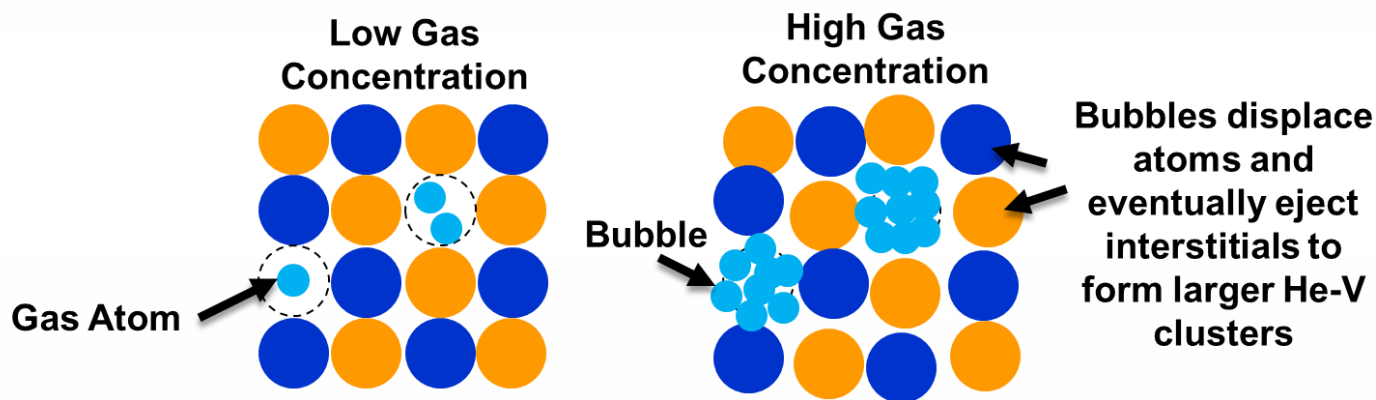
September 6th, 2017



Sandia National Laboratories

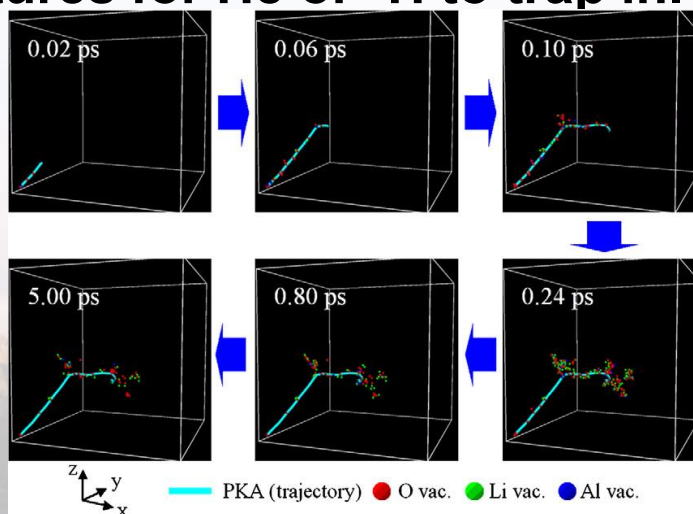
Bubbles May Affect ^3H Release

- Bubbles form due to He trapping in lattice defects



- Neutron irradiation produces displacement cascades, providing complex defect structures for He or ^3H to trap in.

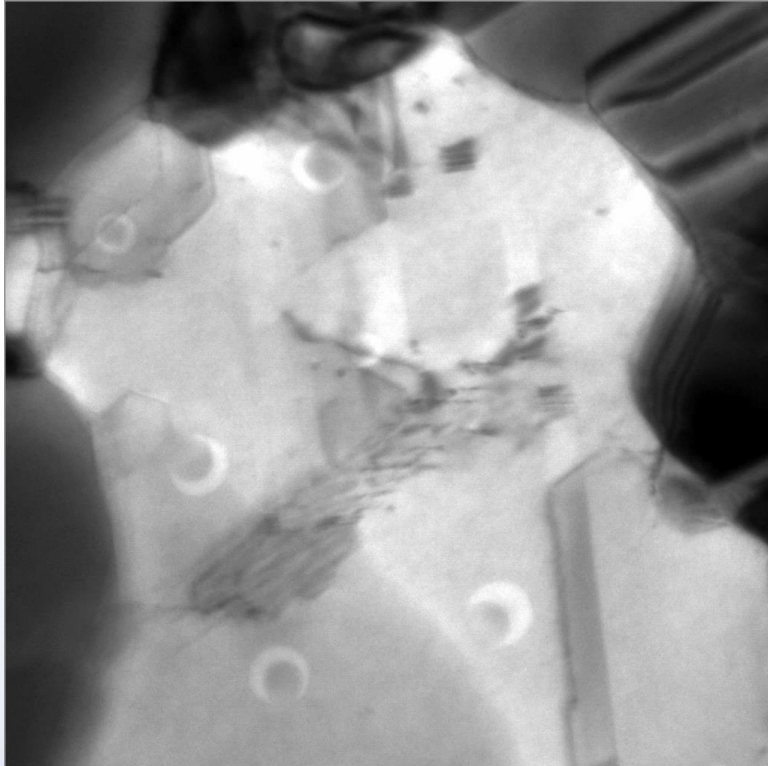
MD simulation of displacement cascade in LiAlO_2 (PKA = 5 keV)



Dose Rate Effects

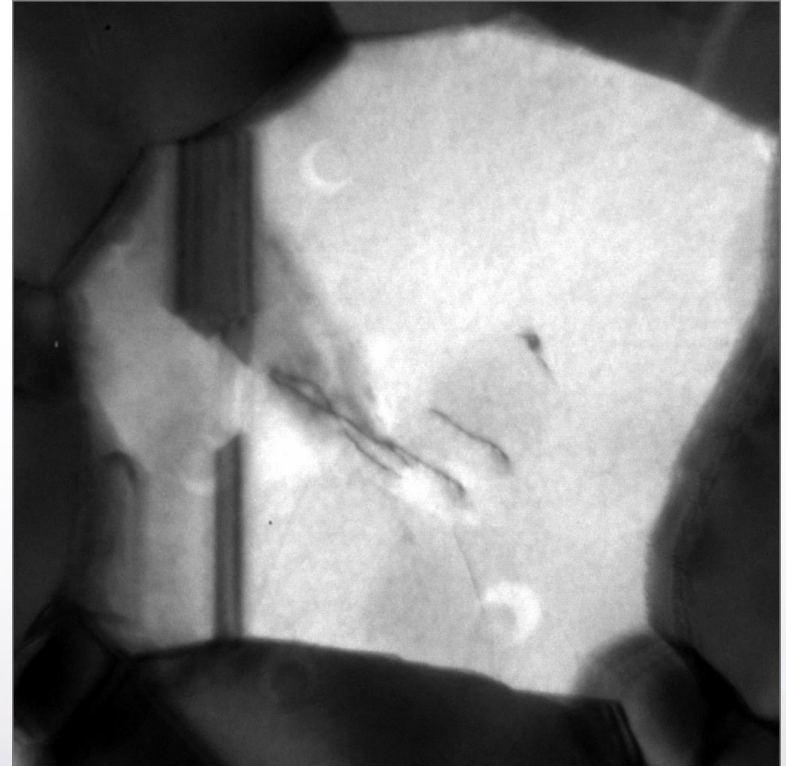
Collaborators: C. Chisholm , P. Hosemann, & A. Minor

7.9×10^9 ions/cm²/s



VS

6.7×10^7 ions/cm²/s



Improved vibrational and ion beam stability permits us to work at 120kx or higher permitting imaging of single cascade events



