

Technical Basis for Engineering Feasibility and Thermal Management

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Outline

- Summary of Previous Technical Feasibility Studies
 - Safety
 - Engineering challenges
 - Thermal management
 - Postclosure criticality
- DPC dimensions and weights
- Emplacement concept
- Waste package handling, transport, emplacement
- Thermal management
 - Why temperature or thermal power limits
 - Disposal power limits are always less than transportation limits
 - Comparison of geologic settings on thermal criteria
 - Time required for DPCs to cool for disposal; fuel age at emplacement
- Postclosure internal criticality review
- Summary

Facts About Potential Direct Disposal of SNF in DPC-Based Waste Packages

- DPCs weigh about the same as Yucca Mountain (YM) canisters sized for 21-pressurized water reactor (PWR) assemblies.

Loaded Magnastor® canister (NAC International) 37-PWR DPC (~50 MT) vs. loaded YM 21-PWR canister (≤ 49.3 MT)

- DPCs are about the same size as YM canisters for commercial SNF.
Magnastor canister dimensional envelope (1.77 m D x 4.87 m L → 12.4 m³) vs. YM canister (1.69 m D x 5.39 m L → 12.1 m³).
- DPC-based waste packages could be lowered down a shaft with a large friction-winder type hoist.

A DPC package (~70 MT) with shield (+75 MT) + carriage would compare to the 175 MT payload for the “DIRECT” conceptual hoist design (BGE Tec).

- Meeting thermal limits for disposal will require fuel aging
Example 1: ~98% of projected DPCs will cool to 10 kW by 2130.
Example 2: ~98% of projected BWR DPCs will cool to 4 kW by 2170.

Summary of Previous (2013–2017) Technical Feasibility Study for DPC Direct Disposal

- **Direct disposal of spent fuel in DPCs is possible with all geologic settings evaluated**
 - Thermal management and postclosure criticality controls vary for geologic settings
 - Relative reliance on natural and engineered barriers also varies
- **Additional considerations**
 - Disposal overpack reliability estimates can be improved
 - DPC basket designs impact structural longevity after package breach
- **Major recommendations**
 - Investigate fillers for all DPCs
 - Investigate screening postclosure criticality on low consequence

Recommendations from Previous (2013-2017) Technical Feasibility Study (1/2)

- **Safety**
 - General attributes of a safe repository also apply for DPCs
 - Performance assessment models need to discern differences
 - Likely need to use cementitious materials in repository construction
- **Engineering Feasibility**
 - Consider fuel and canister condition if extended aging is needed
 - Need to develop transporter and emplacement system concepts
 - Start corrosion testing for packaging materials
 - Update disposal overpack reliability
- **Thermal Management**
 - Continue R&D for high-temperature low-permeability buffer/backfill for crystalline and argillaceous host media (e.g., 150°C or hotter)
 - Develop thermally driven process models (e.g., argillite repository)

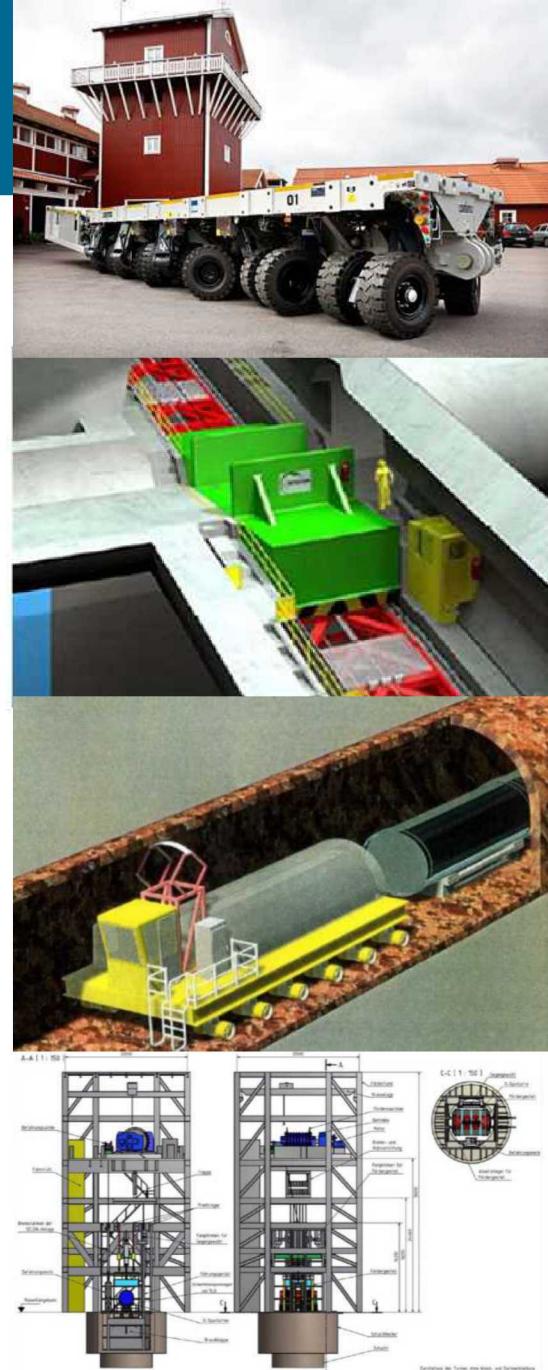
Recommendations from Previous (2013-2017) Technical Feasibility Study (2/2)

