



MELCOR for HTGR Applications

NURETH-19, Advanced Reactors General - II, Track 7, Virtual Meeting

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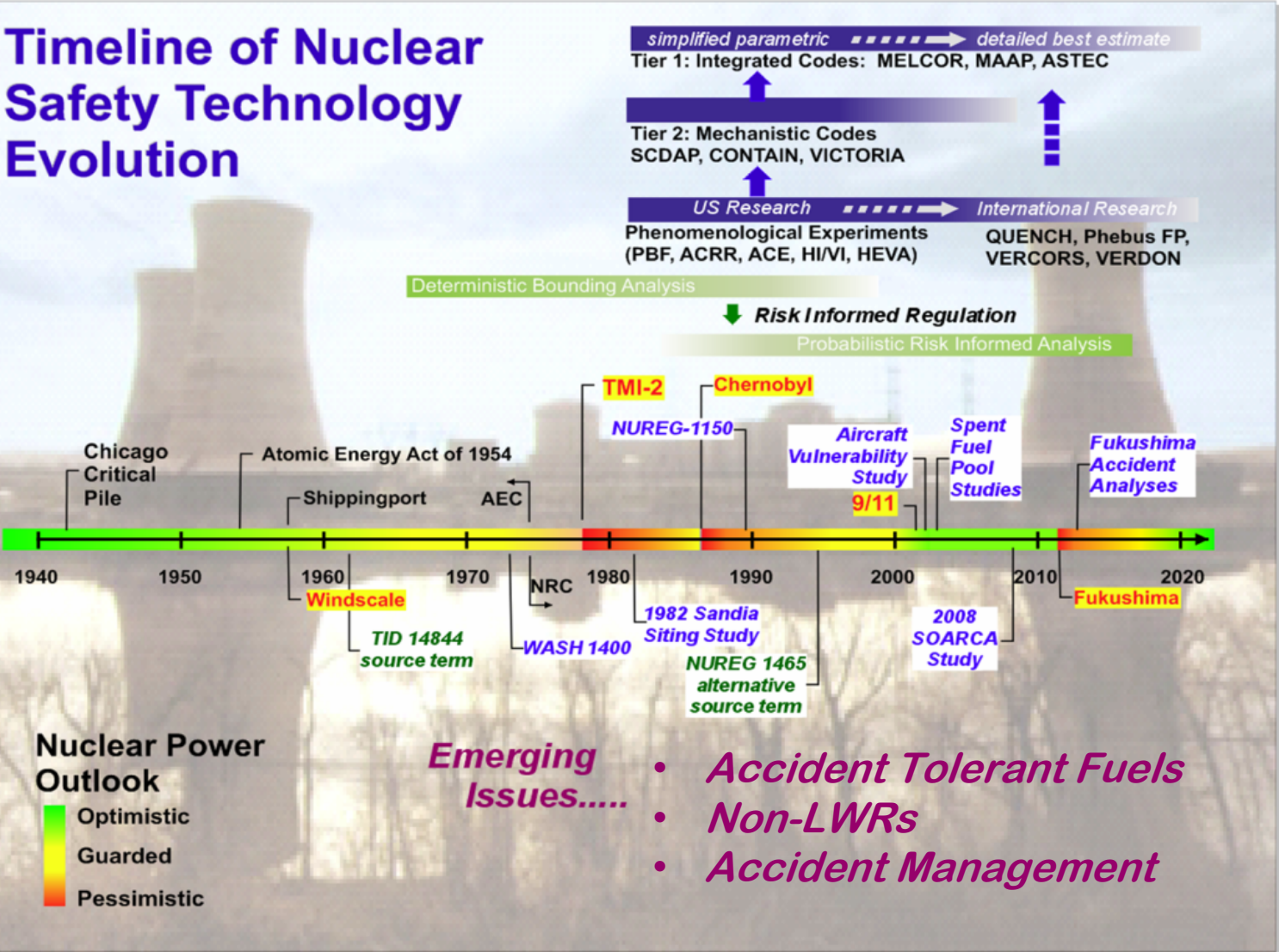
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Outline

