

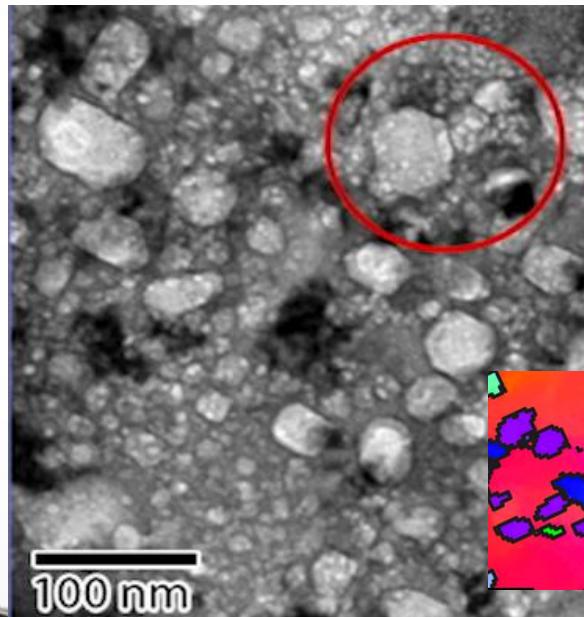
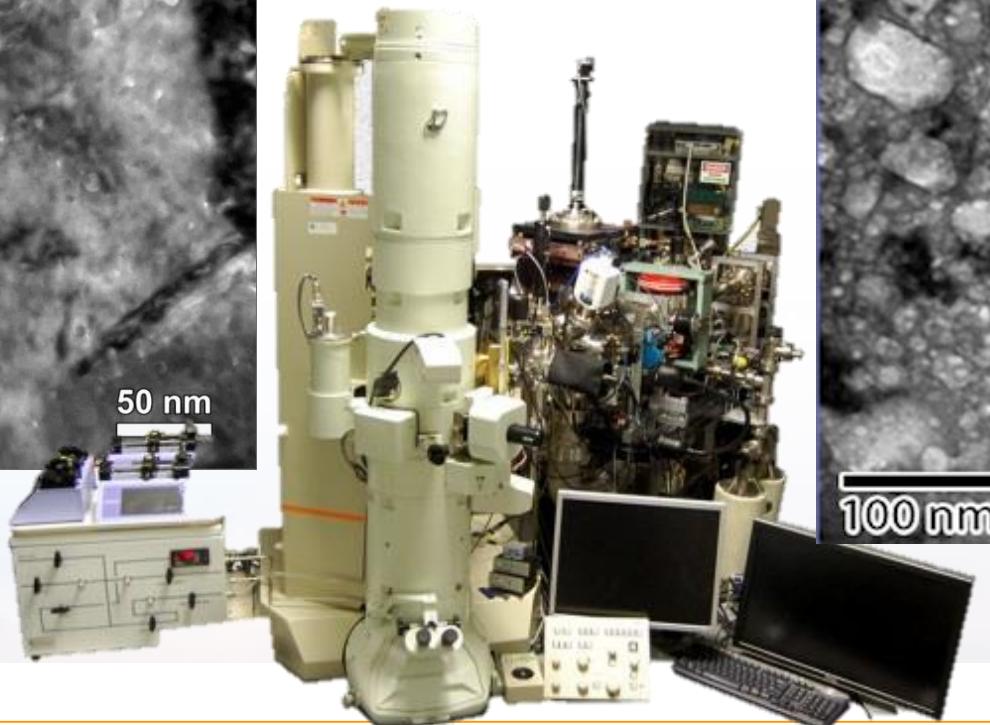
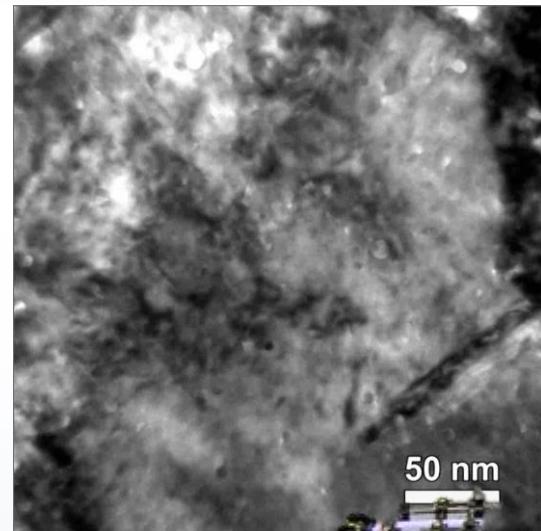
Deconvoluting Reactor Environmental Effects with Nanometer Resolution

SAND2018-0715PE

K. Hattar

Ion Beam Lab at Sandia National Laboratories

January 22nd, 2018



Collaborators:

- IBL: C.Taylor, C.M. Barr, S.A. Briggs, D.C. Bufford, D. Buller, C. Chisholm, B.G. Clark, M.T. Marshall, B. Muntifering, S.H. Pratt, & P. Price
- Sandia: M. Abere, B. Boyce, T.J. Boyle, R. Dingreville, R.F. Hess, A.C. Kilgo, B.E. Klamm, W.M. Mook, J.D. Puskar, J.A. Scott, & J.A. Sharon
- External: A. Aitkaliyeva, H. Bei, P.J. Ferreira, K.J. Ganesh, E.P. George, D. Gross, P. Hosemann, J. Kacher, S. Maloy, A. Minor, J. Qu, S. Rajesekhara, I.M. Robertson, D. Stauffer, & Hysitron Inc.



Sandia National Laboratories

“Exceptional service in the national interest”



THE WHITE HOUSE
WASHINGTON

May 13, 1949

Dear Mr. Wilson:

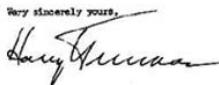
I am informed that the Atomic Energy Commission intends to ask that the Bell Telephone Laboratories accept under contract the direction of the Sandia Laboratory at Albuquerque, New Mexico.

This operation, which is a vital segment of the atomic weapons program, is of extreme importance and urgency in the national defense, and should have the best possible technical direction.

I hope that after you have heard more in detail from the Atomic Energy Commission, your organization will find it possible to undertake this task. In my opinion you have here an opportunity to render an exceptional service in the national interest.

I am writing a similar note direct to Dr. O. E. Buckley.

Very sincerely yours,



Mr. Leroy A. Wilson,
President,
American Telephone and Telegraph Company,
150 Broadway,
New York 7, N. Y.



Livermore, CA



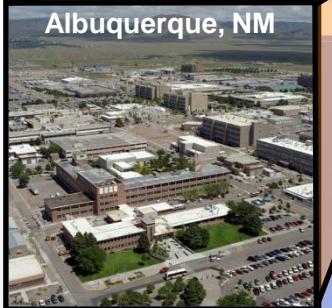
Nevada Test Site



Yucca Mountain



Kauai



Albuquerque, NM



Santa Teresa, NM



Pantex, TX

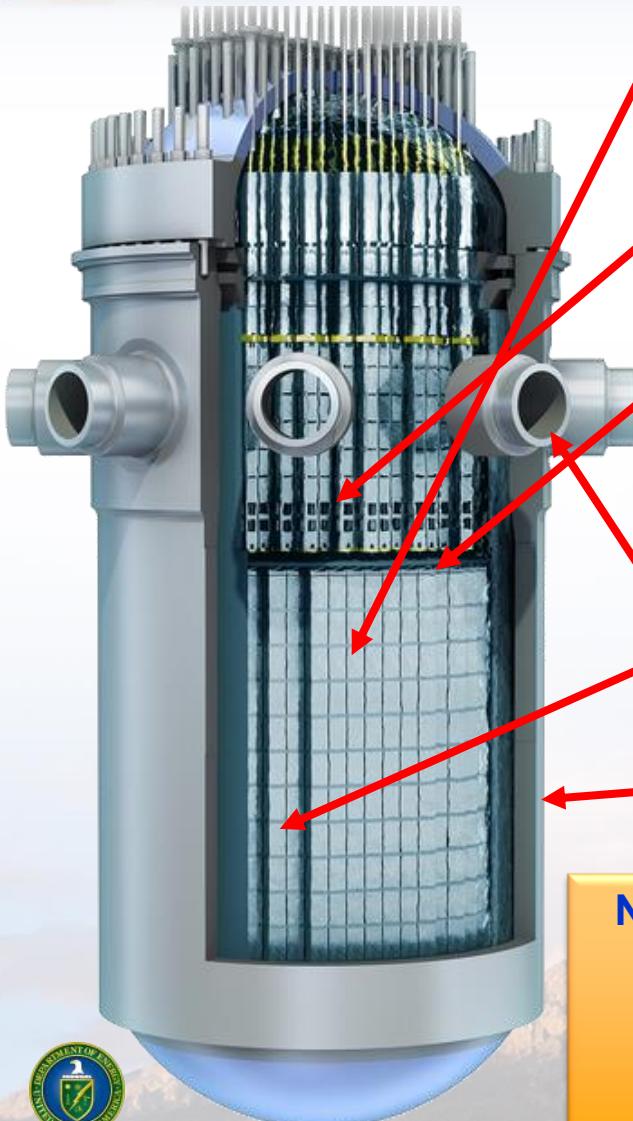


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 **SANDIA NATIONAL LABORATORIES**
President Harry S. Truman Fellowship in
National Security Science and Engineering

**"Sandia develops advanced technologies
to ensure global peace." – S. Younger**

Reactor Materials Challenges

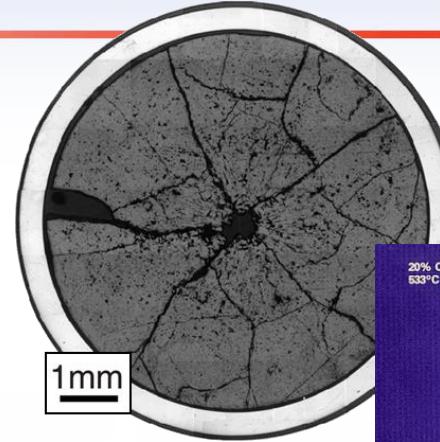


Nuclear Fuels

Displacement
Damage Effects

Transmutation Effects

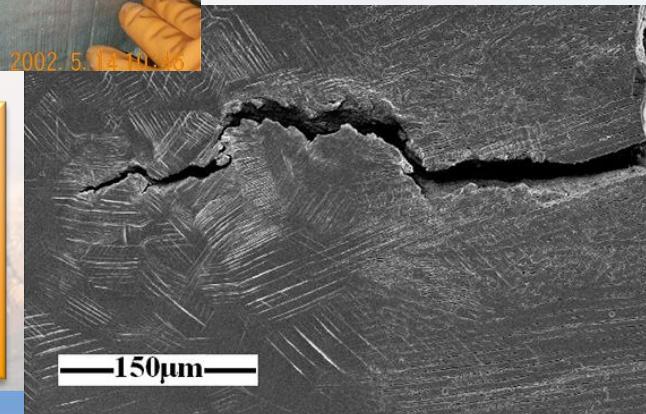
Corrosion
Creep
Fatigue



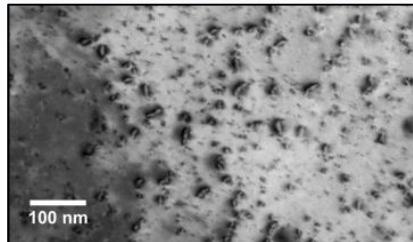
20% CW 316
633°C
UNIRRADIATED CONTROL



Nuclear reactors pose unique materials challenges due to degradation effects from combined environmental stressors



Investigating the nm Scale to Understand the km Scale



1 nm

1 μ m

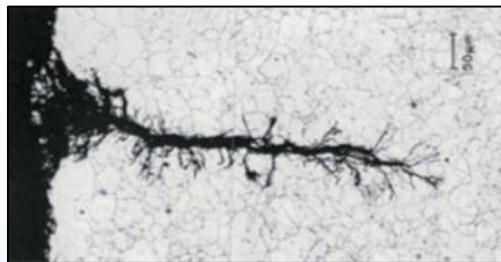


1 mm



1 km

10^7 m



In situ Ion Irradiation TEM (I³TEM)



Ion Beam Lab (IBL)



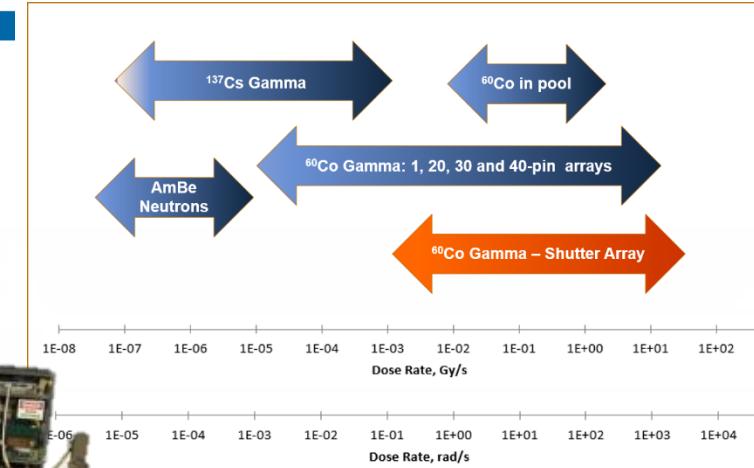
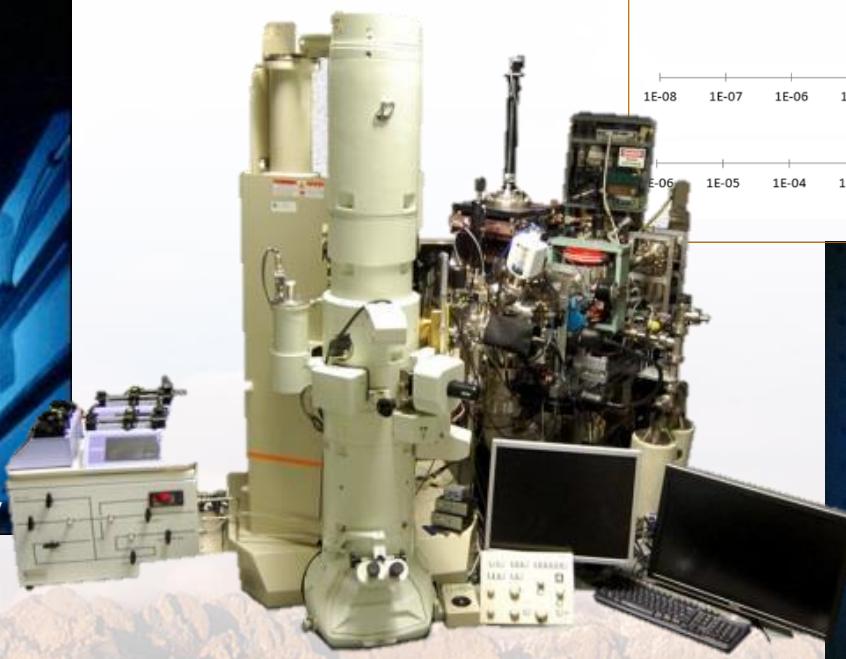
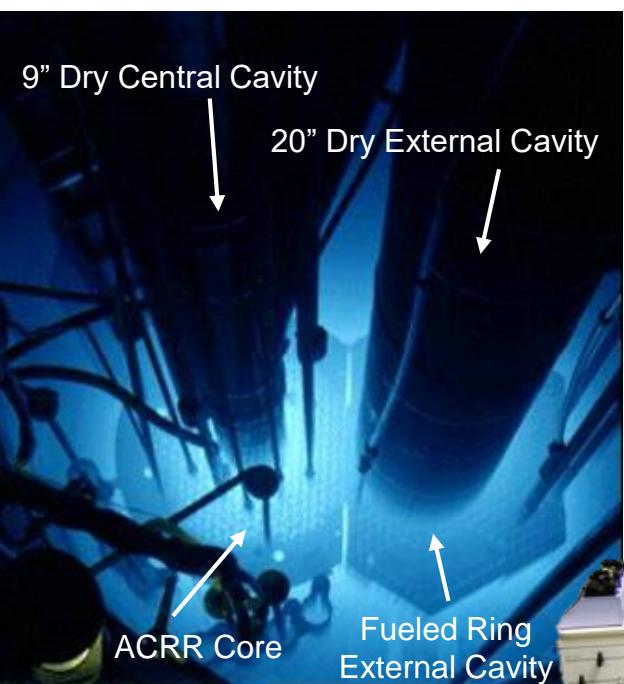
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Sandia's NSUF Capabilities

K. Hattar, D. Hanson, W. Martin, M. Wasiolek



Sandia National Laboratories is a multi-mission laboratory managed and operated by National Technology and Engineering Solutions of Sandia, LLC., a wholly owned subsidiary of Honeywell International, Inc., for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-NA0003525.

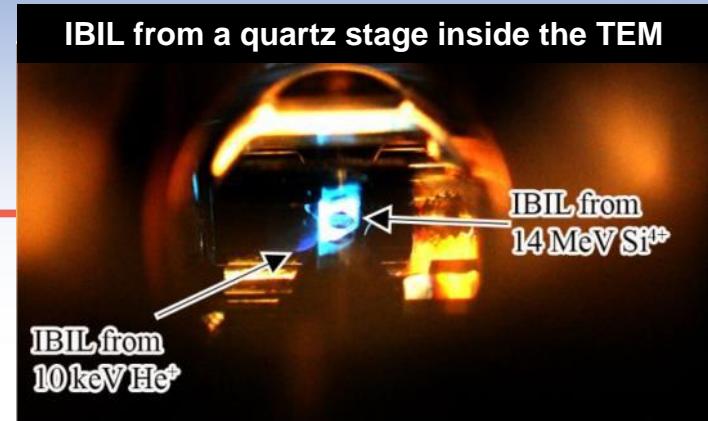
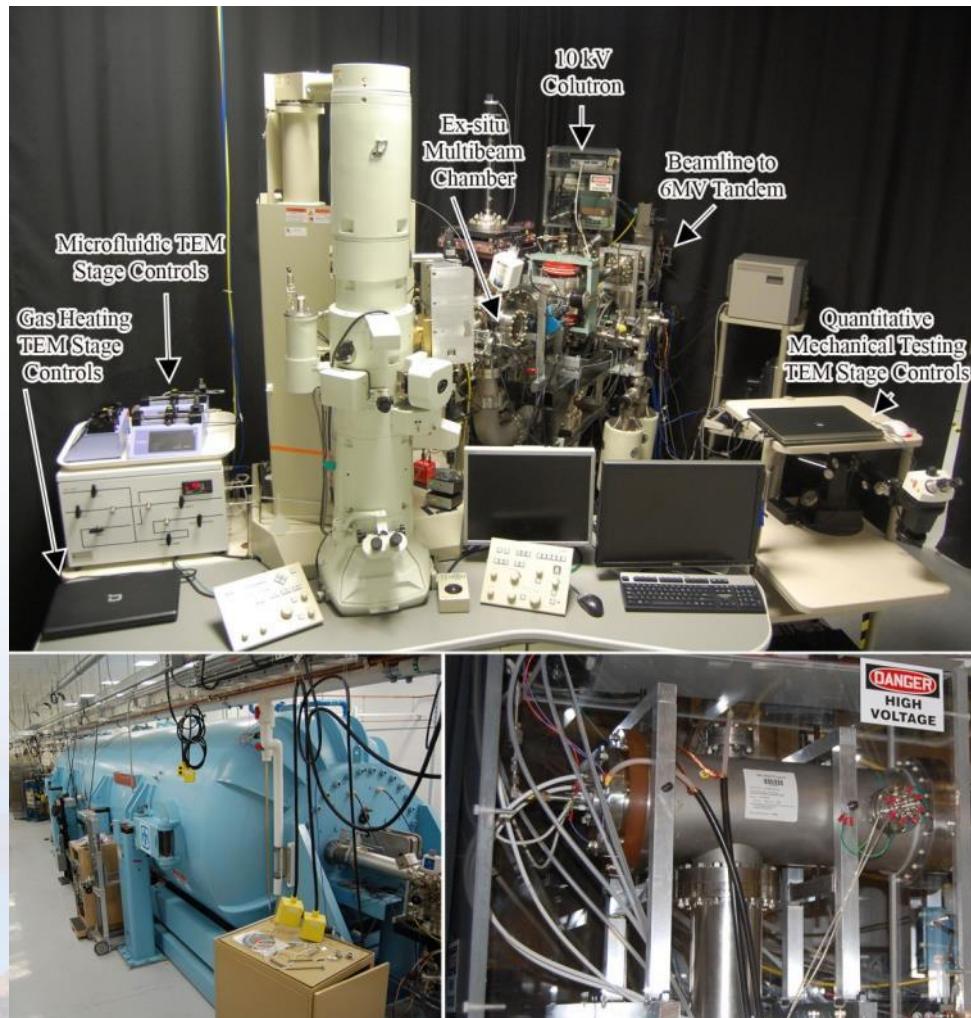


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Sandia's Concurrent *In situ* Ion Irradiation TEM Facility

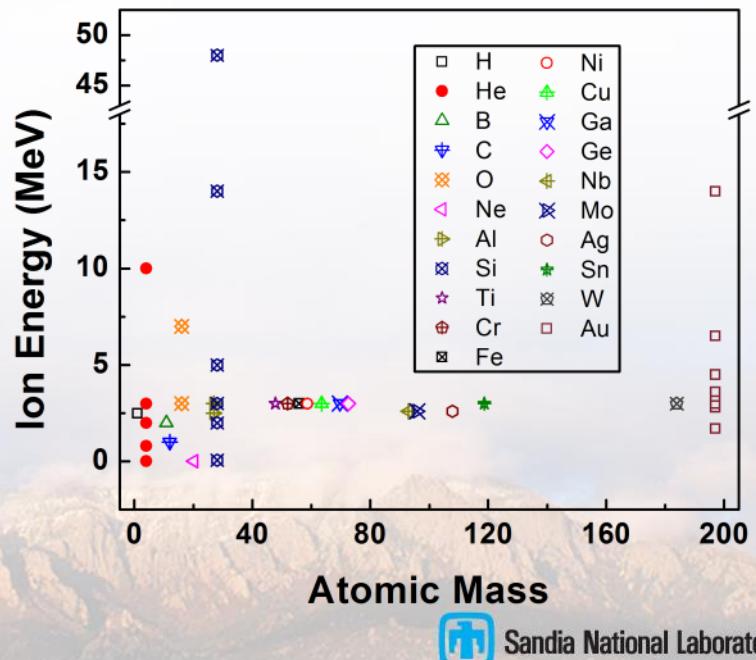
Collaborator: D.L. Buller

10 kV Colutron - 200 kV TEM - 6 MV Tandem



Direct real time observation
of ion irradiation,
ion implantation, or both
with nanometer resolution

Ion species & energy introduced into the TEM



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Benefits & Limitations of *in situ* TEM

Benefits

- ## 1. Real-time nanoscale resolution observations of microstructural dynamics

Limitations

- 1. Predominantly limited to microstructural characterization**
 - Some work in thermal, optical, and mechanical properties
 - 2. Limited to electron transparent films**
 - Can often prefer surface mechanisms to bulk mechanisms
 - Local stresses state in the sample is difficult to predict
 - 3. Electron beam effects**
 - Radiolysis and Knock-on Damage
 - 4. Vacuum conditions**
 - 10^{-7} Torr limits gas and liquid experiments feasibility
 - 5. Local probing**
 - Portions of the world study is small

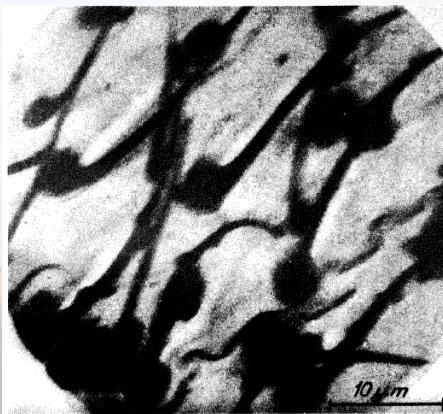


Fig. 6: Wing surface of the house fly.
 (First internal photography, $U = 60$ kV, $M_a = 2200$)
 (Driest, E., and Müller, H.O.: Z. Wiss. Mikroskopie 52, 53-57 (1935))

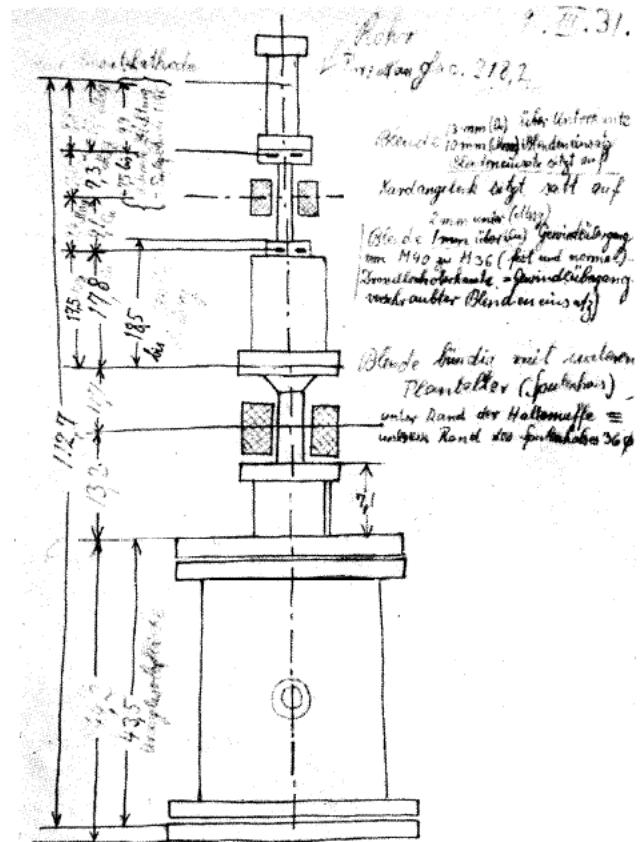
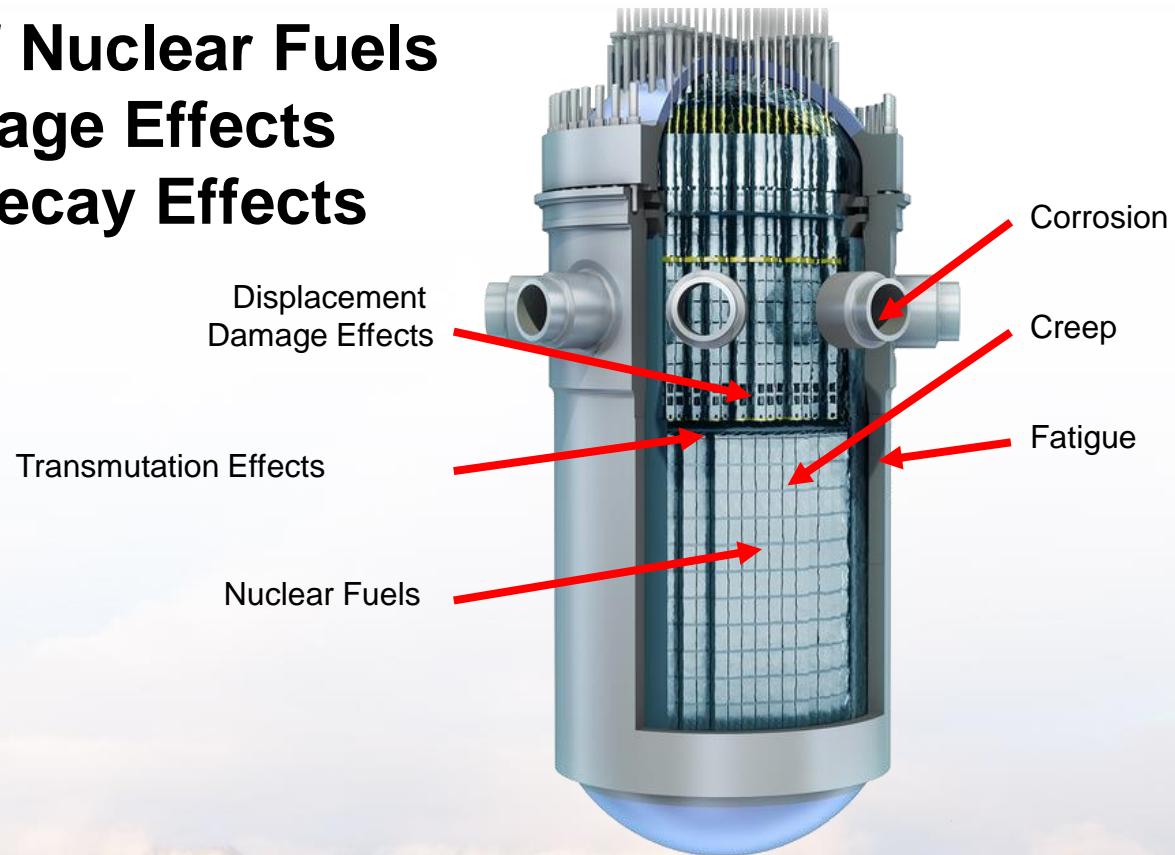


Fig. 2: Sketch by the author (9 March 1931) of the cathode ray tube for testing one-stage and two-stage electron-optical imaging by means of two magnetic electron lenses (electron microscope) [8].



