

NUCLEAR ENERGY & GLOBAL SECURITY



T E C H N O L O G I E S

Probabilistic Risk Assessment (PRA) Presentation to Study Tour for Chinese PUNT Delegation

**May 14, 2007
Timothy Wheeler – PMTS
Sandia National Laboratories**



Sandia has a long history in reactor safety research and development

- **Sandia has supported the US Nuclear Regulatory Commission through regulatory research for 30 years**
- **The historical focus has been on safety of the current US power reactor reactors**
- **Future work for the NRC will have three main thrusts:**
 - 1. Safety of the existing reactors**
 - 2. Licensing support for new light-water reactors in the near-term**
 - 3. Research and Development on safety issues for advanced reactor designs**
- **Sandia will also support the safety element of DOE's Global Nuclear Energy Partnership Program**

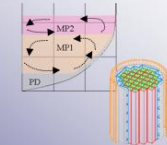
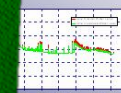
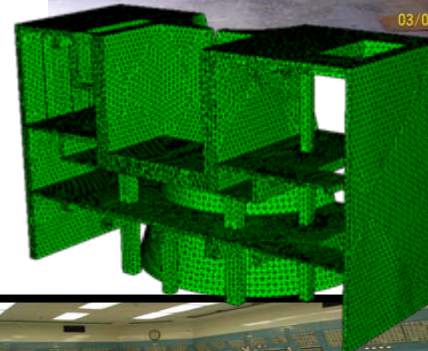
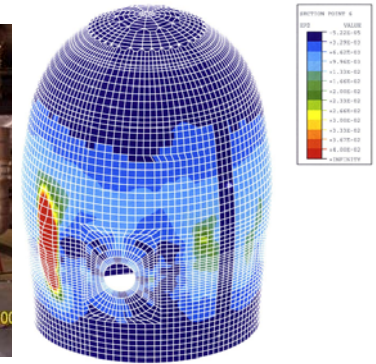


U. S. Nuclear Regulatory Commission Programs

SNL supports the USNRC mission through advanced modeling, testing, and analyses

- **Focus areas**

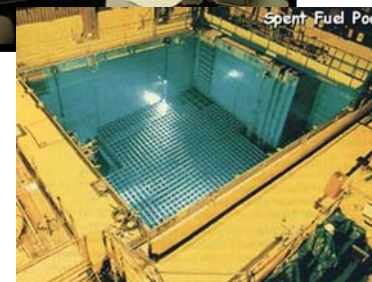
- **Risk-Informed Regulation**
 - Probabilistic risk assessment
 - Fire protection
 - Human reliability analysis
- **Severe Accident Modeling**
- **Security Assessments**
- **Experiments and Structural Analyses**



MELCOR 1.8.6

A Computer Code for Analyzing Severe Accidents in Nuclear Power Plants and Other Facilities

The MELCOR code is developed at Sandia National Laboratories for the United States Nuclear Regulatory Commission





Risk-Informed Regulation

- **Description**
- **Based on 25 years of leadership in PRA and safety technology development**
- **A balance of deterministic and probabilistic approaches to:**
 - **Enhance safety** by focusing resources
 - **Provide a framework** to support regulatory decisions
 - **Allow flexibility** in plant operation and design
- **Leads to *safer*, more effective regulation**
- **Based on Quantitative Safety Goals and a PRA Policy Statement**



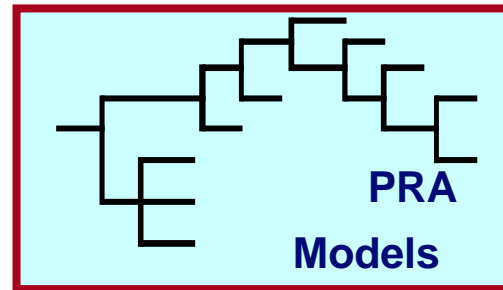
Probabilistic Risk Assessment is a key input to Risk-Informed Regulation

Sandia is prominent in major PRA studies sponsored by the NRC including:

- RSSMAP, IREP, NUREG-1150, RMIEP, Low Power Shut Down, Fire Risk Scoping Study

SNL also has contributed to major PRA application studies:

- Pressurized Thermal Shock, Combustible Gas Control, Decay heat Removal, Station Blackout Study



