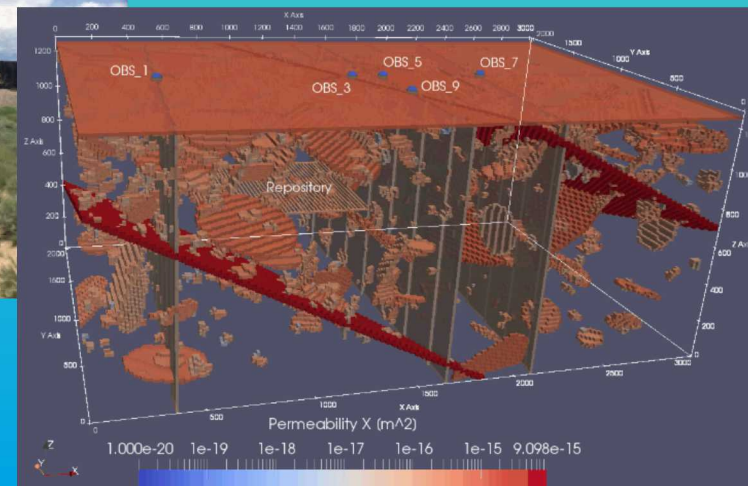




# Technical Basis for Engineering Feasibility and Thermal Management

NWTRB Virtual Meeting  
July 27-28, 2020



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# Outline

- Summary of Previous Technical Feasibility Studies
  - Safety
  - Engineering challenges
  - Thermal management
  - Postclosure criticality
- DPC dimensions and weights
- Emplacement concept
- Waste package handling, transport, emplacement
- Thermal management
  - Why temperature or thermal power limits
  - Disposal power limits are always less than transportation limits
  - Comparison of geologic settings on thermal criteria
  - Time required for DPCs to cool for disposal; fuel age at emplacement
- Postclosure internal criticality review
- Summary

# Facts About Potential Direct Disposal of SNF in DPC-Based Waste Packages

- DPCs weigh about the same as Yucca Mountain (YM) canisters sized for 21-pressurized water reactor (PWR) assemblies.

***Loaded Magnastor® canister (NAC International) 37-PWR DPC (~50 MT) vs. loaded YM 21-PWR canister ( $\leq 49.3$  MT)***

- DPCs are about the same size as YM canisters for commercial SNF.

***Magnastor canister dimensional envelope (1.77 m D x 4.87 m L → 12.4 m<sup>3</sup>) vs. YM canister (1.69 m D x 5.39 m L → 12.1 m<sup>3</sup>).***

- DPC-based waste packages could be lowered down a shaft with a large friction-winder type hoist.

***A DPC package (~70 MT) with shield (+75 MT) + carriage would compare to the 175 MT payload for the “DIREGT” conceptual hoist design (BGE Tec).***

- Meeting thermal limits for disposal will require fuel aging

***Example 1: ~98% of projected DPCs will cool to 10 kW by 2130.***

***Example 2: ~98% of projected BWR DPCs will cool to 4 kW by 2170.***

# Summary of Previous (2013–2017) Technical Feasibility Study for DPC Direct Disposal

- **Direct disposal of spent fuel in DPCs is possible with all geologic settings evaluated**
  - Thermal management and postclosure criticality controls vary for geologic settings
  - Relative reliance on natural and engineered barriers also varies
- **Additional considerations**
  - Disposal overpack reliability estimates can be improved
  - DPC basket designs impact structural longevity after package breach
- **Major recommendations**
  - Investigate fillers for all DPCs
  - Investigate screening postclosure criticality on low consequence

# Recommendations from Previous (2013-2017) Technical Feasibility Study (1/2)

## ■ **Safety**

- General attributes of a safe repository also apply for DPCs
- Performance assessment models need to discern differences
- Likely need to use cementitious materials in repository construction

## ■ **Engineering Feasibility**

- Consider fuel and canister condition if extended aging is needed
- Need to develop transporter and emplacement system concepts
- Start corrosion testing for packaging materials
- Update disposal overpack reliability

## ■ **Thermal Management**

- Continue R&D for high-temperature low-permeability buffer/backfill for crystalline and argillaceous host media (e.g., 150°C or hotter)
- Develop thermally driven process models (e.g., argillite repository)