Outline

1. Characterization of Nuclear Fuels
2. Displacement Damage Effects
3. Transmutation & Decay Effects
4. Corrosion
5. Creep
6. Fatigue

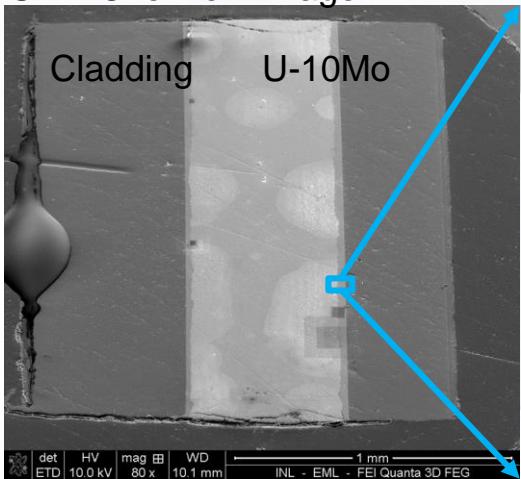


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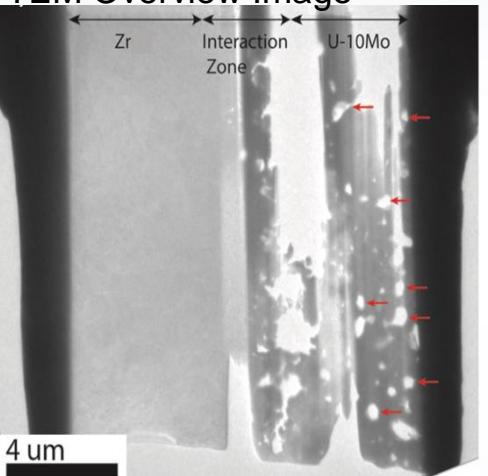
Post Irradiation Characterization of U-10Mo/Zr monolithic interface

Collaborators: C. Barr, A. Aitkaliyeva

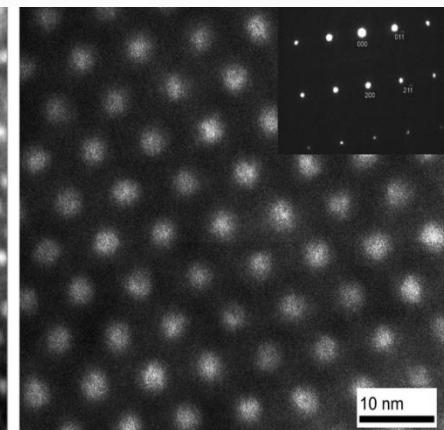
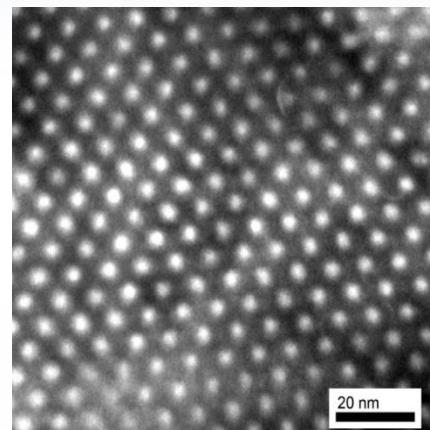
SEM Overview Image



TEM Overview Image

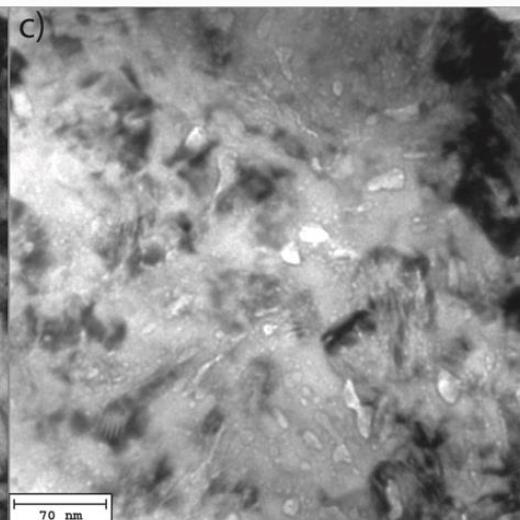
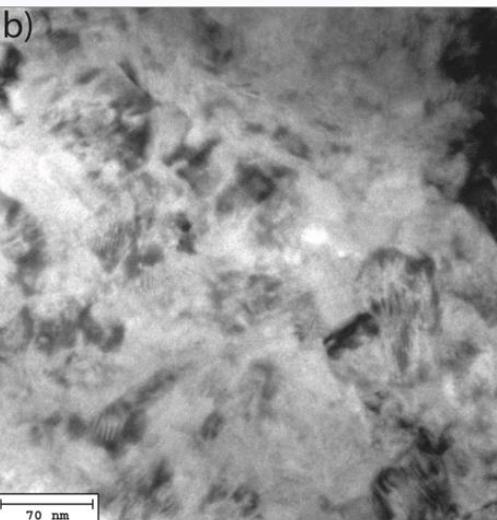
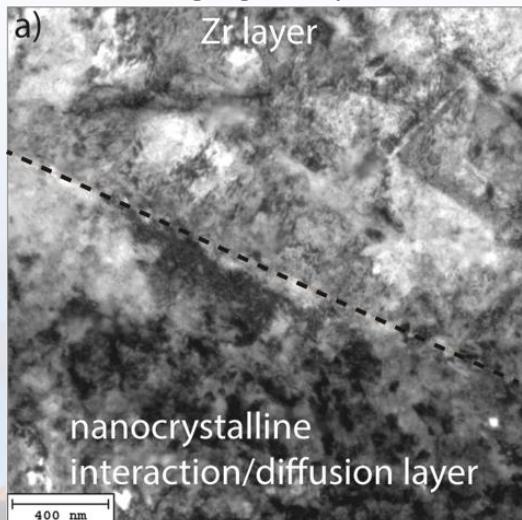


Gas superlattice bubbles ZA [110] in U-10Mo.



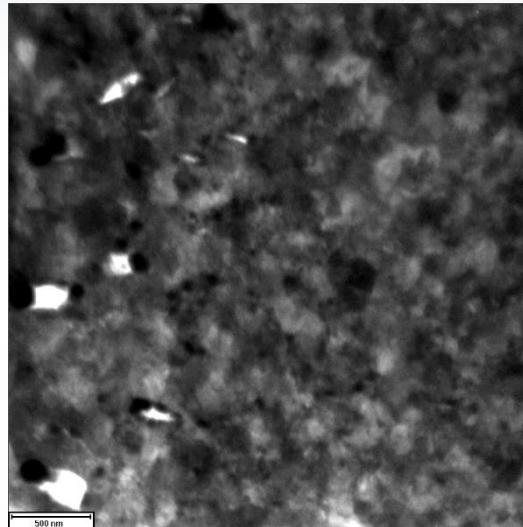
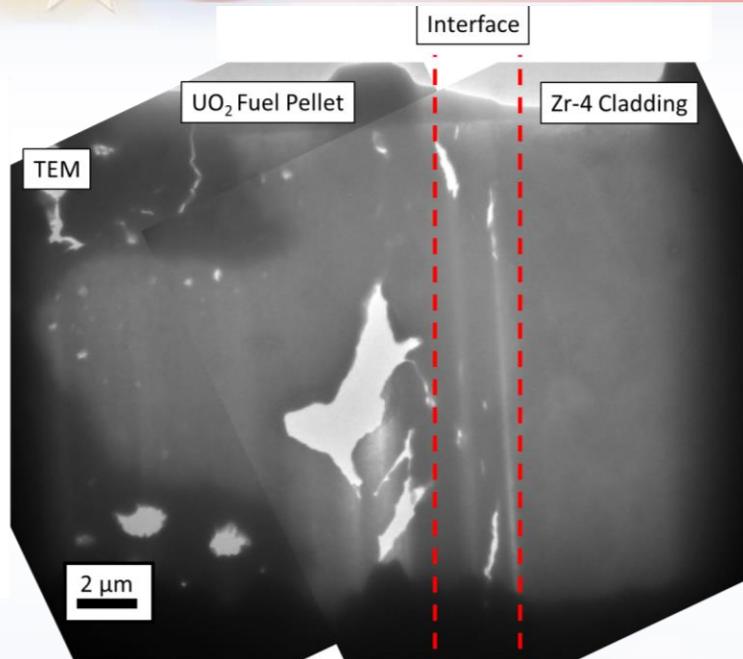
- U-10Mo Monolithic fuel with fission density (fiss/cm^3) = 4.4×10^{21}

- Average gas superlattice bubble diameter distribution is $3.5 \pm 0.25 \text{ nm}$ diameter.

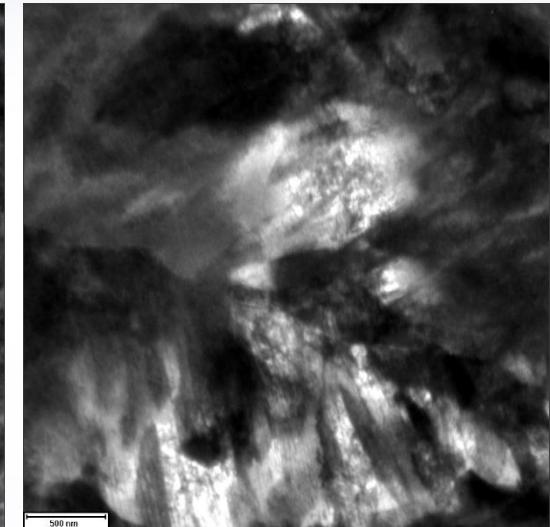


Zr layer and interaction layers where in-focus image of interaction layer (center) and de-focus (-2um) image of interaction layer with high level of porosity apparent
India National Laboratories

Characterization of HB Robinson

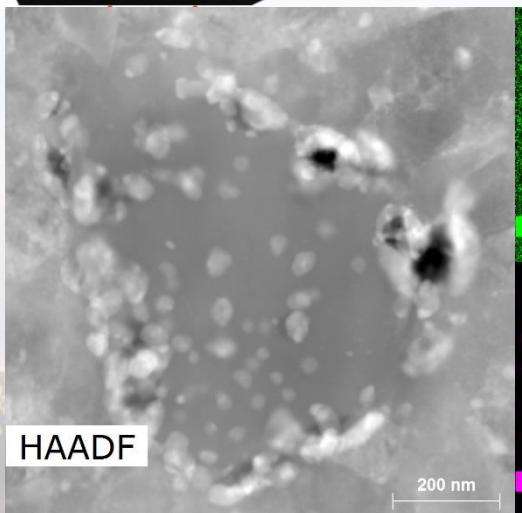


UO_2 Fuel Pellet

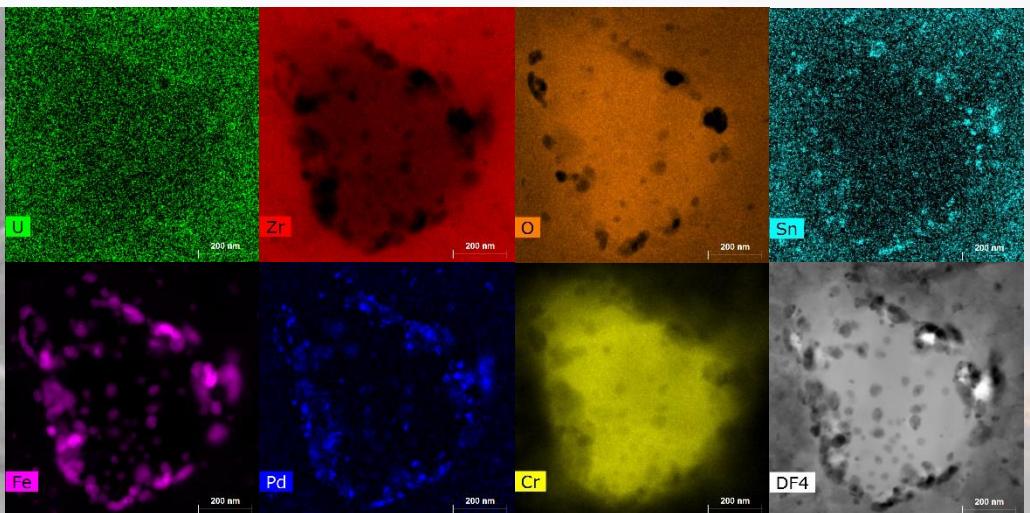


Zr-4 Cladding

Complex
Nanostructured
Material
Resulting from
Typical Nuclear
Reactor
Operation



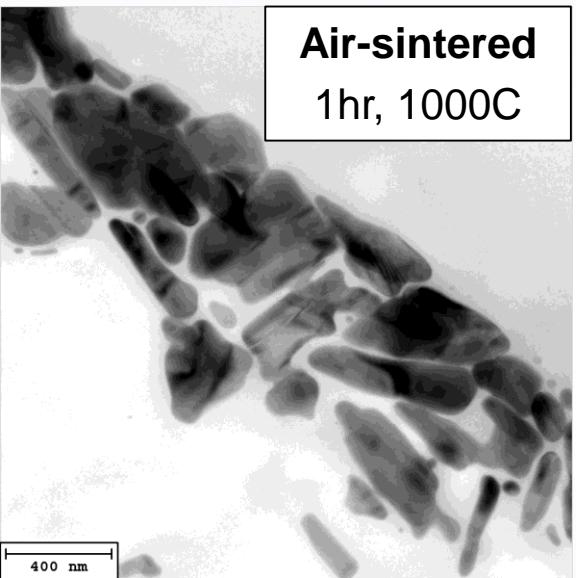
HAADF



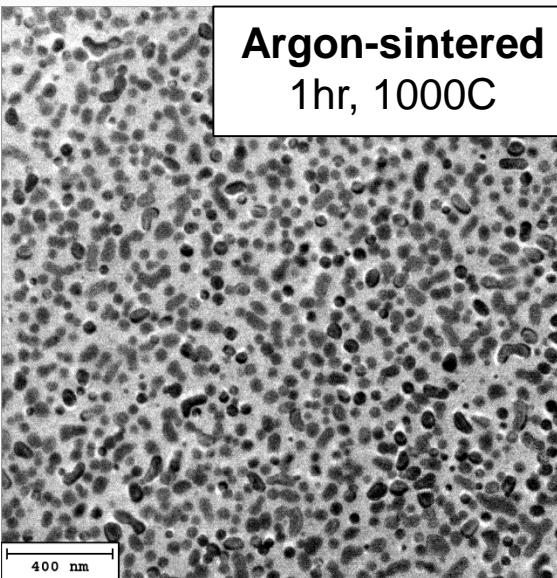
Uranium Oxide Formation in Different Sintering Environments

Collaborators: S.A. Briggs, R. Hess, and B. Klamm

Air-sintered
1hr, 1000C



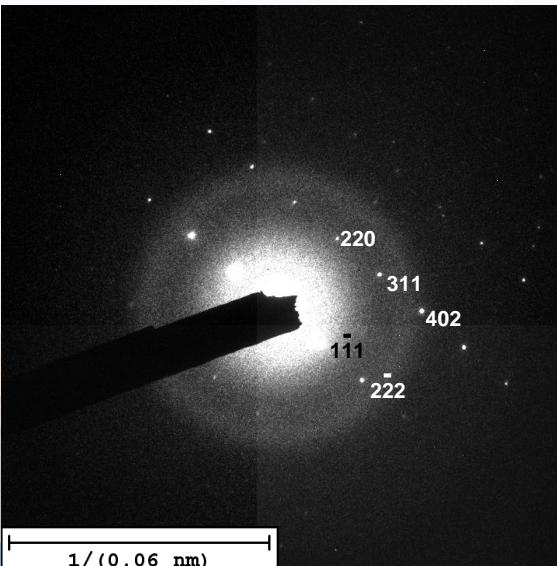
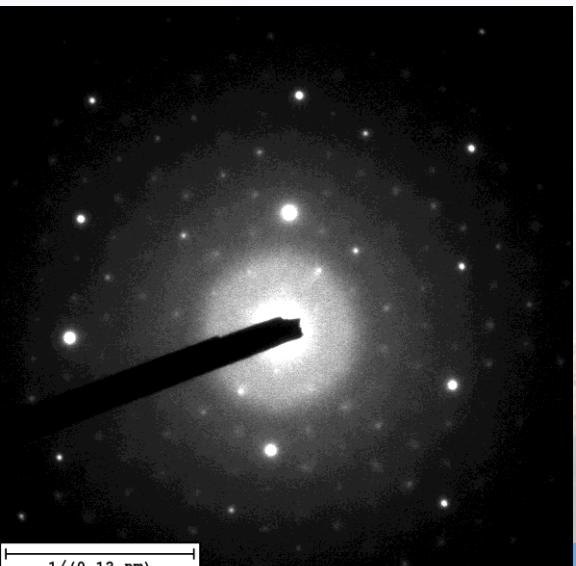
Argon-sintered
1hr, 1000C



Nanoparticles prepared from solution of $\text{UO}_2(\text{NO}_3)_2$, PEI, and EDTA

Ar-sintered specimen phase diffraction patterns map to fluorite/FCC structure characteristic of UO_2

- Larger lattice parameter suggests hypostoichiometric uranium-dioxide phase (UO_{2-x})



Air-sintered specimens do not map to fluorite/FCC

- Likely a hyperstoichiometric uranium-oxide phase (U_3O_8 , U_4O_9)

Characterization of ion-irradiation response is work-in-progress

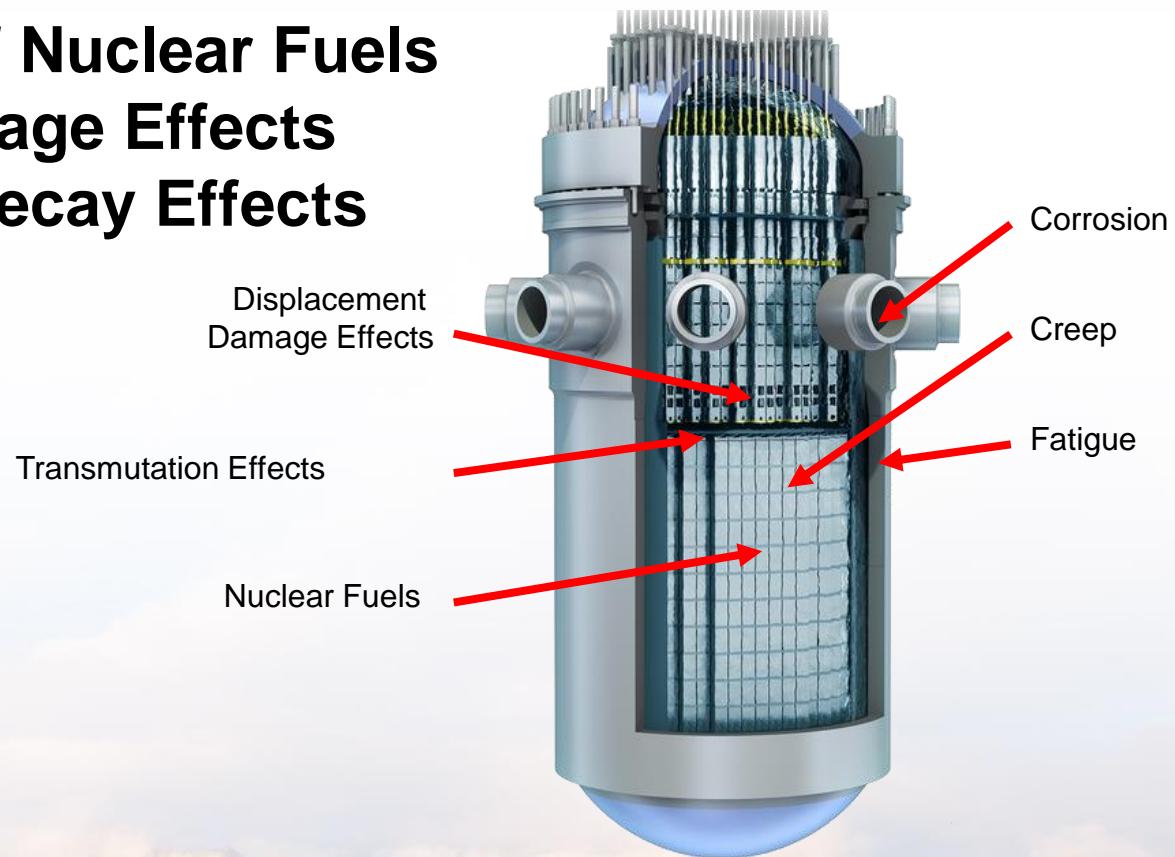


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Outline

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6. Fatigue



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Cumulative Effects of Ion Irradiation as a Function of Ion Energy and Au Particle Size