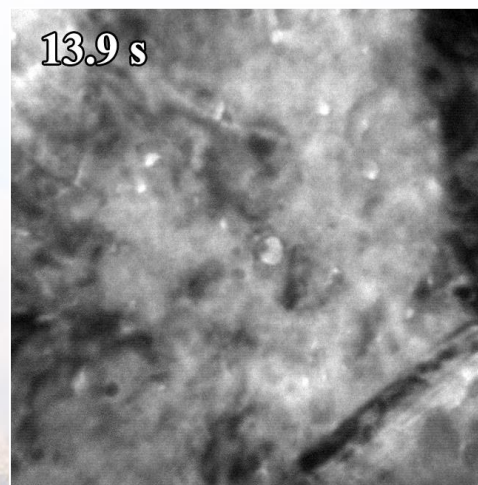
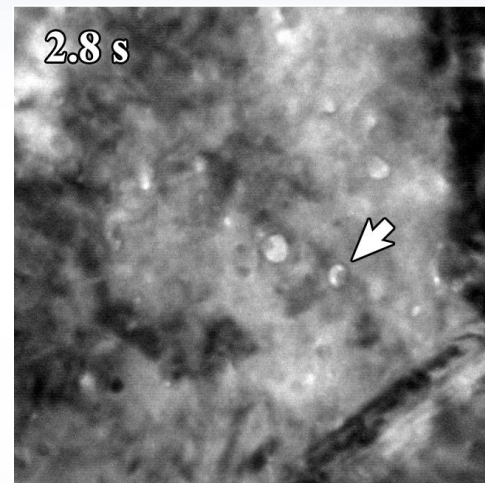
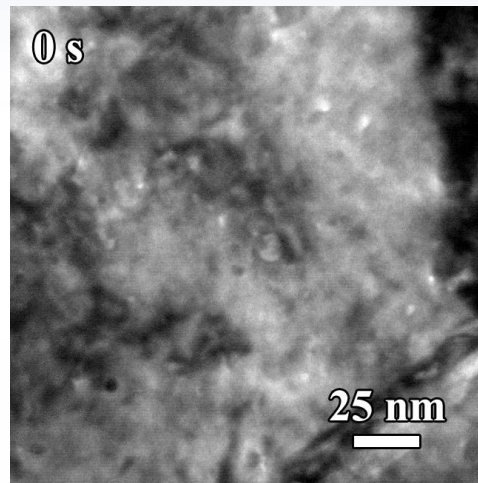
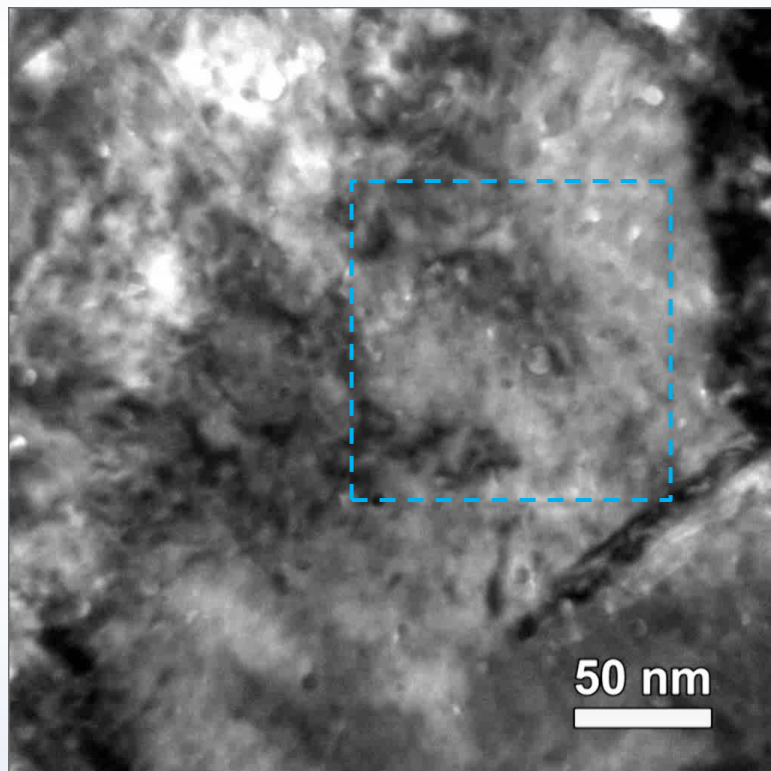
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Simultaneous *In situ* TEM Triple Beam:

2.8 MeV Au⁴⁺ + 10 keV He⁺/D₂⁺

Collaborator: D.C. Bufford

Video playback speed x1.5.



■ Approximate fluence:

- Au 1.2×10^{13} ions/cm²
- He 1.3×10^{15} ions/cm²
- D 2.2×10^{15} ions/cm²

■ Cavity nucleation and disappearance

In-situ triple beam He, D₂, and Au beam irradiation has been demonstrated on Sandia's I³TEM!

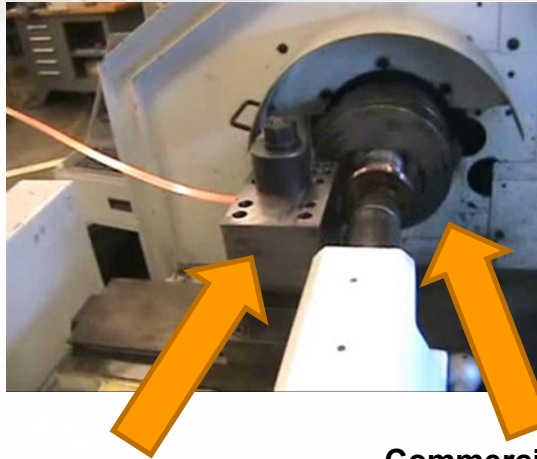
Intensive work is still needed to understand the defect structure evolution that has been observed.



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What Insight into Structural Stability is Gained from I³TEM Experiments?