# Recommendations from Previous (2013-2017) Technical Feasibility Study (2/2)

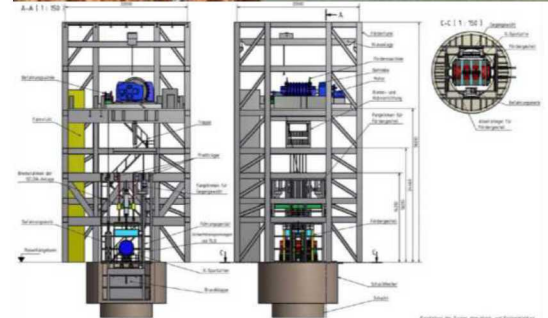
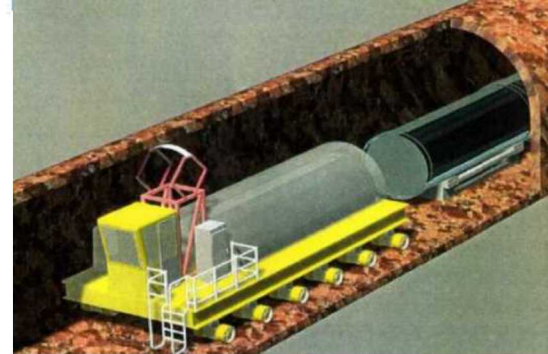
## ■ **Postclosure Criticality Control**

- Continue analysis of “as loaded” DPCs to estimate reactivity margin for degraded, flooded conditions
- Document stylized degradation scenarios
- Develop models of in-package (fuel, basket) degradation including effects from radiolysis
- Develop advanced burnup credit methodology for BWR fuel
- Conduct R&D on fillers for moderator exclusion and neutron absorption



## A large, multi-axle low-bed trailer is shown transporting a massive, red-painted wooden building component. The building has a prominent white balcony with a railing. The trailer is parked on a paved surface in front of other buildings. The image illustrates the heavy-duty capabilities of the truck chassis.

- 



# Repository Concept of Operations

Aspects would be similar for DPC-based packages, as for purpose-designed canisters:

- Repository layout, construction method and sequence
- Shafts for worker access/materials, ventilation, and waste rock
- Waste transport ramp (or shaft, e.g., in evaporites)
- Ground support and invert options
  - Temporary vs. long-term; and use of cementitious materials
- Waste package handling, transport and emplacement
  - Heavy-haul equipment, with shielding and remote operation
- Backfill emplacement drifts to:
  - Hasten reconsolidation (salt)
  - Limit ground water flow (clay/shale and crystalline)
  - Limit EBS damage from rockfall and seismic motion (unsaturated, and other concepts)
- Use plugs/seals as appropriate

## Slide 10

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**PLL1**

Isn't the main purpose of backfill to prevent the drift from becoming a preferential flow path for water?

Price, Laura L, 7/15/2020

**HE1**

Only for some concepts--will clarify

Hardin, Ernest, 7/15/2020

# DPC Overpack Functional Description

- Preclosure functions assigned to overpack:
  - Containment for > 100 yr or until repository closure
  - Structurally robust to withstand handling and drops
  - Unshielded (saving 40+ MT in weight per waste package)
- Postclosure function assigned to overpack:
  - Containment consistent with disposal concept (100 yr to >10,000 yr)
    - Corrosion allowance or resistance
    - Resist impact from rockfall, and crushing from ground water and rock pressures, during containment period



# DPC Canister Size and Weight (1/2)

## ■ Example DPC dimensions, weights (Greene et al. 2013)

S&T DPC System	Cap.	Wt. Loaded	Canister		Storage	Transport
		MT	Diameter, m	Length, m	Cask System	Cask System
<b>MPC-24 series</b>	24 PWR	<b>40.9</b>	<b>1.74</b>	<b>4.83</b>	HI-STORM 100/100U HI-STAR 100	HI-STAR 100
<b>MPC-32 series</b>	32 PWR	<b>40.9</b>	<b>1.74</b>	<b>4.83</b>	HI-STORM 100/100U HI-STAR 100	HI-STAR 100
<b>MPC-68 series</b>	68 BWR	<b>40.9</b>	<b>1.74</b>	<b>4.83</b>	HI-STORM 100/100U	HI-STAR 100
<b>MPC-37</b>	37 PWR	<b>52.9</b>	<b>1.92</b>	<b>4.60</b>	HI-STORM FW/UMAX	HI STAR 190
<b>MPC-68 series</b>	68 BWR	<b>52.9</b>	<b>1.92</b>	<b>4.83</b>	HI-STORM FW/UMAX	HI STAR 190
<b>TSC Class 1-3</b>	24 PWR	<b>33.1</b>	<b>1.71</b>	<b>4.45 – 4.87</b>	VCC Class 1-3	UTC
<b>TSC Class 4-5</b>	56 BWR	<b>34.4</b>	<b>1.71</b>	<b>4.72 – 4.84</b>	VCC Class 4-5	UTC
<b>Magnastor PWR</b>	37 PWR	<b>46.6</b>	<b>1.80</b>	<b>4.70</b>	VCC	MAGNATRAN
<b>Magnastor BWR</b>	87 BWR	<b>47.0</b>	<b>1.80</b>	<b>4.87</b>	VCC	MAGNATRAN
<b>NUHOMS 24 series</b>	24 PWR	<b>37.3 - 43.0</b>	<b>1.71</b>	<b>4.73 – 4.99</b>	HSM-H	MP187/MP197 MP197HB
<b>NUHOMS 32 series</b>	32 PWR	<b>40.1 - 50.0</b>	<b>1.71 – 1.77</b>	<b>4.72 – 5.04</b>	HSM 80 or 102 HSM-H or 102 HSM "Advanced"	MP197HB MP187/MP197
<b>NUHOMS 37 series</b>	37 PWR	<b>49.1 - 49.7</b>	<b>1.77</b>	<b>4.62 – 4.81</b>	HSM-H	MP197HB
<b>NUHOMS 61 series</b>	61 BWR	<b>40.2 - 42.3</b>	<b>1.71</b>	<b>4.98</b>	HSM 80 or 120 HSM-H or -HS HSM "advanced"	MP197/MP197HB
<b>NUHOMS 69BTH</b>	69 BWR	<b>48.2</b>	<b>1.77</b>	<b>4.98</b>	HSM-H/HS	MP197/MP197HB