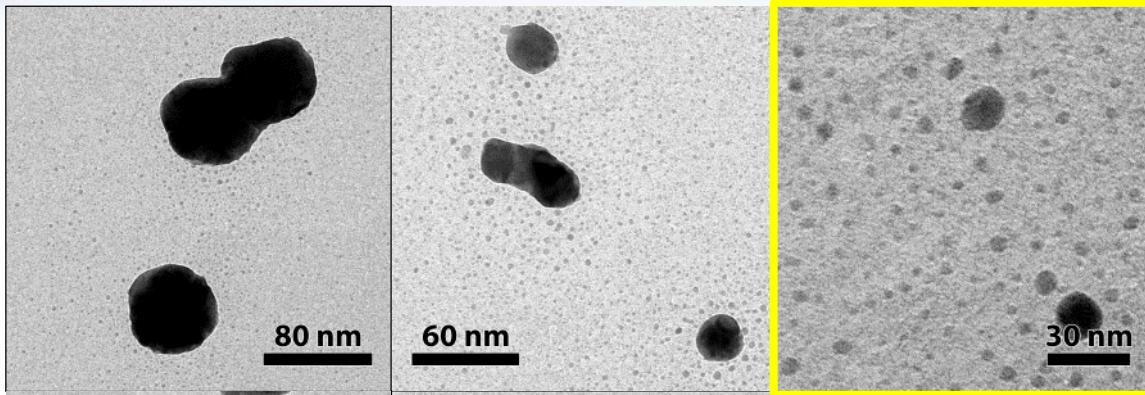
60 nm

20 nm

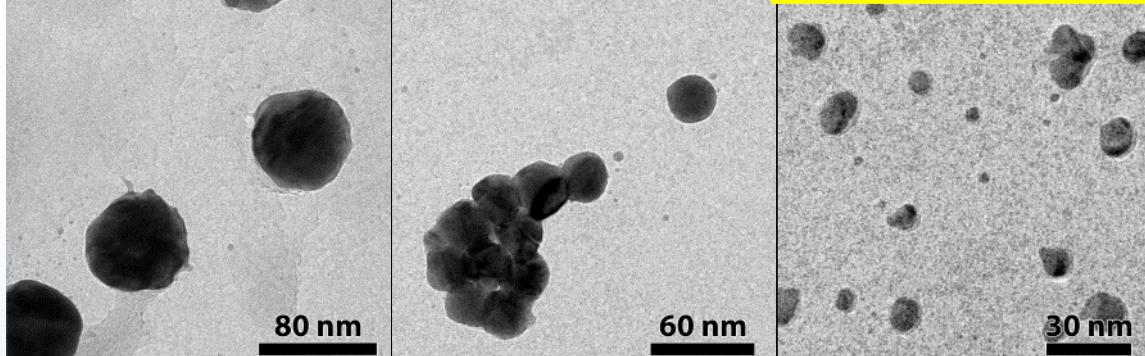
5 nm

Collaborator: D.C. Bufford

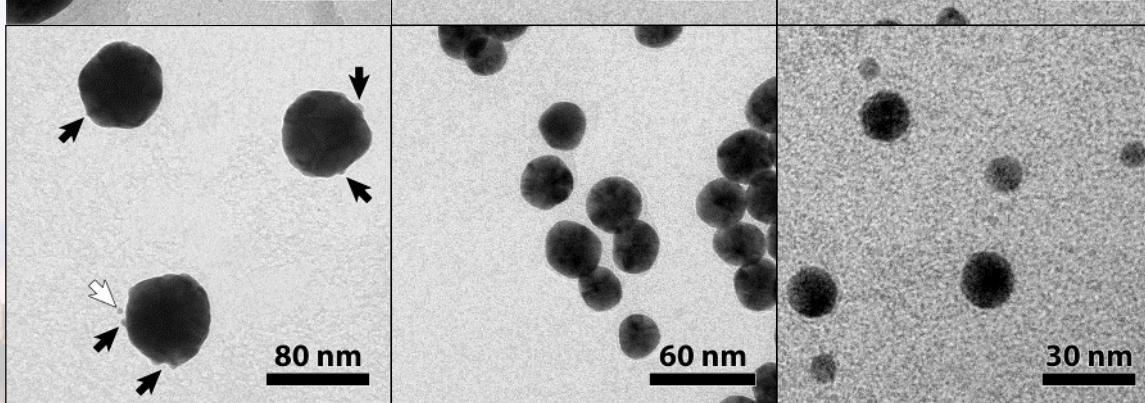
46 keV Au¹⁻
 $3.4 \times 10^{14} / \text{cm}^2$



2.8 MeV Au⁴⁺
 $4 \times 10^{13} / \text{cm}^2$



10 MeV Au⁸⁺
 $1.3 \times 10^{12} / \text{cm}^2$



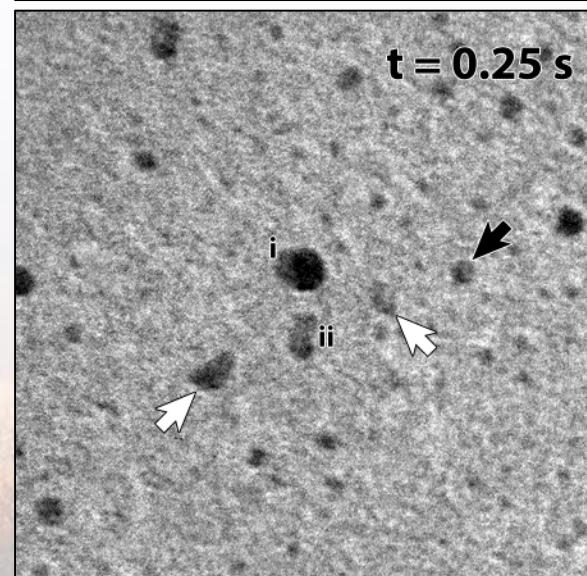
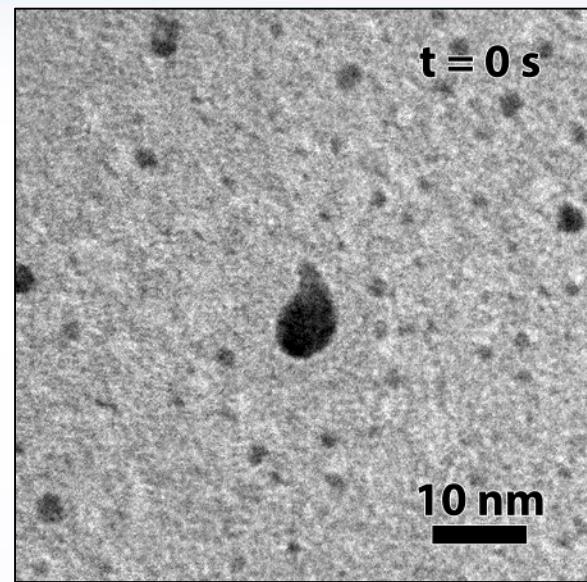
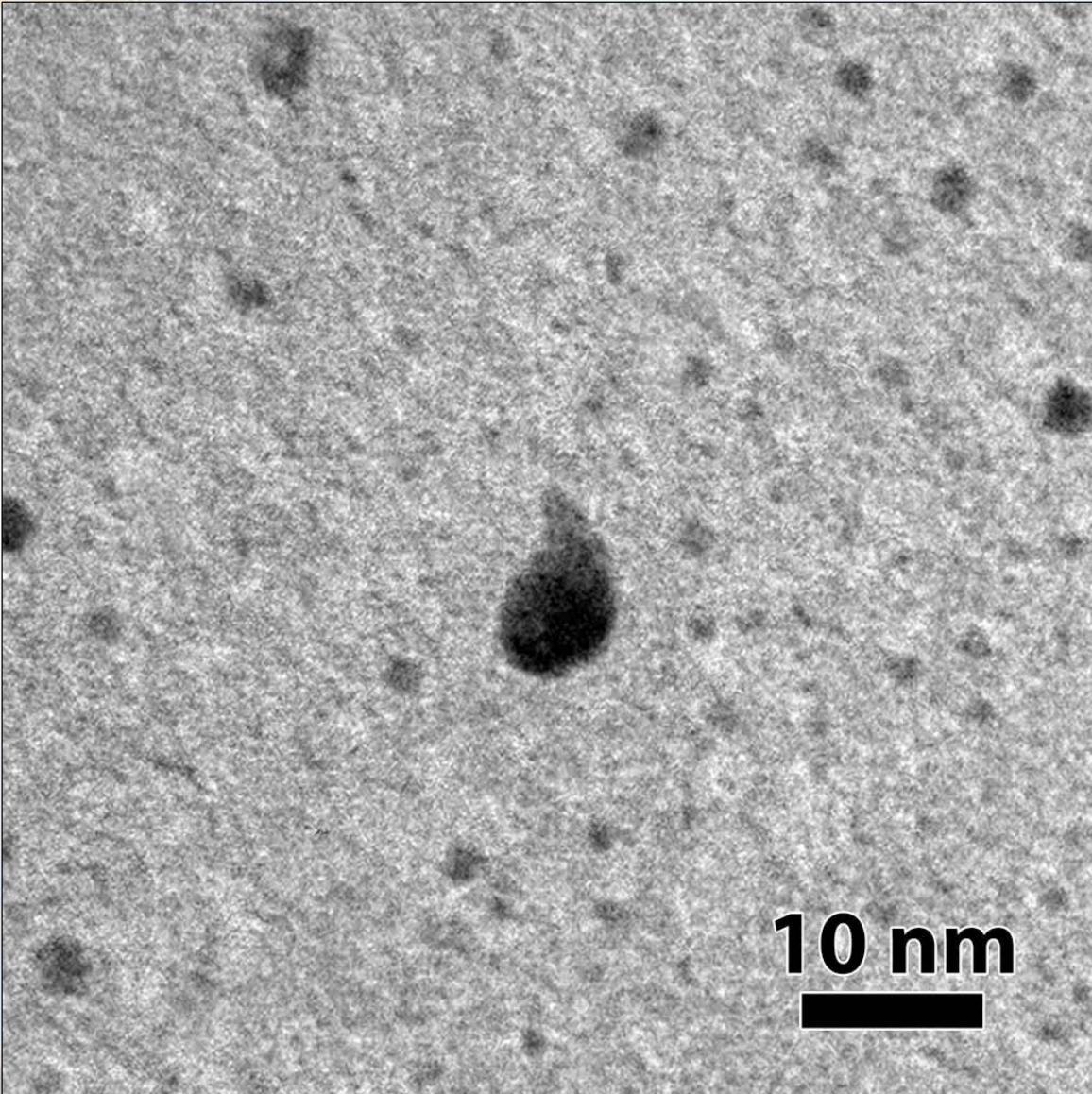
Particle and ion energy dictate the ratio of sputtering, particle motion, particle agglomeration, and other active mechanisms



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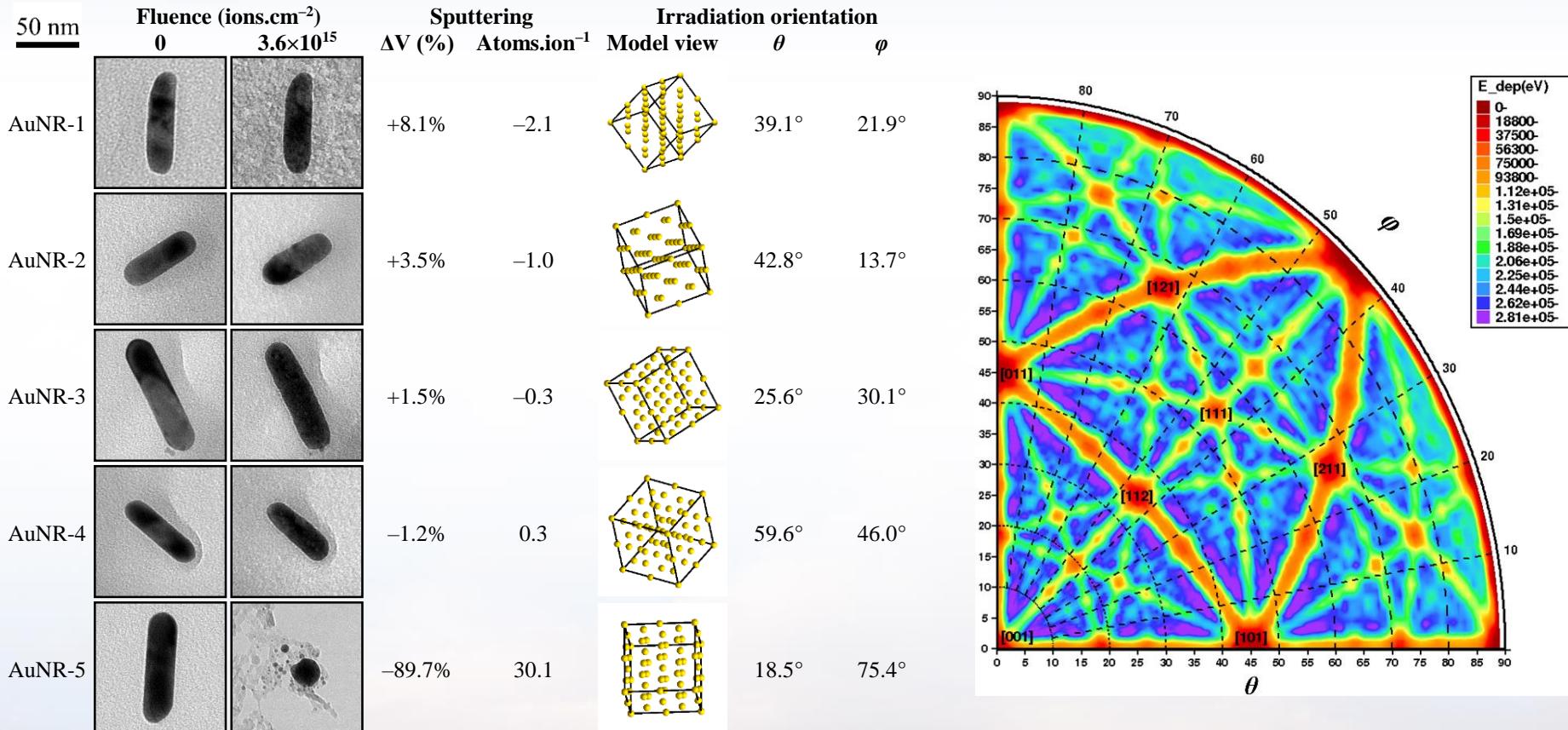
Single Ion Effects with 46 keV Au¹⁻ ions: 5 nm

Collaborator: D.C. Bufford



Exploring Radiation effects in Au Nanorods

Collaborators: J. A. Hinks, F. Hibberd, A. Ilinov, D. C. Bufford, F. Djurabekova, G. Greaves, A. Kuronen, S. E. Donnelly & K. Nordlund

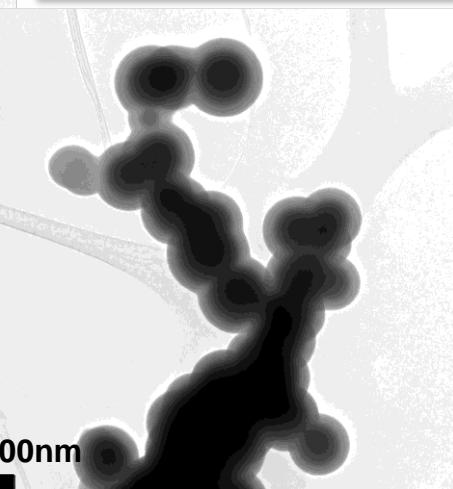
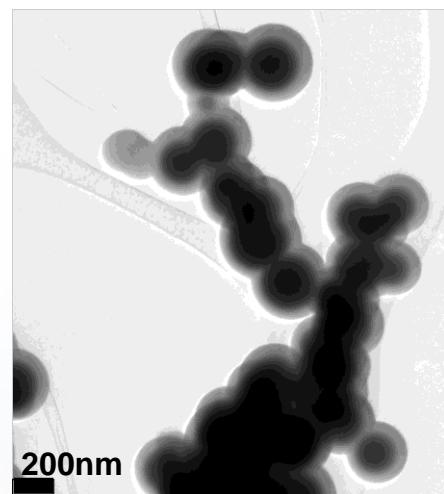
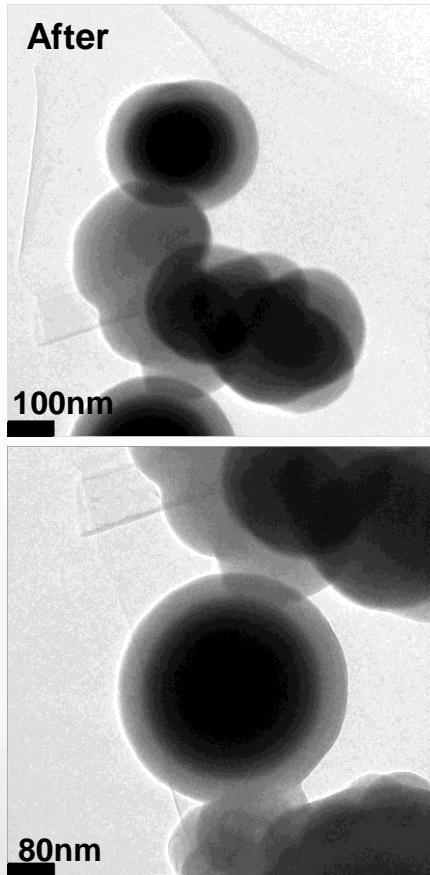
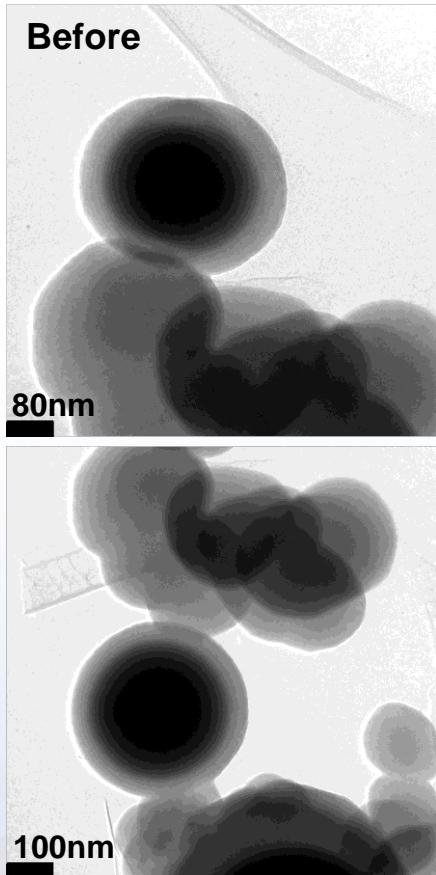


Crystal Orientation Matters!



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Irradiation of Amorphous Hf Oxide Nanoparticles with 10 nA of 3 MeV Cu Resulted in No Obvious Changes



Sample was placed in tomography holder, tilted to 80°, and irradiated for an additional 2.5 h. No obvious changes.

3 hour irradiation: no obvious changes.



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Radiation Tolerance is Needed in Advanced Scintillators for Non-proliferation Applications

Contributors: S.M. Hoppe, B.A. Hernandez-Sanchez, T. Boyle



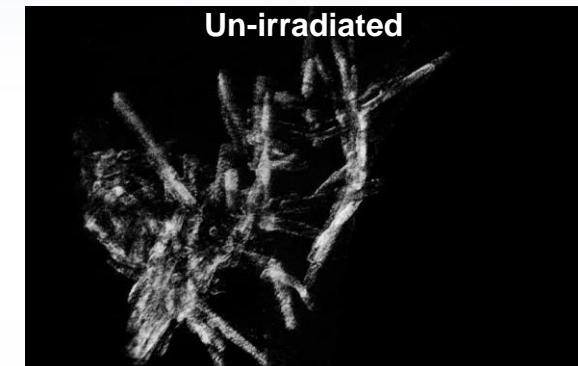
In situ Ion Irradiation TEM (I³TEM)



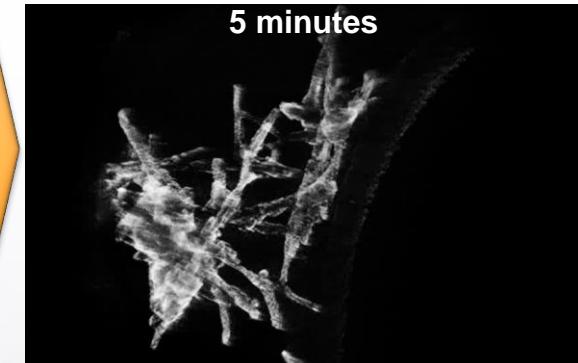
Hummingbird
tomography stage



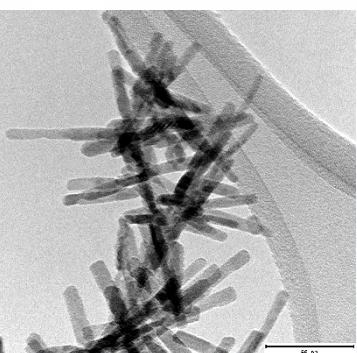
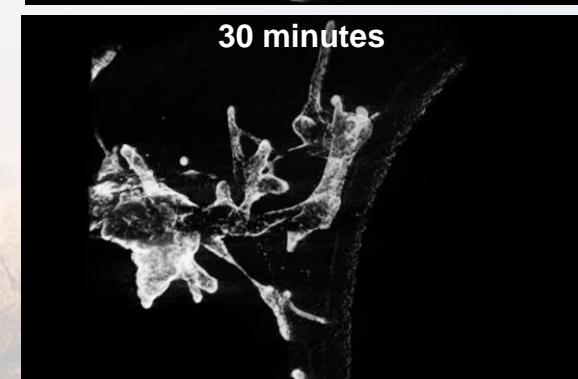
Tomography of Irradiated CdWO₄:
3 MeV Cu³⁺ at ~30 nA



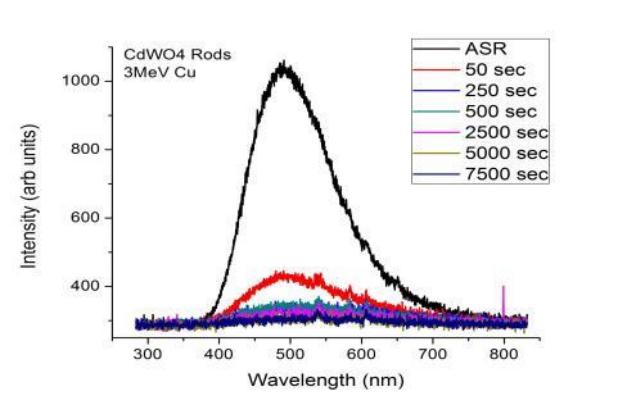
5 minutes



30 minutes



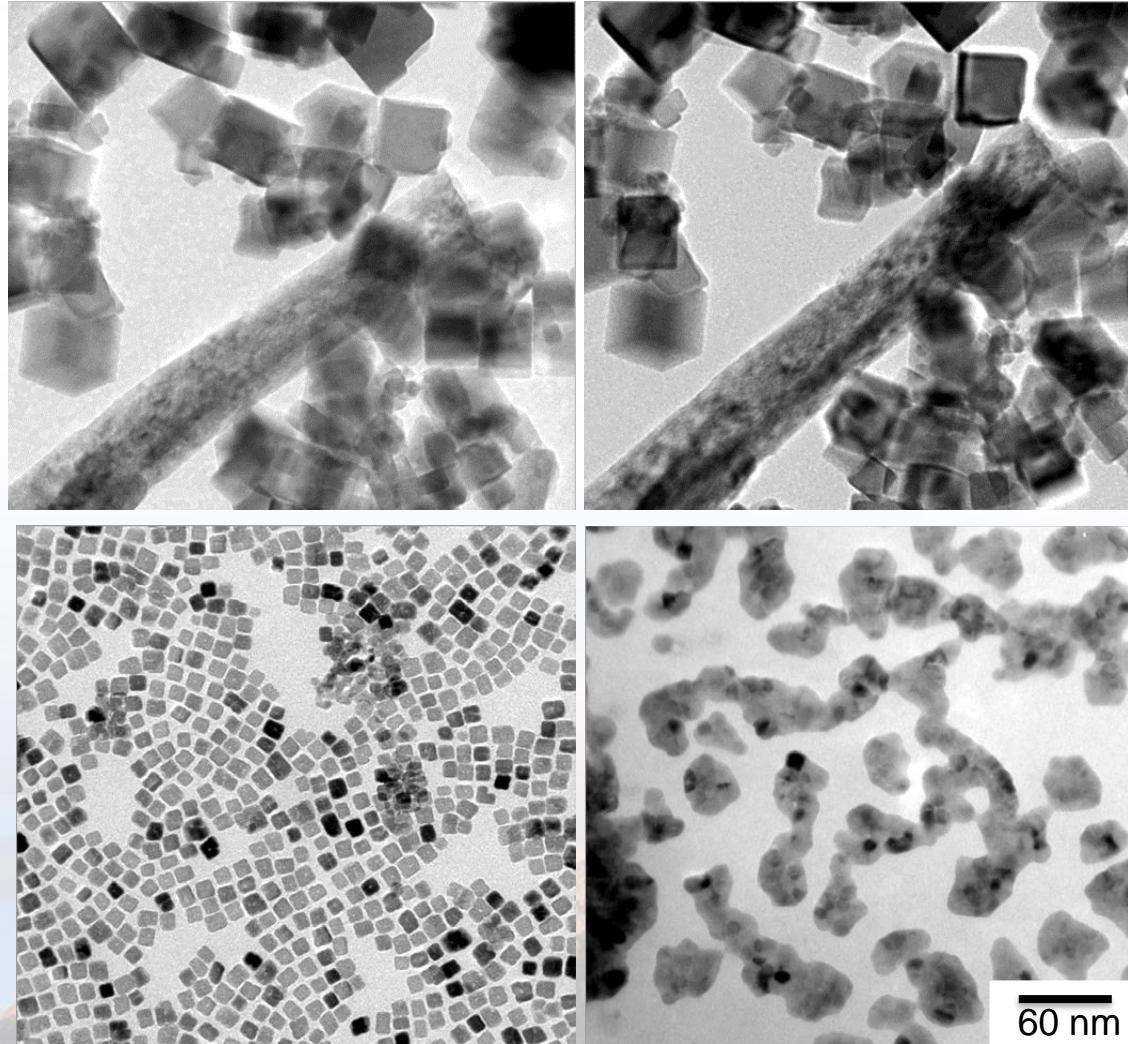
High-Z
nanoparticles
(CdWO₄) are
promising,
but are
radiation
sensitive



Details of Radiation Environment is Important!

Collaborators: S.A. Briggs, C. Taylor, J. Kolar, T. Boyle

Radiation Stability of CeO_x Nanoparticles



In situ

- 2.7 MeV Si+, RT, $8.83\text{E}11$ ions/cm₂s, $7.96\text{E}-5$ dpa/s
- CeCl_3 irradiated for 6 mins, $2.87\text{E}-2$ dpa
- CeNO_3 irradiated for 18 minutes $8.60\text{E}-2$ dpa

Ex situ

- 2.7 MeV Si+, RT, $2.41\text{E}11$ ions/cm₂s, $2.2\text{E}-5$ dpa/s
- Both irradiated for 66 mins to $8.7\text{E}-2$ dpa

Difference between a broad DC beam vs. rastered beam is significant!

