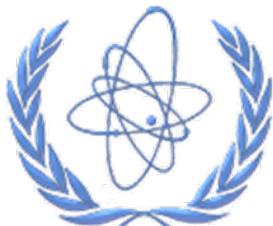


Advanced Conceptual and Numerical Methods for Modeling Subsurface Processes Regarding Nuclear Waste Repository Systems

IAEA Network of Centers of Excellence



Overview of Safety Assessment Methods



Clifford Hansen
Sandia National Laboratories
June 23, 2010

Sandia is a multiprogram laboratory operated by Sandia Corporation, a Lockheed Martin Company, for the United States Department of Energy's National Nuclear Security Administration under contract DE-AC04-94AL85000. This presentation is SAND2010-XXXX X.

Outline

- Definitions
- Basic Structure for Safety Assessments
- Iterative Process
 - Screen Features, Events and Processes
 - Develop Model
 - Characterize Uncertainty
 - Construct System Model
 - Evaluate System Performance
- Summary

Definitions

- Safety Case

A collection of arguments and evidence to demonstrate the safety of a facility

Developed in concert with the facility as scientific understanding advances

Includes:

Pre- and post-closure safety assessments

Descriptions of barriers and their performance

Supporting evidence (e.g., geologic analogues)

Acknowledges unresolved issues

Geologic Disposal of Radioactive Waste, IAEA, 2006

Definitions

- Post-Closure Safety Assessment

Systematic analysis of:

the hazards associated with the facility and
the ability of the site and the design of the facility to
provide for the safety functions and meet technical
requirements

Quantifies performance and associated uncertainties

Compares to relevant safety standards

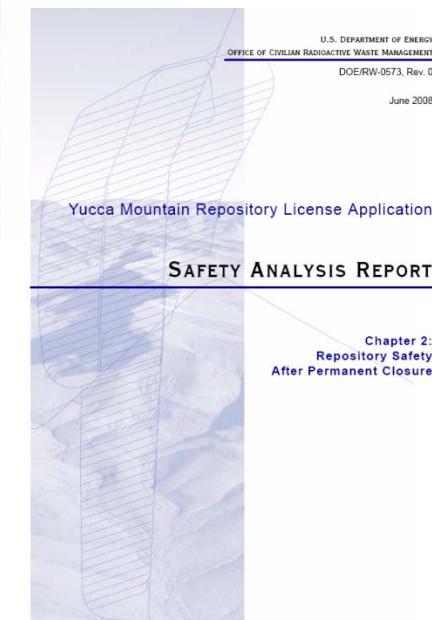
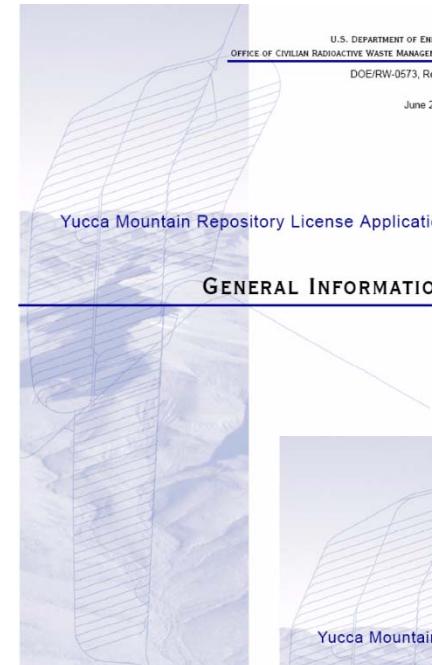
Safety assessments are site and design specific
Constructed to address regulatory requirements

Geologic Disposal of Radioactive Waste, IAEA, 2006

Safety Case Example: Yucca Mountain Repository License Application

DOE/RW-0573 Rev 0
June 2008

- General Information (GI)
 - General Description
 - Proposed Schedules for Construction, Receipt and Emplacement of Waste
 - Physical Protection Plan
 - Material Control and Accounting Program
 - Site Characterization
- Safety Analysis Report (SAR)
 - Repository Safety Before Permanent Closure
 - Repository Safety After Permanent Closure
 - Research and Development Program to Resolve Safety Questions
 - Performance Confirmation Program
 - Management Systems
- Available from the NRC (<http://www.nrc.gov/waste/hlw-disposal/yucca-lic-app.html#appdocuments>)



Example

- Safety Case: Compliance Certification Application for the Waste Isolation Pilot Plant (and applications for recertification)
- Safety Assessment: 1996 (and 2004 and 2009) Performance Assessment for the WIPP
- Safety Case includes Safety Assessment(s)
prepared by repository developer
reviewed by regulator

Basic Structure for Safety Assessment

- Framework for quantitative risk assessment
 1. What events and processes can take place at the facility?
 2. How likely are these events and processes?
 3. What are the consequences of the events and processes?