▪ Postclosure Criticality Control

- Continue analysis of “as loaded” DPCs to estimate reactivity margin for degraded, flooded conditions
- Document stylized degradation scenarios
- Develop models of in-package (fuel, basket) degradation including effects from radiolysis
- Develop advanced burnup credit methodology for BWR fuel
- Conduct R&D on fillers for moderator exclusion and neutron absorption

Engineering Challenges Can Be Met

- **Handling/Packaging: Use Current Practices**
- **Surface-Underground Transport**
 - Spiral ramp (~10% grade, rubber-tire)
 - Linear ramp (>10% grade, funicular)
 - Shallow ramp ($\leq 3\%$ grade, standard rail)
 - Heavy shaft hoist (up to 175 MT payload)
- **Drift Opening Stability Constraints**
 - Salt (a few years with little attention or heating; longer with rock bolts and maintenance)
 - Hard rock (50 years or longer)
 - Sedimentary (50 years may be feasible, or longer depending on geologic setting)



Repository Concept of Operations

Aspects would be similar for DPC-based packages, as for purpose-designed canisters:

- Repository layout, construction method and sequence
- Shafts for worker access/materials, ventilation, and waste rock
- Waste transport ramp (or shaft, e.g., in evaporites)
- Ground support and invert options
 - Temporary vs. long-term; and use of cementitious materials
- Waste package handling, transport and emplacement
 - Heavy-haul equipment, with shielding and remote operation
- Backfill emplacement drifts to:
 - Hasten reconsolidation (salt)
 - Limit ground water flow (clay/shale and crystalline)
PL1
HE1
 - Limit EBS damage from rockfall and seismic motion (unsaturated, and other concepts)
- Use plugs/seals as appropriate

Slide 10

PLL1 Isn't the main purpose of backfill to prevent the drift from becoming a preferential flow path for water?
Price, Laura L, 7/15/2020

HE1 Only for some concepts--will clarify
Hardin, Ernest, 7/15/2020

DPC Overpack Functional Description

- Preclosure functions assigned to overpack:
 - Containment for > 100 yr or until repository closure
 - Structurally robust to withstand handling and drops
 - Unshielded (saving 40+ MT in weight per waste package)
- Postclosure function assigned to overpack:
 - Containment consistent with disposal concept (100 yr to >10,000 yr)
 - Corrosion allowance or resistance
 - Resist impact from rockfall, and crushing from ground water and rock pressures, during containment period

DPC Canister Size and Weight (1/2)

- Example DPC dimensions, weights (Greene et al. 2013)

S&T DPC System	Cap.	Wt. Loaded MT	Canister		Storage Cask System	Transport Cask System
			Diameter, m	Length, m		
MPC-24 series	24 PWR	40.9	1.74	4.83	HI-STORM 100/100U HI-STAR 100	HI-STAR 100
MPC-32 series	32 PWR	40.9	1.74	4.83	HI-STORM 100/100U HI-STAR 100	HI-STAR 100
MPC-68 series	68 BWR	40.9	1.74	4.83	HI-STORM 100/100U	HI-STAR 100
MPC-37	37 PWR	52.9	1.92	4.60	HI-STORM FW/UMAX	HI STAR 190
MPC-68 series	68 BWR	52.9	1.92	4.83	HI-STORM FW/UMAX	HI STAR 190
TSC Class 1-3	24 PWR	33.1	1.71	4.45 – 4.87	VCC Class 1-3	UTC
TSC Class 4-5	56 BWR	34.4	1.71	4.72 – 4.84	VCC Class 4-5	UTC
Magnastor PWR	37 PWR	46.6	1.80	4.70	VCC	MAGNATRAN
Magnastor BWR	87 BWR	47.0	1.80	4.87	VCC	MAGNATRAN
NUHOMS 24 series	24 PWR	37.3 - 43.0	1.71	4.73 – 4.99	HSM-H	MP187/MP197 MP197HB
NUHOMS 32 series	32 PWR	40.1 - 50.0	1.71 – 1.77	4.72 – 5.04	HSM 80 or 102 HSM-H or 102 HSM "Advanced"	MP197HB MP187/MP197
NUHOMS 37 series	37 PWR	49.1 - 49.7	1.77	4.62 – 4.81	HSM-H	MP197HB
NUHOMS 61 series	61 BWR	40.2 - 42.3	1.71	4.98	HSM 80 or 120 HSM-H or -HS HSM "advanced"	MP197/MP197HB
NUHOMS 69BTH	69 BWR	48.2	1.77	4.98	HSM-H/HS	MP197/MP197HB