Collaborators: O. El-Atwani, J. P. Allain, D. Buller, & J.A. Scott

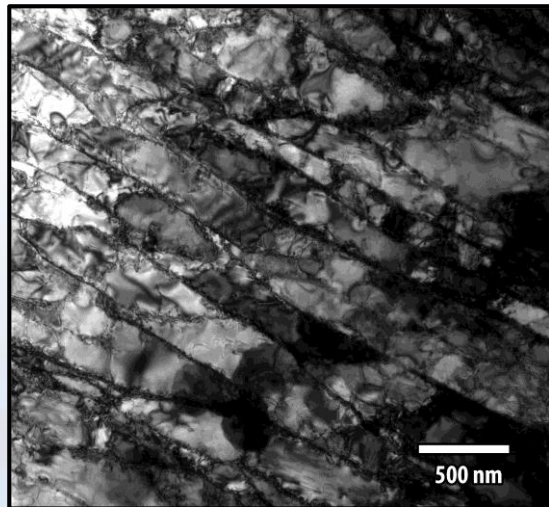


Extrusion
Machining tooling

Commercially
available lathe

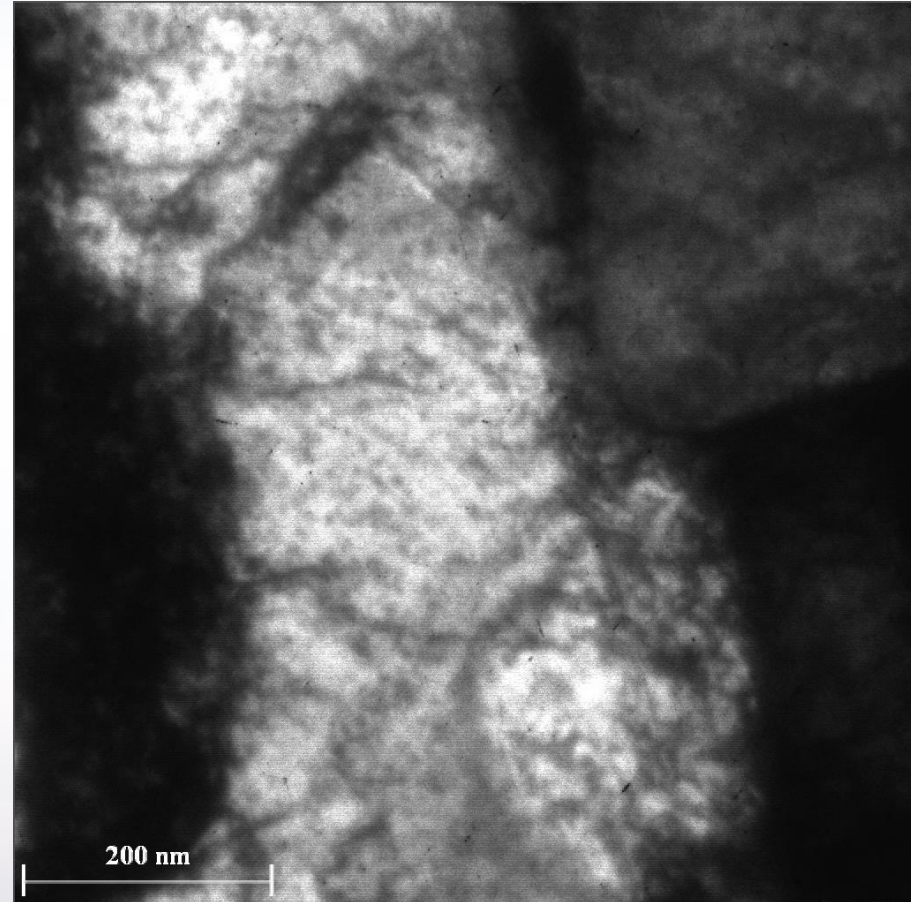
From NW
components through
proposed NE
cladding to waste
storage:

Understanding
Radiation Damage is
Essential



UFG Tungsten

- I³TEM W irradiation and He implantation of SPD-W developed for ITER applications



I³TEM is providing insight into:

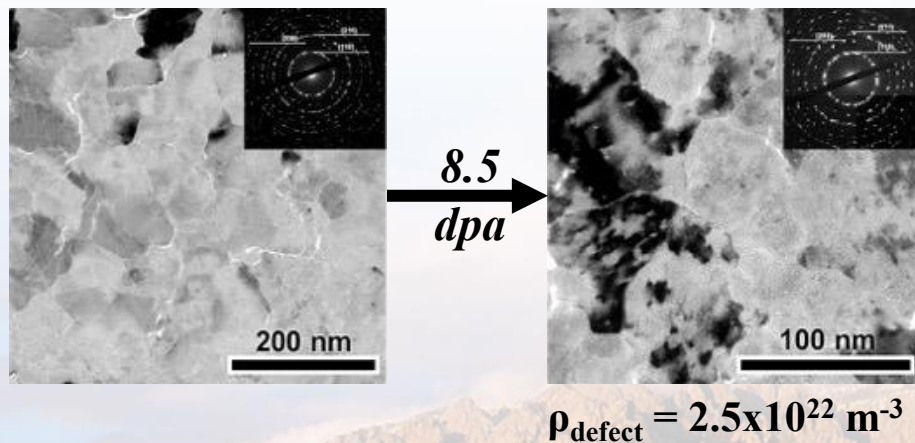
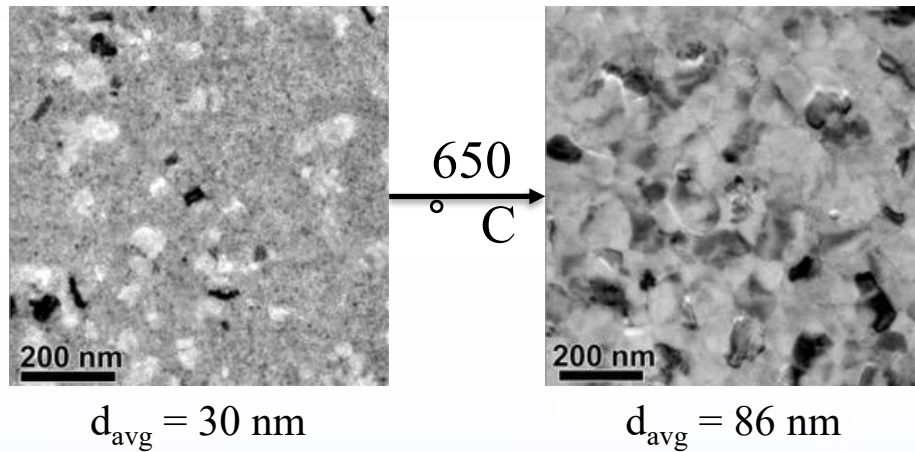
- 1) Loop formation
- 2) Loop stability & migration
- 3) Rad & structural defect interactions



Scaling down to Nanocrystalline Tungsten Alloys

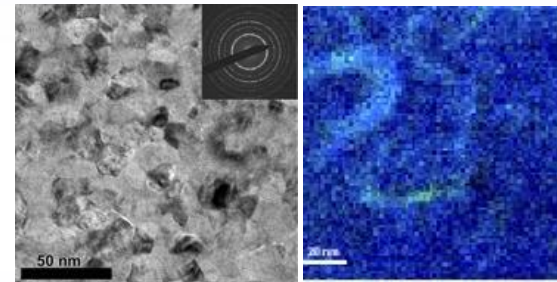
Collaborators: O.K. Donaldson, T. Kaub, G. Thompson, and J. Trelewicz

Nanocrystalline W

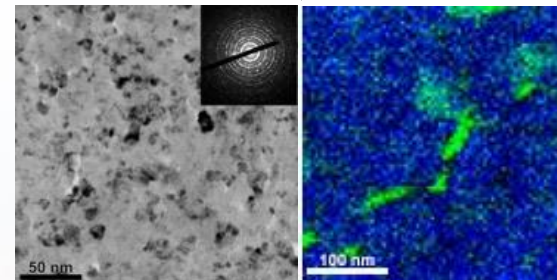


Alloying does not negatively effect radiation tolerance, while improving thermal and mechanical properties