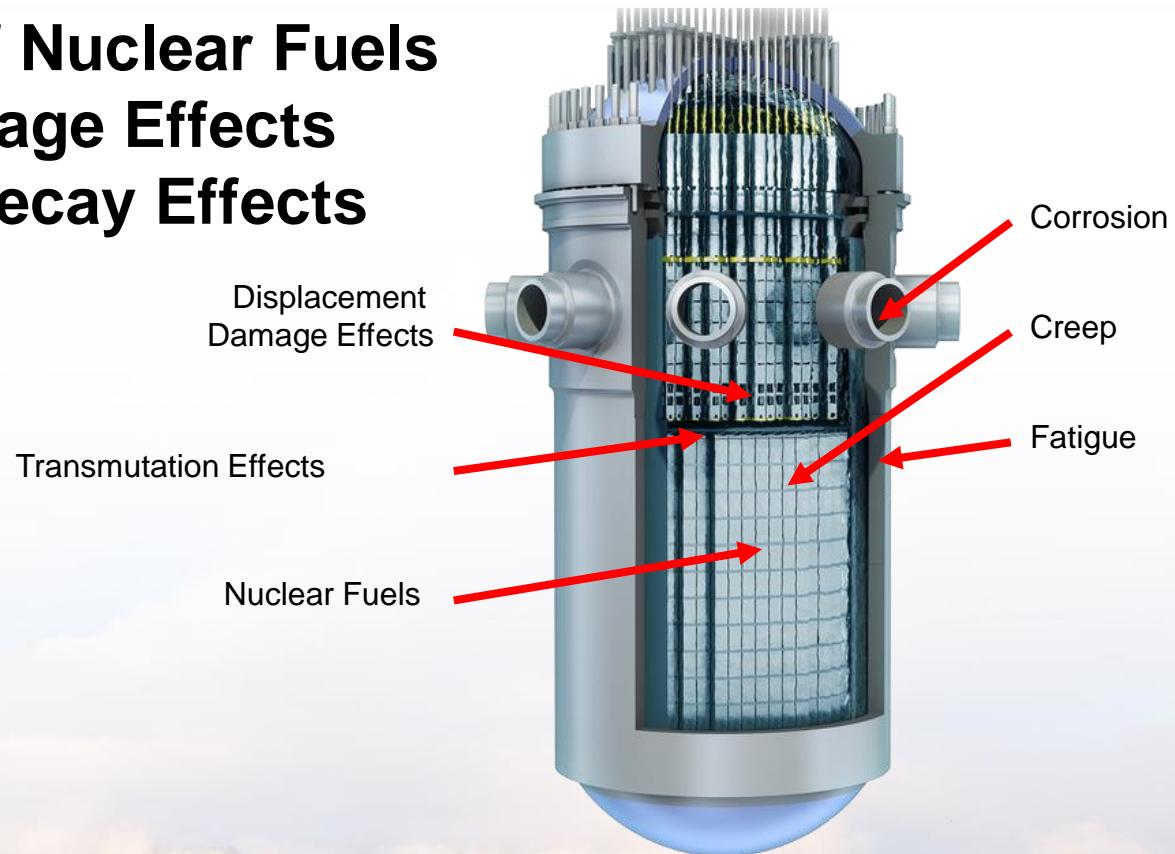


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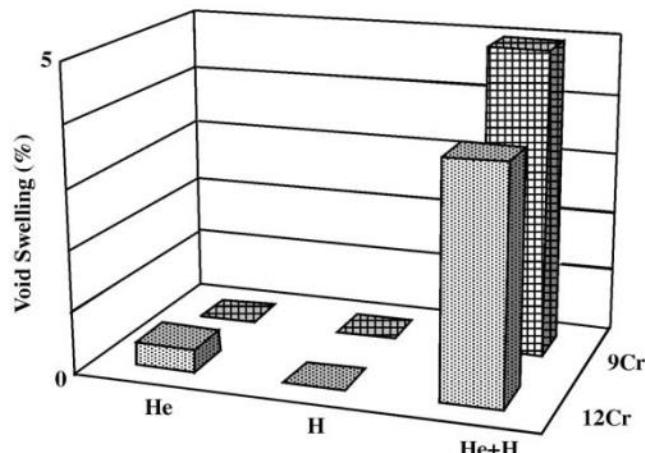
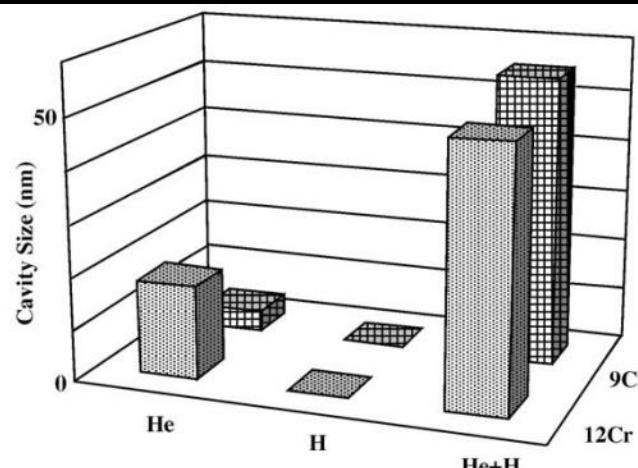
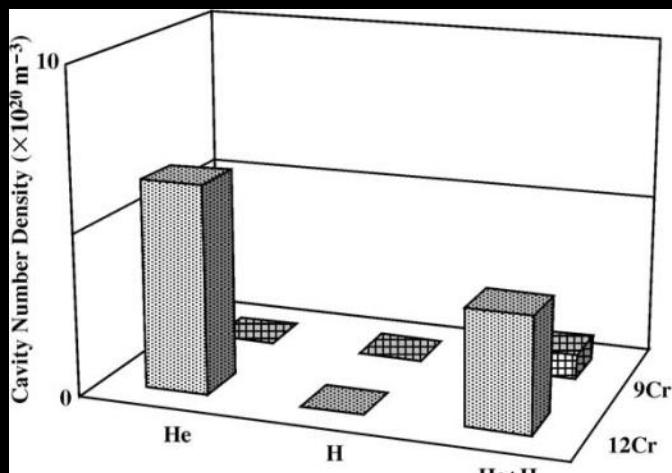
Outline

- 1. Characterization of Nuclear Fuels**
- 2. Displacement Damage Effects**
- 3. Transmutation & Decay Effects**
- 4. Corrosion**
- 5. Creep**
- 6. Fatigue**



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H, He, and Displacement Damage Synergy



T. Tanaka et al. "Synergistic effect of helium and hydrogen for defect evolution under multi-ion irradiation of Fe-Cr ferritic alloys"

Coupling Effect

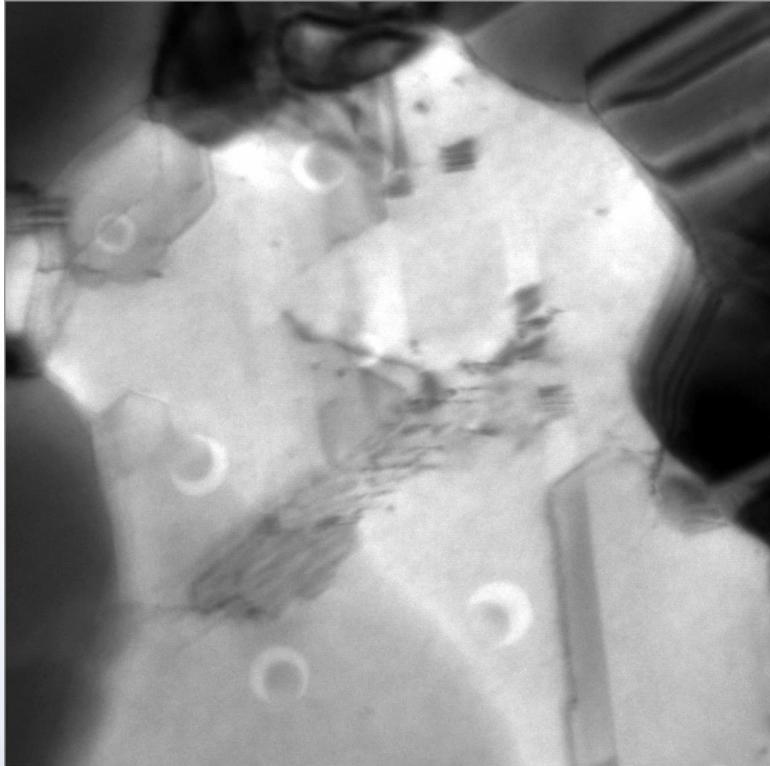
- H and He are produced as decay products
- The relationship between the point defects present, the interstitial hydrogen, and the He bubbles in the system that results in the increased void swelling has only been theorized.
- The mechanisms which governs the increased void swelling under the presence of He and H have never been experimentally determined

No capability currently exists for triple beam irradiation in the U.S. and No capability for triple beam TEM ion irradiation exists in the world

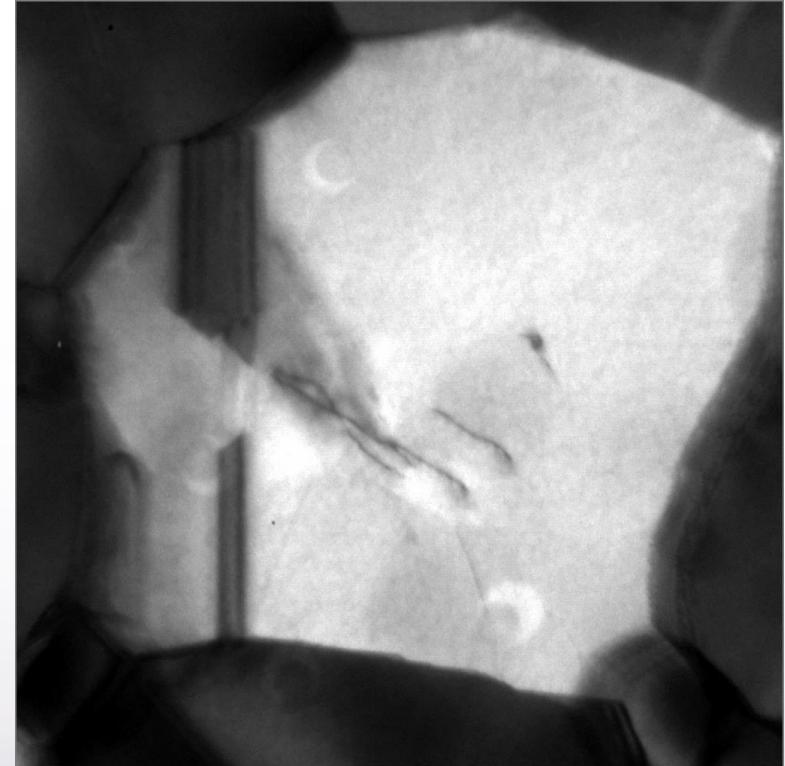
Dose Rate Effects in Nanocrystalline Metals

Collaborators: C. Chisholm , P. Hosemann, & A. Minor

7.9×10^9 ions/cm²/s



6.7×10^7 ions/cm²/s



VS

Improved vibrational and ion beam stability permits us to work at 120kx or higher permitting imaging of single cascade events

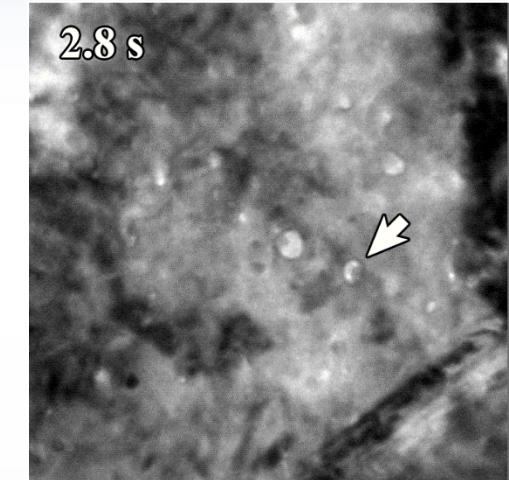
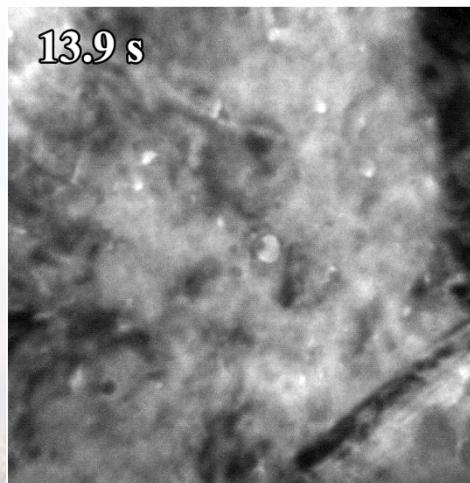
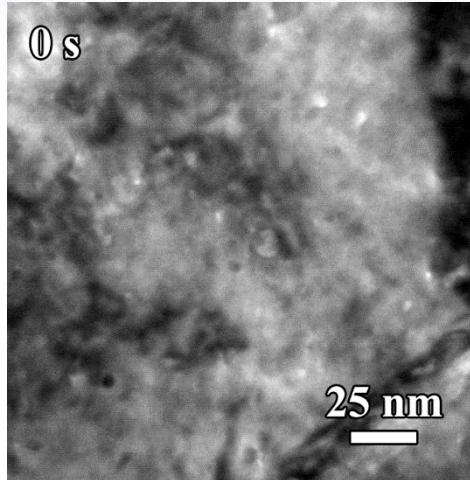
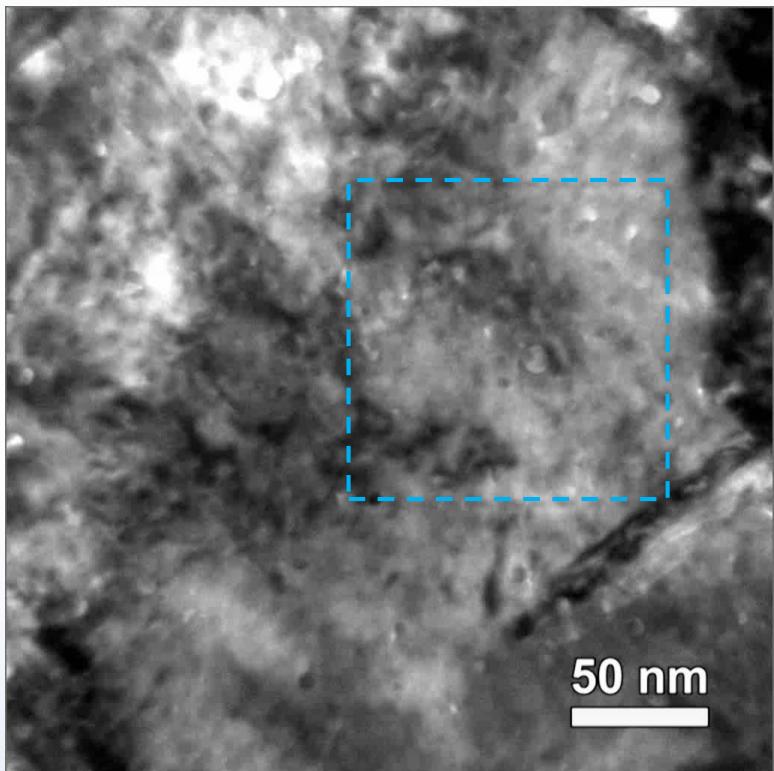


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Simultaneous *In situ* TEM Triple Beam: 2.8 MeV Au⁴⁺ + 10 keV He⁺/D₂⁺

Collaborator: D.C. Bufford

Video playback speed x1.5.



- Approximate fluence:
 - Au 1.2×10^{13} ions/cm²
 - He 1.3×10^{15} ions/cm²
 - D 2.2×10^{15} ions/cm²
- Cavity nucleation and disappearance

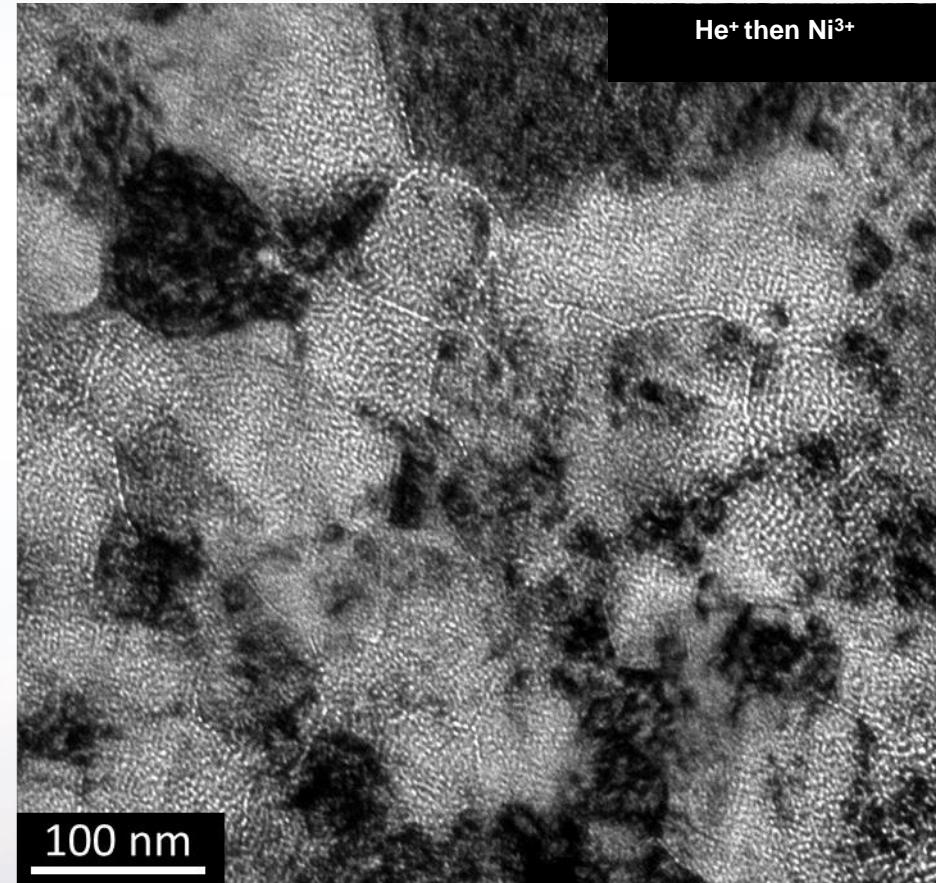
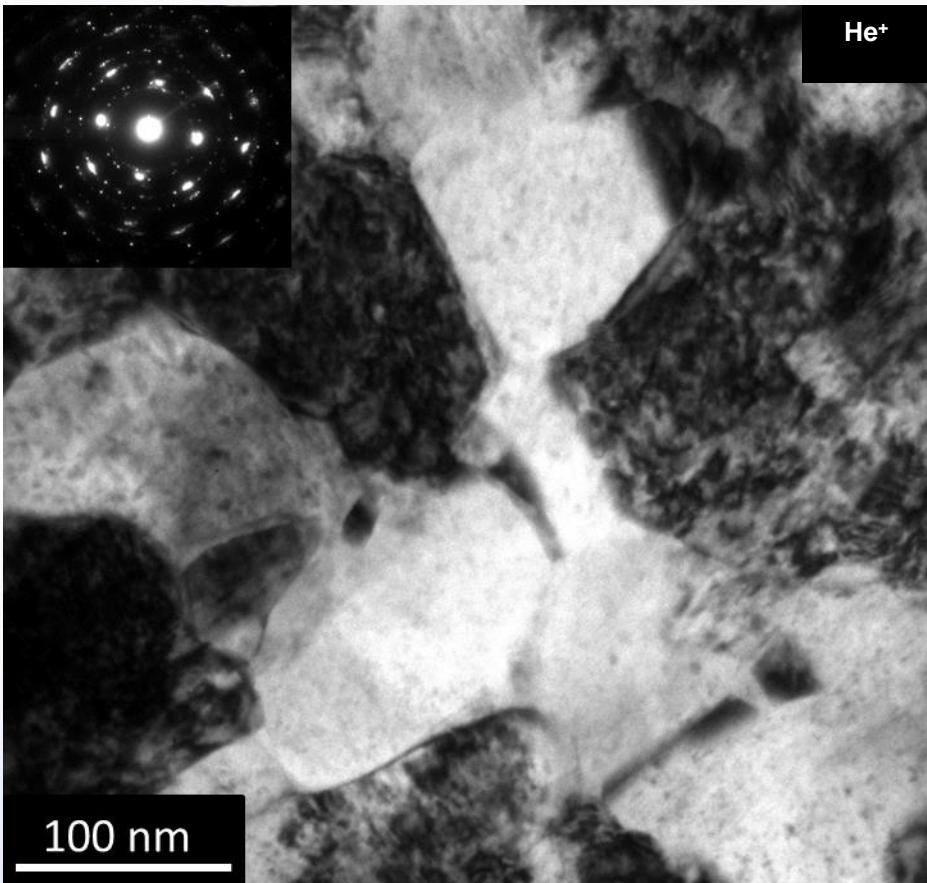
In-situ triple beam He, D₂, and Au beam irradiation has been demonstrated on Sandia's I³TEM! Intensive work is still needed to understand the defect structure evolution that has been observed.



Sandia National Laboratories

Heterogeneous Bubble Formation under Some Radiation Environments

Collaborator: B. Muntifering & J. Qu



10^{17} He⁺/cm²

Visible damage to the sample

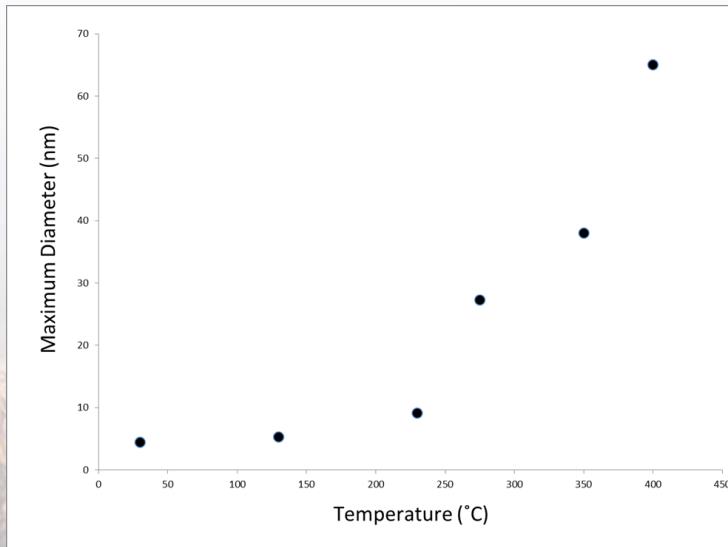
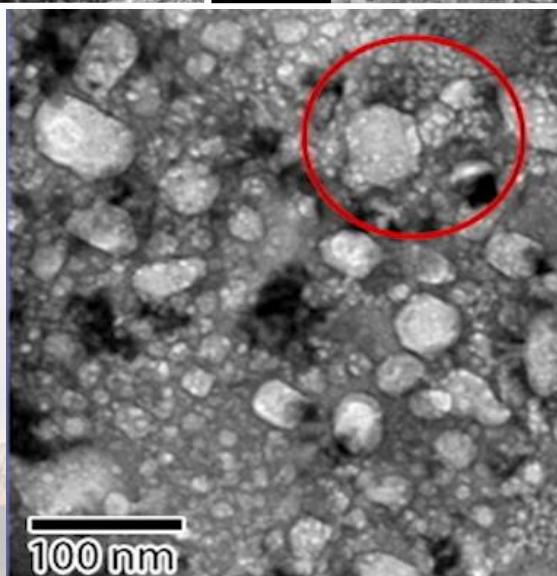
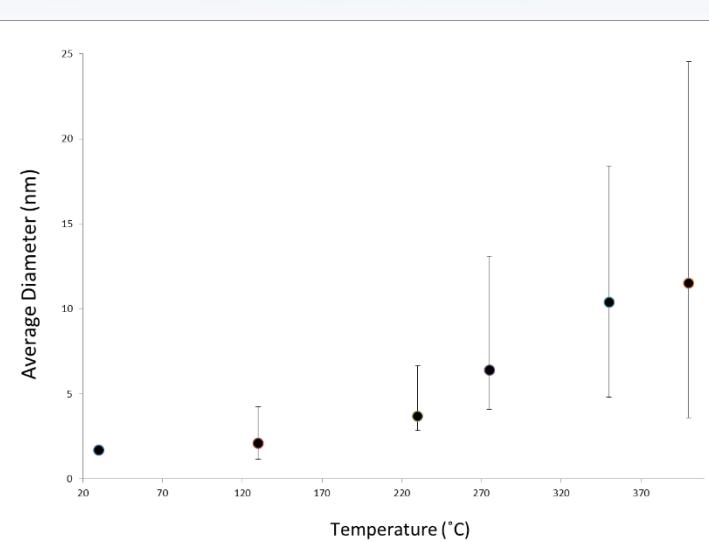
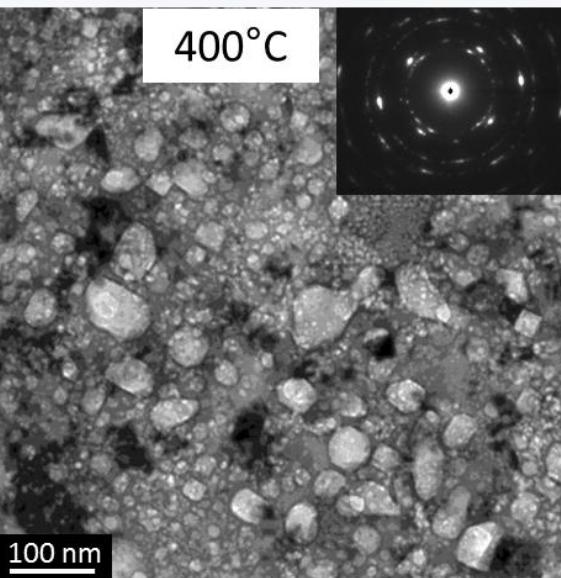
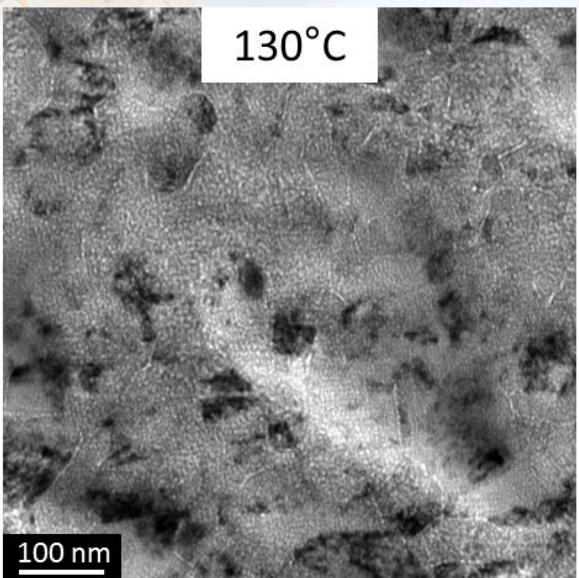
0.7 dpa Ni³⁺ irradiation