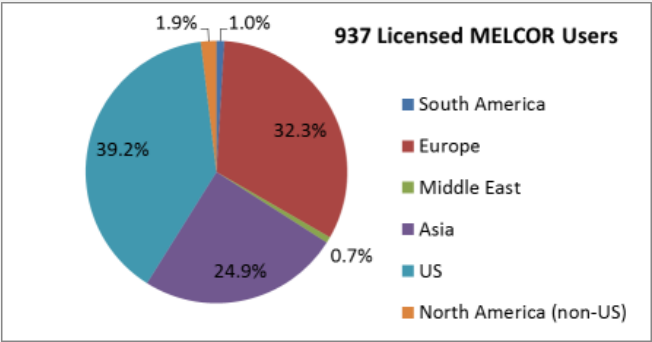
- MELCOR code introduction
- Describe MELCOR physics models and capabilities for HTGRs in limited detail
 - Diffusional fission product release model
 - Core component conduction (intracell, intercell, core boundary)
 - Core control volume convection/flow
 - Graphite oxidation
 - Point reactor kinetics equations
- Conclusions

MELCOR – History and Introduction

Timeline of Nuclear Safety Technology Evolution



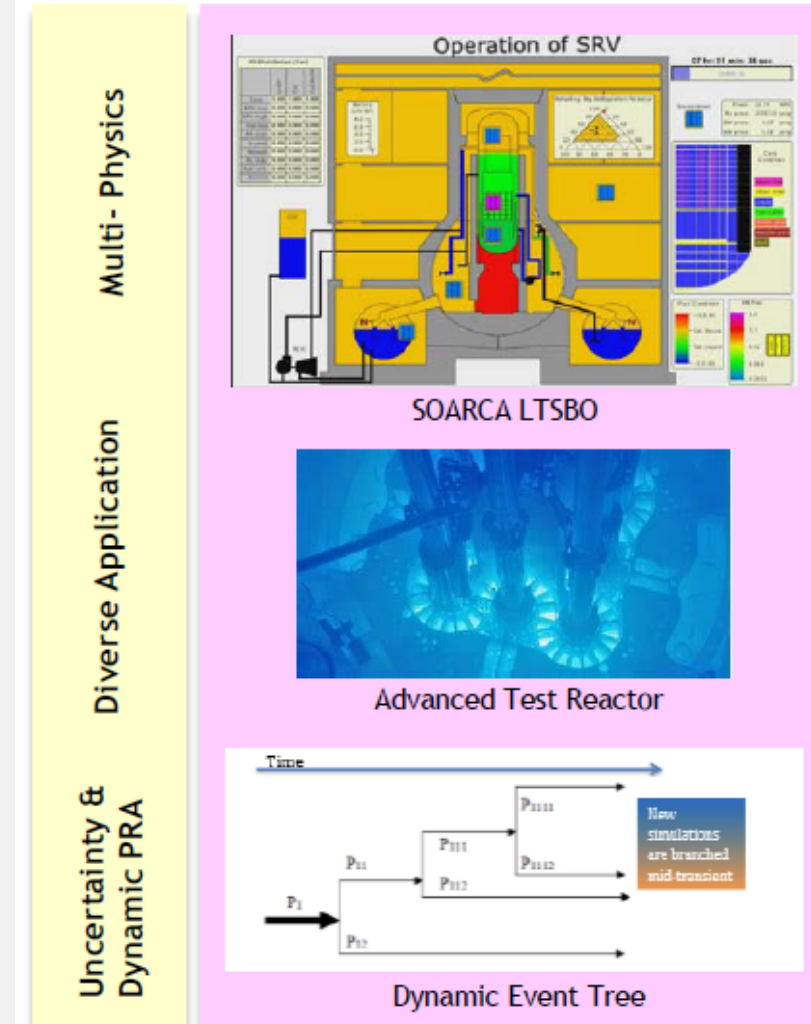
- Began in 1982 shortly after TMI-2
- Replaced Source Term Code Package
- Systems-level approach to modeling
- Emphasis on “best-estimate”
- Repository of knowledge
- Global standard (used by 31+ nations)
 - Users’ groups (AMUG & EMUG)
 - Annual CSARP/MCAP meetings