Nanocrystalline W-20at.%Ti

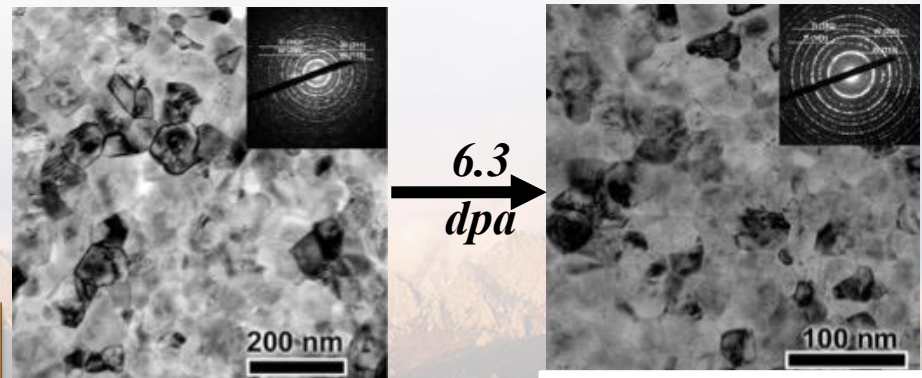


1000° C Anneal



Grain growth is hampered by the addition of Ti

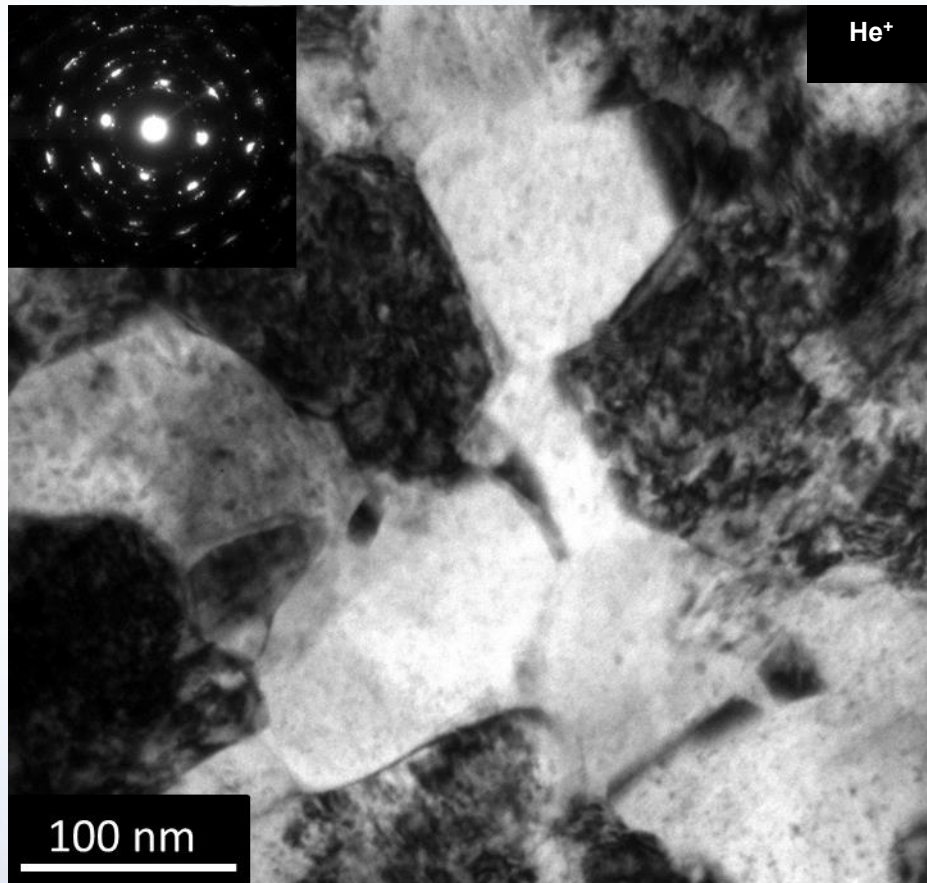
Ti solute is heterogeneously distributed after annealing



$\rho_{\text{defect}} = 2.8 \times 10^{22} \text{ m}^{-3}$

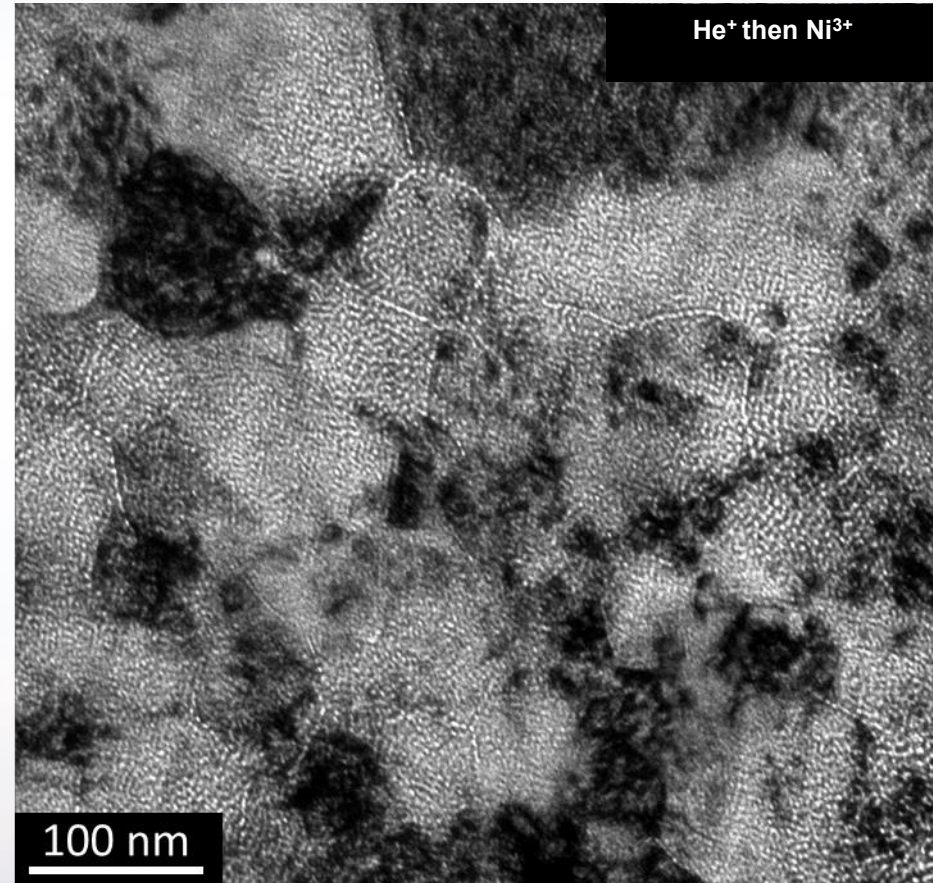
10 keV He⁺ Implantation followed by 3 MeV Ni³⁺ Irradiation

