

LA-UR-23-29659

Approved for public release; distribution is unlimited.

Title: Preliminary results of lead shielding effects on neutron assay of 233U contain items

Author(s): Jackson, Daniel Erik
Lockhart, Madeline Louise
Henzl, Vlad

Intended for: I will be presenting this as part of a independent review (Schubert Review) for the Thorium Fuel Safeguards project.

Issued: 2023-08-31 (rev.1)



Los Alamos National Laboratory, an affirmative action/equal opportunity employer, is operated by Triad National Security, LLC for the National Nuclear Security Administration of U.S. Department of Energy under contract 89233218CNA000001. By approving this article, the publisher recognizes that the U.S. Government retains nonexclusive, royalty-free license to publish or reproduce the published form of this contribution, or to allow others to do so, for U.S. Government purposes. Los Alamos National Laboratory requests that the publisher identify this article as work performed under the auspices of the U.S. Department of Energy. Los Alamos National Laboratory strongly supports academic freedom and a researcher's right to publish; as an institution, however, the Laboratory does not endorse the viewpoint of a publication or guarantee its technical correctness.



Preliminary results of lead shielding effects on neutron assay of ^{233}U containing items.

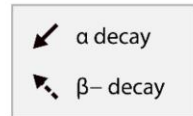
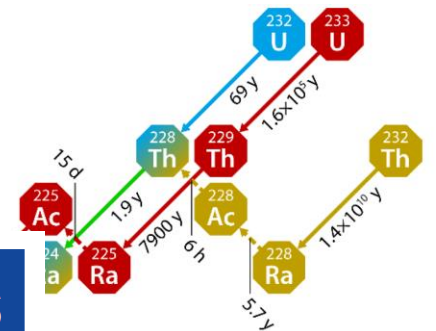
Daniel Jackson, Madeline Lockhart, Vlad Henzl

8/15/2023

Lead Shielding and ^{233}U containing items

- ^{232}U impurity causes a build up of ^{208}Tl in the material
- ^{208}Tl has a short half-life and associated gamma rays
- Lead shield workers

Metal	Dose Rate (rem/hr)	Hours
Weapon-grade plutonium	0.0013	3800
Reactor-grade plutonium	0.0082	610
^{233}U containing 1 ppm ^{232}U	0.013	380
^{233}U containing 5 ppm ^{232}U	0.059	80
^{233}U containing 100 ppm ^{232}U	1.27	4
^{233}U containing 1% ^{232}U	127	0.04



Safeguards Technology for Thorium Fuel Cycles: Research and Development Needs Assessment and Recommendations (2021)

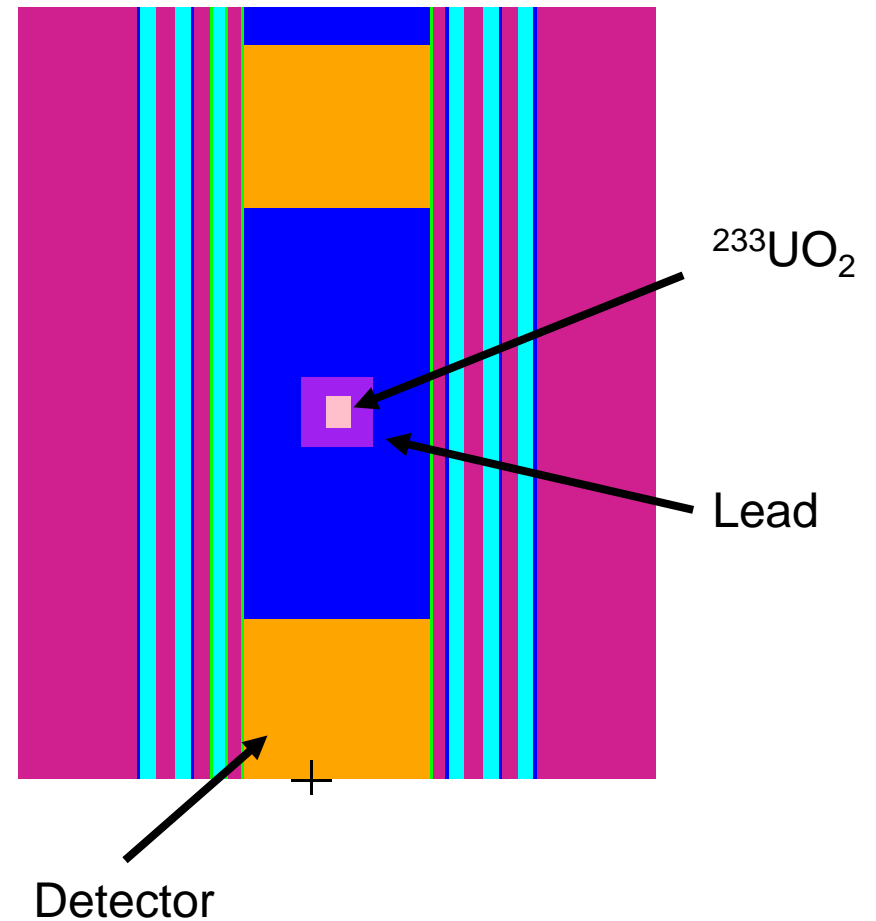
Possible effects of lead shielding

- 1 Moderation – Bad moderator, still could affect the detectors efficiency.
2. N-2N reactions – fission spectrums overlap with lead N-2N could cause increase in doubles.
3. Neutron Reflector – Neutrons reflected back into the sample inducing fission.