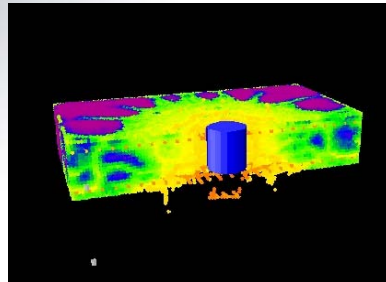
SNL has supported development and application of PRA licensing evaluation tools

- Accident Sequence Precursor evaluations
- Significant Determination Process model development
- Generic Safety Issue evaluation

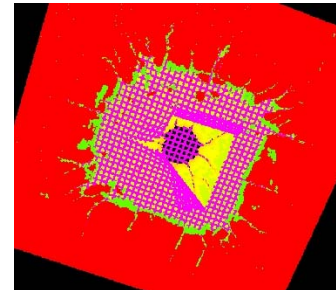


PRA integrates the various aspects of severe accident analysis

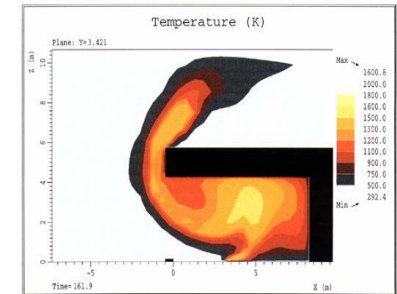
Computing



Penetration & Structures

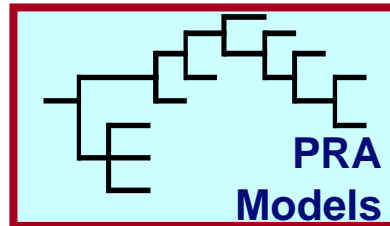


Explosives

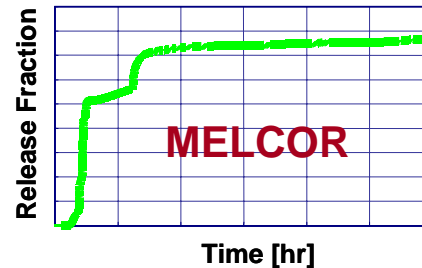


Fire

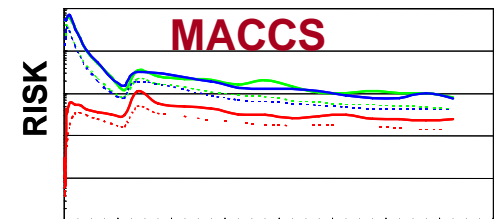
Nuclear
Reactor
Safety



Plant Responses



Radiation Release



Health Effects

Testing



Sled Track



Fire



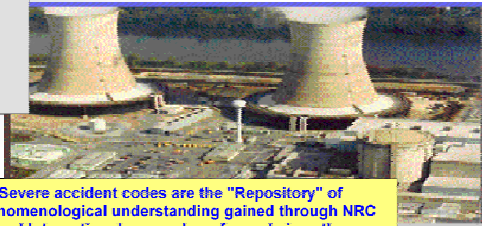
Structures





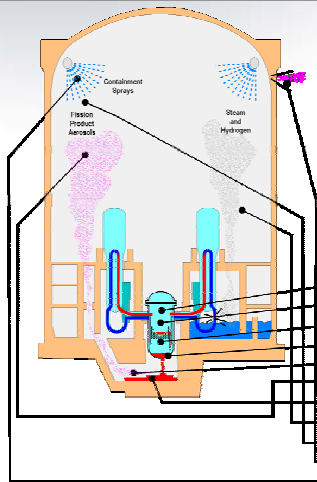
Severe Accident Modeling

Modeling and Analysis of Severe Accidents in Nuclear Power Plants



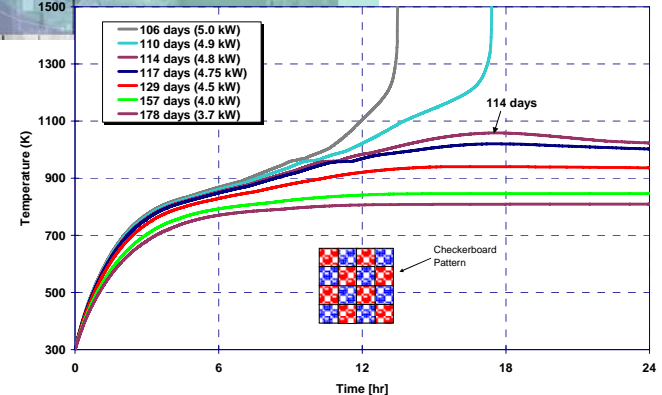
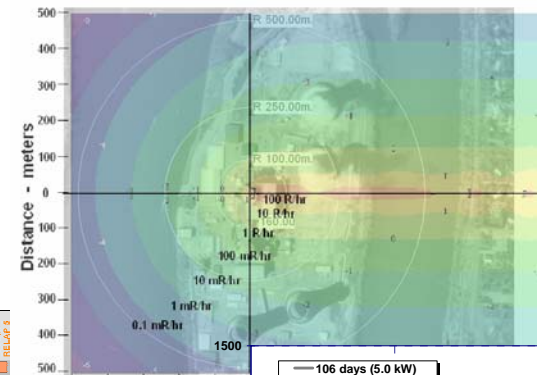
Severe accident codes are the "Repository" of phenomenological understanding gained through NRC and International research performed since the TMI-2 accident in 1979