Greene et al. 2013. *Storage and Transport Cask Data for Used Commercial Nuclear Fuel*. ATI-TR-13047. Energx. Oak Ridge, TN.

DPC Canister Size and Weight (2/2)

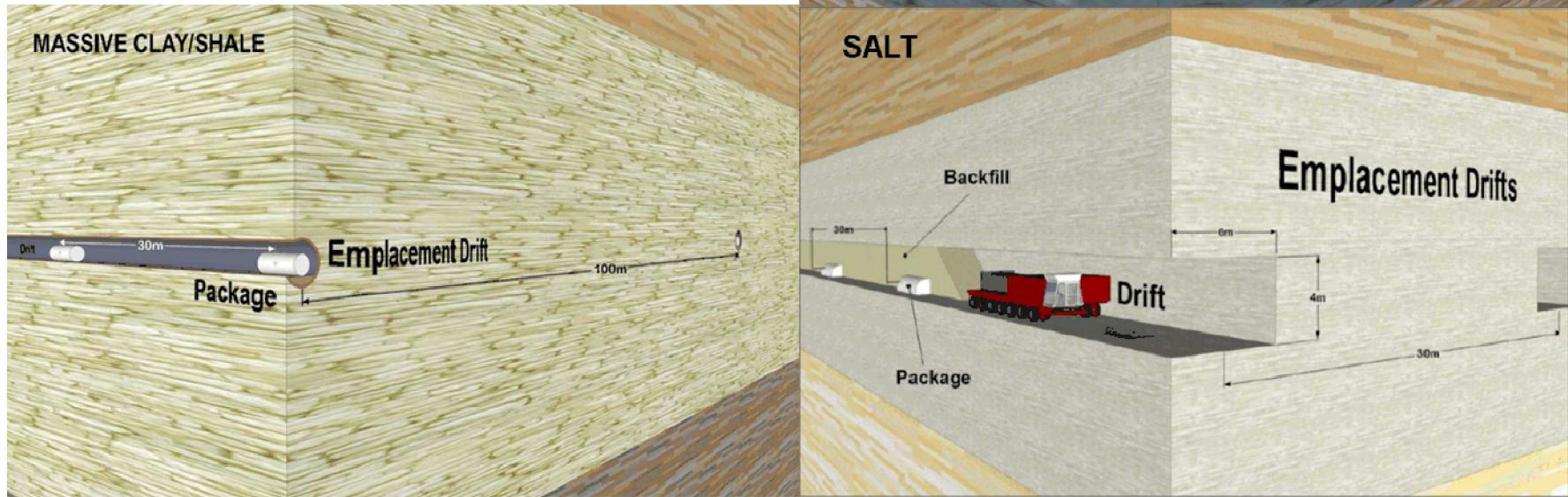
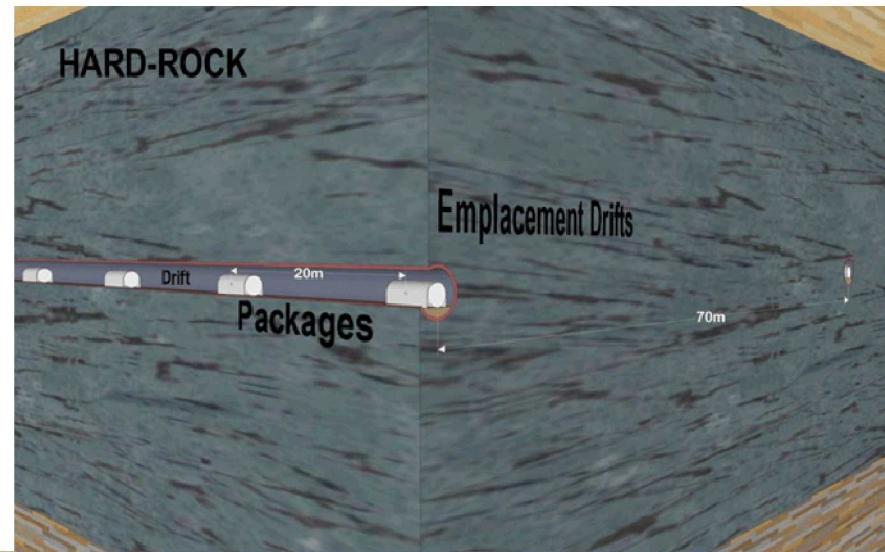
	Yucca Mountain Transport-Aging-Disposal (TAD) Canister	Largest DPC (3 major vendors) *
Capacity	21-PWR/44-BWR	37-PWR/89-BWR
Diameter	1.69 m	1.92 m
Length	5.39 m	4.87 m
Weight	49.3 MT (loaded)	52.9 MT (loaded)