MCNP simulations of ENMC with lead shielded sources

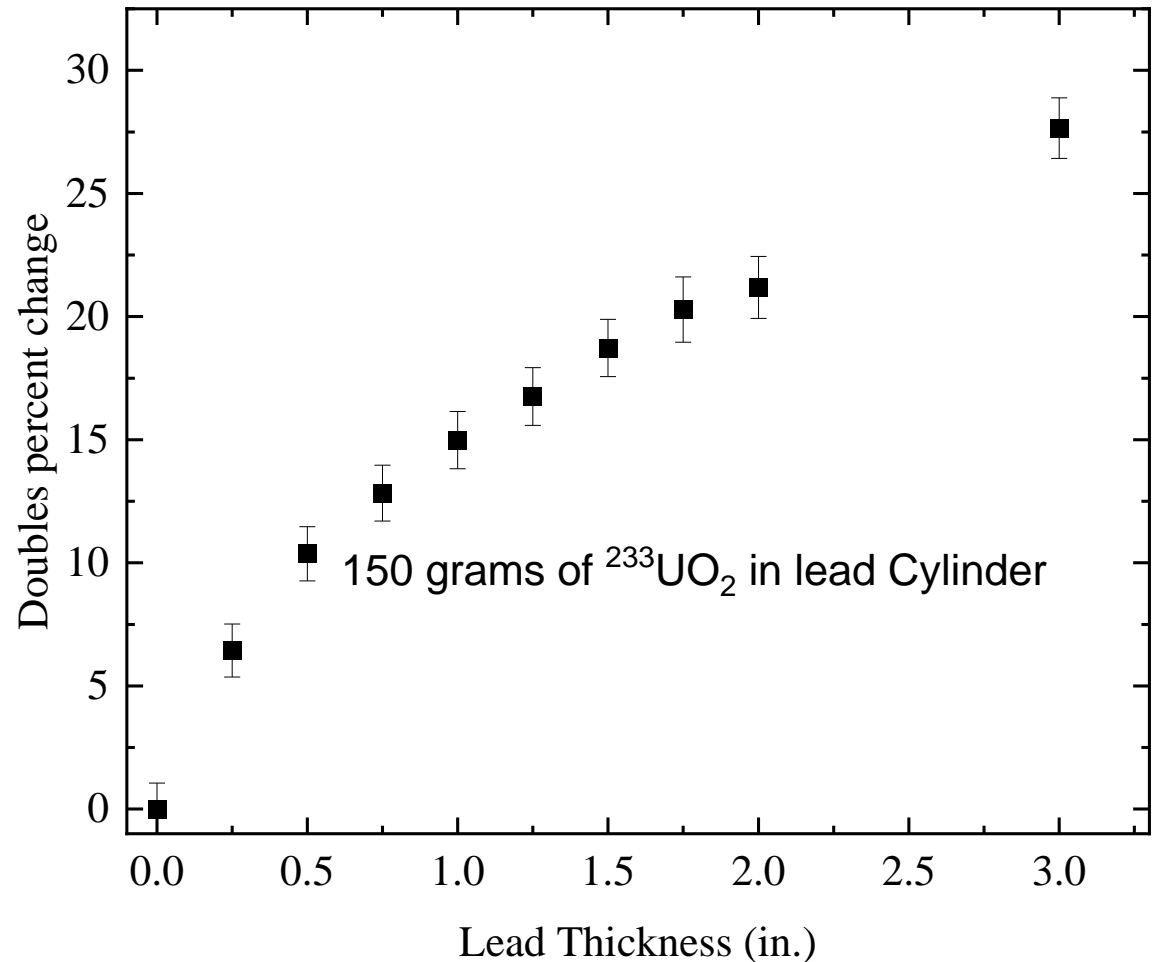
- Same mass and shape of $^{233}\text{UO}_2$
- Lead shielding thickness was changed to get trend
- Simulation results were run through in house software to generate rates based off standard gate lengths of ENMC
- High fidelity model of ENMC that has been bench marked to past experiments.