Integrated models required for self consistent analysis



Important Severe Accident Phenomena

- Accident initiation
- Reactor coolant thermal hydraulics
- Loss of core coolant
- Core meltdown and fission product release
- Reactor vessel failure
- Transport of fission products in RCS and Containment
- Fission product aerosol dynamics
- Molten core/basemat interactions
- Containment thermal hydraulics
- Fission product removal processes
- Release of fission products to environment
- Engineered safety systems - sprays, fan coolers, etc
- Iodine chemistry, and more



Sandia-developed codes MELCOR and MACCS are the premier computer tools for modeling nuclear power plant accident progression and consequences, world-wide.



Fire Protection and Risk



SNL is at the forefront of fire risk methods development and application

Steve Nowlen will present more

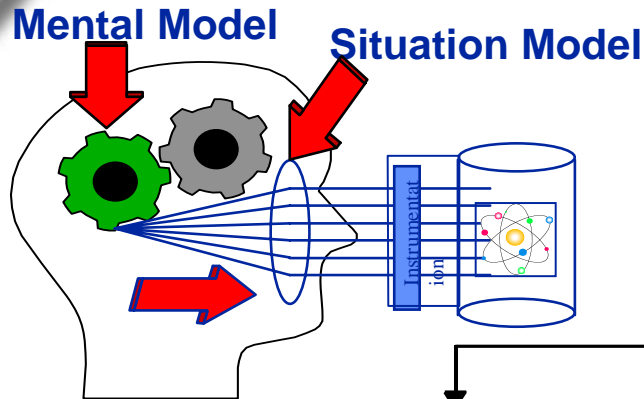


Fire Protection

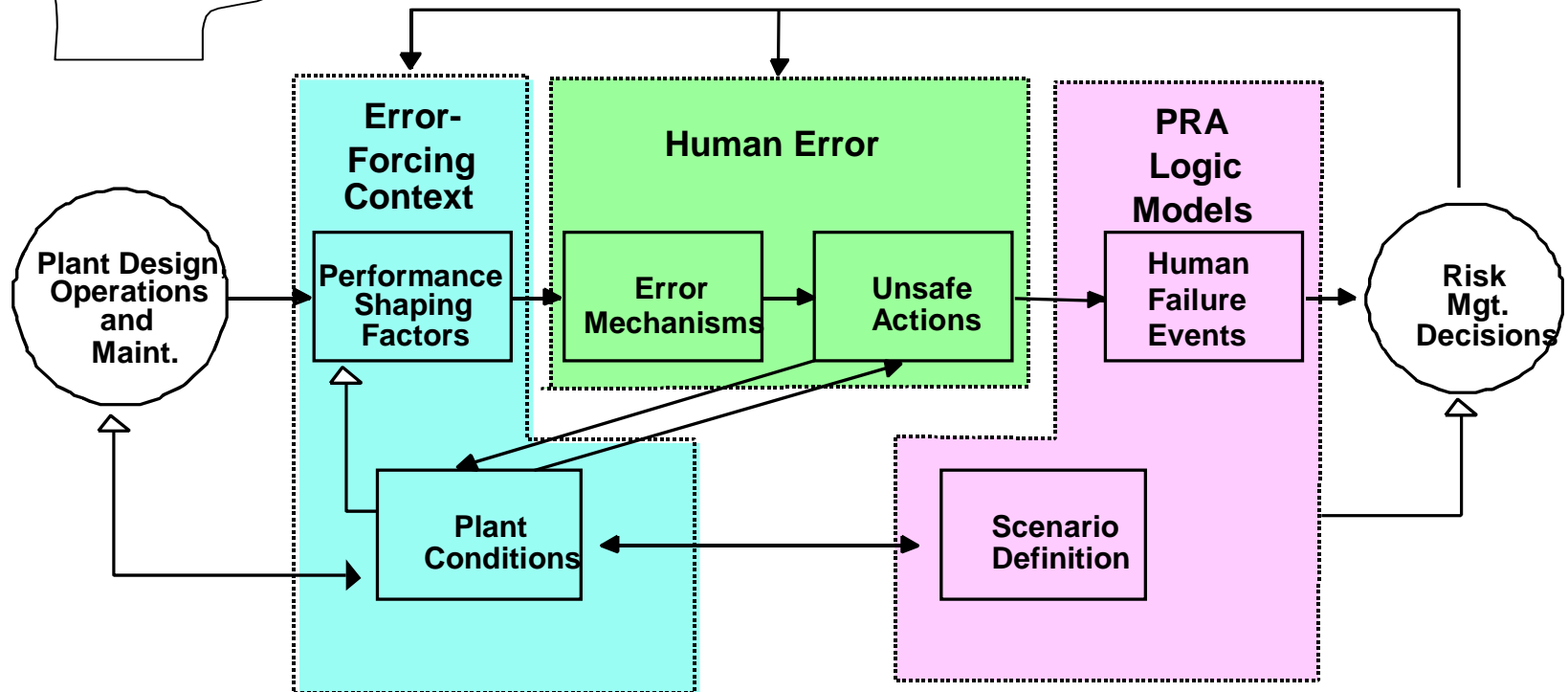
- Inspections
- NFPA 805
- Fire Protection SDP
- Cable Fire Testing
- Hemyc and MT fire barrier tests