Collaborator: B. Muntifering & J. Qu



10^{17} He⁺/cm²

Visible damage to the sample

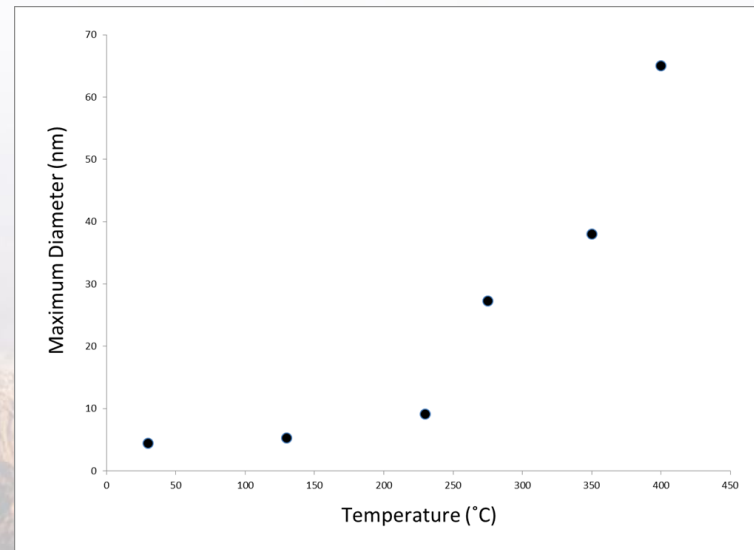
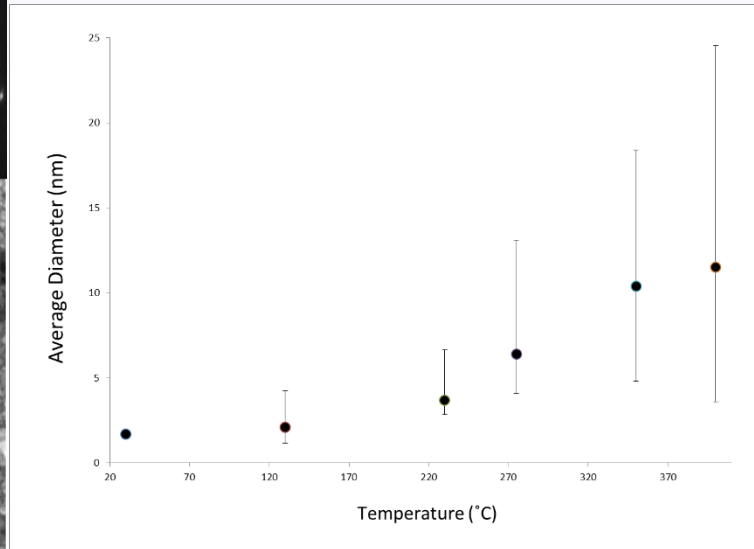
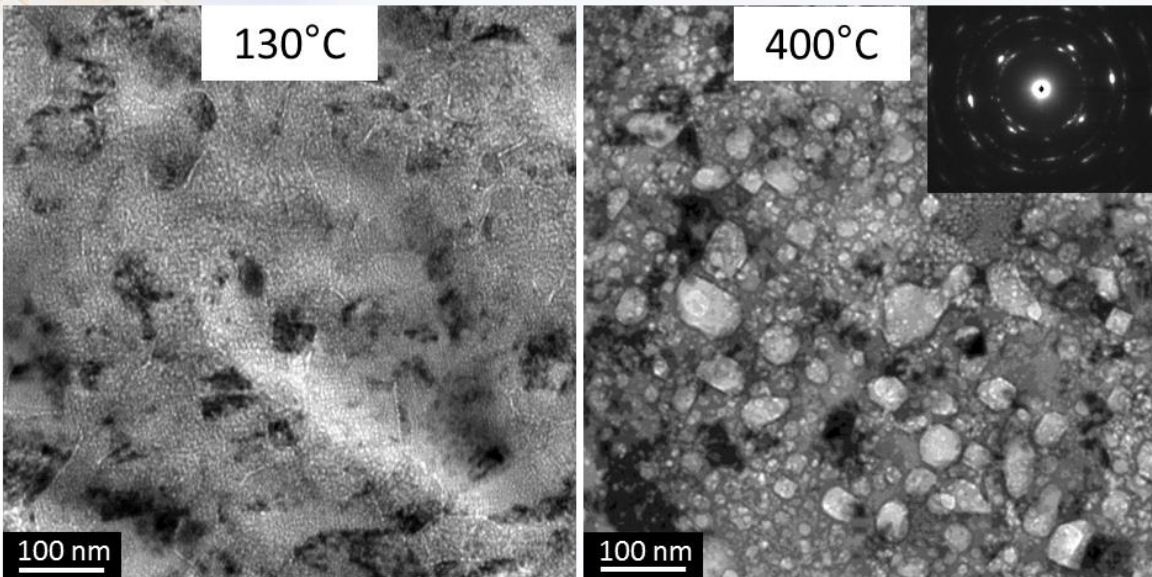


0.7 dpa Ni³⁺ irradiation

High concentration of cavities along
grain boundaries



Cavity Growth during In-situ Annealing of 10 keV He⁺ Implanted and then 3 MeV Irradiated Ni³⁺



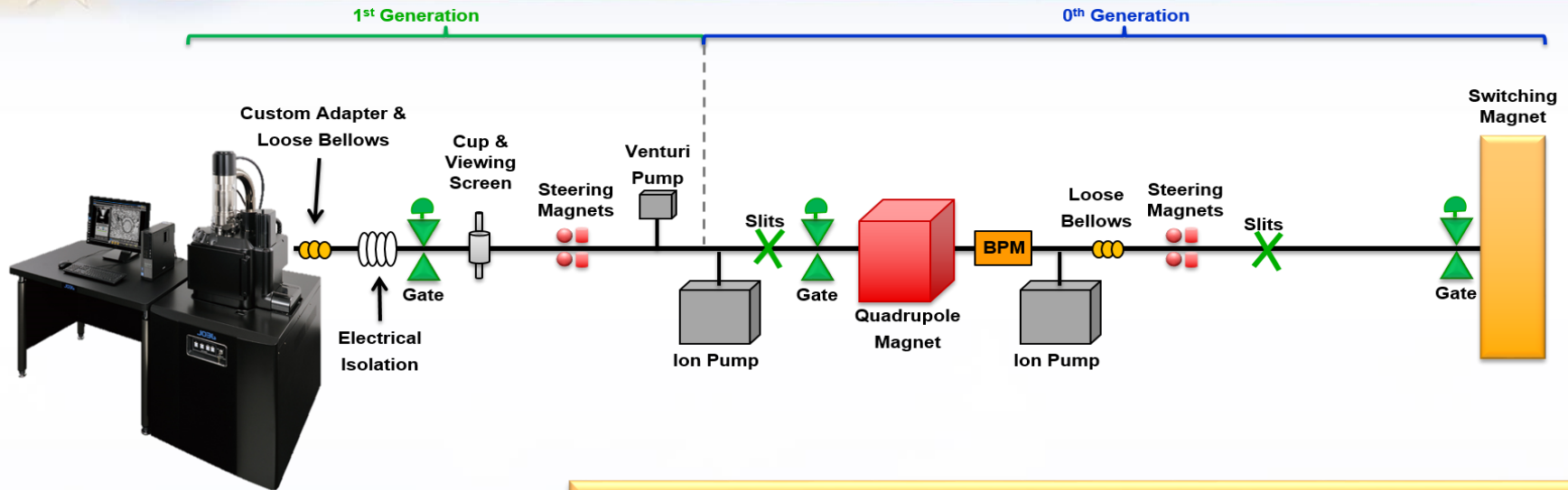
Bubble to cavity transition and cavity evolution can be directly studied