150 grams $^{233}\text{UO}_2$ inside of lead shielding



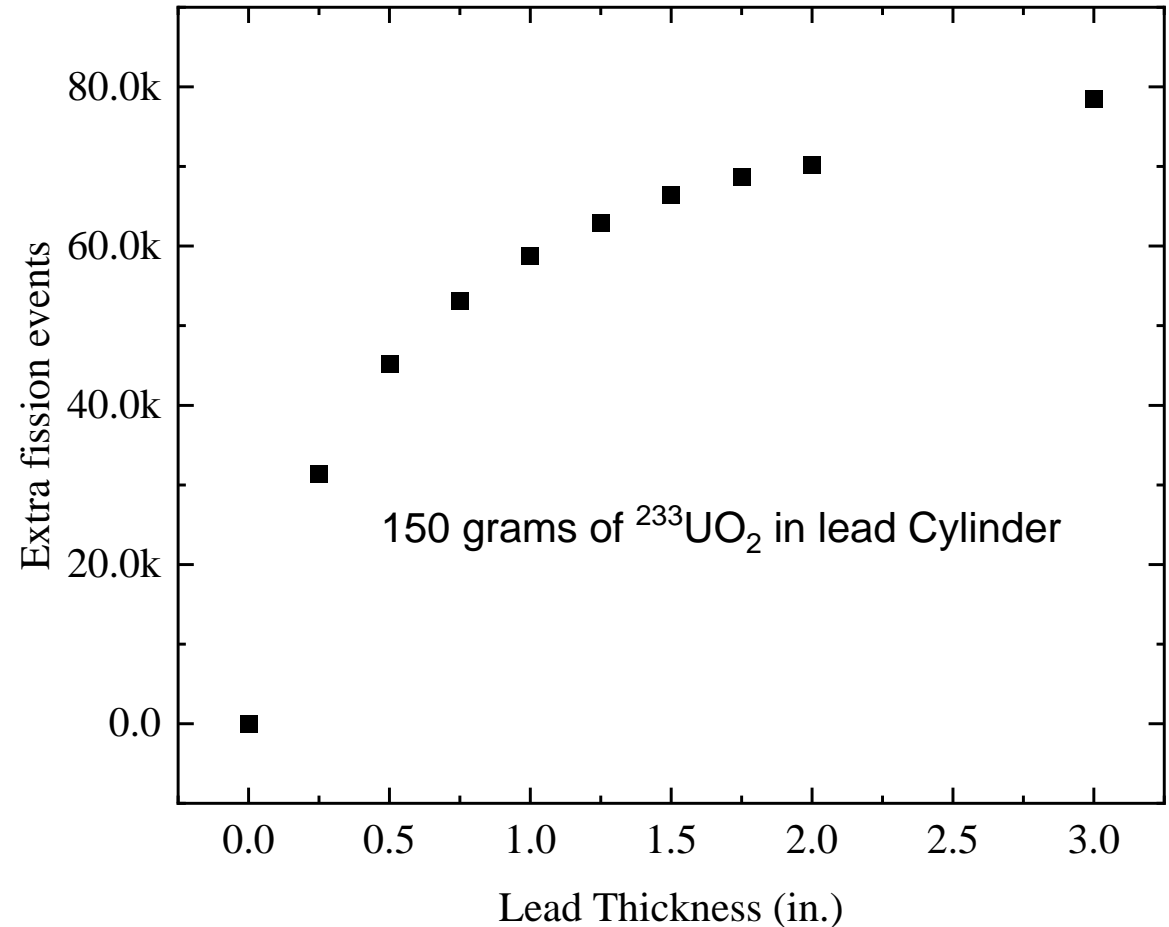
MCNP Simulations of $^{233}\text{UO}_2$ in lead shielding

- Lead shielding resulted in a significant increase in the doubles rate
- 3 inches is the maximum lead shielding fitting in ENMC