- **Conclusions:**
 - Handling and packaging of DPCs for disposal is within the industrial state of practice
 - TAD canisters would be robust

* See example DPC dimensions, previous slide

DPC Direct Disposal Concepts

- In-drift emplacement
- Unshielded packages
- Rubber-tired transport
- Some thermal aging (or ventilation in situ) is needed
- Backfill (except unsaturated hard rock; not shown)
- Remote operations



(Hardin et al. 2013. FCRD-UFD-2013-000171 Rev. 1)

Why Thermal Limits for Disposal?

- Cladding protection (ISG-3 Rev. 3 limits adapted to postclosure, e.g., max. 350°C)
- Packaging material limits (de-alloying/sensitization, e.g., 300°C for Alloy 22)
- Repository temperature limits
 - Buffer/backfill alteration (100 to 200°C)
 - Microcracking of siliceous rock (~200°C)
 - Salt decrepitation (~270°C)
- Injectable fillers (limit internal pressure during filling operations)
- Waste package handling (e.g., 18 kW/package for YM transport-emplacement-vehicle)

DPC Thermal Power Limits for Storage and Transportation

- Example thermal limits for licensed DPC storage/transport systems (Greene et al. 2013)

S&T DPC System	Cap.	Wt. Loaded	Heat Rejection	Licensing	Storage	Transport
		MT	Storage/Transport., kW	Status (2013)	Cask System	Cask System
MPC-24 series	24 PWR	40.9	36.9 / 20.0 19.0 / 20.0	S&T	HI-STORM 100/100U HI-STAR 100	HI-STAR 100
MPC-32 series	32 PWR	40.9	36.9 / 20.0	S&T	HI-STORM 100/100U HI-STAR 100	HI-STAR 100
MPC-68 series	68 BWR	40.9	36.9 / 18.5 18.5 / 18.5	S&T	HI-STORM 100/100U	HI-STAR 100
MPC-37	37 PWR	52.9	47.0 / 38.0	S	HI-STORM FW/UMAX	HI STAR 190
MPC-68 series	68 BWR	52.9	46.3 / 38.0	S	HI-STORM FW/UMAX	HI STAR 190
TSC Class 1-3	24 PWR	33.1	23.0 / 20.0	S&T	VCC Class 1-3	UTC
TSC Class 4-5	56 BWR	34.4	23.0 / 16.0	S&T	VCC Class 4-5	UTC
Magnastor PWR	37 PWR	46.6	35.5 / 33.0	S&T	VCC	MAGNATRAN
Magnastor BWR	87 BWR	47.0	35.5 / 33.0	S&T	VCC	MAGNATRAN
NUHOMS 24 series	24 PWR	37.3 - 43.0	24.0 - 40.8 / 24.0 - 32.0	S	HSM-H	MP187/MP197 MP197HB
NUHOMS 32 series	32 PWR	40.1 - 50.0	24.0 - 40.8 / 24.0 - 32.0	S	HSM 80 or 102 HSM-H or 102 HSM "Advanced"	MP197HB MP187/MP197
NUHOMS 37 series	37 PWR	49.1 - 49.7	30.0 / 30.0	S&T	HSM-H	MP197HB
NUHOMS 61 series	61 BWR	40.2 - 42.3	18.3 - 31.2 / 15.9 - 31.2	S&T	HSM 80 or 120 HSM-H or -HS HSM "advanced"	MP197/MP197HB
NUHOMS 69BTH	69 BWR	48.2	26.0 - 32.0 / 26.0 to 32.0	T	HSM-H/HS	MP197/MP197HB