(Kaplan and Garrick (1979) 'risk triplet')
- How certain are the answers to these questions?
- Iterative process for answering these questions

Sources of Uncertainty

- Lack of knowledge about the future state of the system
 - probabilities of disruptive events
- Incomplete data
 - for example, limited hydrologic data from test wells
- Spatial variability and scaling issues
 - data may be available from small volumes (for example, porosity measurements from core samples), but may be used in the models to represent large volumes
- Abstraction of physical processes into models

Classification of Uncertainties

Aleatory Uncertainty

- Inherent randomness in events that could occur in the future
- Alternative descriptors: irreducible, stochastic, intrinsic, type A
- Examples:
 - *Time and size of an igneous event*
 - *Time and size of a seismic event*

Epistemic uncertainty

- Lack of knowledge about appropriate value or model to use
- Alternative descriptors: reducible, subjective, state of knowledge, type B
- Examples:
 - *Spatially averaged permeabilities, porosities, sorption coefficients, ...*
 - *Rates defining Poisson processes*

Steps in Iterative Performance Assessment

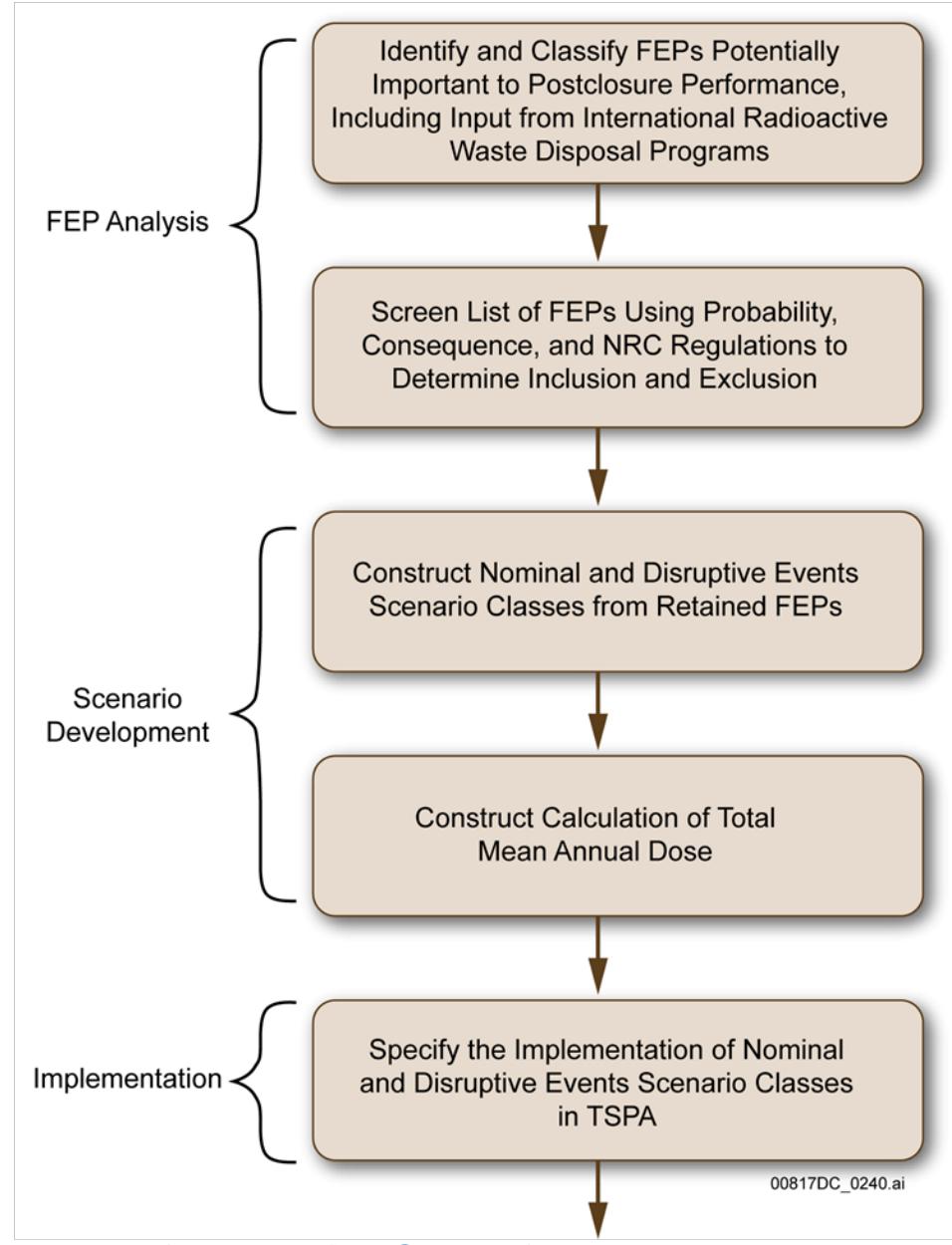
- *Screen Features, Events, and Processes (FEPs) and develop scenario classes*
Answers first two questions: what can happen? How likely?
- *Develop models and abstractions, along with their scientific basis, for logical groupings of FEPs within scenario classes*
Answers third question: what are the consequences?
- *Characterize uncertainty in model inputs*
Answers fourth question: how certain are the answers?
- *Construct integrated system model using retained FEPs and perform calculations for the scenario classes*
- *Evaluate system performance, incorporating uncertainty*