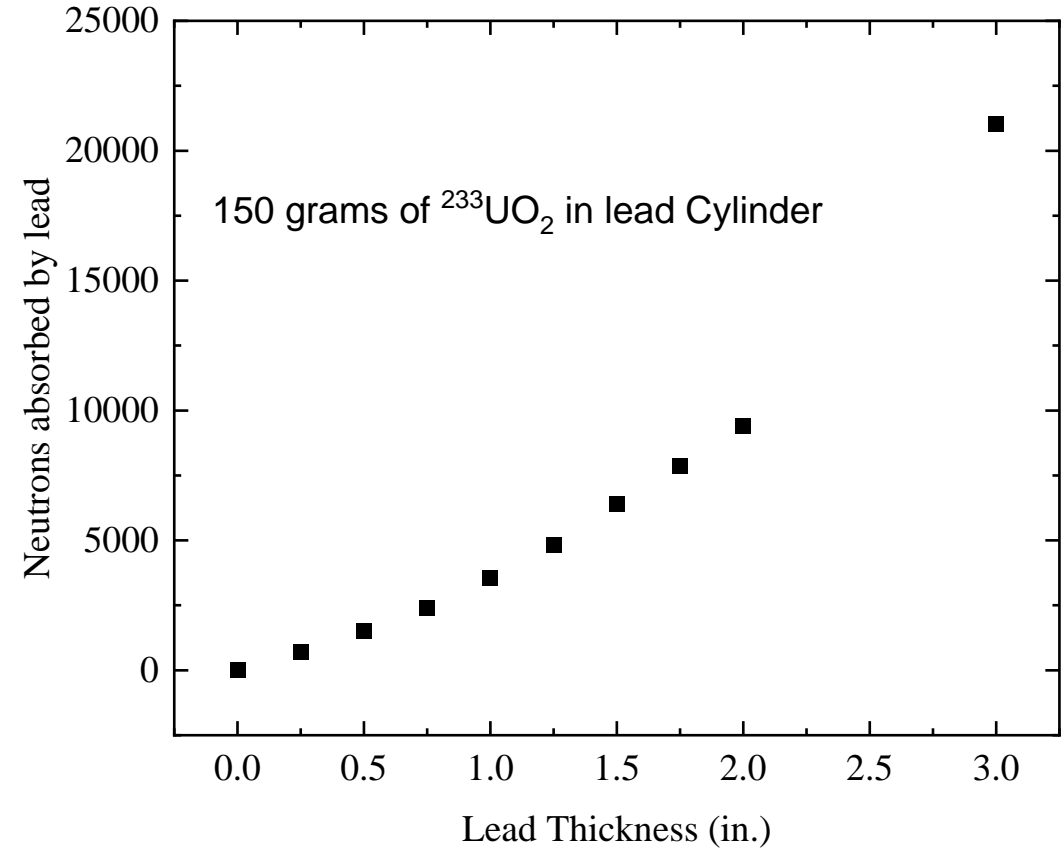
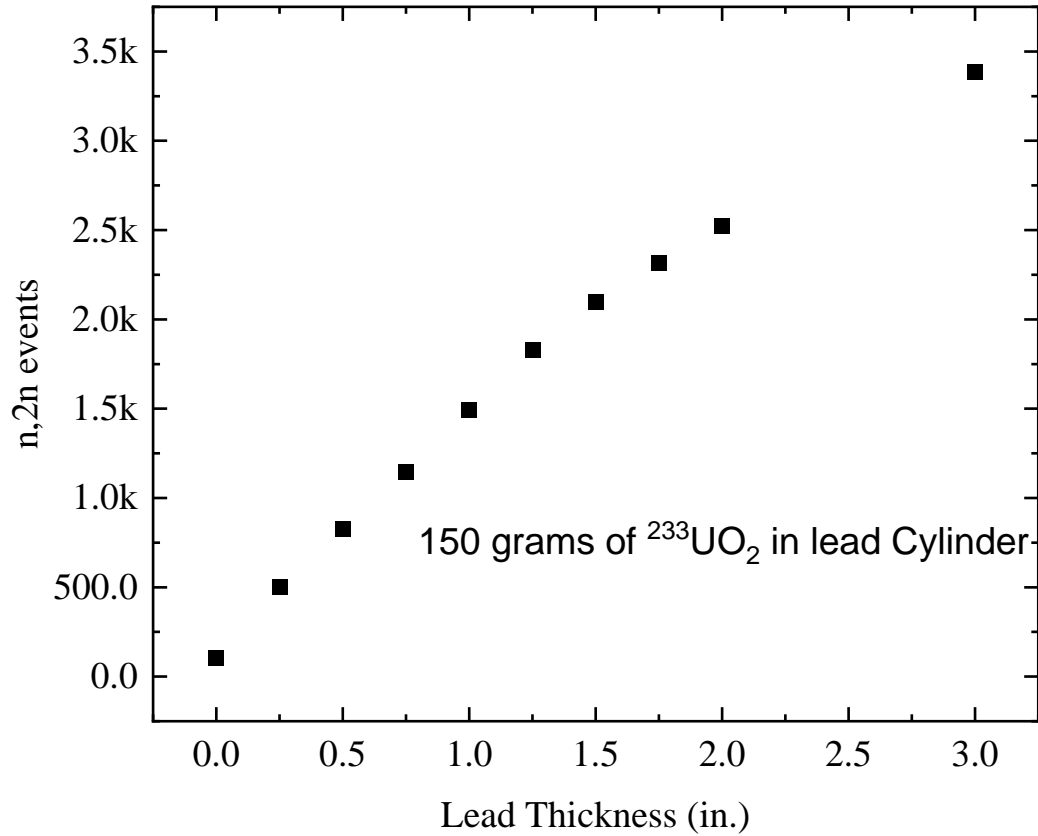


MCNP Simulations of $^{233}\text{UO}_2$ in lead shielding

- MCNP simulation reports number of fission events