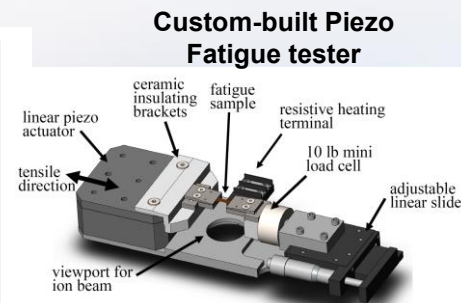
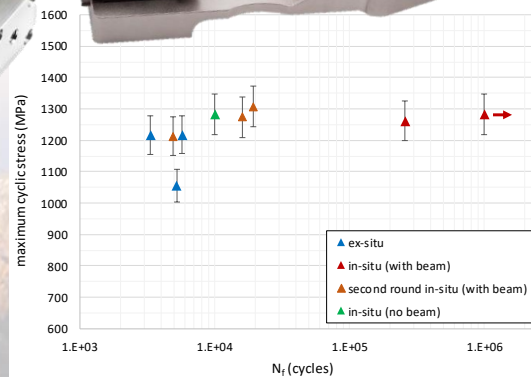
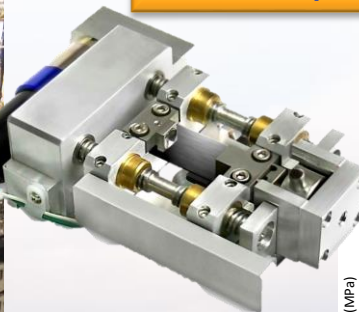
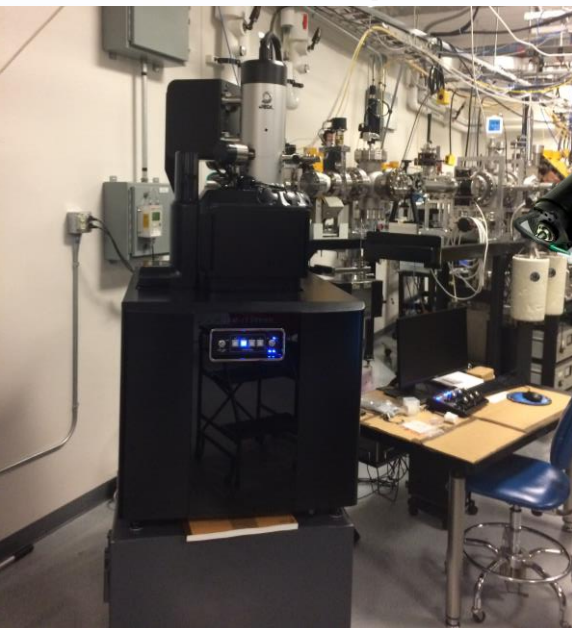
Schematic of the *In situ* SEM Beamline

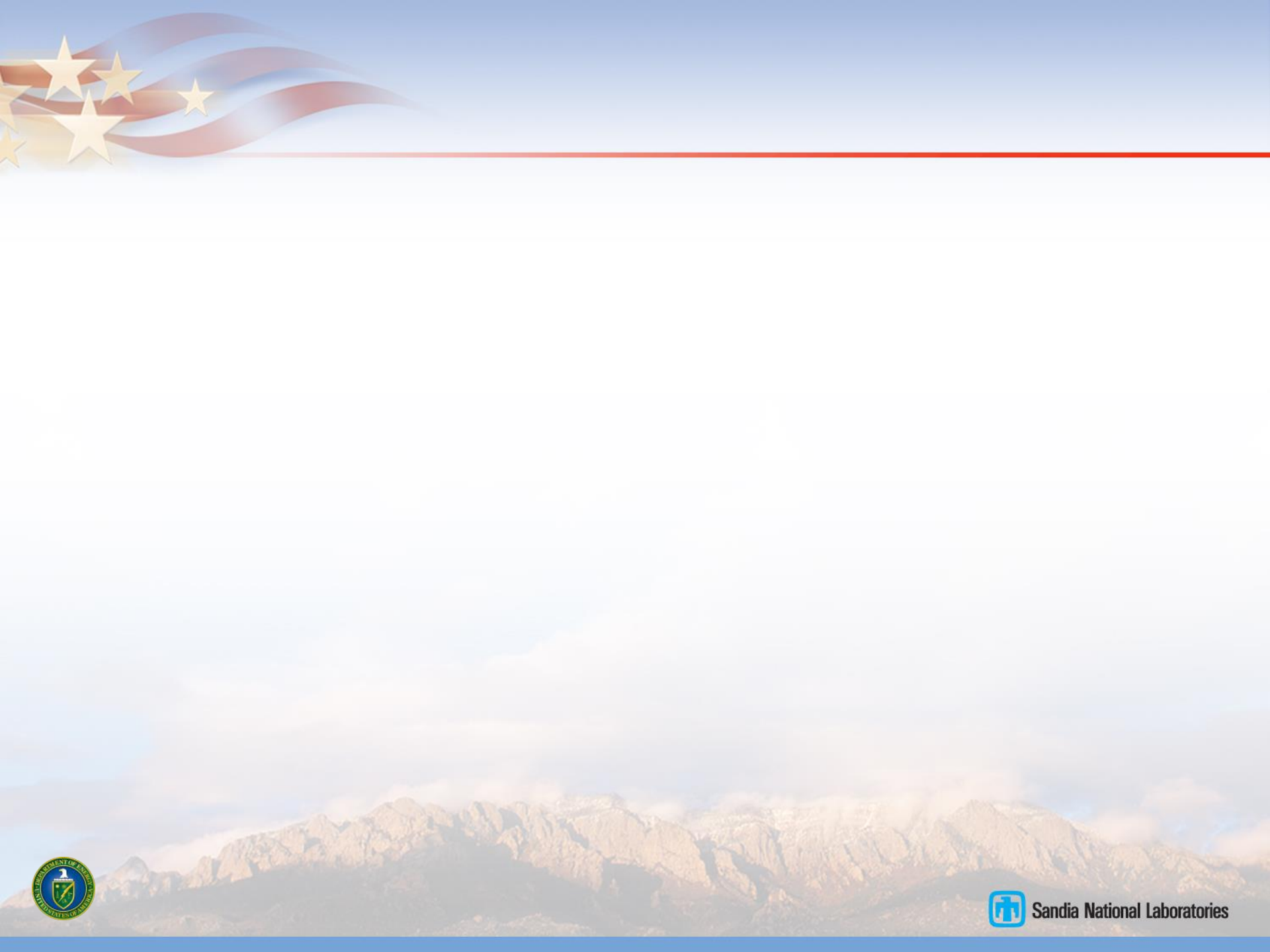
Collaborators: D.L. Buller & S. Briggs

8/24/2017



Beam Line planned for the *in situ* SEM will be developed in phases. Ultimate plan is for multiple accelerators being attached for dual or triple beam experiments.





Feasibility of Studying Zircaloy 2 at Nominally 1 atm

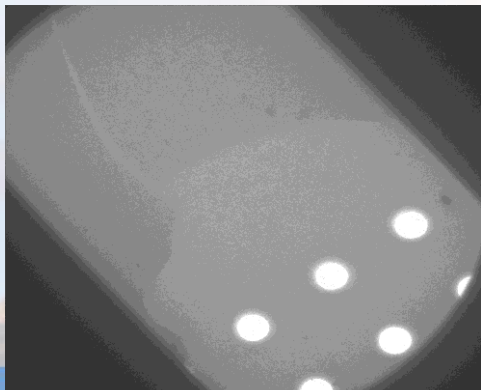
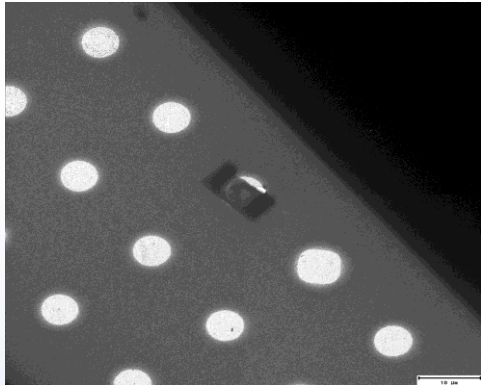
Collaborators: S. Rajasekhara and B.G. Clark