High concentration of cavities along grain boundaries



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Cavity Growth during In-situ Annealing of 10 keV He⁺ Implanted and then 3 MeV Irradiated Ni³⁺



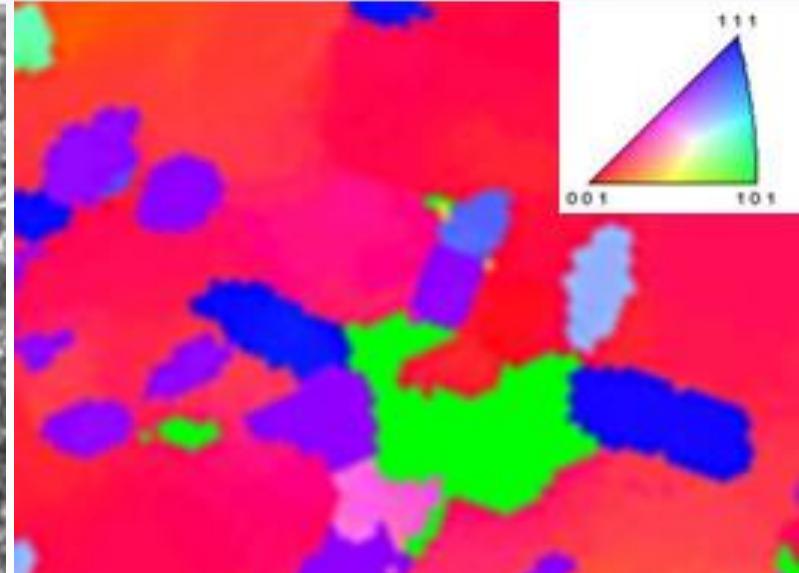
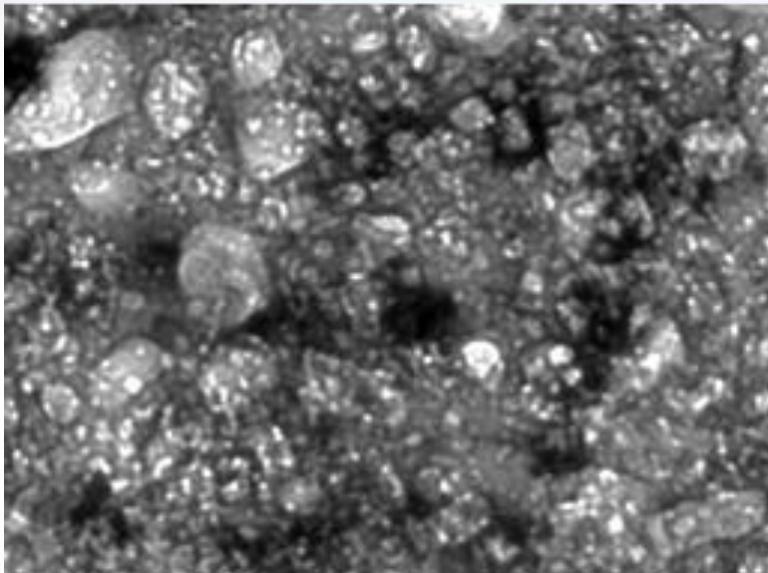
Bubble to cavity transition and cavity evolution can be directly studied



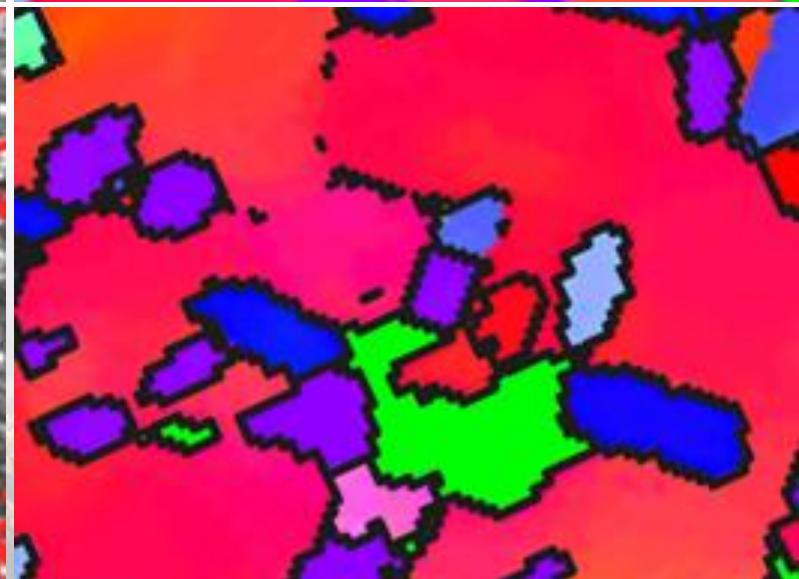
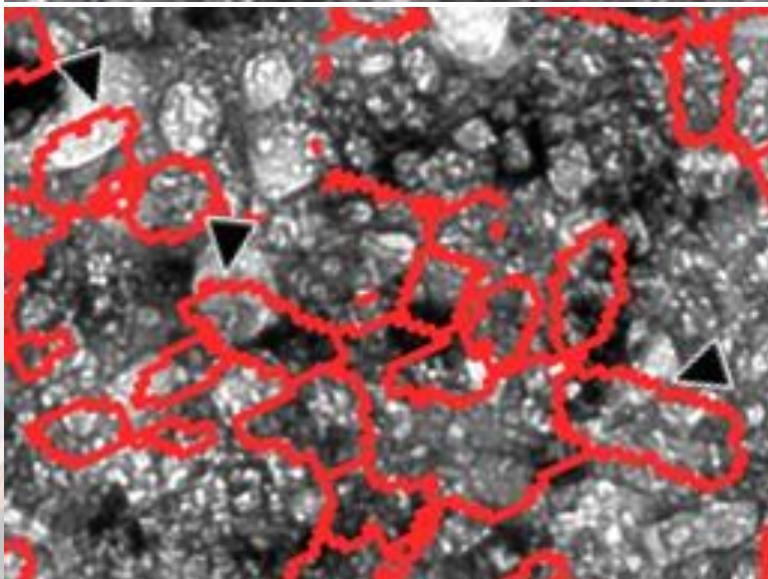


Precession Electron Diffraction Reveals Hidden Grain Structure

Cavities in helium implanted, self-ion irradiated, nc nickel film annealed to 400 °C



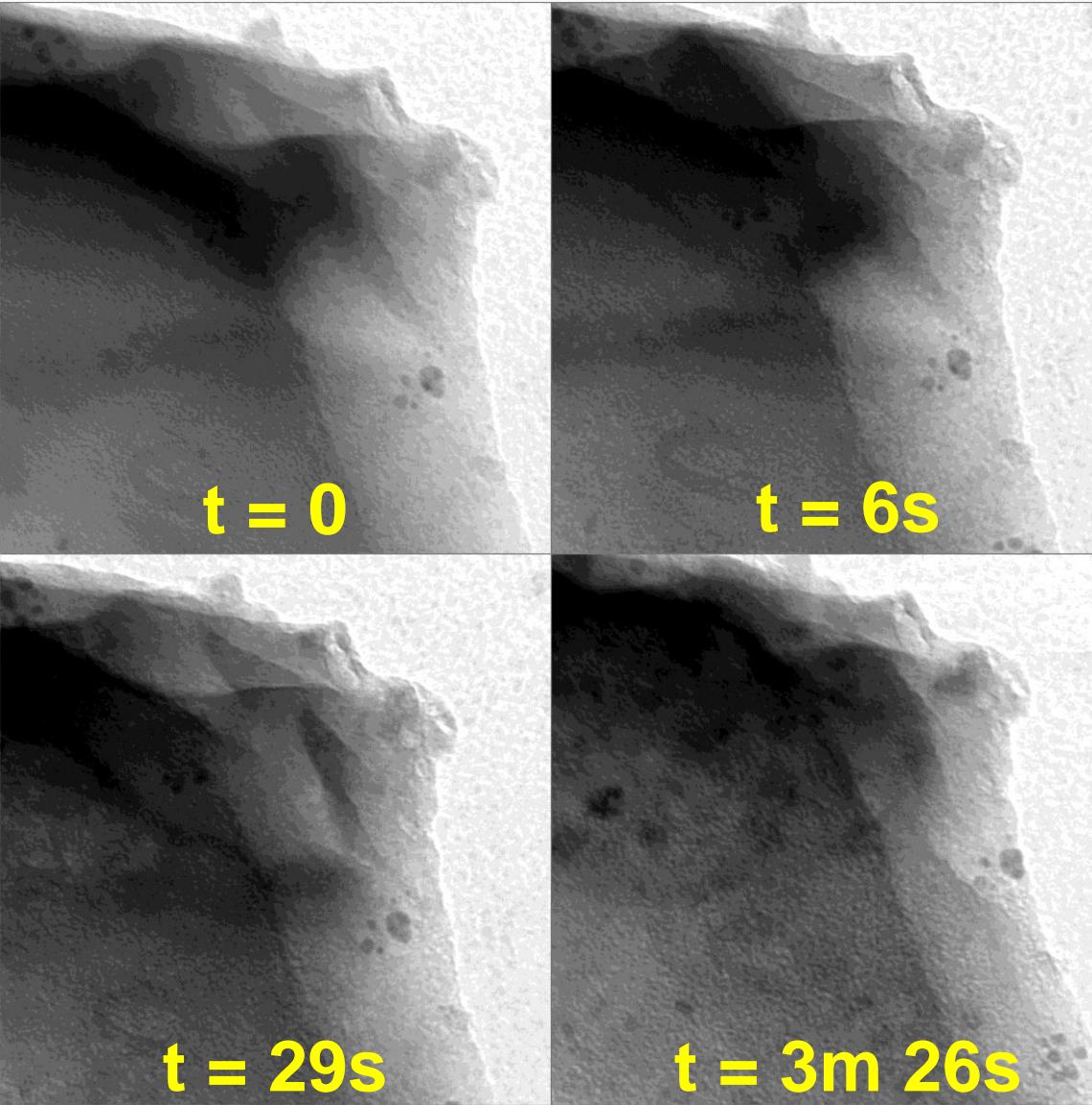
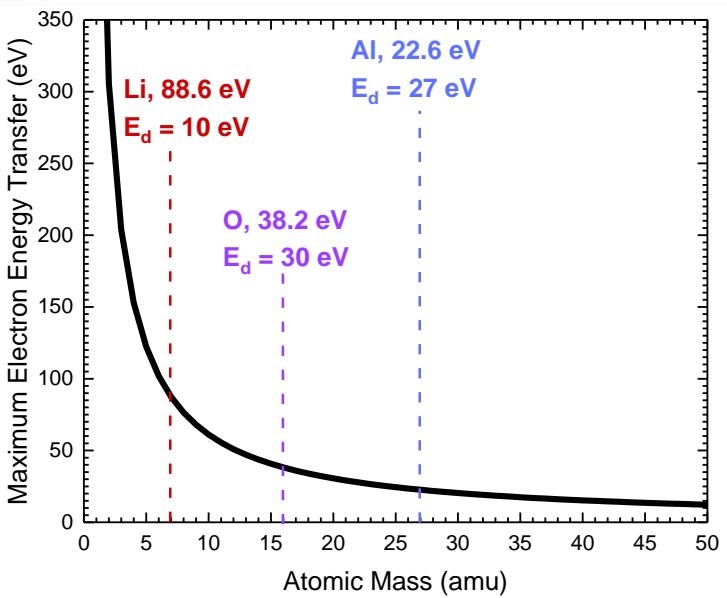
Cavities span multiple grains at identified grain boundaries



100 nm

Electron beam induced void growth

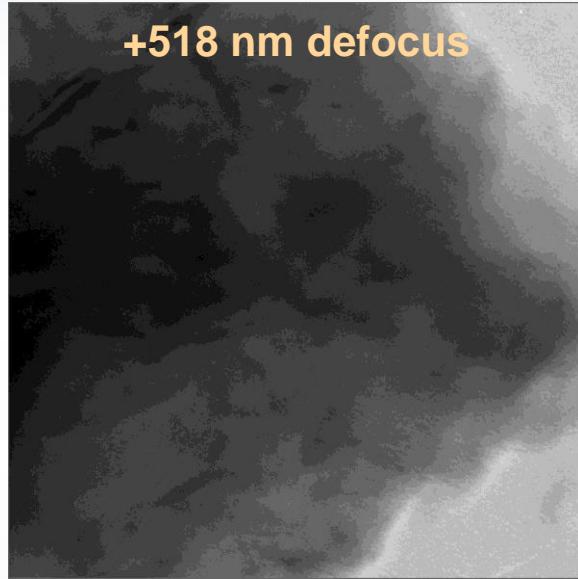
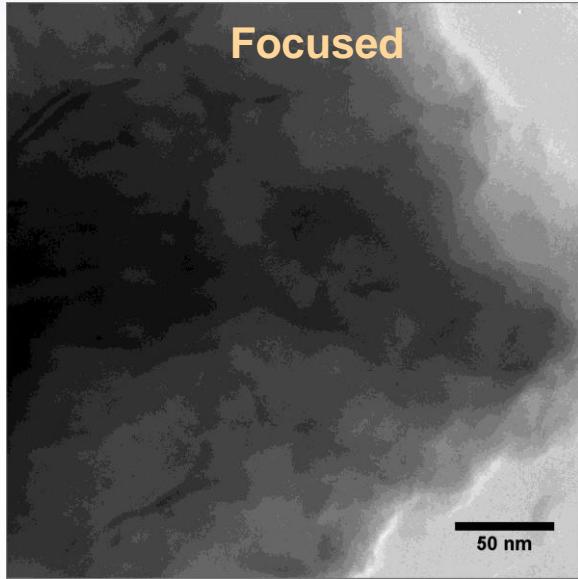
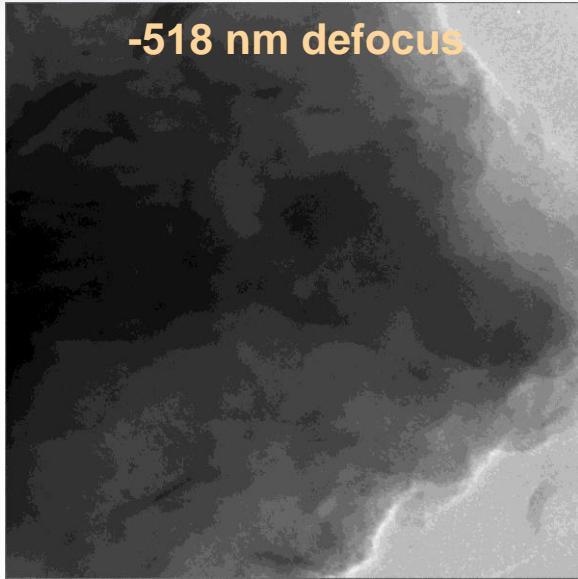
- Voids were observed to form under the electron beam in several particles
- Rate of void formation is not consistent between particles
- Possibly due to electron beam displacing Li and O atoms



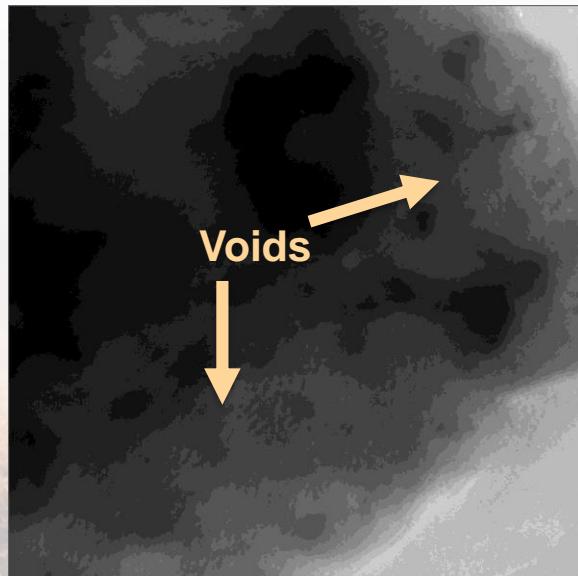
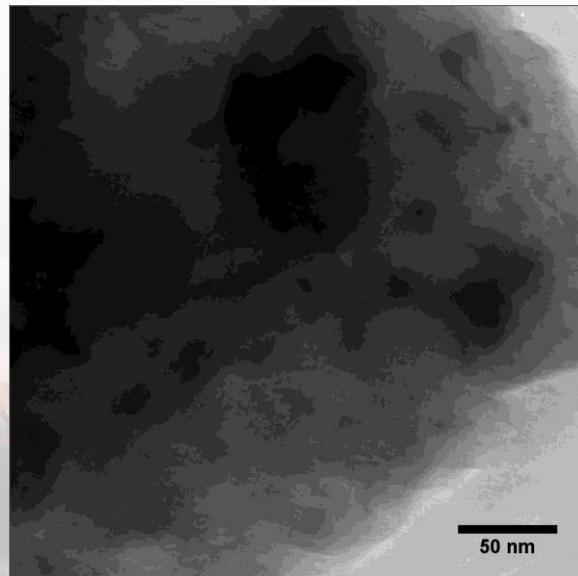
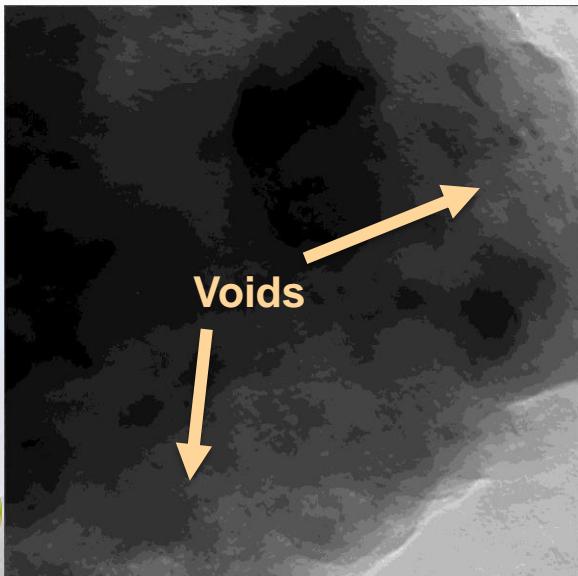
In-situ He implantation @ 310°C

Bubbles formed after ~13 min (1.5×10^{17} He/cm²)

Before



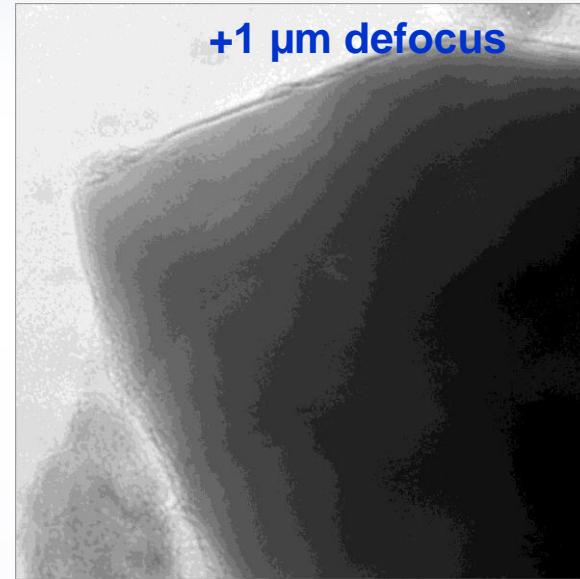
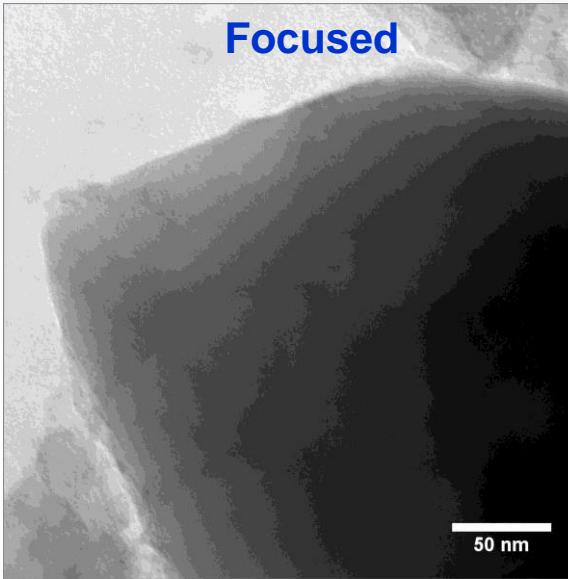
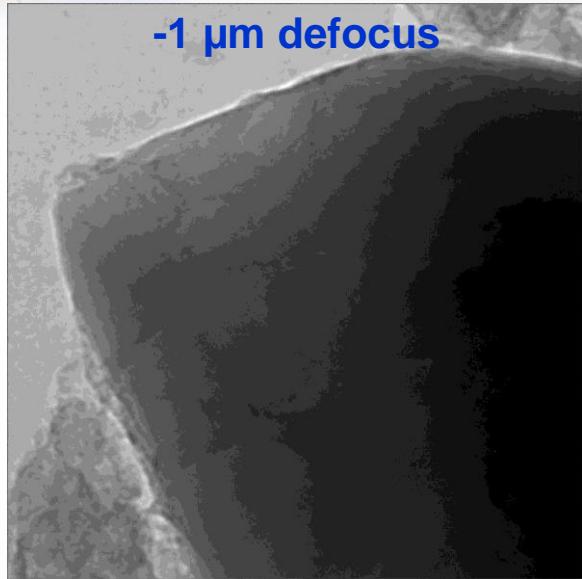
After



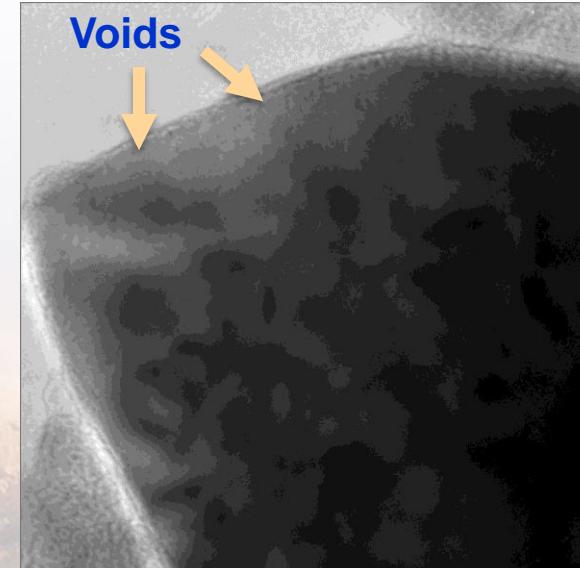
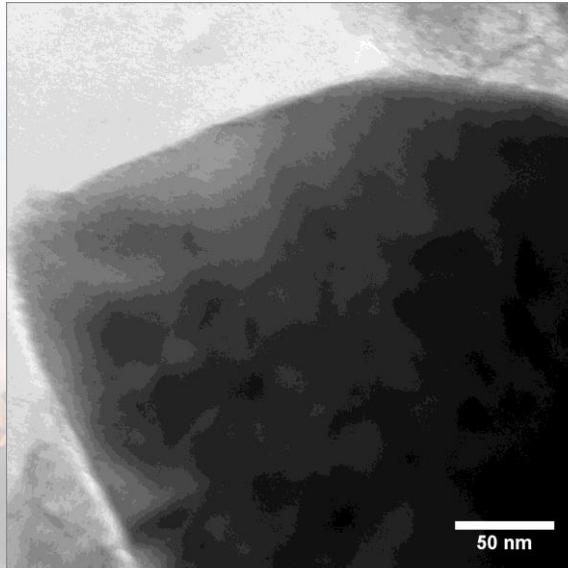
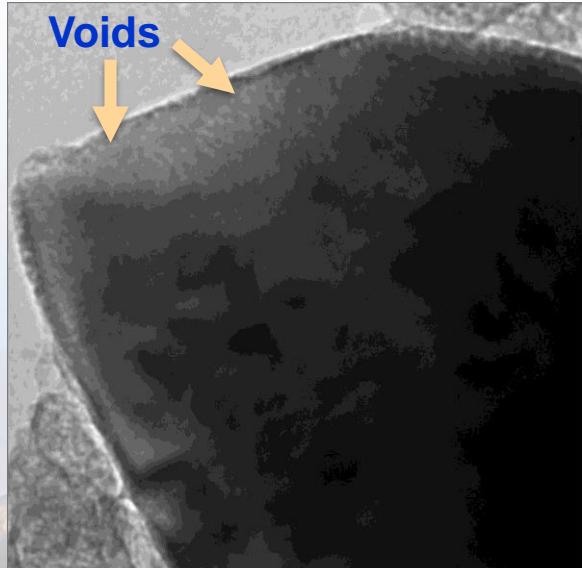
In-situ He + D irradiation @ 310°C

Bubbles formed after ~60 min (1.7×10^{17} He/cm², 3.4×10^{17} D/cm²)

Before



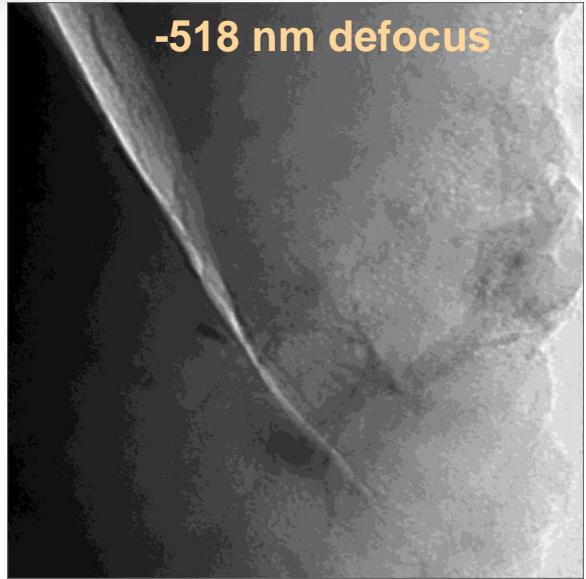
After



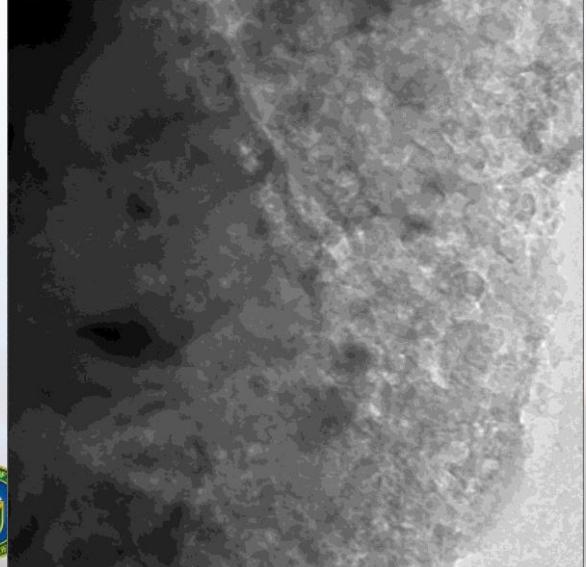
In-situ He + D + Au @ 310°C

No single eucentric: Drastic increase in surface roughness!