Greene et al. 2013. Storage and Transport Cask Data for Used Commercial Nuclear Fuel. ATI-TR-13047. Energx. Oak

DPC Thermal Power Limits for Transportation vs. Disposal

- Typical disposal power limits:
 - Yucca Mountain License Application: ≤ 18 kW/package at emplacement; ≤ 11.8 kW/package at closure
 - Emplacement power limits of 10 kW/package or less, for generic disposal concepts in various media
- Conclusions:
 - 1) Thermal power limits for storage and transport are greater than limits for disposal, and
 - 2) Thermal aging (or ventilation *in situ*) will be needed for DPC direct disposal, with duration depending on EBS and host rock temperature limits

DPC Thermal Power Limits for Different Disposal Concepts

DPC Direct Disposal Concepts: Thermal Comparison

Setting	Host Rock Temperature Tolerance (°C)	Host Rock Thermal Cond. (W/m-K) ^A	Power Limit at Emplacement (& Backfilling; in kW)	Comments
Argillite (clay/shale)	~100	1.1 to 2.3	4 ^B	<ul style="list-style-type: none"> Overheat the near field host rock (~125°C). Space packages apart (20 m) to limit peak temp. for clay-based backfill between packages (<100°C).
Crystalline	200+	2.4 to 3.2	3 ^C	<ul style="list-style-type: none"> Power limited by peak allowable buffer temp. (100 to 200°C).
Salt	200+	2.7 to 5.4	10 ^D	<ul style="list-style-type: none"> Protect halite and other salts from decrepitation. Conductivity range given for 200 to 27°C. Lower thermal conductivity, but no temperature limit for crushed salt backfill.
Unsaturated	200 ^E	0.9 to 2	~10	<ul style="list-style-type: none"> By analogy to the Yucca Mountain repository thermal strategy: 1.45 kW/m line load w/ 11.8 kW max. package (at closure or backfilling). Peak package temp. >300°C with backfill.

Sources:

^A Hardin et al. 2012. *Parameter Uncertainty for Repository Thermal Analysis*. FCRD-UFD-2012-000097. April 2012. Range represents variability between formations, and includes anisotropic variation for shales, unless indicated otherwise.

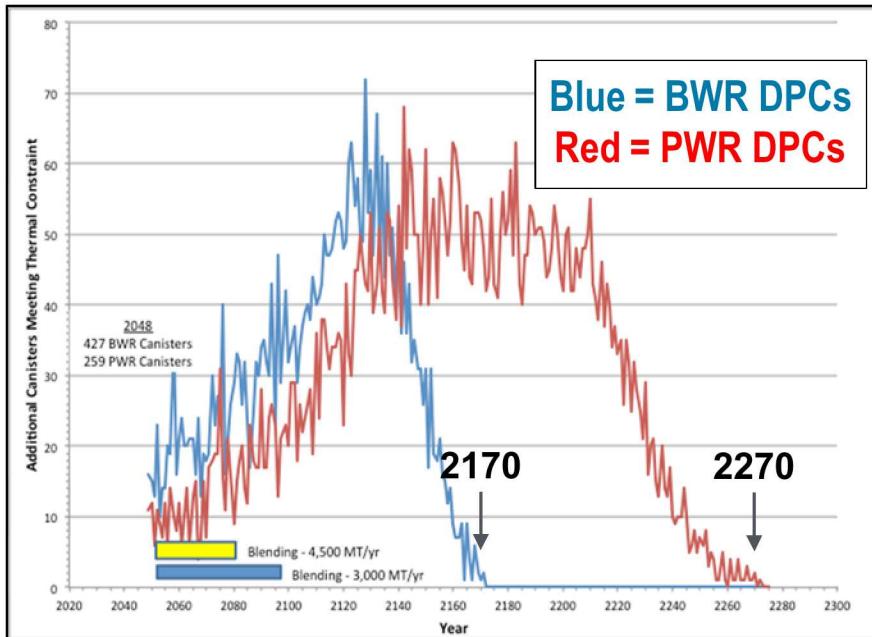
^B SNL 2020. *High Temperature Argillite Reference Case*. (in prep.).

^C Hardin, E. 2013. *Temperature-Package Power Correlations for Open-Mode Geologic Disposal Concepts*. SAND2013-1425.