Advanced Human Reliability Analysis Techniques



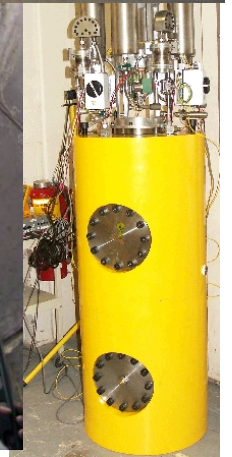
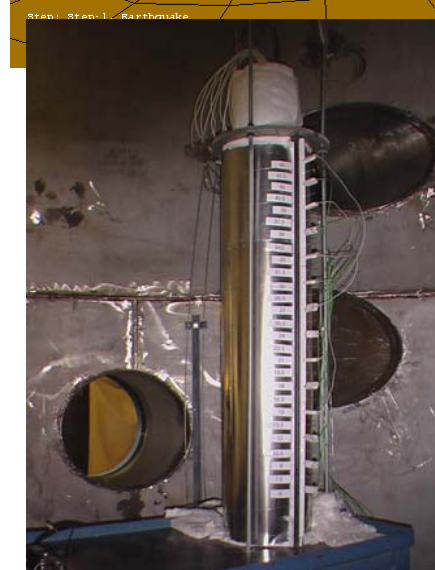
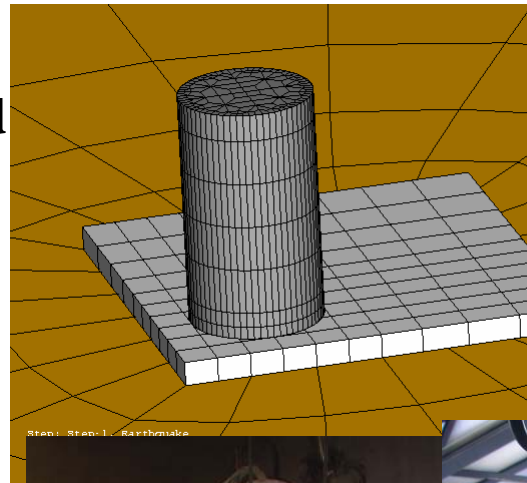
A systematic process for evaluating human errors of commission





Experiments and Structural Programs

- Nuclear power plant containment: experimental and modeling programs
- Seismic analyses of transportation/storage casks
- Analysis of Oyster Creek drywell corrosion for license renewal
- Spent fuel pool thermal experiments
- Dispersal of spent nuclear fuel experiments





Future Focus Areas

- **Support for new reactor licensing**
 - **First Combined Operating Licenses for advanced LWR designs expected in late 2007, with many more to follow**
- **Support for licensing of advanced non-light-water reactors**
- **Global Nuclear Energy Partnership: safety and licensing are of critical importance**
- **Continue to support NRC in regulatory research and security areas**