- Used by USNRC, USDOE & US industry
- Used for naval reactors (US/UK)
- Evolves to meet regulatory needs

MELCOR – History and Introduction

- Fully integrated, multi-physics engineering-level code
 - Thermal-hydraulic response in the reactor coolant system, reactor cavity, containment, and confinement buildings
 - Core heat-up, degradation, and relocation
 - Core-concrete attack
 - Hydrogen production, transport, and combustion
 - Fission product release and transport behavior
- Diverse application
 - Multiple core designs
 - Models built from basic code constructs
 - Adaptability to new or non-traditional reactor designs (ATR, Naval, VVER)
- Validated physics models (ISP's, benchmarks, experiments, accidents)
- Uncertainty analysis & dynamic PRA (fast-running, reliable, access to parameters)
- User convenience
 - Windows/Linux versions
 - User utilities and post-processing/visualization capabilities
 - Extensive code documentation

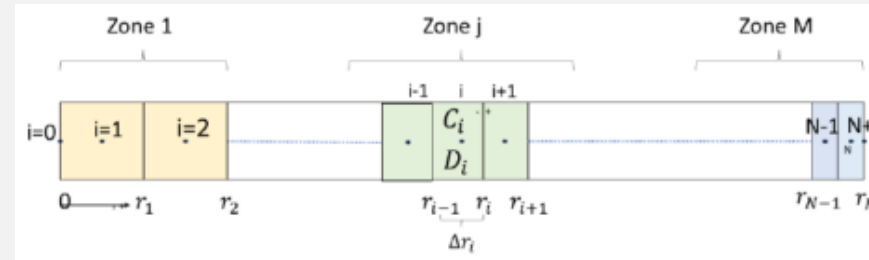


Diffusional Fission Product Release

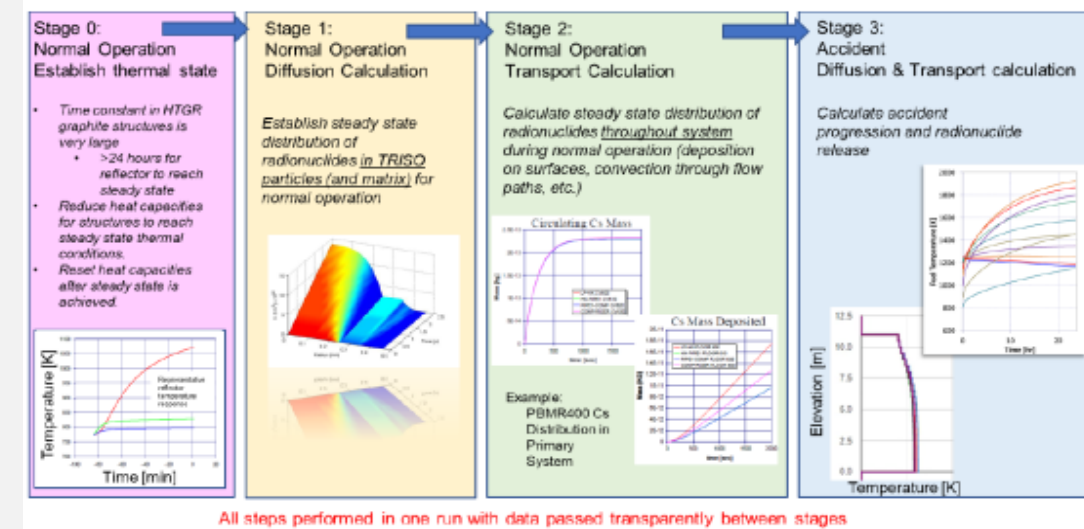
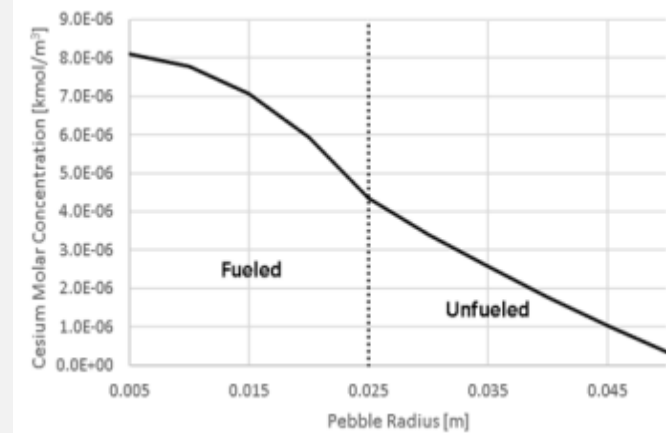
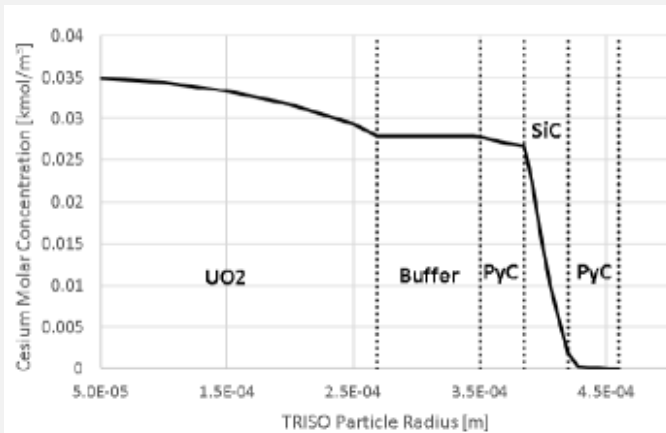
- Overall goals of the HTGR diffusional release model:
 - Predict radionuclide distributions within fuel elements in the core, and
 - Predict radionuclide release from fuel to coolant
- Entails a finite volume diffusion solver and a specially-tailored analytic release model for failed TRISO fuel