- Lead increased number of fission events by 80,000 at 3 inches of lead.
- Neutron reflection strong effect on $^{233}\text{UO}_2$

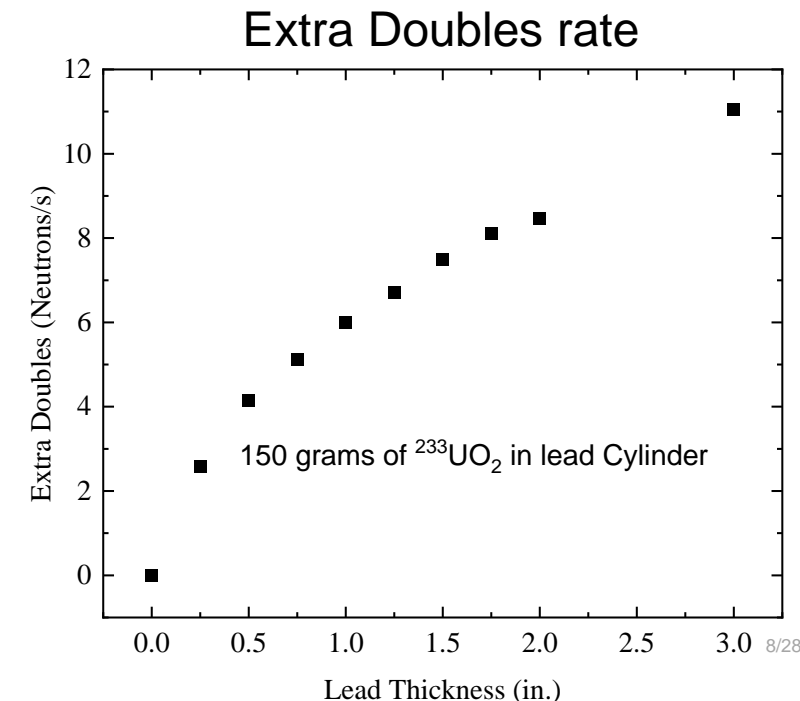
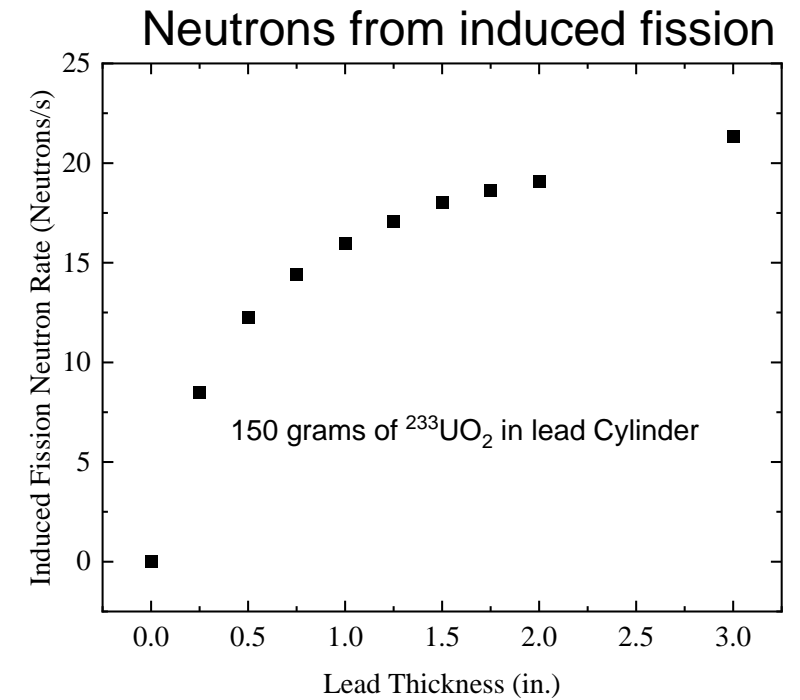