Greene et al. 2013. *Storage and Transport Cask Data for Used Commercial Nuclear Fuel*. ATI-TR-13047. Energx. Oak Ridge, TN.

# DPC Canister Size and Weight (2/2)

	Yucca Mountain Transport-Aging-Disposal (TAD) Canister	Largest DPC (3 major vendors) *
Capacity	21-PWR/44-BWR	37-PWR/89-BWR
Diameter	1.69 m	1.92 m
Length	5.39 m	4.87 m
Weight	49.3 MT (loaded)	52.9 MT (loaded)

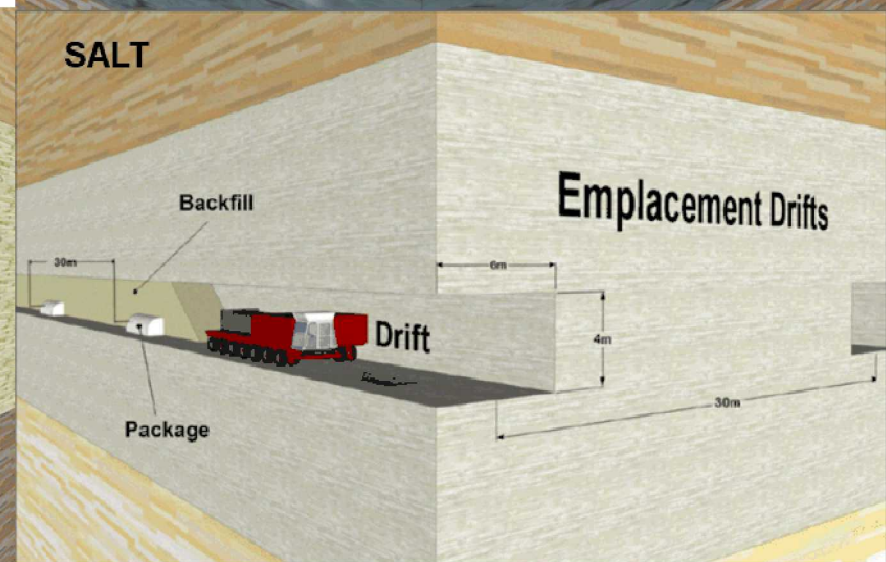
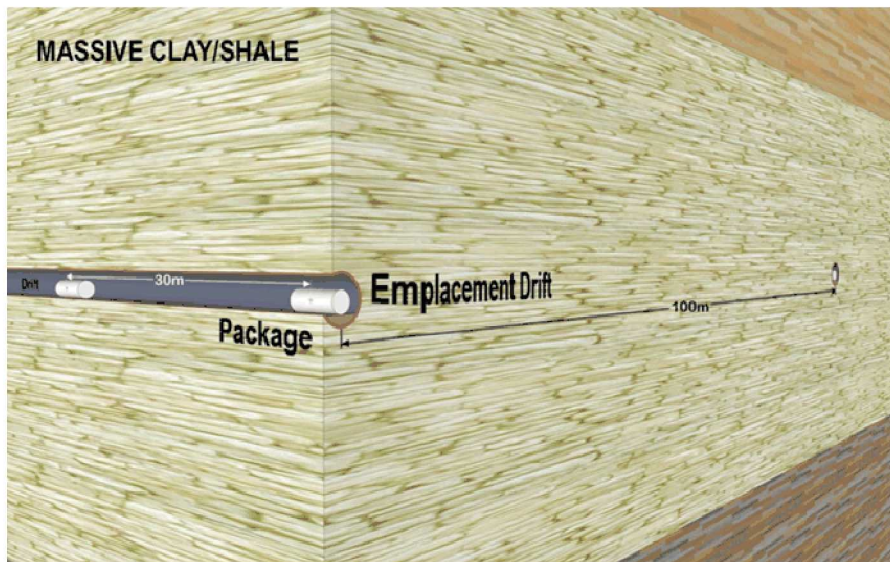
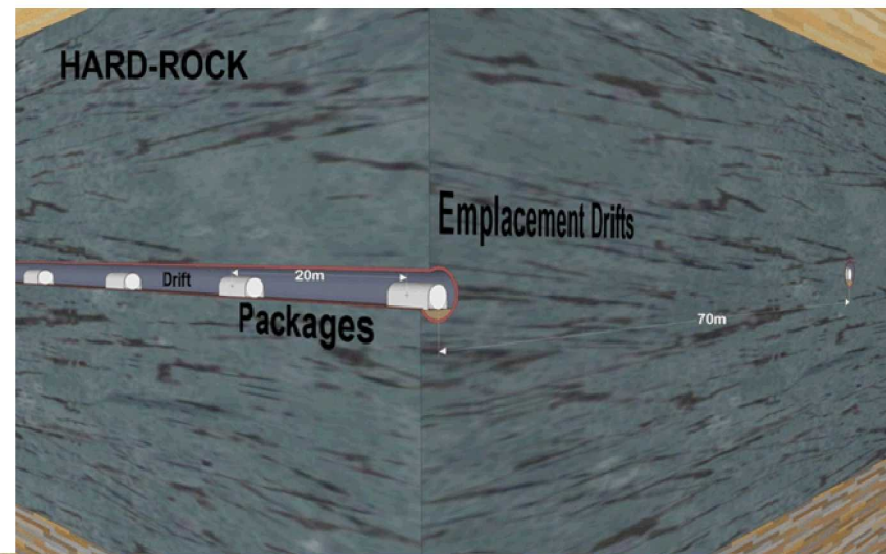
## ■ Conclusions:

- Handling and packaging of DPCs for disposal is within the industrial state of practice
- TAD canisters would be robust

\* See example DPC dimensions, previous slide

# DPC Direct Disposal Concepts

- In-drift emplacement
- Unshielded packages
- Rubber-tired transport
- Some thermal aging (or ventilation in situ) is needed
- Backfill (except unsaturated hard rock; not shown)
- Remote operations



(Hardin et al. 2013. FCRD-UFD-2013-000171 Rev. 1)



# Why Thermal Limits for Disposal?

- Cladding protection (ISG-3 Rev. 3 limits adapted to postclosure, e.g., max. 350°C)
- Packaging material limits (de-alloying/sensitization, e.g., 300°C for Alloy 22)
- Repository temperature limits
  - Buffer/backfill alteration (100 to 200°C)
  - Microcracking of siliceous rock (~200°C)
  - Salt decrepitation (~270°C)
- Injectable fillers (limit internal pressure during filling operations)
- Waste package handling (e.g., 18 kW/package for YM transport-emplacement-vehicle)



# DPC Thermal Power Limits for Storage and Transportation

- Example thermal limits for licensed DPC storage/transport systems (Greene et al. 2013)

S&T DPC System	Cap.	Wt. Loaded MT	Heat Rejection Storage/Transport., kW	Licensing Status (2013)	Storage Cask System	Transport Cask System
<b>MPC-24 series</b>	24 PWR	40.9	36.9 / 20.0 19.0 / 20.0	S&T	HI-STORM 100/100U HI-STAR 100	HI-STAR 100
<b>MPC-32 series</b>	32 PWR	40.9	36.9 / 20.0	S&T	HI-STORM 100/100U HI-STAR 100	HI-STAR 100
<b>MPC-68 series</b>	68 BWR	40.9	36.9 / 18.5 18.5 / 18.5	S&T	HI-STORM 100/100U	HI-STAR 100
<b>MPC-37</b>	37 PWR	52.9	47.0 / 38.0	S	HI-STORM FW/UMAX	HI STAR 190
<b>MPC-68 series</b>	68 BWR	52.9	46.3 / 38.0	S	HI-STORM FW/UMAX	HI STAR 190
<b>TSC Class 1-3</b>	24 PWR	33.1	23.0 / 20.0	S&T	VCC Class 1-3	UTC
<b>TSC Class 4-5</b>	56 BWR	34.4	23.0 / 16.0	S&T	VCC Class 4-5	UTC
<b>Magnastor PWR</b>	37 PWR	46.6	35.5 / 33.0	S&T	VCC	MAGNATRAN
<b>Magnastor BWR</b>	87 BWR	47.0	35.5 / 33.0	S&T	VCC	MAGNATRAN
<b>NUHOMS 24 series</b>	24 PWR	37.3 - 43.0	24.0 - 40.8 / 24.0 - 32.0	S	HSM-H	MP187/MP197 MP197HB
<b>NUHOMS 32 series</b>	32 PWR	40.1 - 50.0	24.0 - 40.8 / 24.0 - 32.0	S	HSM 80 or 102 HSM-H or 102 HSM "Advanced"	MP197HB MP187/MP197
<b>NUHOMS 37 series</b>	37 PWR	49.1 - 49.7	30.0 / 30.0	S&T	HSM-H	MP197HB
<b>NUHOMS 61 series</b>	61 BWR	40.2 - 42.3	18.3 - 31.2 / 15.9 - 31.2	S&T	HSM 80 or 120 HSM-H or -HS HSM "advanced"	MP197/MP197HB
<b>NUHOMS 69BTH</b>	69 BWR	48.2	26.0 - 32.0 / 26.0 to 32.0	T	HSM-H/HS	MP197/MP197HB

Greene et al. 2013. *Storage and Transport Cask Data for Used Commercial Nuclear Fuel*. ATI-TR-13047. Energx. Oak

Ridge, TN.