$$x \frac{\partial C}{\partial t} = \frac{1}{r^n} \frac{\partial}{\partial r} \left(r^n D \frac{\partial C}{\partial r} \right) - \lambda C + \beta$$

$$D(T) = D_0 e^{-Q/RT}$$



- Solution algorithm and run-time strategy
- Get molar concentration profiles and diffusional release



Conduction

- Intercell
 - Between like or unlike components in axially or radially adjacent core cells, pending conduction logic
 - Formulated as a conductance times a temperature difference with the conductance equaling a parallel combination of:
 - FU term (geometry, solid material conductivity)
 - MX term (geometry and effective conductivity consisting of radiation, fluid conduction, and solid conduction terms)
 - Tanaka and Chisaka formulation for PMR
 - Zehner-Schlunder-Bauer formulation for PBR
- Intracell
 - Between components within the same core cell, more precisely fuel (FU) and matrix (MX) component in the context of HTGRs
 - Formulated as a conductance times a temperature difference with the conductance equaling a parallel combination of:
 - FU term (geometry, solid material conductivity)
 - MX term (geometry, solid material conductivity)
 - Another user-specified conductivity if necessary
- Boundary
 - Allow thermal energy to pass from core periphery (usually a reflector region) to bounding heat structures (usually the core barrel region)
 - Accounts for heat transfer across a user-specified gap
 - Key for modeling passive safety features of HTGRs

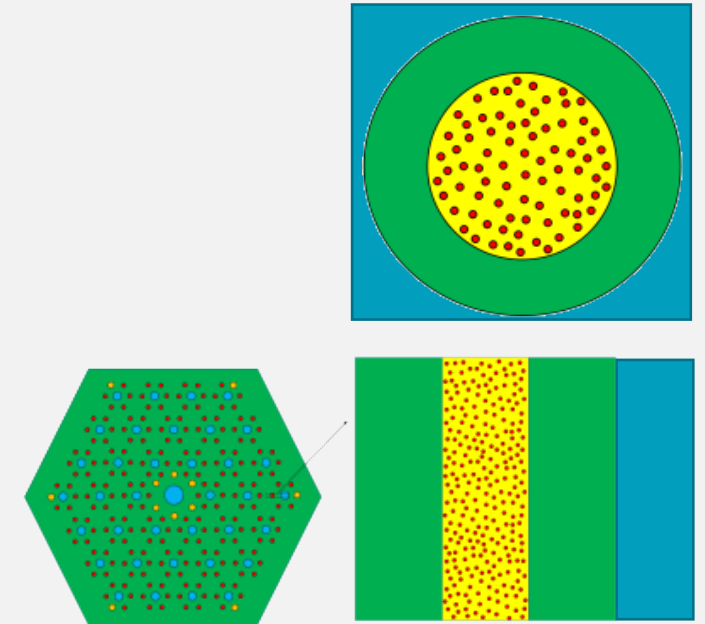
Convection/Flow

- Nusselt number correlations for heat transfer coefficients
 - Pebble fueled systems use correlations for isolated spherical particles:

$$NU_{forced} = 2.0 + 0.6Re_f^{1/2}Pr_f^{1/3}$$

$$NU_{natural} = 2.0 + 0.6Gr_f^{1/4}Pr_f^{1/3}$$

- Prismatic fuel systems utilize conventional Dittus-Boelter correlation forms
 - Forced/natural laminar/turbulent Nusselt numbers computed and maximum used
 - Sensitivity coefficient access to all correlation multipliers, constants, exponents
- For either pebble bed or prismatic cores, the matrix component thickness contributes to heat transfer resistance
- For pebble bed cores, use packed bed correlations (Ergun, Achenbach)
 - Compute flow resistance and pressure loss
 - Frame as K-loss for convenience given phasic velocity equation forms



$$K_L(\epsilon, Re) = \left[C_1 + C_2 \frac{1-\epsilon}{Re} + C_3 \left(\frac{1-\epsilon}{Re} \right)^{C_4} \right] \frac{(1-\epsilon)L}{\epsilon D_p}$$

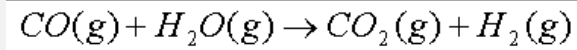
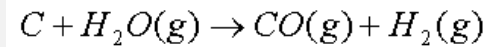
Correlation	C ₁	C ₂	C ₃	C ₄
Ergun (original)	3.5	300.	0.0	-
Modified Ergun (smooth)	3.6	360.	0.0	-
Modified Ergun (rough)	8.0	360.	0.0	-
Achenbach	1.75	320.	20.0	0.4

Graphite Oxidation

Steam oxidation

$$R_{OX,steam} = \frac{k_4 P_{H_2O}}{1 + k_5 P_{H_2}^{0.5} + k_6 P_{H_2O}}$$

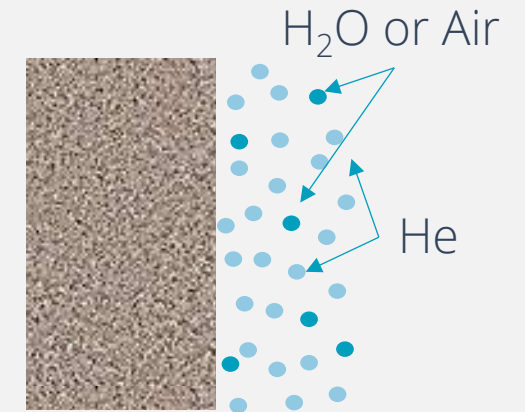
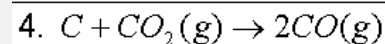
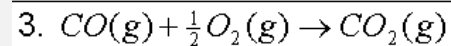
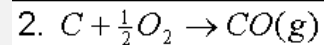
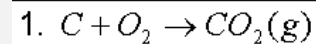
Reaction



Air oxidation

$$R_{OX} = 1.7804 \times 10^4 \exp\left(-\frac{20129}{T}\right) \left(\frac{P}{0.21228 \times 10^5}\right)^{0.5}$$

Reaction



*Both steam and air include rate limit due to steam/air diffusion towards active oxidation surface

- Graphite oxidation active in the context of a new generalized oxidation model
- For air oxidation, empirical correlation determines relative production of CO/CO₂