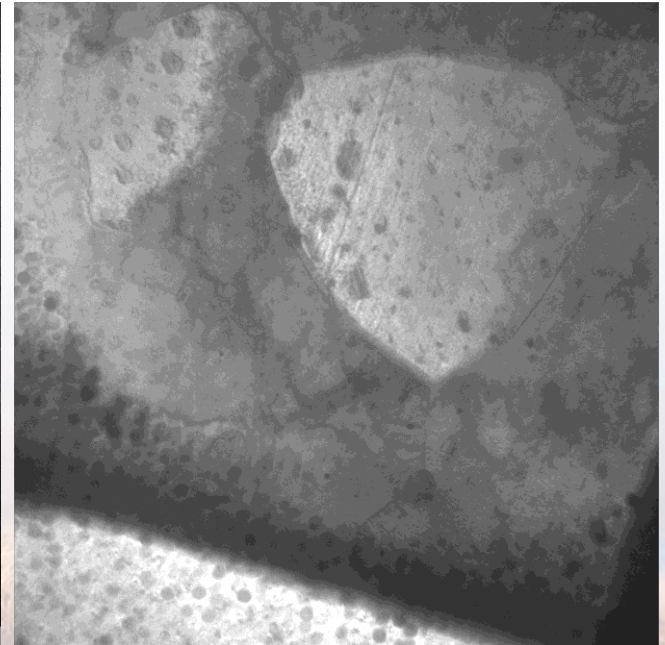
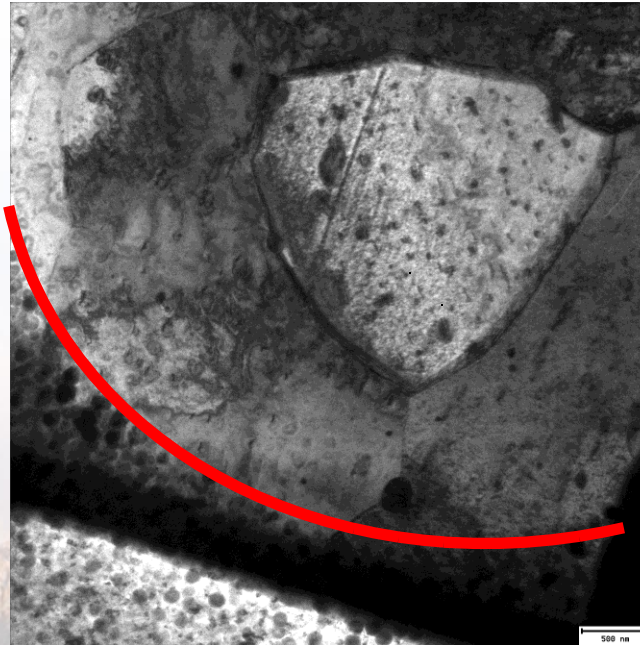
Vapor-Phase Heating TEM Stage

- Compatible with a range of gases
- *In situ* resistive heating
- Continuous observation of the reaction channel
- Chamber dimensions are controllable
- Compatible with MS and other analytical tools

Vacuum & Single Window



Nominally 1 atm H₂ & Two Windows

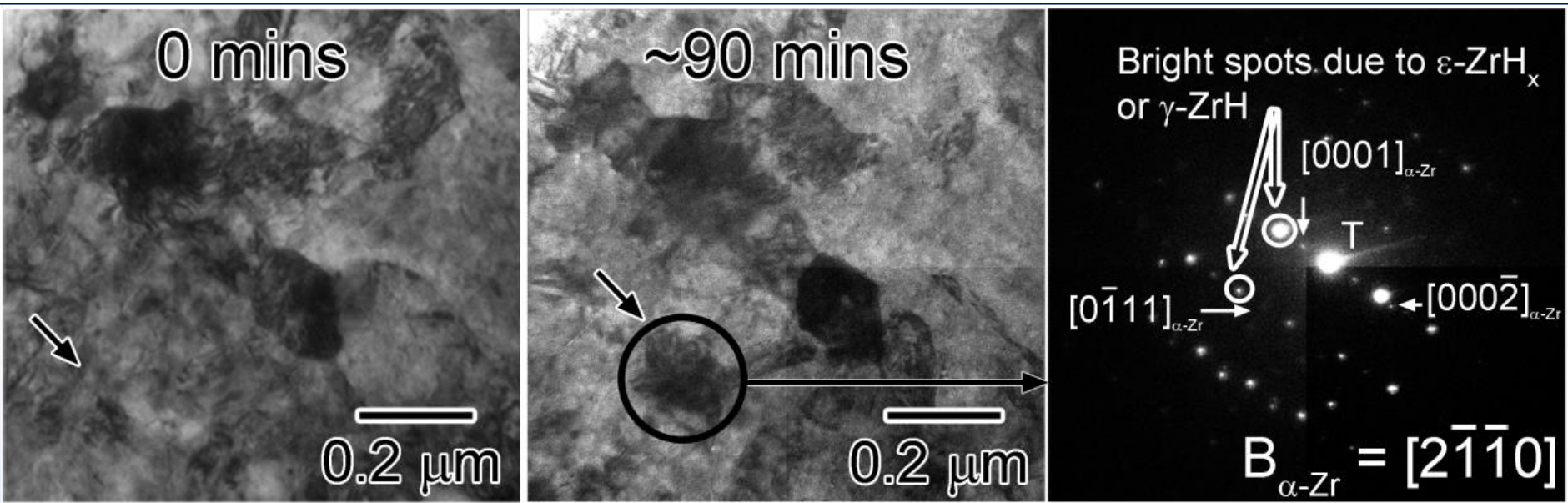


Most features are observed in both despite the decreased resolution resulting from the additional SiN window and 5 μm of air

In situ Observation of Hydride Formation in Zirlo

Collaborators: S. Rajasekhara and B.G. Clark

Absolute hydrogen pressure: 327 torr (~ 0.5 atm),
Ramp rate: 1°C/s , Final temperature: $\sim 400^\circ\text{C}$, Dwell time: ~ 90 mins



Hydride formation shown, for the first time by use of a novel TEM gas-cell stage, at elevated temperature and hydrogen pressure



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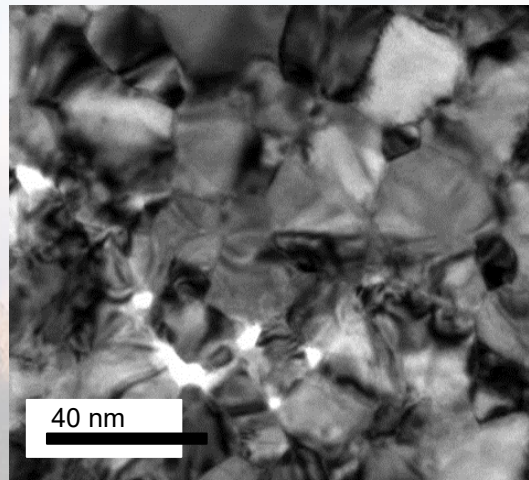
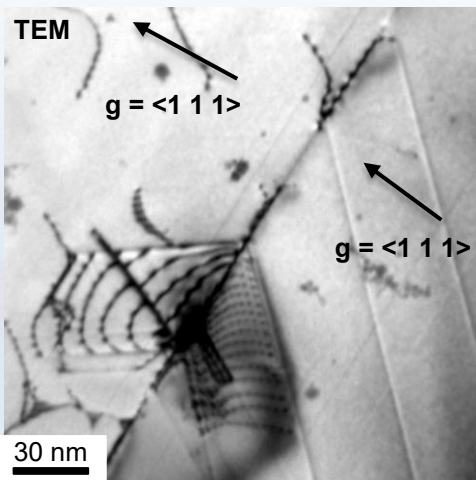
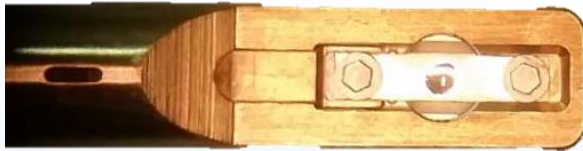
In situ Mechanical Testing