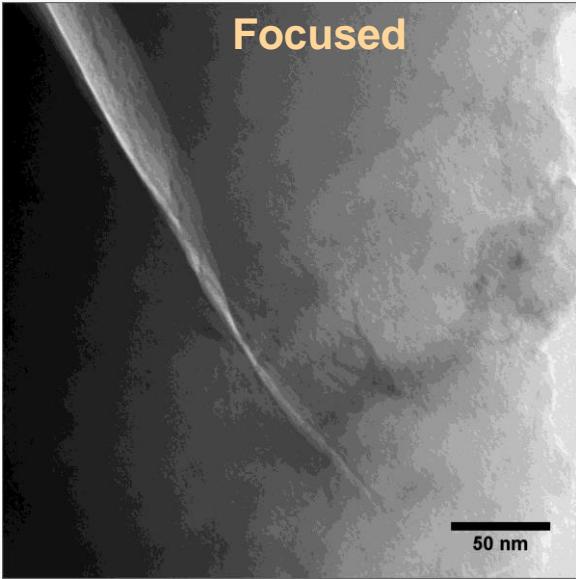
Before



After

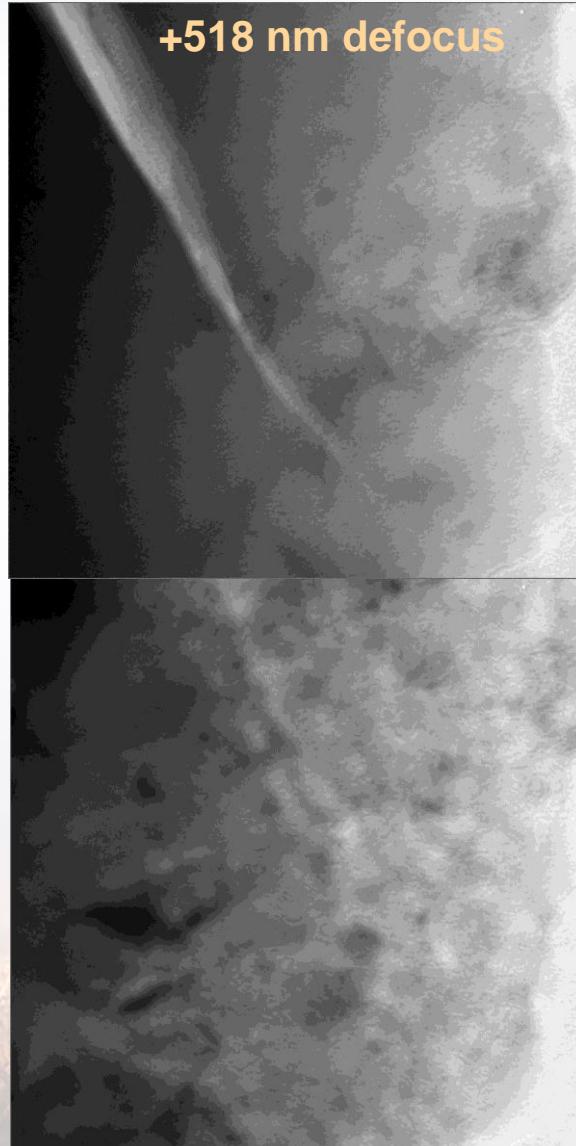


Focused



50 nm

+518 nm defocus



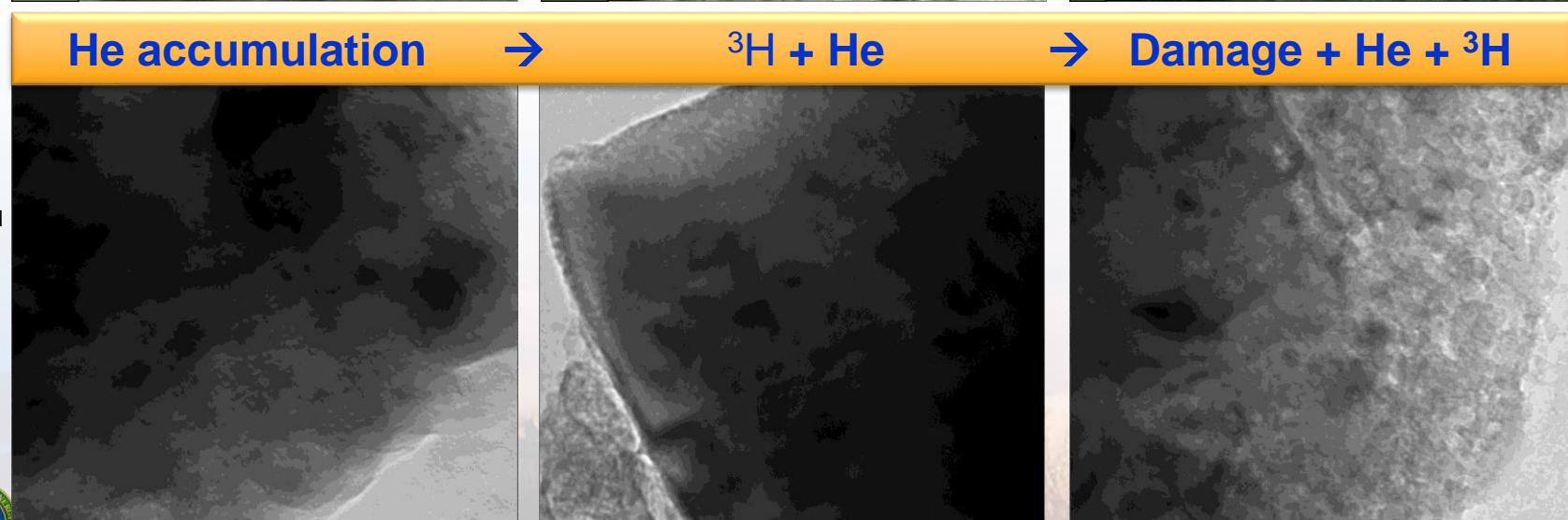
50 nm

Comparison between LiAlO_2 and Zr-4

Zr Alloys



LiAlO_2



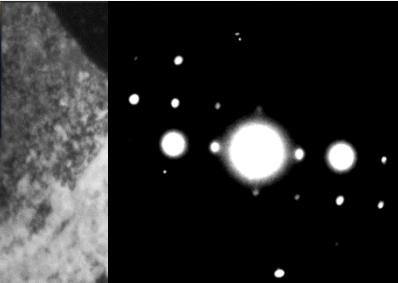
Sandia National Laboratories is a multi-mission laboratory managed and operated by National Technology & Engineering Solutions of Sandia, LLC., a wholly owned subsidiary of Honeywell International, Inc., for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-NA0003525.



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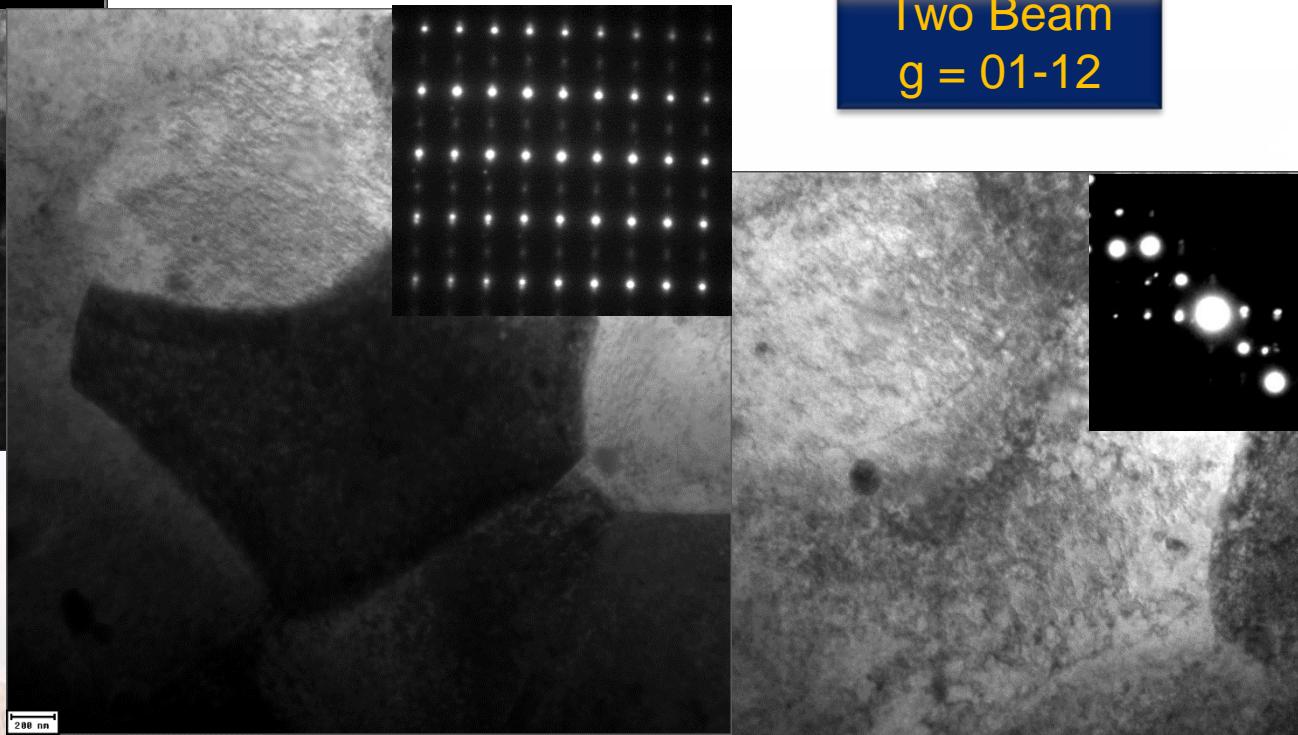
10 keV He⁺ Implantation Then 3 MeV Self Ion Irradiation into Zr-4 at 310 C

Two Beam
g = 0002



On Axis

Two Beam
g = 01-12

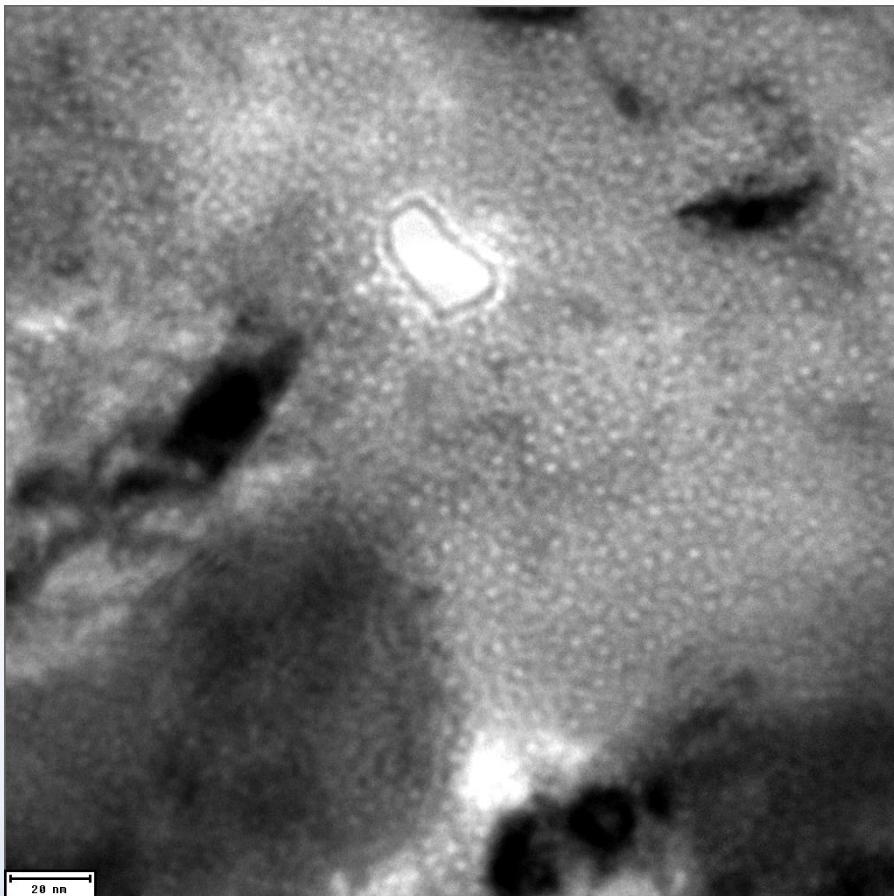


Sequence matters, as
well as imaging
condition!

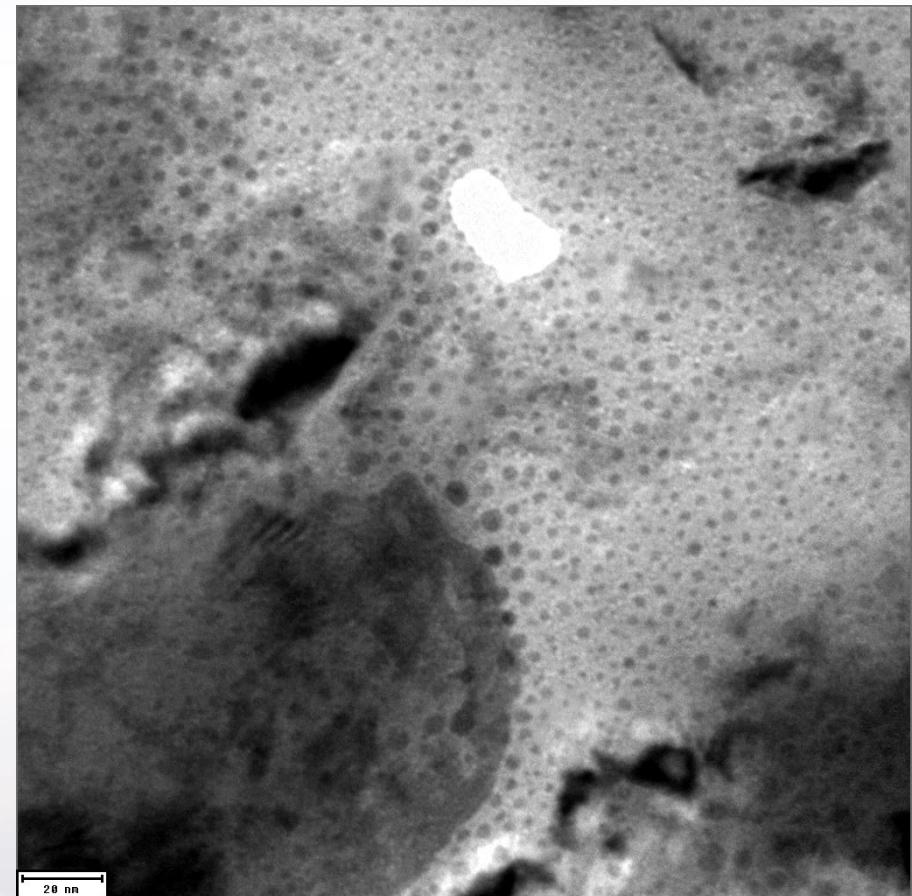




PIE: Through Focus Images 30 Days After the *In situ* Experiment



Under Focus



Over Focus

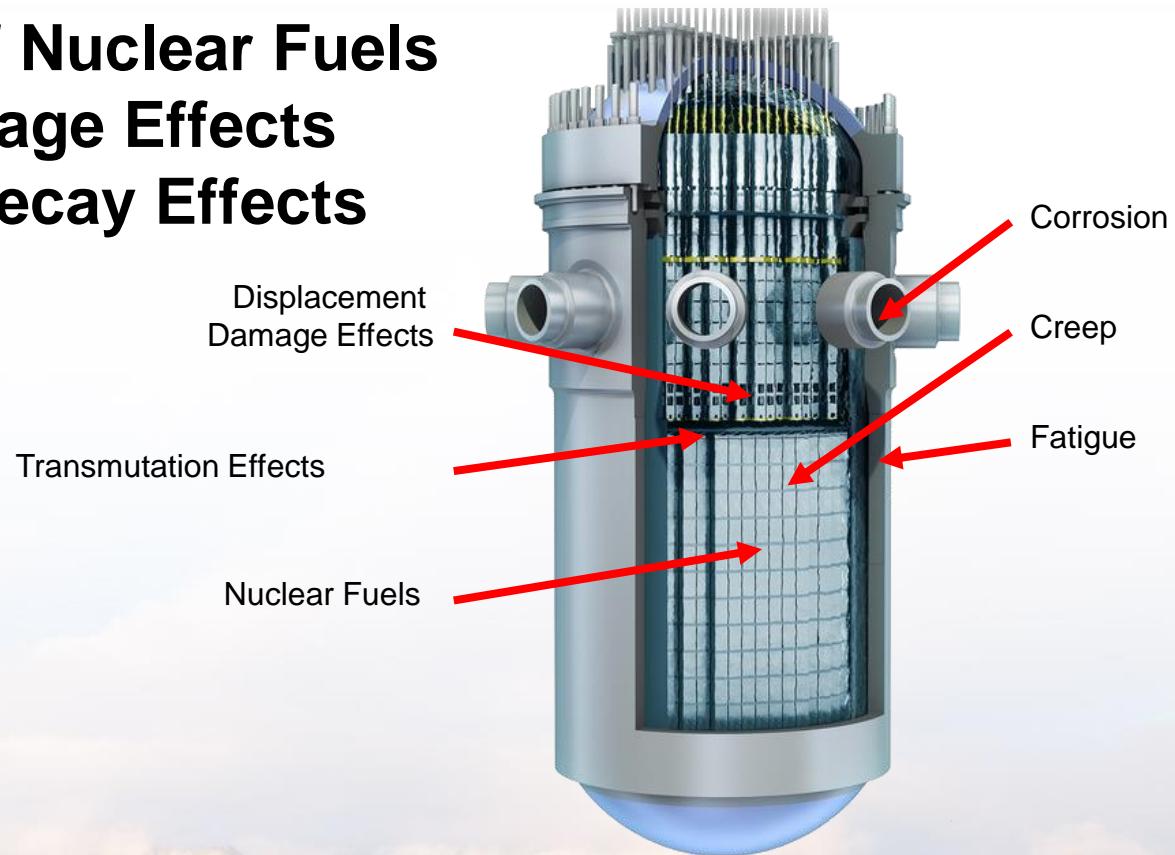
The end of accelerated aging experiment is not the end of the materials aging





Outline

- 1. Characterization of Nuclear Fuels**
- 2. Displacement Damage Effects**
- 3. Transmutation & Decay Effects**
- 4. Corrosion**
- 5. Creep**
- 6. Fatigue**



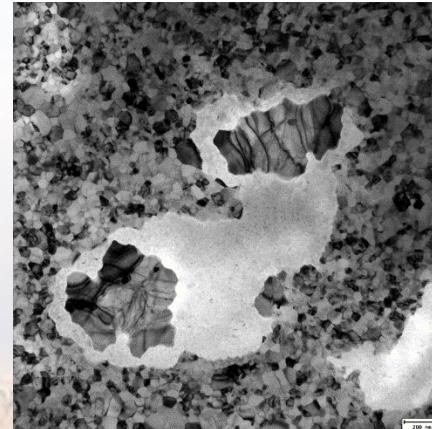
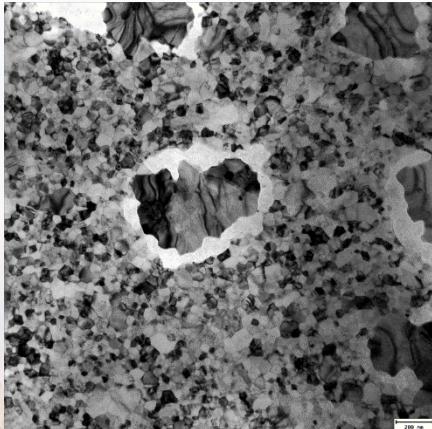
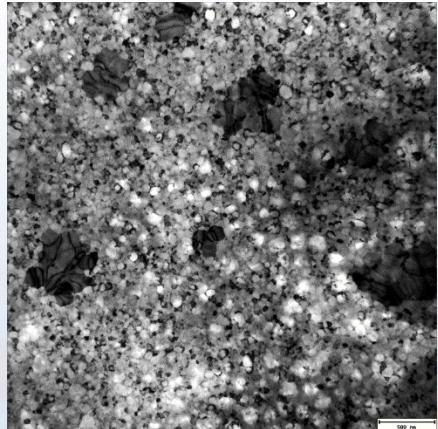
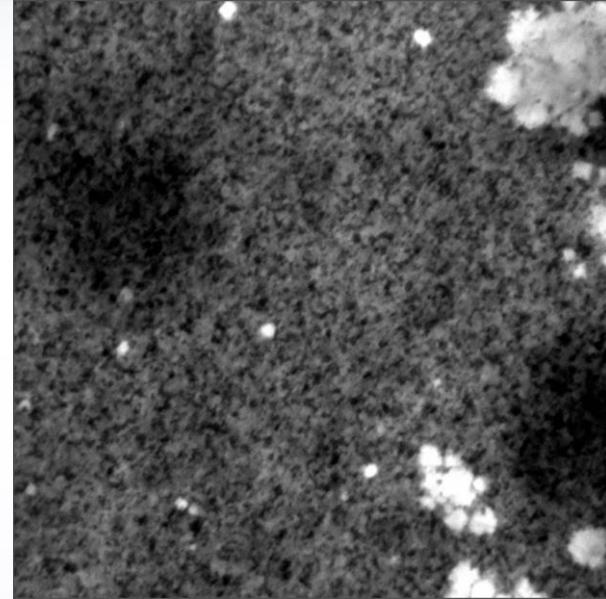
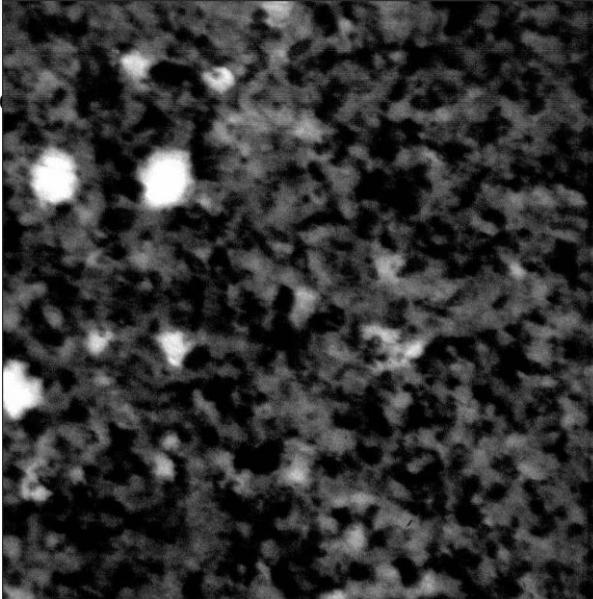
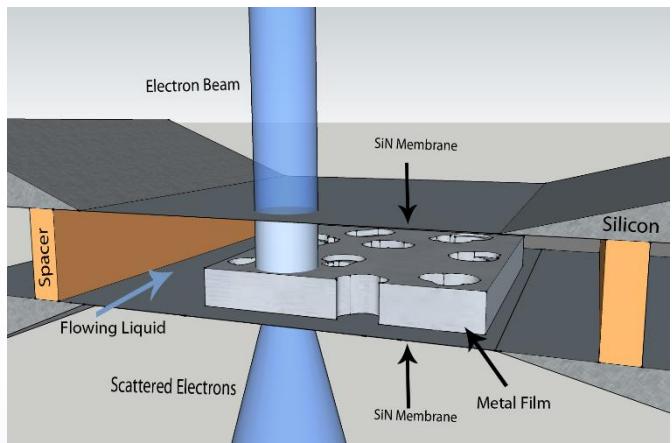
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Can We Gain Insight into the Corrosion Process through *In situ* TEM?