# DPC Thermal Power Limits for Transportation vs. Disposal

- Typical disposal power limits:
  - Yucca Mountain License Application:  $\leq 18$  kW/package at emplacement;  $\leq 11.8$  kW/package at closure
  - Emplacement power limits of 10 kW/package or less, for generic disposal concepts in various media
- Conclusions:
  - 1) Thermal power limits for storage and transport are greater than limits for disposal, and
  - 2) Thermal aging (or ventilation in situ) will be needed for DPC direct disposal, with duration depending on EBS and host rock temperature limits

# DPC Thermal Power Limits for Different Disposal Concepts

# DPC Direct Disposal Concepts: Thermal Comparison

Setting	Host Rock Temperature Tolerance (°C)	Host Rock Thermal Cond. (W/m-K) <sup>A</sup>	Power Limit at Emplacement (& Backfilling; in kW)	Comments
Argillite (clay/shale)	~100	1.1 to 2.3	4 <sup>B</sup>	<ul style="list-style-type: none"> <li>• Overheat the near field host rock (~125°C).</li> <li>• Space packages apart (20 m) to limit peak temp. for clay-based backfill between packages (&lt;100°C).</li> </ul>
Crystalline	200+	2.4 to 3.2	3 <sup>C</sup>	<ul style="list-style-type: none"> <li>• Power limited by peak allowable buffer temp. (100 to 200°C).</li> </ul>
Salt	200+	2.7 to 5.4	10 <sup>D</sup>	<ul style="list-style-type: none"> <li>• Protect halite and other salts from decrepitation.</li> <li>• Conductivity range given for 200 to 27°C.</li> <li>• Lower thermal conductivity, but no temperature limit for crushed salt backfill.</li> </ul>
Unsaturated	200 <sup>E</sup>	0.9 to 2	~10	<ul style="list-style-type: none"> <li>• By analogy to the Yucca Mountain repository thermal strategy: 1.45 kW/m line load w/ 11.8 kW max. package (at closure or backfilling).</li> <li>• Peak package temp. &gt;300°C with backfill.</li> </ul>

## Sources:

<sup>A</sup> Hardin et al. 2012. *Parameter Uncertainty for Repository Thermal Analysis*. FCRD-UFD-2012-000097. April 2012. Range represents variability between formations, and includes anisotropic variation for shales, unless indicated otherwise.

<sup>B</sup> SNL 2020. *High Temperature Argillite Reference Case*. (in prep.).

<sup>C</sup> Hardin, E. 2013. *Temperature-Package Power Correlations for Open-Mode Geologic Disposal Concepts*. SAND2013-1425.

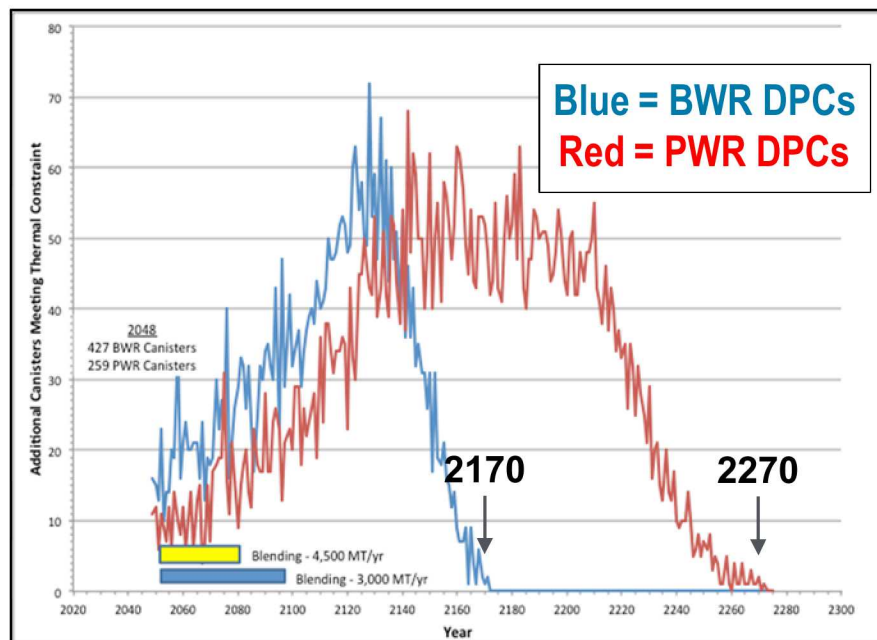
<sup>D</sup> SNL 2019. *A Salt Repository Concept for CSNF in 21-PWR Size Canisters*. SFWD-IWM-2017-000246 Rev. 2.