Qualitative “Bulk” Mechanical Testing

Minimal control over displacement and no “out-of-box” force information

- Successful in studies in observing dislocation-GB interactions/mechanisms
- Ideally both grains have kinematic BF 2-beam conditions: challenging in ST holder

Traditional Gatan Heating and Straining Holder



Quantitative Mechanical Testing

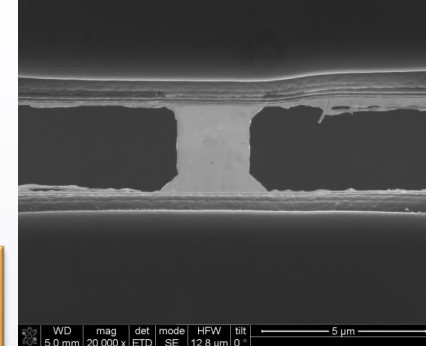
Minimal control over displacement and no “out-of-box” force information

- Sub nanometer displacement resolution
- Quantitative force information with μN resolution

Hysitron PI-95 Holder



Micro Tension Bars



Notched Bar



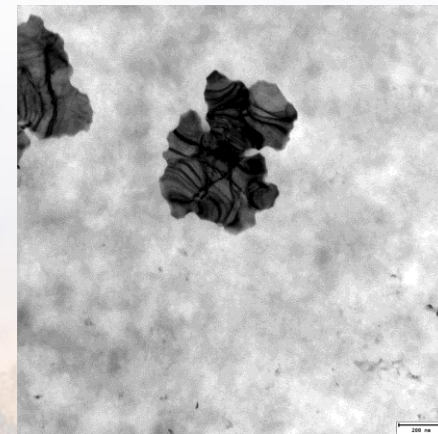
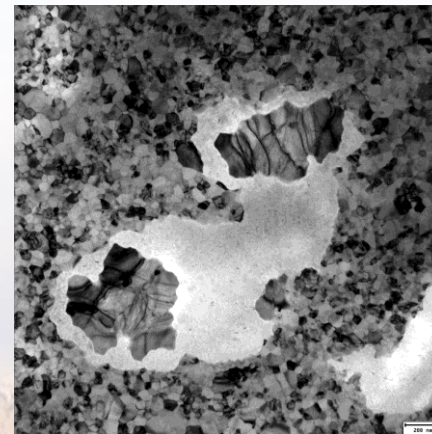
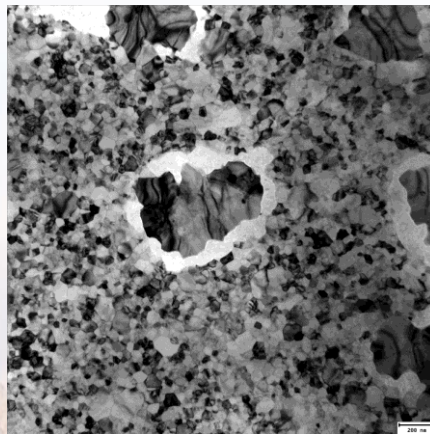
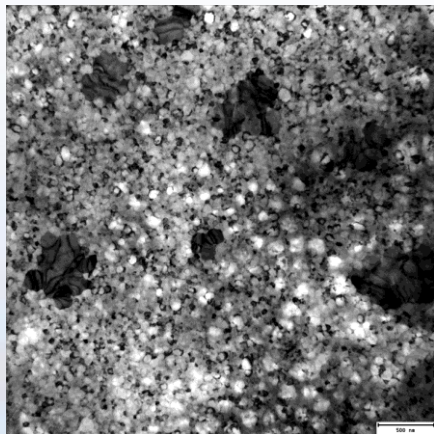
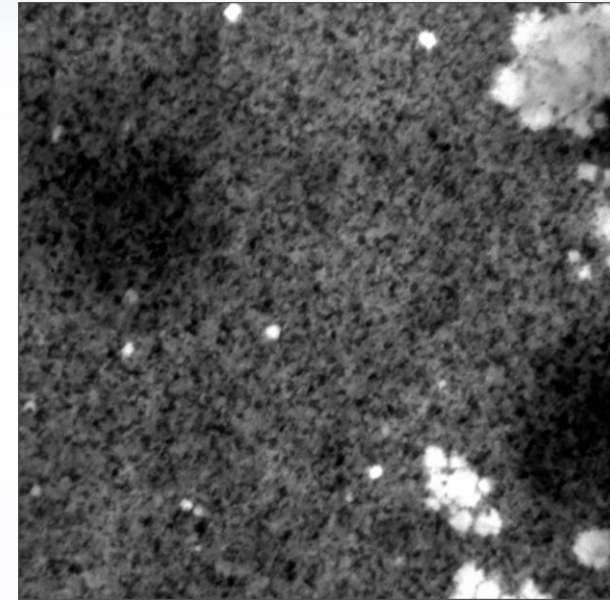
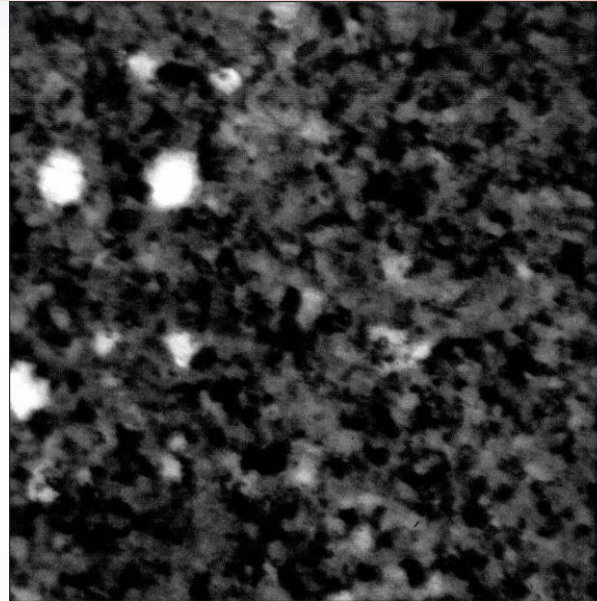
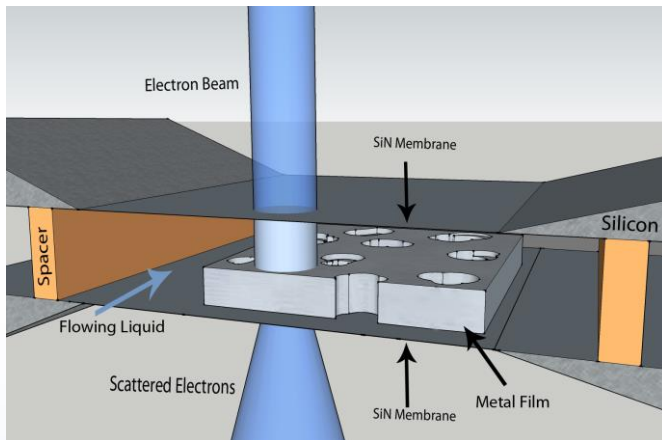
- 1) Indentation
- 2) Tension
- 3) Fatigue
- 4) Creep
- 5) Compression
- 6) Bend

Can We Gain Insight into the Corrosion Process through *In situ* TEM?

Contributors: D. Gross, J. Kacher, I.M. Robertson & Protochips, Inc.

Microfluidic Stage

- Mixing of two or more channels
- Continuous observation of the reaction channel



**Pitting mechanisms during dilute flow of acetic acid over 99.95% nc-PLD Fe involves many grains.
Large grains resulting from annealing appear more corrosion tolerant**