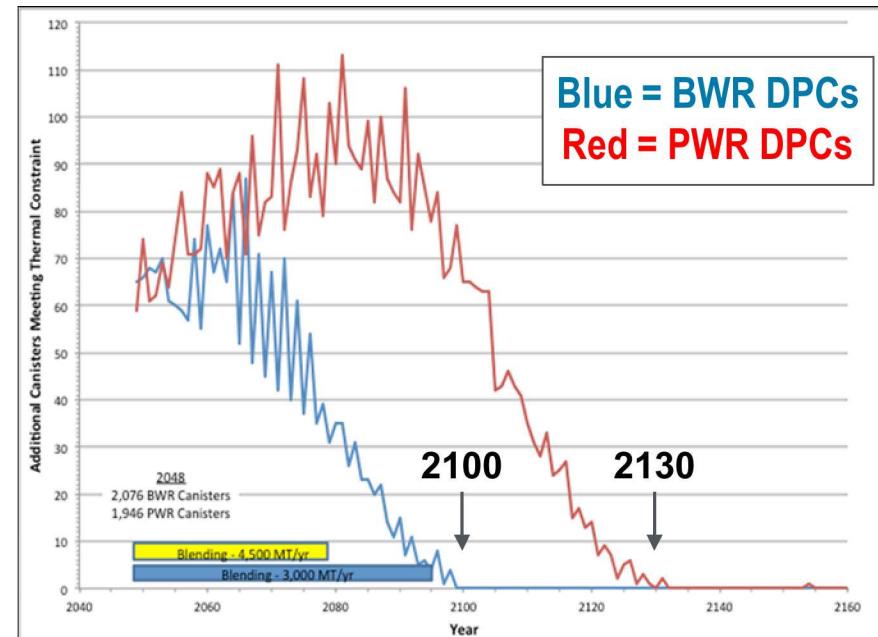
^D SNL 2019. *A Salt Repository Concept for CSNF in 21-PWR Size Canisters*. SFWD-IWM-2017-000246 Rev. 2.

^E For welded tuff (Hardin et al. 1997. *Synthesis Report on Thermally Driven Coupled Processes*. UCRL-ID-128495). Temperature tolerance for other media such as alluvium has not been determined.

Projections of All DPCs to be Loaded Cooling: to Meet Disposal Thermal Power Limits



Number of DPCs that cool
to **4 kW** each year
(argillite or crystalline
disposal concepts with
clay-based buffer/backfill).



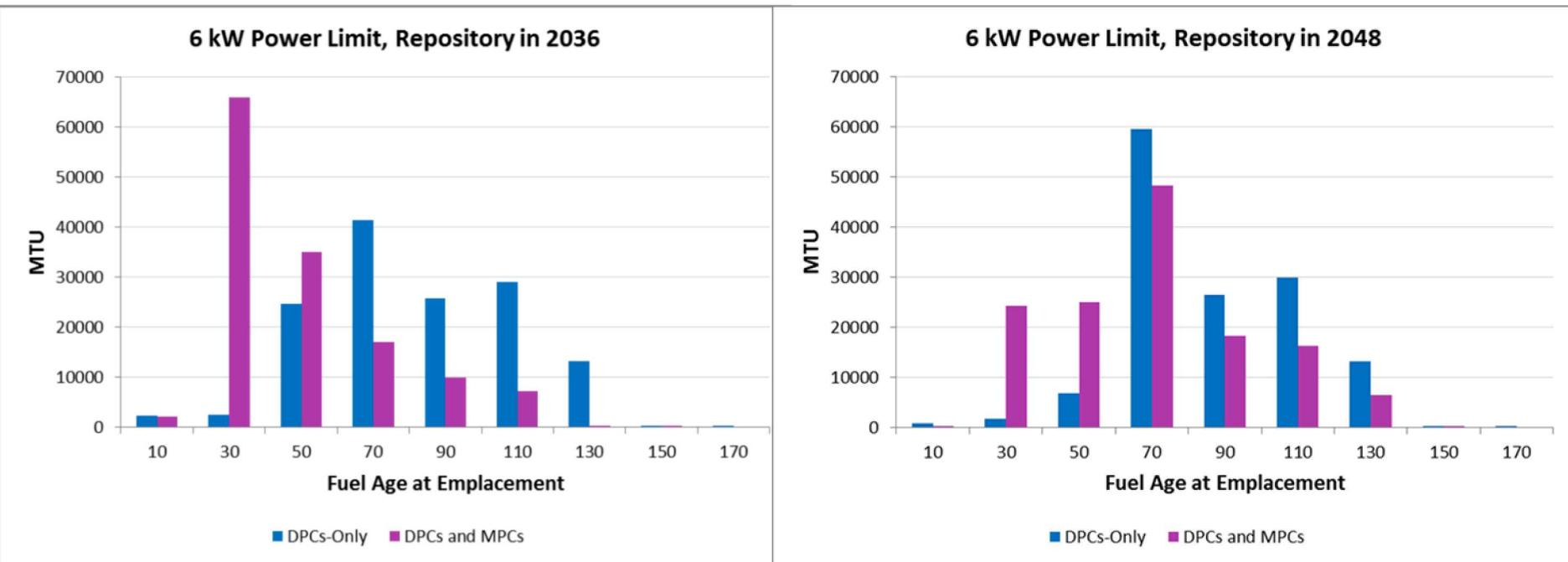
Number of DPCs that cool
to **10 kW** each year (salt,
unsaturated hard rock
disposal concepts).