<sup>E</sup> For welded tuff (Hardin et al. 1997. *Synthesis Report on Thermally Driven Coupled Processes*. UCRL-ID-128495). Temperature tolerance for other media such as alluvium has not been determined.

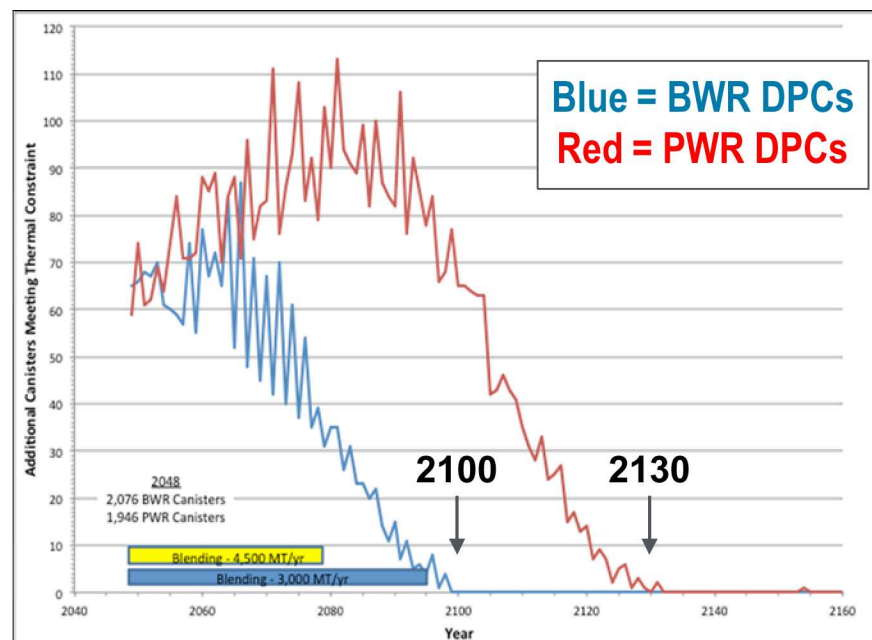
(Hardin et al. 2013. FCRD-UFD-2013-000171 Rev. 1)



# Projections of All DPCs to be Loaded Cooling: to Meet Disposal Thermal Power Limits



Number of DPCs that cool to **4 kW** each year (argillite or crystalline disposal concepts with clay-based buffer/backfill).



Number of DPCs that cool to **10 kW** each year (salt, unsaturated hard rock disposal concepts).

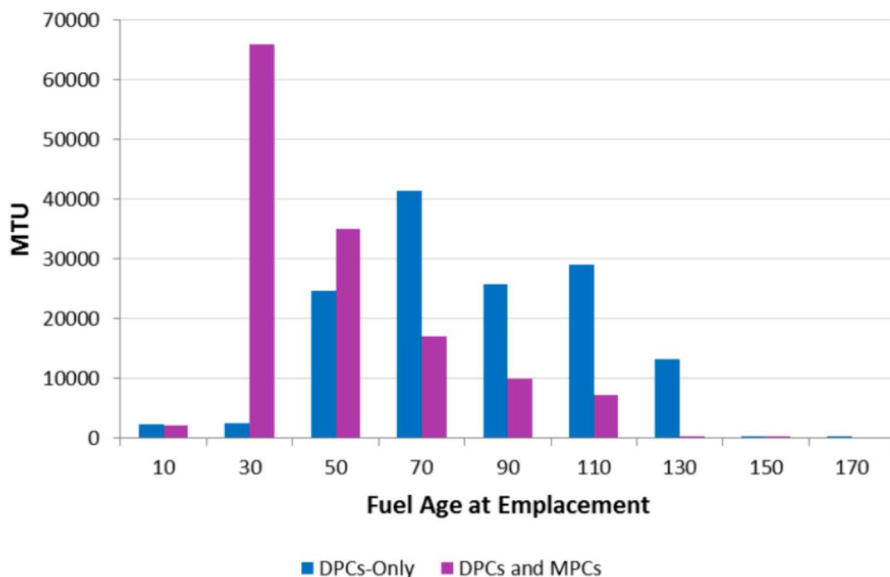
# Fuel Age Out-of-Reactor at Disposal

Fuel age at emplacement is potentially important if constraints on canister or fuel condition are related to aging time.