N-2N and lead absorption of neutrons

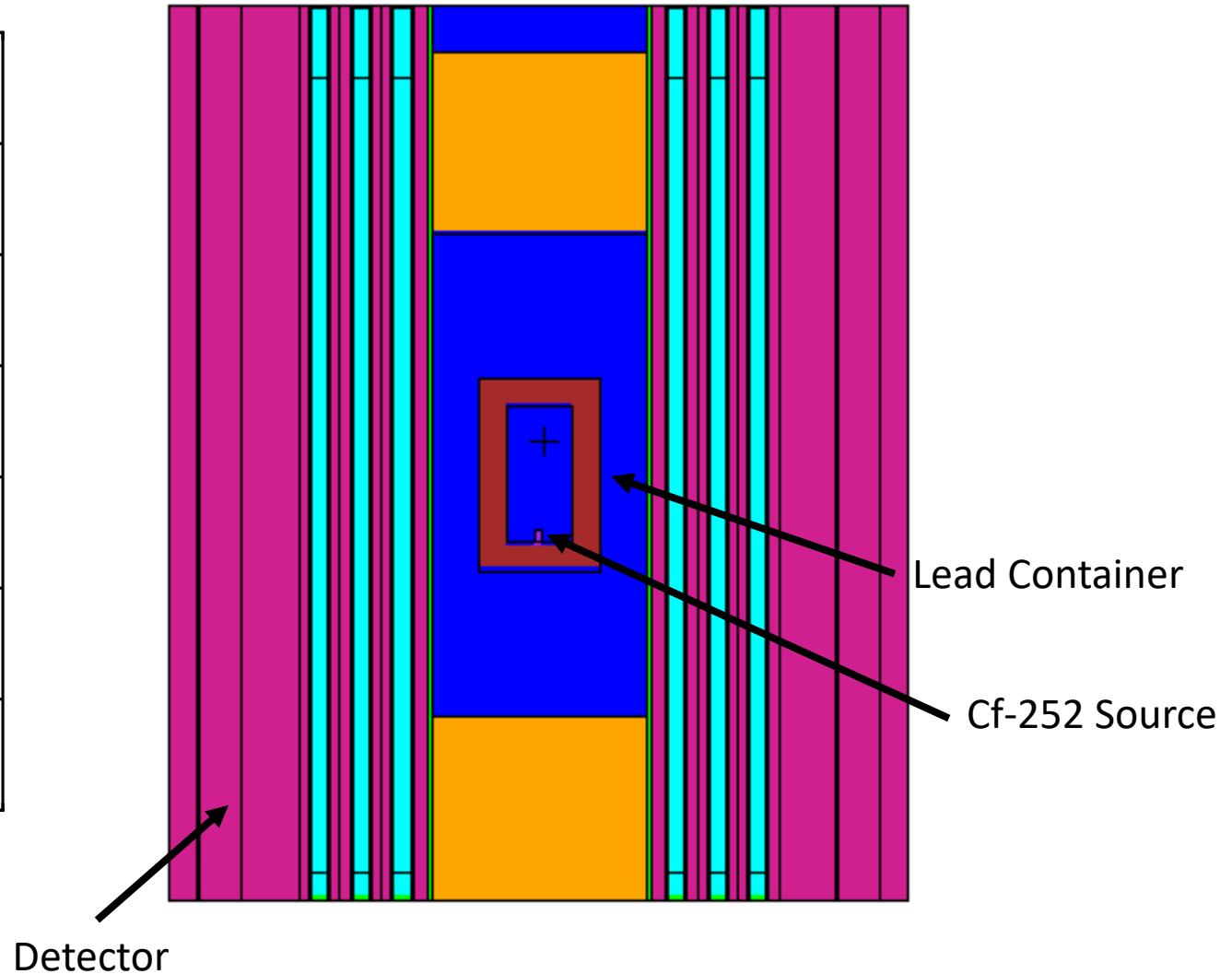
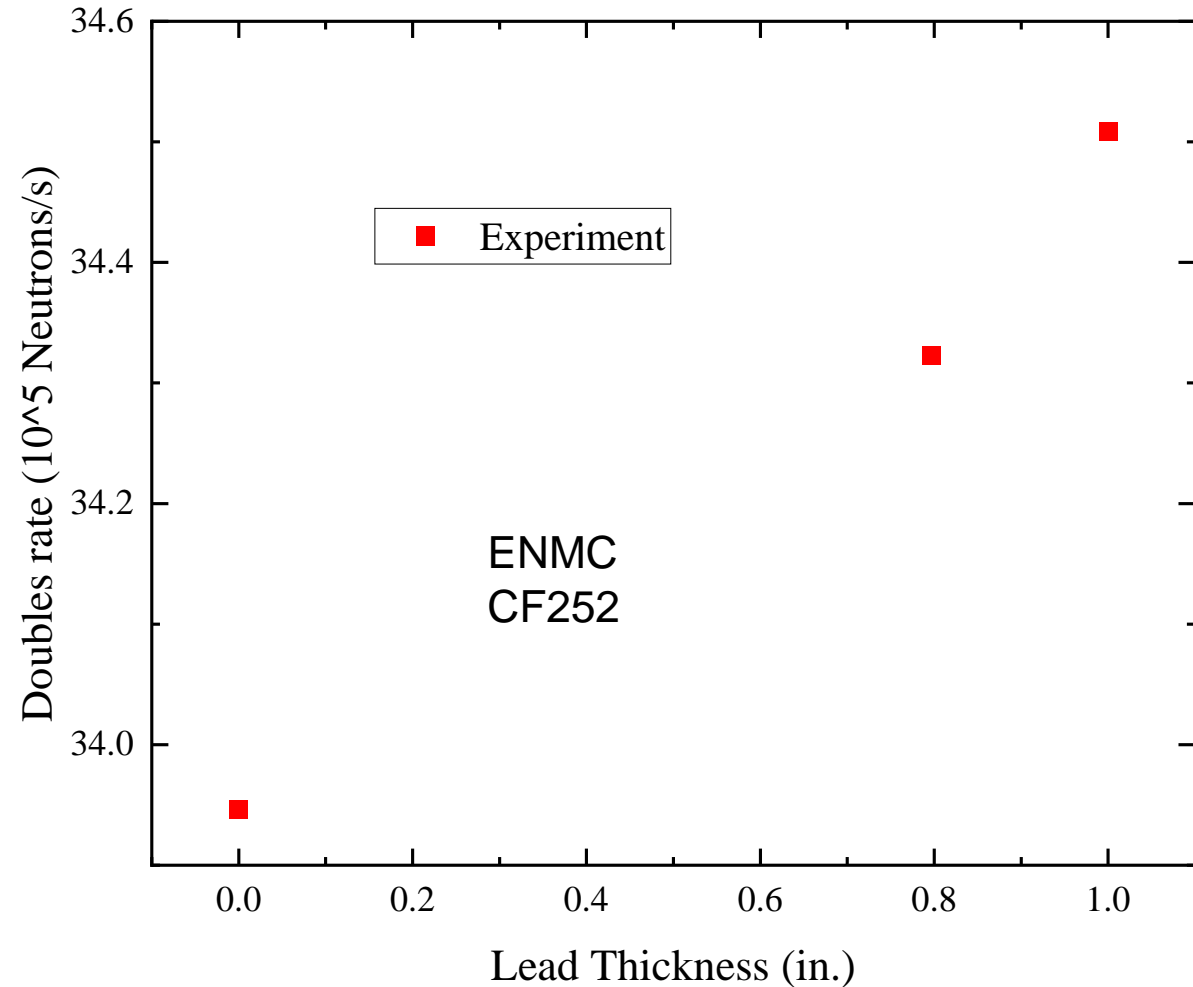