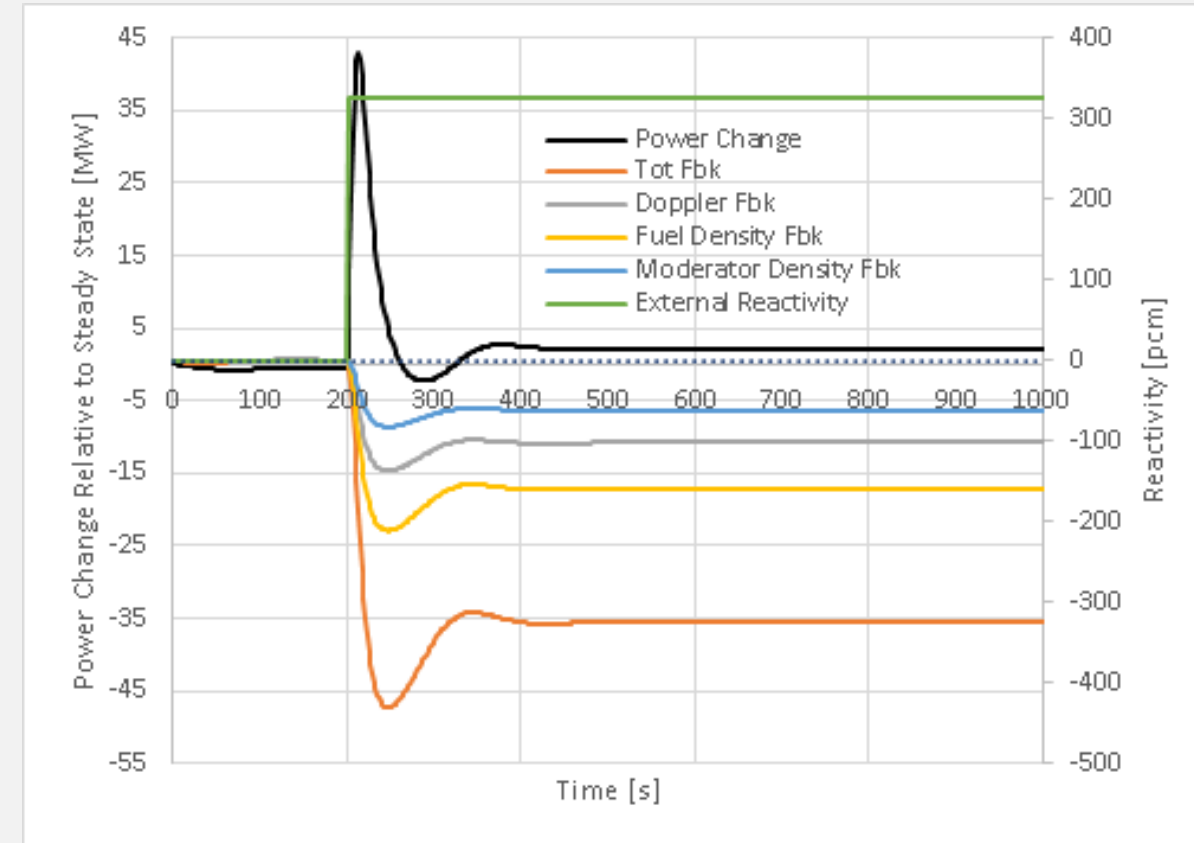
$$f = \chi_{CO} / \chi_{CO_2} = 7396 * \exp\left(-\frac{69604}{8.314 * T_{MX|RF|SS}}\right)$$

Point Reactor Kinetics Equations

- Standard PRKE treatment

$$\frac{dP}{dt} = \left(\frac{\rho - \beta}{\Lambda} \right) P + \sum_{i=1}^6 \lambda_i Y_i + S_0$$
$$\frac{dY_i}{dt} = \left(\frac{\beta_i}{\Lambda} \right) P - \lambda_i C_i, \quad \text{for } i = 1 \dots 6$$

- Kinetics data accessible by SC
- Feedback models
 - CF-specified external component
 - Doppler
 - Fuel and moderator density
- Define COR cell ranges as regions over which averages are taken to inform feedback models
- Useful capability for ATWS-type scenarios



Conclusions

- To facilitate MELCOR modeling of HTGRs, several physics models and capabilities were developed and integrated
- An approach consistent with the systems-level modeling philosophy of MELCOR was taken
- Noteworthy aspects of HTGR physics models and/or modeling capabilities presented here
 - Further detail in NURETH-19 paper
 - Still further detail in MELCOR code documentation
 - Publicly-released demonstration calculation performed for USNRC