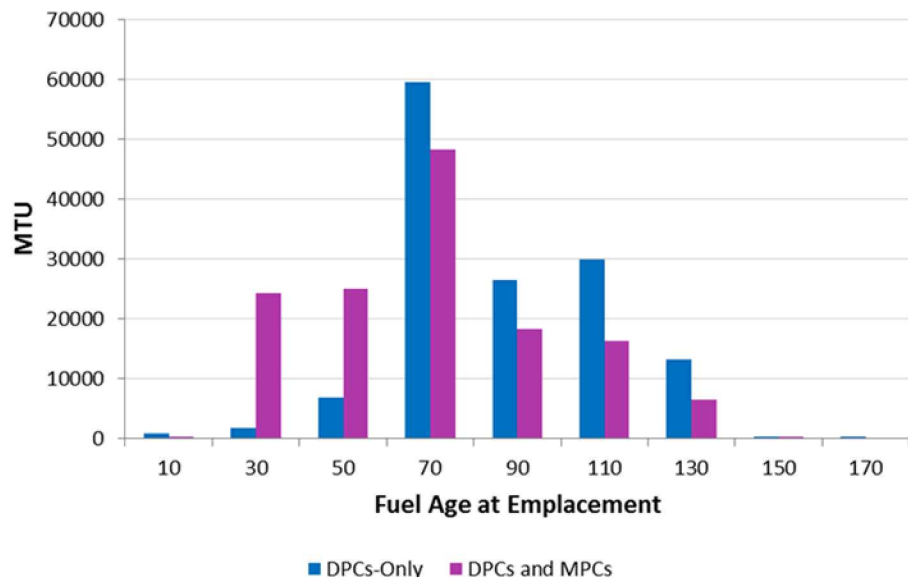
- Minimum fuel age at emplacement is obtained by re-packaging all DPCs into smaller canisters (e.g., 4-PWR), thus decreasing thermal aging time.
- For a future transition from DPCs to smaller canisters, without re-packaging the DPCs, fuel age at emplacement is comparable to repackaging if the emplacement power limit is high enough ( $\geq 10$  kW).
- To maintain comparable fuel age at emplacement for a lower emplacement power limit (6 kW) two changes would be needed:
  - Transition to smaller canisters, and
  - Early repository start (e.g., 2048 or sooner).

# Fuel Age (out-of-reactor) at Emplacement: Example TSL-CALVIN Projection

6 kW Power Limit, Repository in 2036



6 kW Power Limit, Repository in 2048

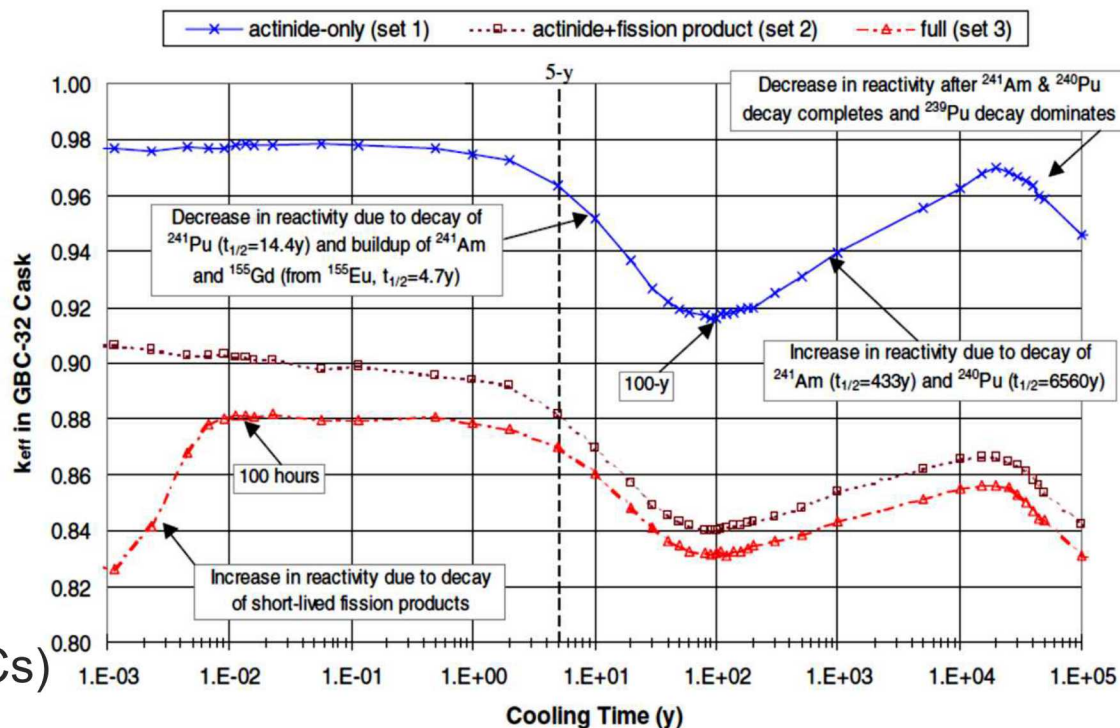


- DPC direct disposal compared to repackaging all fuel into purpose-designed 4 PWR/9 BWR packages (MPCs).
- Repackaging starts 5 years before repository opening.
- DPC case produces the oldest fuel at disposal because of thermal aging.
- MPC case produces the youngest because no thermal aging is needed after repackaging.