Induced fission drives the increase in doubles rate

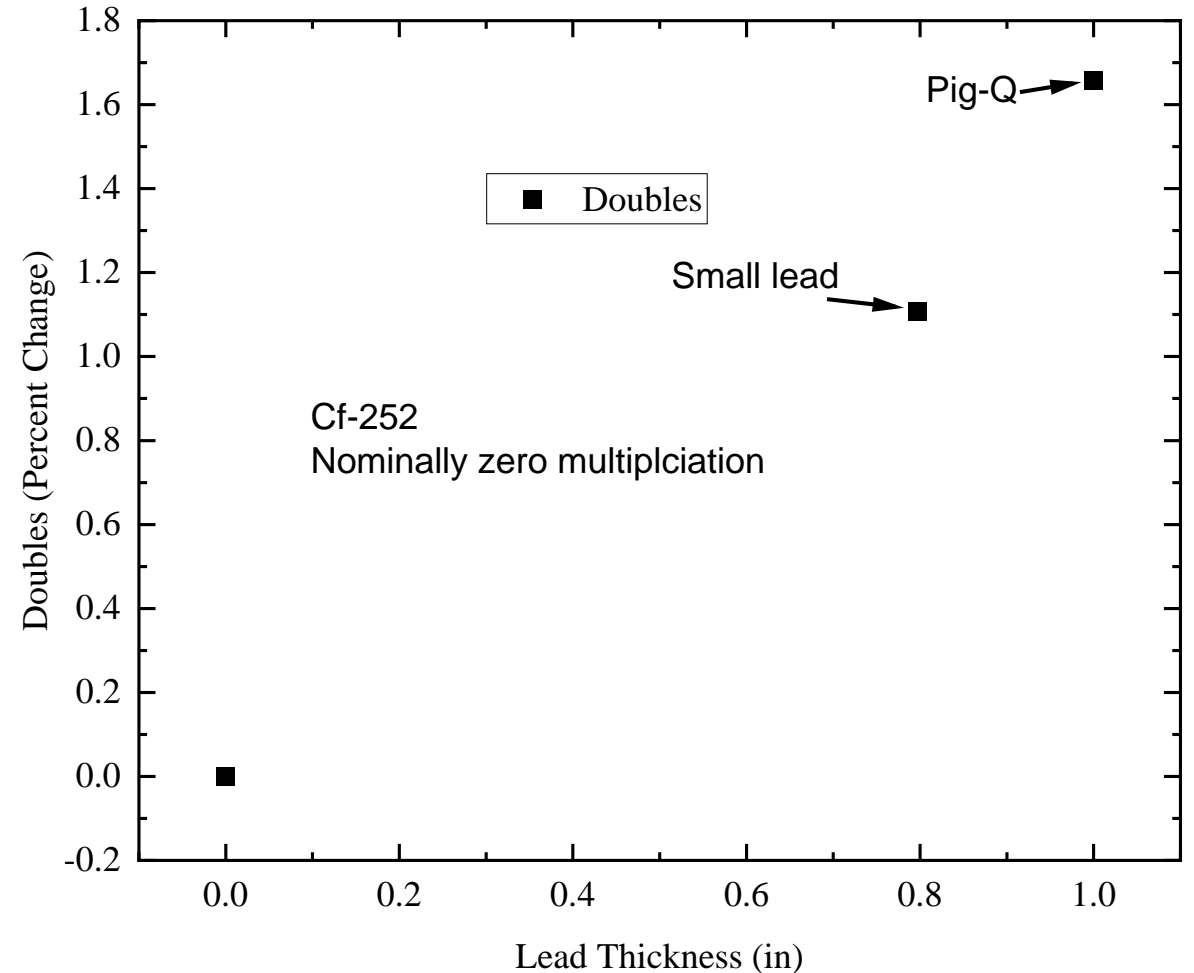
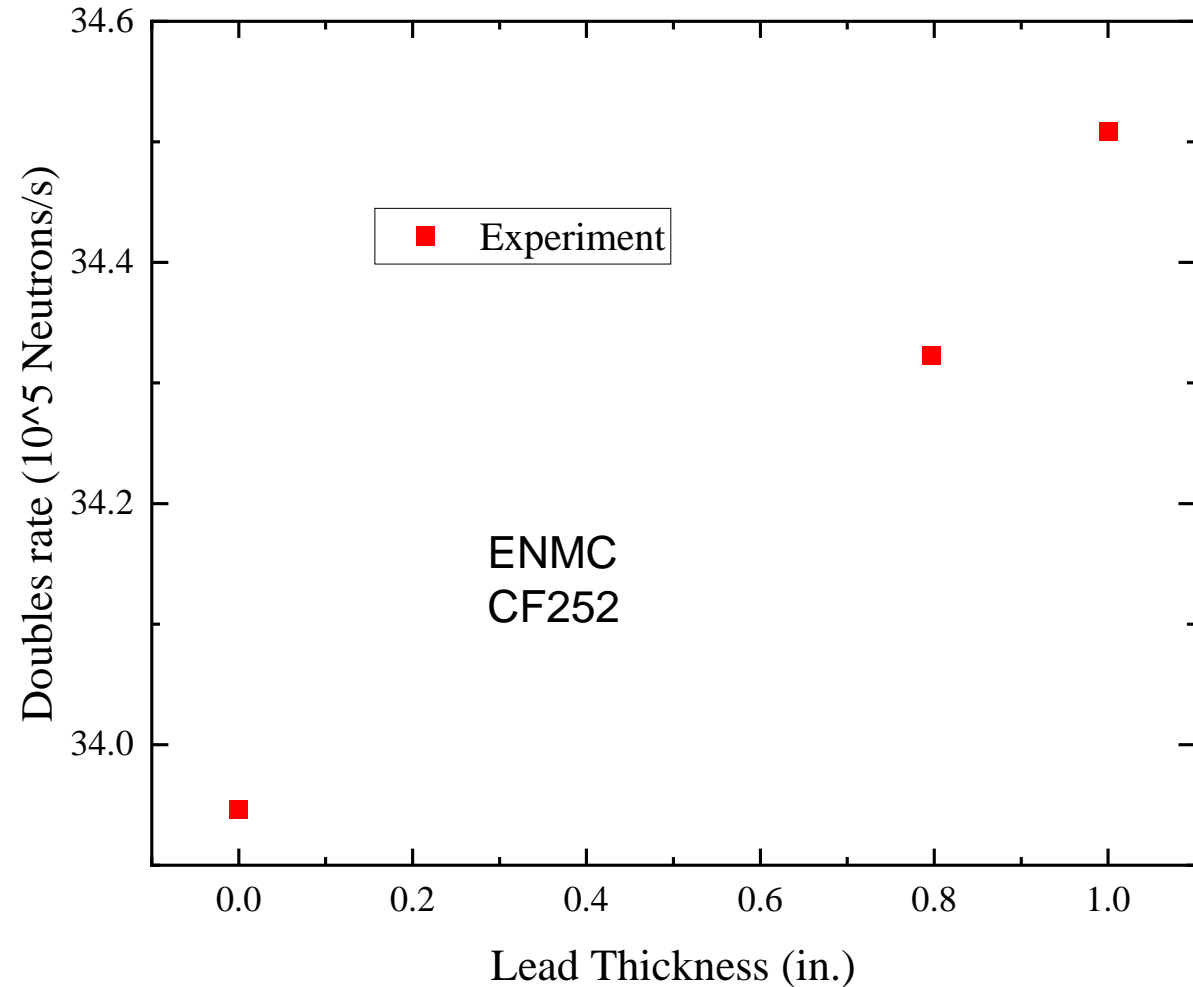
- The adding of lead shielding for $^{233}\text{UO}_2$ is predicted to significantly increase the doubles rate.
- The doubles rate in neutron measurements is often used to determine the mass of material present.
- Understanding how to adjust for this increase may require using triples as the amount of lead and material may both be unknown.