Fuel Age Out-of-Reactor at Disposal

Fuel age at emplacement is potentially important if constraints on canister or fuel condition are related to aging time.

- Minimum fuel age at emplacement is obtained by re-packaging all DPCs into smaller canisters (e.g., 4-PWR), thus decreasing thermal aging time.
- For a future transition from DPCs to smaller canisters, without re-packaging the DPCs, fuel age at emplacement is comparable to repackaging if the emplacement power limit is high enough (≥ 10 kW).
- To maintain comparable fuel age at emplacement for a lower emplacement power limit (6 kW) two changes would be needed:
 - Transition to smaller canisters, and
 - Early repository start (e.g., 2048 or sooner).

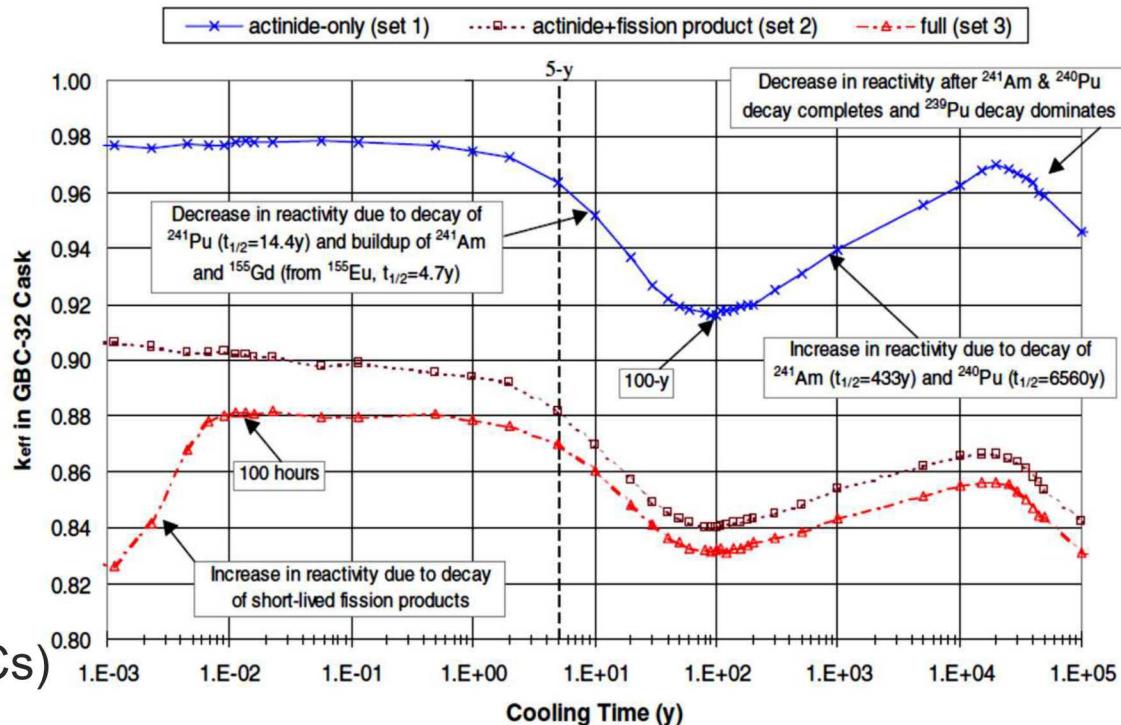
Fuel Age (out-of-reactor) at Emplacement: Example TSL-CALVIN Projection



- DPC direct disposal compared to repackaging all fuel into purpose-designed 4 PWR/9 BWR packages (MPCs).
- Repackaging starts 5 years before repository opening.
- DPC case produces the oldest fuel at disposal because of thermal aging.
- MPC case produces the youngest because no thermal aging is needed after repackaging.