FCRD-UFD-2014-000069 Rev. 0 Investigations of Dual-Purpose Canister Direct Disposal Feasibility (FY14)

# Postclosure Nuclear Criticality Control

- **Disposal Environment**
  - Groundwater availability
  - Chloride in groundwater
- **Moderator Exclusion**
  - Overpack integrity
- **Moderator Displacement**
  - Fillers
- **Add Neutron Absorbers**
  - Fillers (e.g.,  $B_4C$  loaded)
  - Control hardware (future DPCs)
- **Zone Loading**
- **Criticality Analysis Methodology**
  - Burnup credit, as-loaded, stylized degradation cases
  - Peak reactivity occurs at >10,000 years
  - Reactivity margin (many DPCs)



**Neutron multiplication factor ( $k_{eff}$ ) vs. time**  
Generic burnup-credit 32-PWR cask  
PWR fuel (4% enriched, 40 GW-d/MT burnup)

Wagner and Parks 2001 (NUREG/CR-6781)



# Postclosure Criticality Control Measures (1/2)

- **Alternative: Reactivity Margin**

- Many (not all) DPCs are subcritical in stylized degradation cases.

- **Alternative: Criticality Control Features**

- PWR or BWR fuel assembly disposal control rods (EPRI 2008)
- BWR fuel rechanneling \*
- Chevron inserts (patents extant) \*
- Zone loading (future DPCs; EPRI 2008)

\* Requires corrosion resistant neutron absorber material

- **Alternative: Injectable Fillers**

- Cut off covers over existing DPC vent/drain ports

- **Alternative: High-Performance Disposal Overpack**

- May not be sufficiently reliability for low-probability exclusion of internal criticality

EPRI (Electric Power Research Institute) 2008. *Feasibility of Direct Disposal of Dual-Purpose Canisters: Options for Assuring Criticality Control*. #1016629.

# Postclosure Criticality Control Measures (2/2)

## ■ Cut DPC Lids Off?

- Skiving (wet or dry)
- Dry filler tests: steel shot (Cogar 1996); glass beads (Forsberg 1997)
- Particle filling would be done dry (inert gas cover)
- Criticality control hardware installation (e.g., disposal control rods, rechanneling) could be done wet
- Requires re-welding

Cogar, J. 1996. *Waste Package Filler Material Testing Report*. BBA000000-01717-2500-00008 Rev 01. OCRWM.

Forsberg, C.W. 1997. *Description of the Canadian Particulate-Fill Waste Package (WP) System for Spent Nuclear Fuel (SNF) and its Applicability to Light-Water Reactor SNF WPs with Depleted Uranium Dioxide Fill*. ORNL/TM-13502.

# Summary (1/3)

Technical feasibility investigations for direct disposal of commercial SNF in DPCs established:

- ***At least some DPCs are disposable for all of the generic geologic settings evaluated (and excluding postclosure criticality from PA on low probability).***
- **Preclosure operational safety:** Similar to the current state-of-the-practice in fuel handling and packaging
- **Postclosure waste isolation:** No substantial difference compared to site-specific, purpose-designed, possibly smaller canisters.
- **Engineering challenges:** Can be met (including a first-of-a-kind heavy shaft hoist if needed)

# Summary (2/3)

- **Postclosure internal criticality:**

- Unlikely for disposal concepts that don't allow package flooding
- A fraction of existing DPCs have sufficient reactivity margin to remain subcritical if degraded and flooded
- There are many types of DPCs (50 or more) with various types of degradation on exposure to ground water, and different fuel characteristics

- **Thermal management:**

- Disposal power limit of 10 kW allows 98% of projected DPCs to cool by 2130 (6 kW DPCs by 2170, 4 kW BWR DPCs by 2170)
- Favors disposal concepts with  $\geq 200^{\circ}\text{C}$  temperature tolerance (e.g., at package surface) and greater thermal conductivity
- BWR DPCs cool significantly faster (e.g., 4 kW BWR DPCs cool  $\sim 100$  yr sooner than PWR DPCs)



# Summary (3/3)

## Review of Recommendations from Technical Feasibility Study through 2017

- Information needs analyzed (SNL 2015)
- Continue to collect and analyze information on existing DPCs
- Develop burnup credit approach for BWR fuel
- Ensure DPC service lifetime ( $\geq 100$  yr) needed for thermal aging
- Investigate disposal concepts with greater host-medium thermal conductivity and temperature tolerance
- Research injectable fillers for postclosure criticality control in DPCs by moderator displacement
- Perform consequence analysis for criticality event exclusion from, or inclusion in performance assessment

SNL 2015. *Summary of Investigations on Technical Feasibility of Direct Disposal of Dual-Purpose Canisters*. FCRD-UFD-2015-000129 Rev. 0

Liljenfeldt, H. et al. 2016. *Summary of Investigations on Technical Feasibility of Direct Disposal of Dual-Purpose Canisters*. SFWD-SFWST-2017-000045 (calculations update to SNL 2015).

Questions?