Contributors: D. Gross, J. Kacher, I.M. Robertson & Protochips, Inc.

Microfluidic Stage

- Mixing of two or more channels
- Continuous observation of the reaction channel



Pitting mechanisms during dilute flow of acetic acid over 99.95% nc-PLD Fe involves many grains. Large grains resulting from annealing appear more corrosion tolerant

Feasibility of Studying Zircaloy 2 at Nominally 1 atm

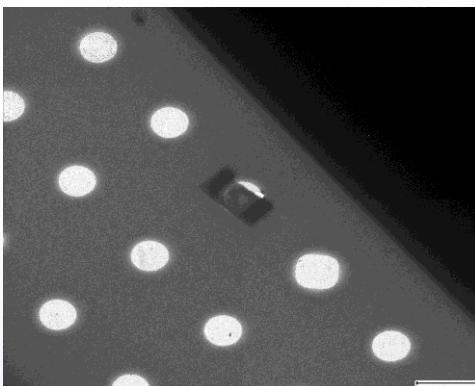
Collaborators: S. Rajasekhara and B.G. Clark



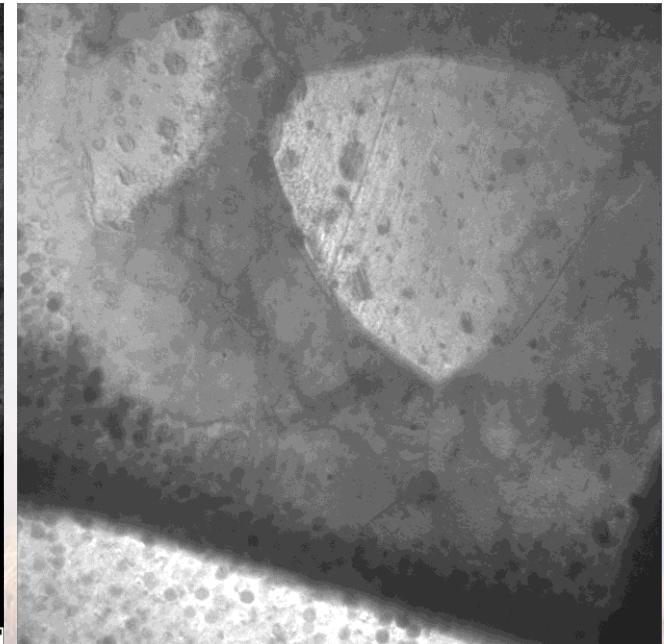
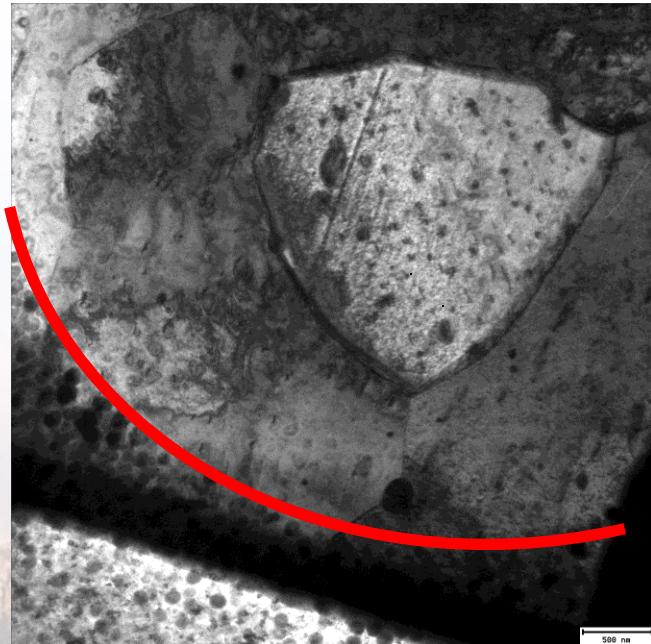
Vapor-Phase Heating TEM Stage

- Compatible with a range of gases
- *In situ* resistive heating
- Continuous observation of the reaction channel
- Chamber dimensions are controllable
- Compatible with MS and other analytical tools

Vacuum & Single Window



Nominally 1 atm H₂ & Two Windows

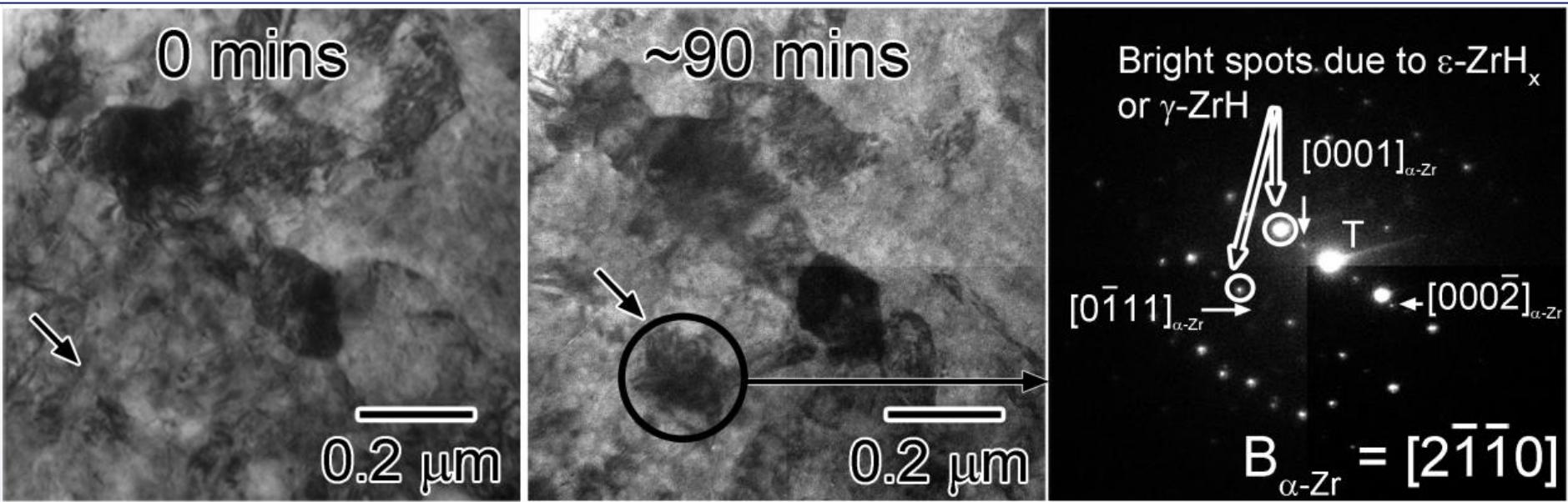


Most features are observed in both despite the decreased resolution resulting from the additional SiN window and 5 μm of air

In situ Observation of Hydride Formation in Zirlo

Collaborators: S. Rajasekhara and B.G. Clark

Absolute hydrogen pressure: 327 torr (~ 0.5 atm),
Ramp rate: 1 °C/s, Final temperature: ~ 400 C, Dwell time: ~ 90 mins



Hydride formation shown, for the first time by use of a novel TEM gas-cell stage, at elevated temperature and hydrogen pressure

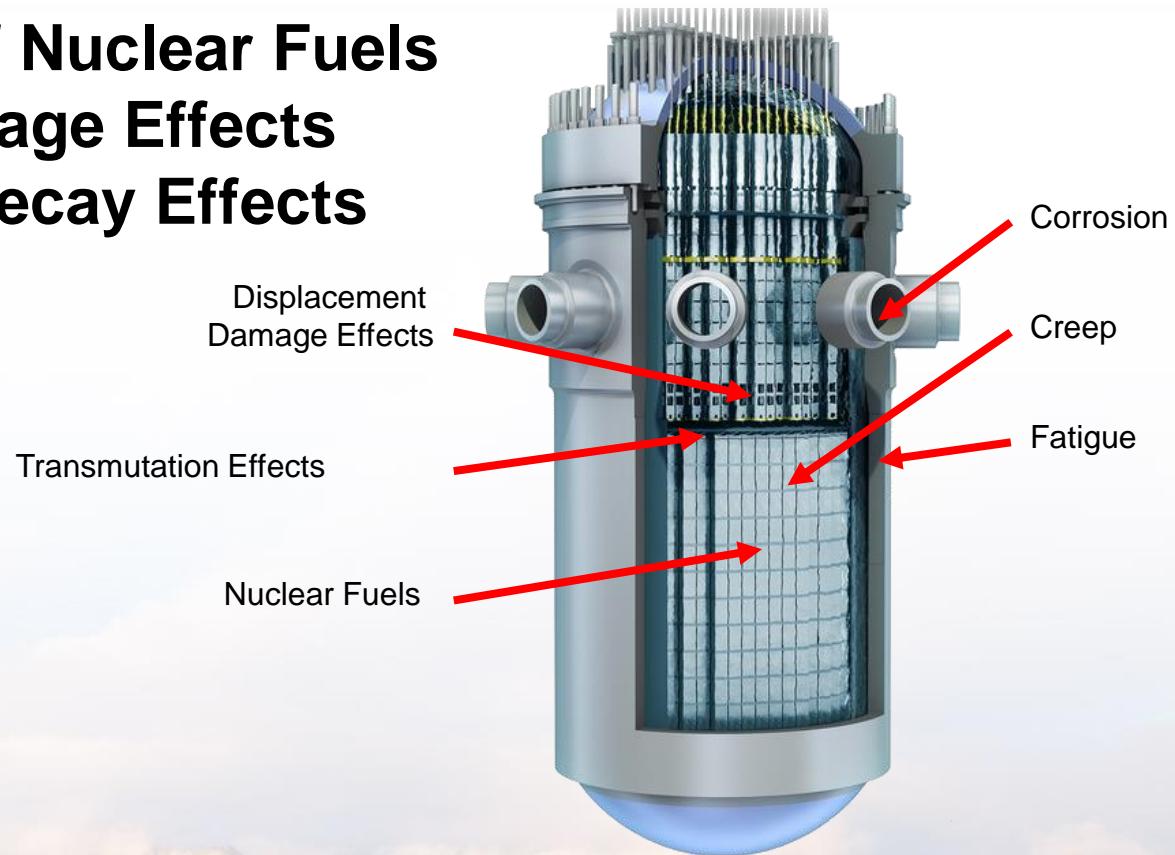


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Outline

1. Characterization of Nuclear Fuels
2. Displacement Damage Effects
3. Transmutation & Decay Effects
4. Corrosion
5. Creep
6. Fatigue



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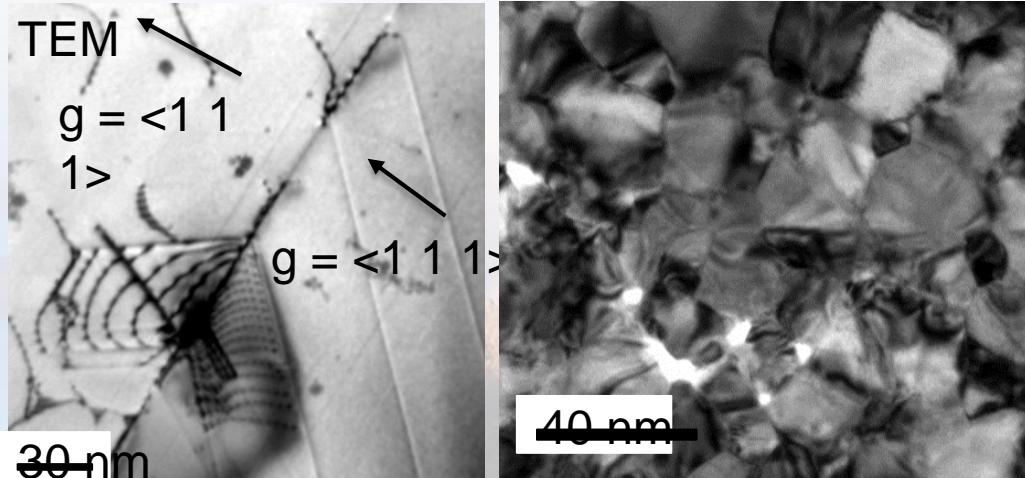
In situ Mechanical Testing

Qualitative “Bulk” Mechanical Testing

Minimal control over displacement and no “out-of-box” force information

- Successful in studies in observing dislocation-GB interactions/mechanisms
- Ideally both grains have kinematic BF 2-beam conditions: challenging in ST holder

Traditional Gatan Heating and Straining Holder



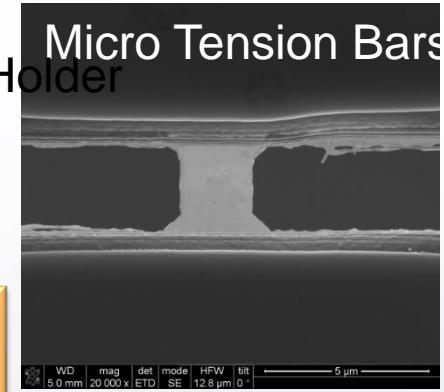
Quantitative Mechanical Testing

Minimal control over displacement and no “out-of-box” force information

- Sub nanometer displacement resolution
- Quantitative force information with μN resolution

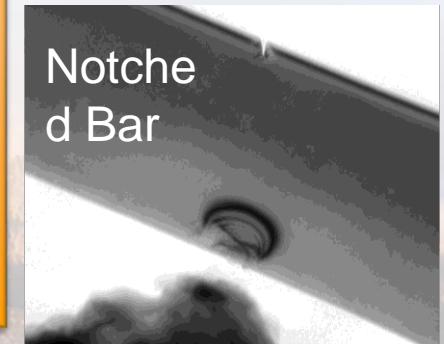


Hysitron PI-95 Holder



Micro Tension Bars Holder

- 1) Indentation
- 2) Tension
- 3) Fatigue
- 4) Creep
- 5) Compression
- 6) Bend

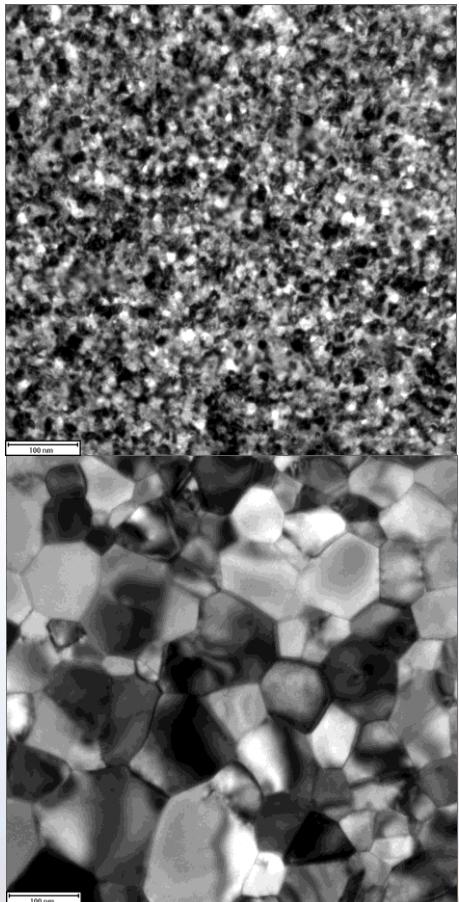


Notched Bar

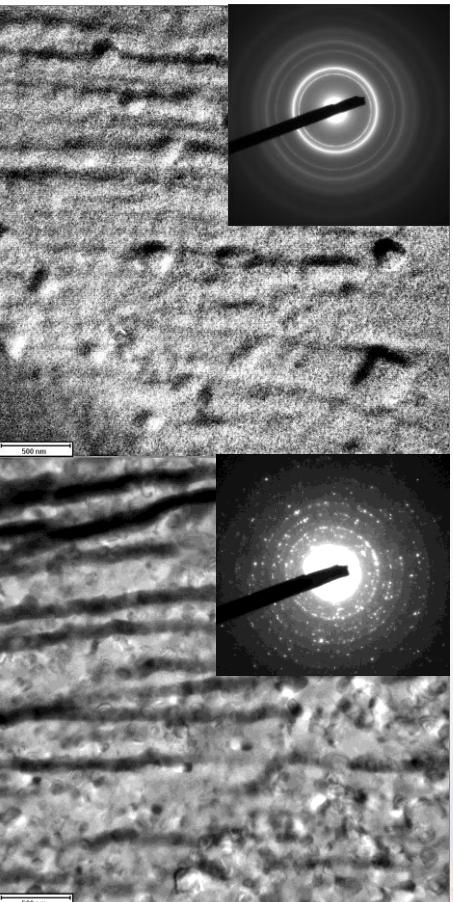
Laser Initiation of Multilayer Reactive Foils

Collaborator: P. Price, C.M. Barr, D. Adams, M. Abere

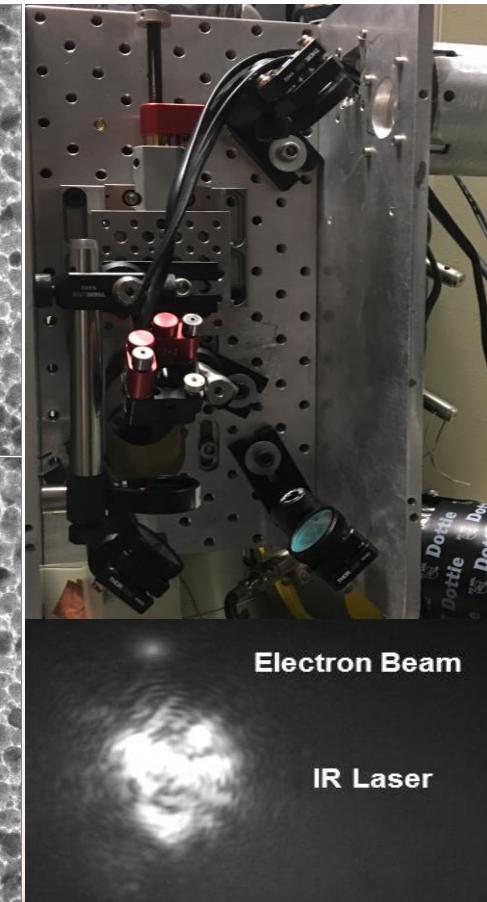
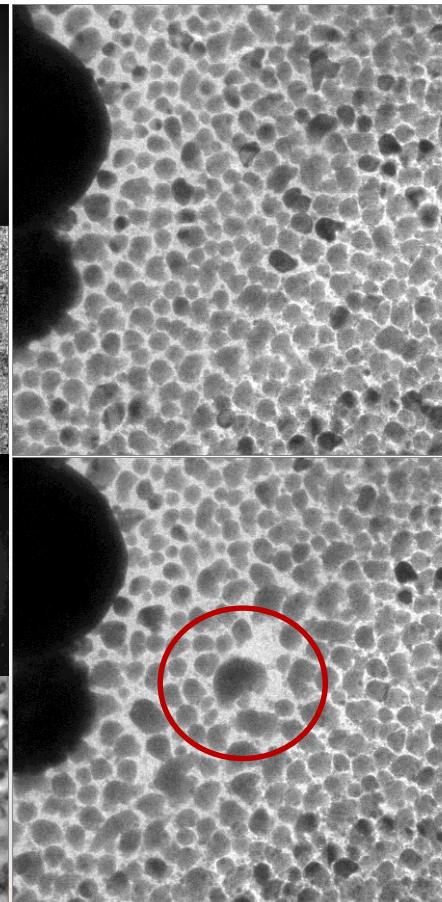
Pt Grain Growth



Reactive Multilayer Films



Nanoparticle Sintering

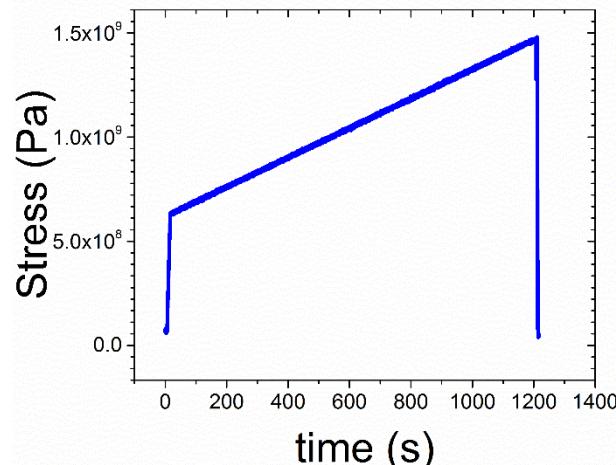


Understanding materials in extreme thermal environments as well as additive manufacturing processes

Irradiation Creep (4 MeV Cu³⁺ 10⁻² DPA/s)

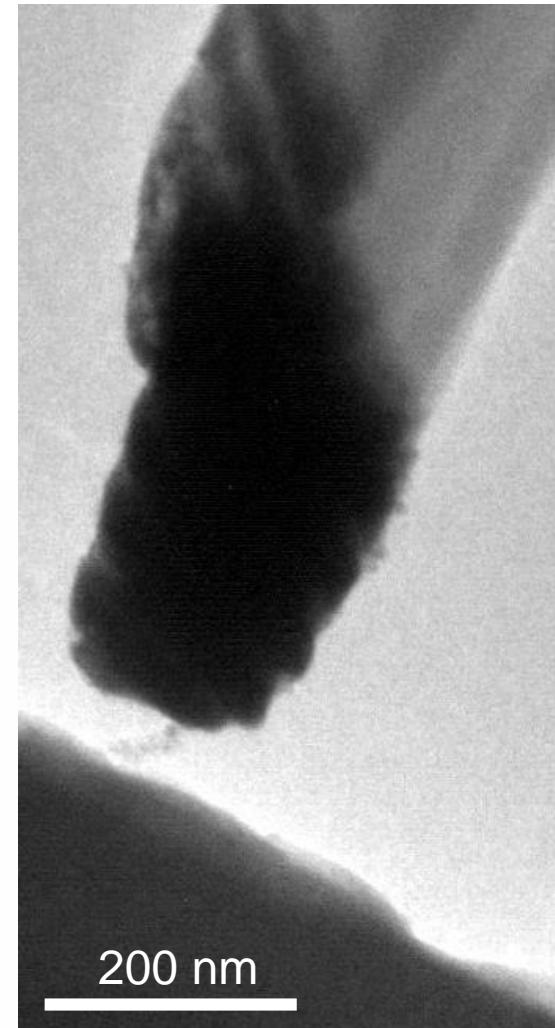
Contributors: S. Dillon & R.S. Averback

Controlled Loading Rate Experiments

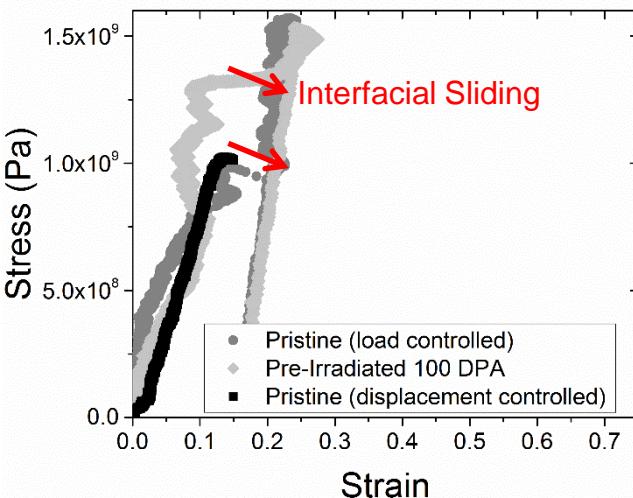


In-situ TEM
radiation
creep is
feasible!