Postclosure Nuclear Criticality Control

- **Disposal Environment**
 - Groundwater availability
 - Chloride in groundwater
- **Moderator Exclusion**
 - Overpack integrity
- **Moderator Displacement**
 - Fillers
- **Add Neutron Absorbers**
 - Fillers (e.g., B_4C loaded)
 - Control hardware (future DPCs)
- **Zone Loading**
- **Criticality Analysis Methodology**
 - Burnup credit, as-loaded, stylized degradation cases
 - Peak reactivity occurs at >10,000 years
 - Reactivity margin (many DPCs)



Wagner and Parks 2001 (NUREG/CR-6781)

Postclosure Criticality Control Measures (1/2)

▪ Alternative: Reactivity Margin

- Many (not all) DPCs are subcritical in stylized degradation cases.

▪ Alternative: Criticality Control Features

- PWR or BWR fuel assembly disposal control rods (EPRI 2008)
- BWR fuel rechanneling *
- Chevron inserts (patents extant) *
- Zone loading (future DPCs; EPRI 2008)

* Requires corrosion
resistant neutron
absorber material

▪ Alternative: Injectable Fillers

- Cut off covers over existing DPC vent/drain ports

▪ Alternative: High-Performance Disposal Overpack

- May not be sufficiently reliable for low-probability exclusion of internal criticality

EPRI (Electric Power Research Institute) 2008. *Feasibility of Direct Disposal of Dual-Purpose Canisters: Options for Assuring Criticality Control.* #1016629.

Postclosure Criticality Control Measures (2/2)

- **Cut DPC Lids Off?**

- Skiving (wet or dry)
- Dry filler tests: steel shot (Cogar 1996); glass beads (Forsberg 1997)
- Particle filling would be done dry (inert gas cover)
- Criticality control hardware installation (e.g., disposal control rods, rechanneling) could be done wet
- Requires re-welding

Cogar, J. 1996. *Waste Package Filler Material Testing Report*. BBA000000-01717-2500-00008 Rev 01. OCRWM.

Forsberg, C.W. 1997. *Description of the Canadian Particulate-Fill Waste Package (WP) System for Spent Nuclear Fuel (SNF) and its Applicability to Light-Water Reactor SNF WPs with Depleted Uranium Dioxide Fill*. ORNL/TM-13502.

Summary (1/3)

Technical feasibility investigations for direct disposal of commercial SNF in DPCs established:

- ***At least some DPCs are disposable for all of the generic geologic settings evaluated (and excluding postclosure criticality from PA on low probability).***
- **Preclosure operational safety:** Similar to the current state-of-the-practice in fuel handling and packaging
- **Postclosure waste isolation:** No substantial difference compared to site-specific, purpose-designed, possibly smaller canisters.
- **Engineering challenges:** Can be met (including a first-of-a-kind heavy shaft hoist if needed)

Summary (2/3)

- **Postclosure internal criticality:**
 - Unlikely for disposal concepts that don't allow package flooding
 - A fraction of existing DPCs have sufficient reactivity margin to remain subcritical if degraded and flooded
 - There are many types of DPCs (50 or more) with various types of degradation on exposure to ground water, and different fuel characteristics
- **Thermal management:**
 - Disposal power limit of 10 kW allows 98% of projected DPCs to cool by 2130 (6 kW DPCs by 2170, 4 kW BWR DPCs by 2170)
 - Favors disposal concepts with $\geq 200^{\circ}\text{C}$ temperature tolerance (e.g., at package surface) and greater thermal conductivity
 - BWR DPCs cool significantly faster (e.g., 4 kW BWR DPCs cool ~ 100 yr sooner than PWR DPCs)

Summary (3/3)

Review of Recommendations from Technical Feasibility Study through 2017

- Information needs analyzed (SNL 2015)
- Continue to collect and analyze information on existing DPCs
- Develop burnup credit approach for BWR fuel
- Ensure DPC service lifetime (≥ 100 yr) needed for thermal aging
- Investigate disposal concepts with greater host-medium thermal conductivity and temperature tolerance
- Research injectable fillers for postclosure criticality control in DPCs by moderator displacement
- Perform consequence analysis for criticality event exclusion from, or inclusion in performance assessment

SNL 2015. *Summary of Investigations on Technical Feasibility of Direct Disposal of Dual-Purpose Canisters*. FCRD-UFD-2015-000129 Rev. 0

Liljenfeldt, H. et al. 2016. *Summary of Investigations on Technical Feasibility of Direct Disposal of Dual-Purpose Canisters*. SFWD-SFWST-2017-000045 (calculations update to SNL 2015).

Questions?