Measurement of Cf-252 to help benchmark effects of lead without access to sufficient $^{233}\text{UO}_2$ samples

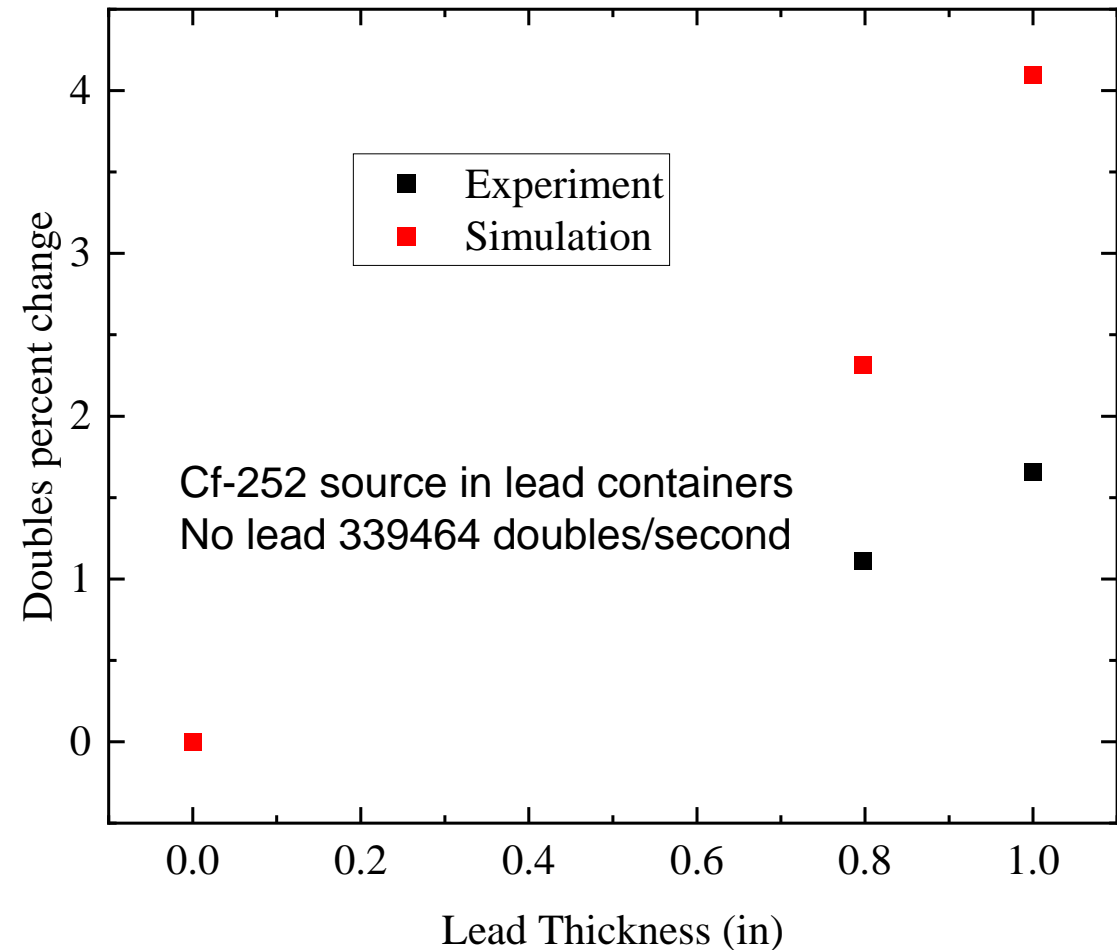


Measurement of Cf-252 to help benchmark effects of lead without access to sufficient $^{233}\text{UO}_2$ samples



Initial simulations of Cf-252 are not completely consistent

- Rate of Cf-252 sample in experiment was 852K neutrons per second
- ENMC captures 62% of neutrons and not typically used for sources of such high intensity
- Simulation data is being run through in house software including dead time adjustments..
- Future measurement with less intense source, and with sources with higher induced fission total cross section (Pu or ^{235}U) in lead.



Future work to lock down lead effects

- Measure Cf-252 source with more similar activity to 150 grams of $^{233}\text{UO}_2$.
- Measure other materials such as plutonium in lead (large enough to get induced fission from reflection).
- Simulations of varying masses of $^{233}\text{UO}_2$ in lead.
- Experiments of samples in lead when $^{233}\text{UO}_2$ is available.