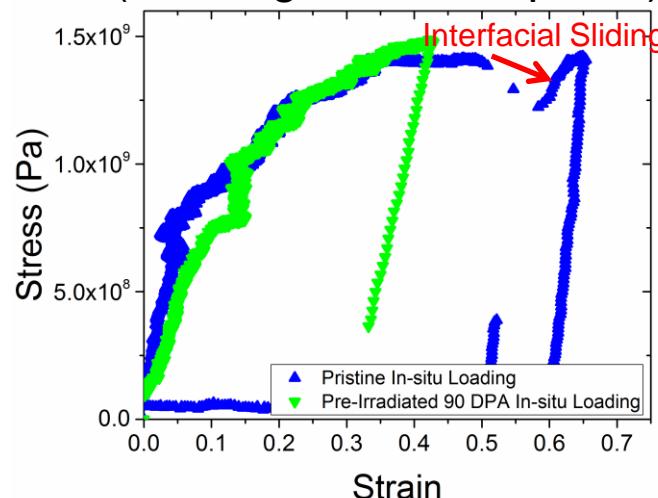
50 nm Cu-W multilayer
20 Min



No Irradiation
(Loading rate 0.6 Mpa s⁻¹)

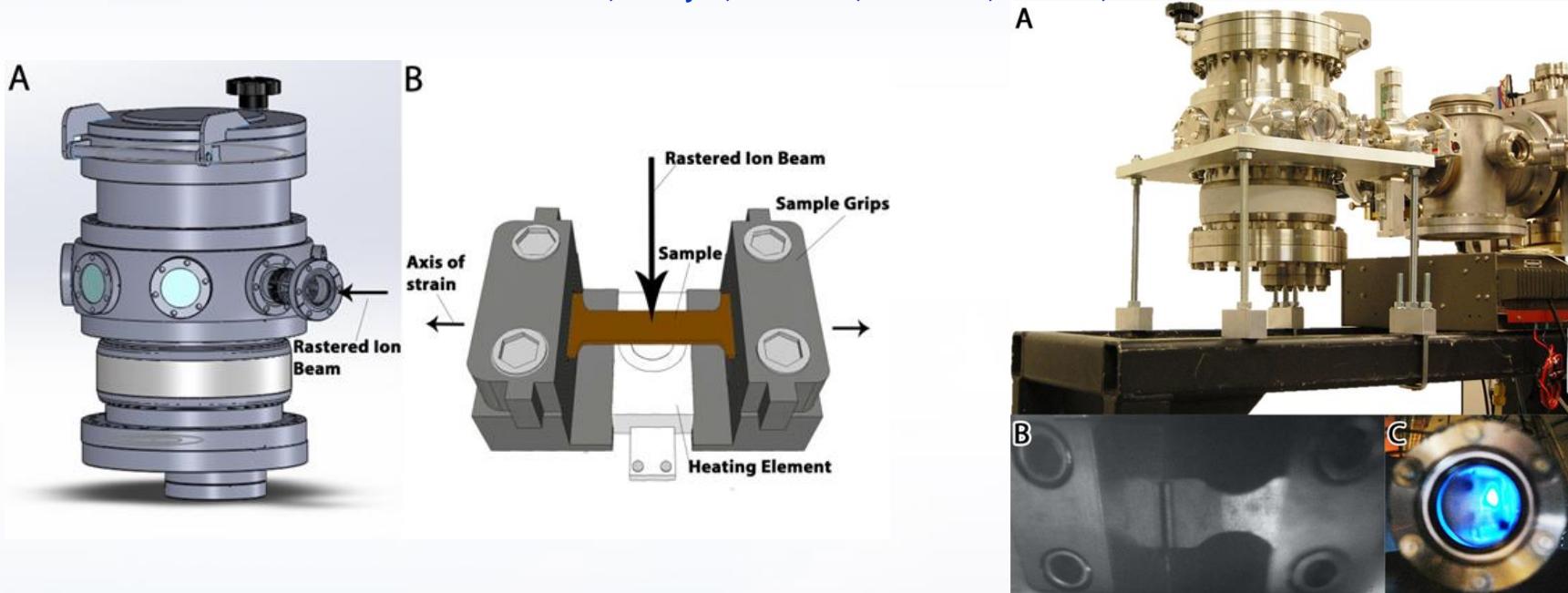


Irradiation Creep
(Loading rate 0.6 Mpa s⁻¹)



Mechanical Testing End Station

Collaborator: M. Steckbeck, B. Boyce, T. Furnish, D. Bufford, D. Buller, C. Barr



End-station developed consists of a two possible micro-mechanical test frames situated to receive a variety of ion species at energies up to 88 MeV from the 6 MV Van de Graff Tandem accelerator:

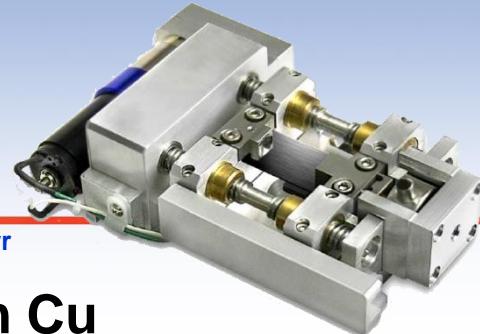
1. Commercial MTI/Fullham Multi-use Mechanical Stage (4000 N max): tensile, three point bend, creep, stress relaxation, others
2. Fatigue (custom design) for thin (5 to 10 micron samples)



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Irradiation and Stress Relaxation

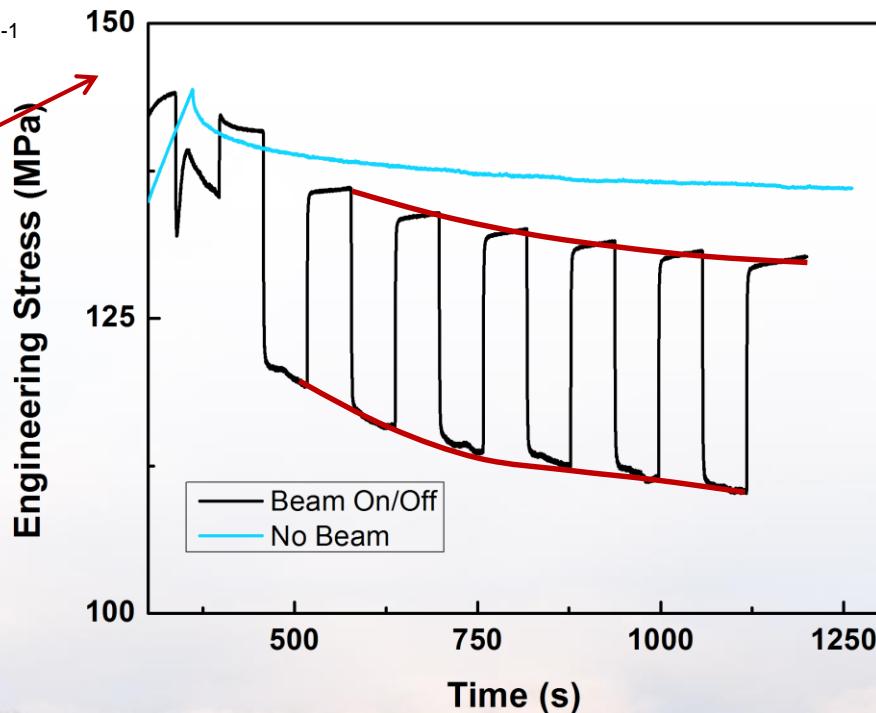
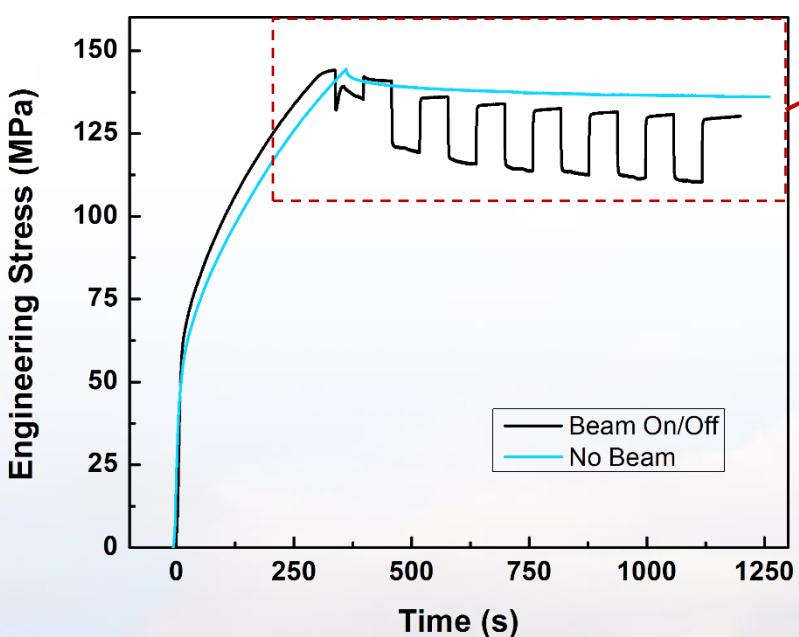
Collaborator: M. Steckbeck, B. Boyce, T. Furnish, D. Bufford, D. Buller, C. Barr



■ 0.25mm/min elongation rate to 22.5 N load in 50 um Cu

- Approximately 75% of typical ultimate tensile load
- 900 s hold at constant position
- Beam cycled on/off at 60 s intervals
- Beam conditions: 4.5 MeV H⁺: 2.1×10^{11} ions cm⁻²s⁻¹

MTI/Fullham SEMTester



- Offset likely an artifact
- Different relaxation rates with beam on and off

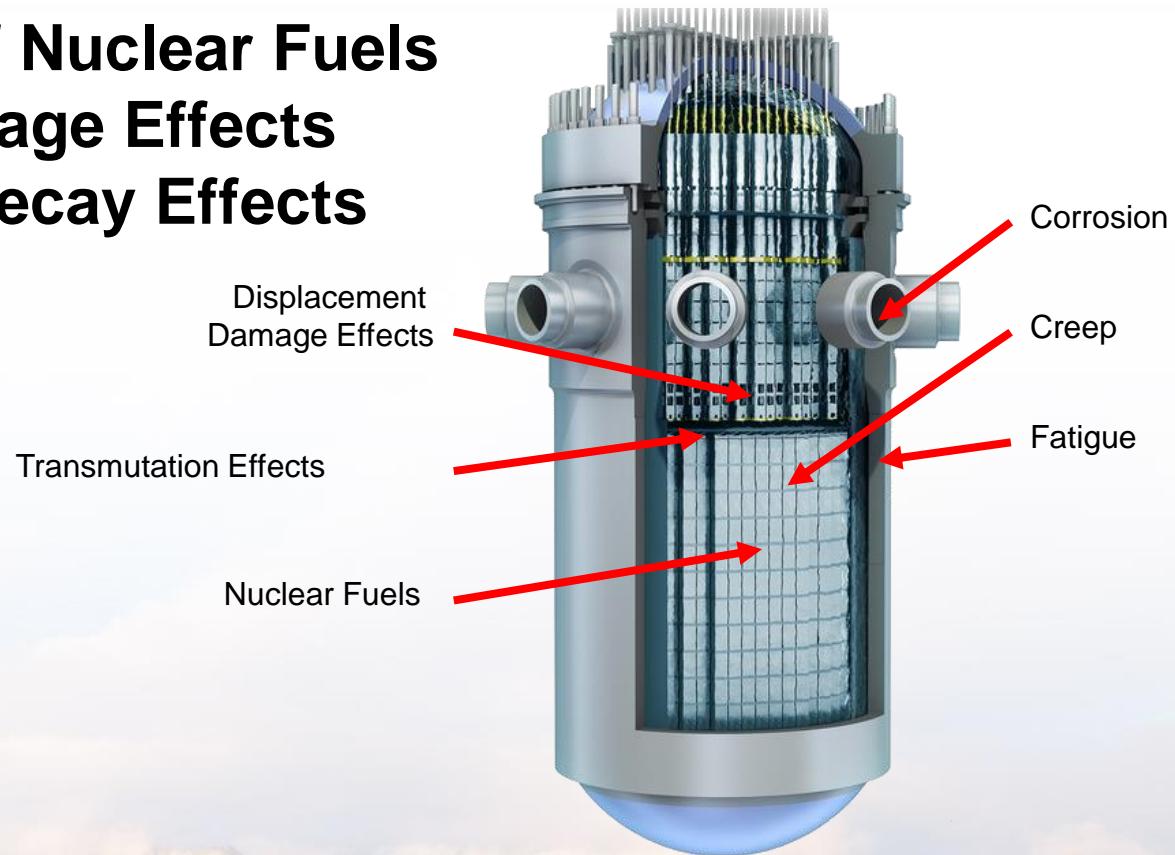


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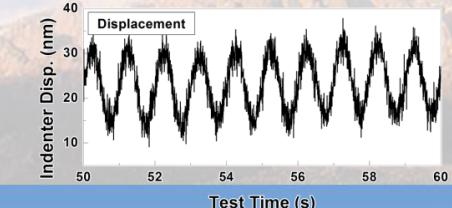
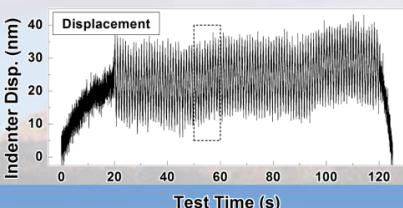
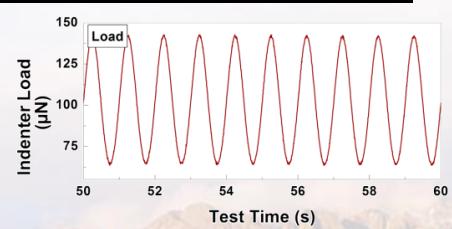
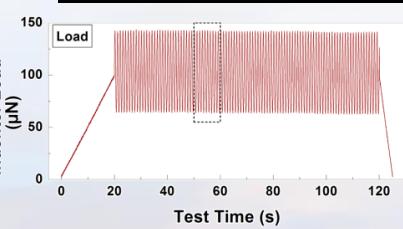


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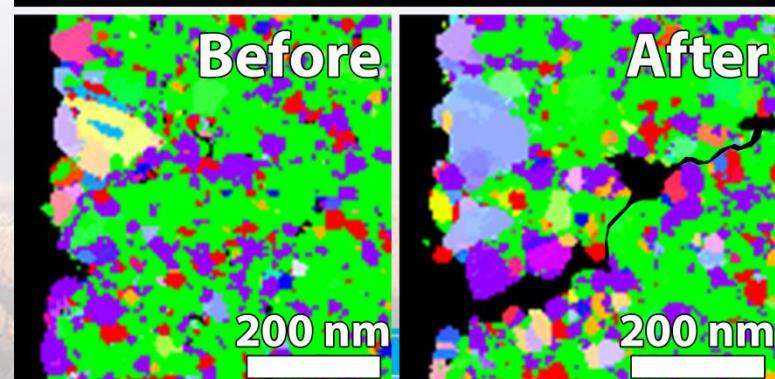
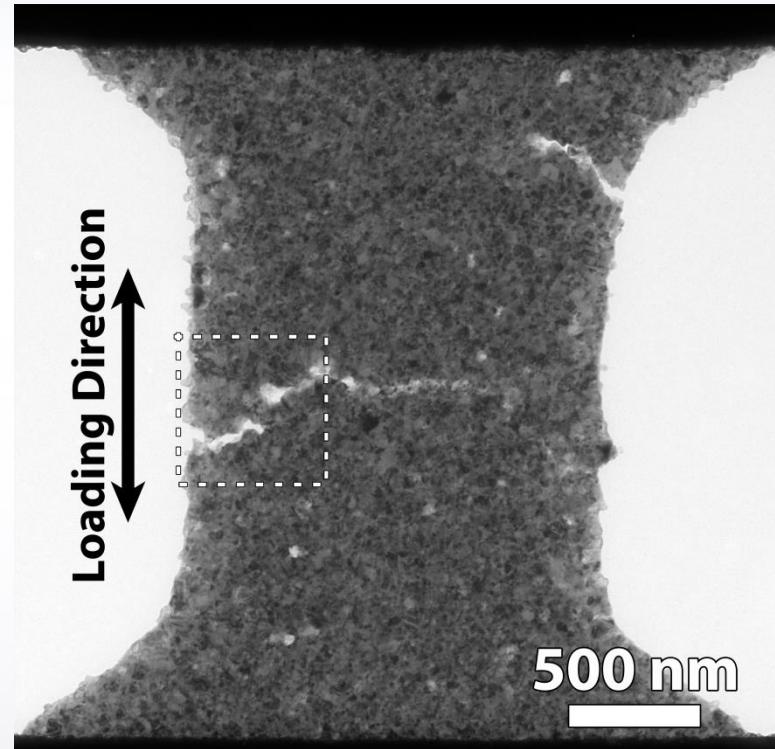
In situ TEM Quantitative Fatigue Testing

Contributors: D.C. Bufford, D. Stauffer, W. Mook

3x playback speed

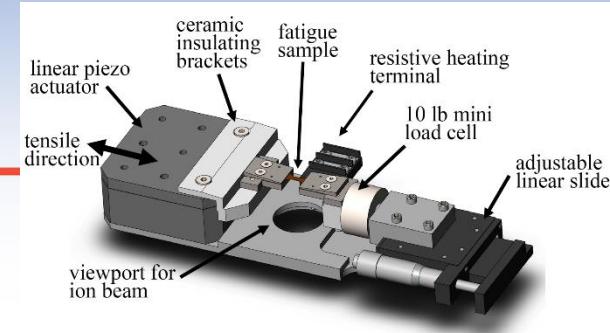


High cycle fatigue in real time with nanometer resolution

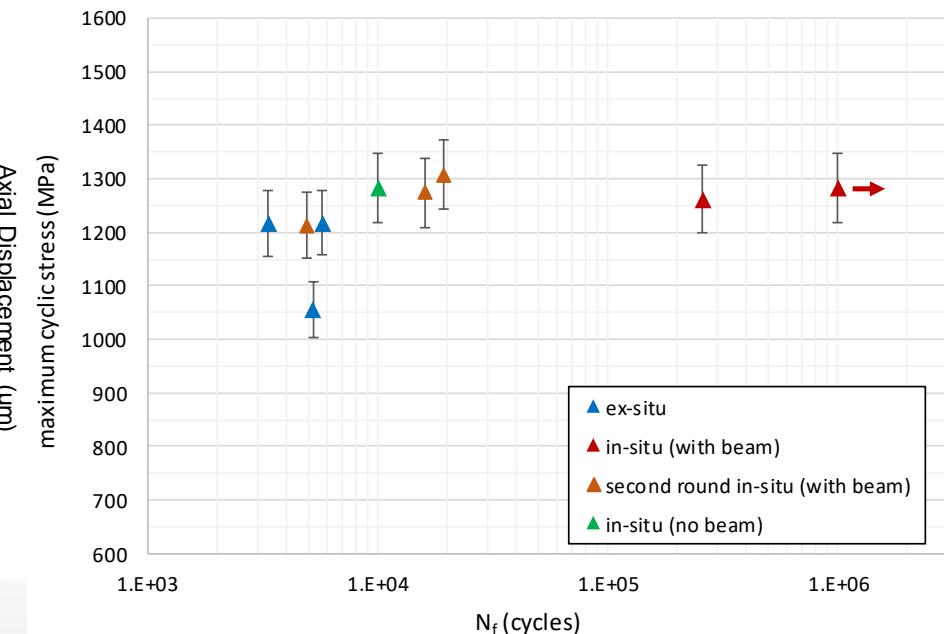
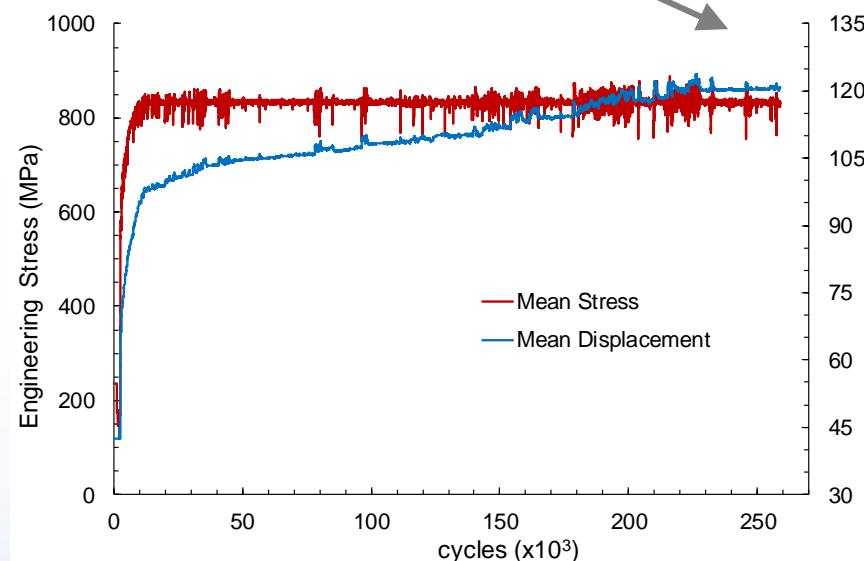


Irradiation and Fatigue

Collaborator: B. Boyce, T. Furnish, D. Buller



Fatigue Failure after 259k cycles



- Preliminary fatigue results in Ni-Fe alloy with both irradiation beam on sample (10 MeV He⁺) and beam off sample.
- Custom mechanical stage designed for thin samples (5 to 10 μ m thickness: suitable for allowing entire thickness to be irradiated with raster or defocused beam)

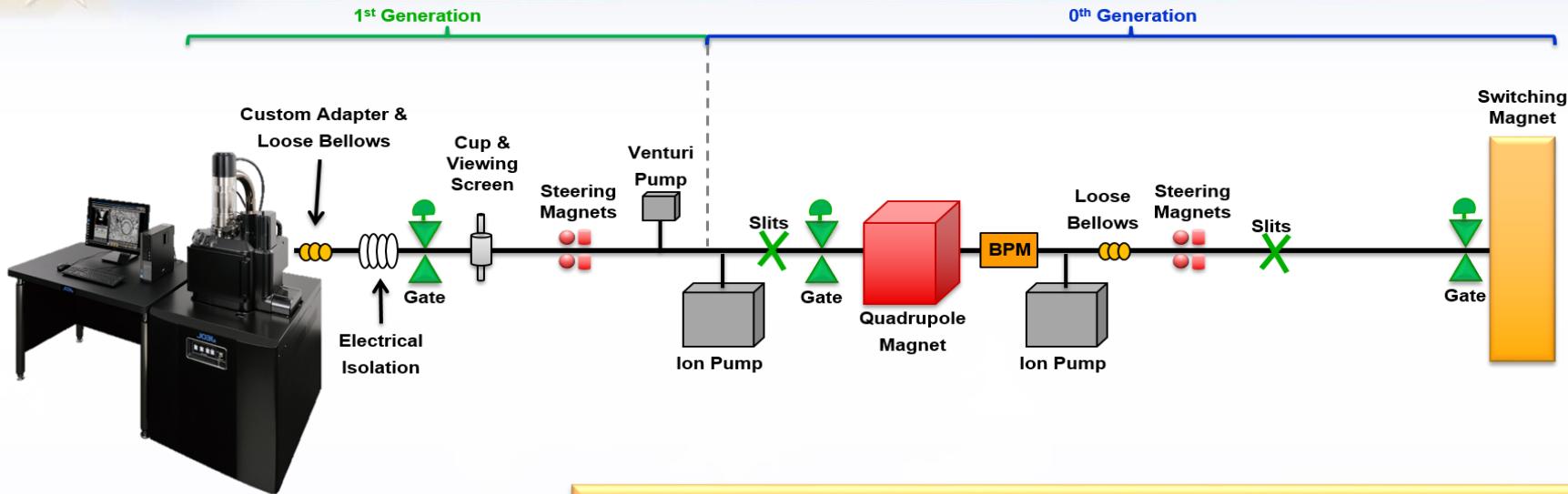


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Schematic of the *In situ* SEM Beamlne

Collaborators: D.L. Buller & S. Briggs

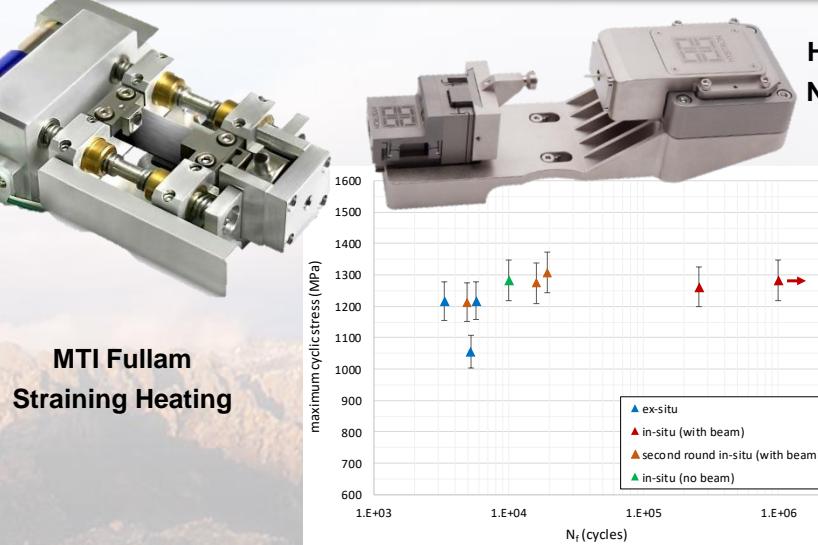
8/24/2017



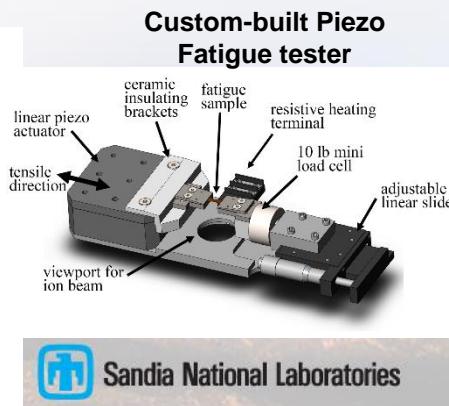
Beam Line planned for the *in situ* SEM will be developed in phases. Ultimate plan is for multiple accelerators being attached for dual or triple beam experiments.



MTI Fullam
Straining Heating



Hysitron PI85
Nanoindenter

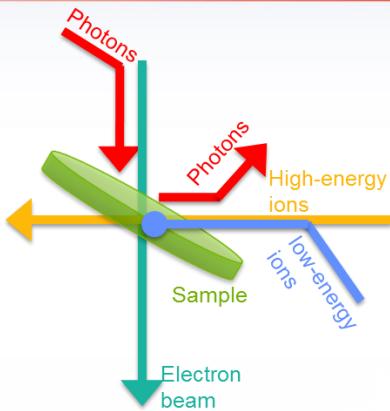


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Summary & Still Father-out Future Directions

Sandia's I³TEM capabilities:

- *In situ* high energy ion irradiation from H to Au
- *In situ* gas implantation
- Heating up to 1,000 °C
- Quantitative and bulk straining
- Two-port microfluidic cell
- Gas flow/heating stage
- Electron tomography
- Precession Electron Diffraction



Currently applying the current I³TEM capabilities to various material systems in sequential or combined harsh environmental conditions

Sandia's I³TEM future capabilities being developed:

- In situ ion irradiation TEM in liquid or gas (currently capable)
- DTEM: Nanosecond resolution (laser optics being developed)
- Beamline: Add 1 MV NEC Tandem & convert 90° magnet to bend beams 45°

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