Definitions: FEPs

- Features
 - Elements of engineered or natural system that are important to represent in disposal system models
 - E.g., waste containers, fractures in host rock
- Events
 - Future occurrences that affect evolution of the disposal system
 - E.g., seismic events
- Processes
 - Physical processes that describe the evolution of the disposal system
 - E.g., water flow, metal corrosion, gas generation from chemical reactions

Evaluating FEPs: Yucca Mountain Example

- Probability and significance criteria for FEPs provided in 10 CFR 63.114
- 374 FEPs evaluated
 - 222 excluded
 - 152 included
- Relatively few Events, many more Features and Processes



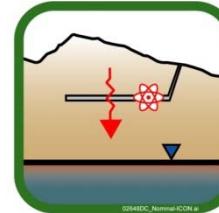
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Form Scenarios: Yucca Mountain Example

Group events by similar effects to form Scenario Classes

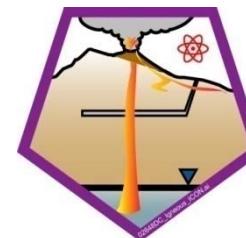
Nominal Scenario Class

- Nominal Modeling Case



Early Failure Scenario Class

- Waste Package Modeling Case
- Drip Shield Modeling Case



Igneous Scenario Class

- Intrusion Modeling Case
- Eruption Modeling Case



Seismic Scenario Class

- Ground Motion Modeling Case
- Fault Displacement Modeling Case

Characterizing Aleatory Uncertainty

- What can happen? Define a vector \mathbf{a} that describes an individual future, e.g.,

$$\mathbf{a} = [nEW, nED, nII, nIE, nSG, nSF, \mathbf{a}_{EW}, \mathbf{a}_{ED}, \mathbf{a}_{II}, \mathbf{a}_{IE}, \mathbf{a}_{SG}, \mathbf{a}_{SF}]$$

- nEW = number of early WP failures
- nED = number of early DS failures
- nII = number of igneous intrusive events
- nIE = number of igneous eruptive events
- nSG = number of seismic ground motion events
- nSF = number of fault displacement events
- \mathbf{a}_{EW} = vector defining nEW early WP failures
- \mathbf{a}_{ED} = vector defining nED early DS failures
- \mathbf{a}_{II} = vector defining nII igneous intrusive events
- \mathbf{a}_{IE} = vector defining nIE igneous eruptive events
- \mathbf{a}_{SG} = vector defining nSG seismic ground motion events
- \mathbf{a}_{SF} = vector defining nSF fault displacement events

Form the set \mathcal{A} of all such vectors (description of all possible futures)

$$\mathcal{A} = \{\mathbf{a} : \mathbf{a} = [nEW, nED, nII, nIE, nSG, nSF, \mathbf{a}_{EW}, \mathbf{a}_{ED}, \mathbf{a}_{II}, \mathbf{a}_{IE}, \mathbf{a}_{SG}, \mathbf{a}_{SF}]\}$$

- How likely?

Quantitative approach: characterize each element of \mathbf{a} with a probability distribution

Qualitative approach: consider a few subsets of \mathcal{A} separately

Characterizing Aleatory Uncertainty

- Scenario-based approach

Define

Reference or nominal scenario – evolution of the disposal system in the absence of unlikely disturbances

Altered evolution scenarios – unlikely events

Bounding scenarios – extreme events

Stylized scenarios – events for which no likelihood can be expressed

Results from different scenarios are not combined; rather, are compared separately to safety standards

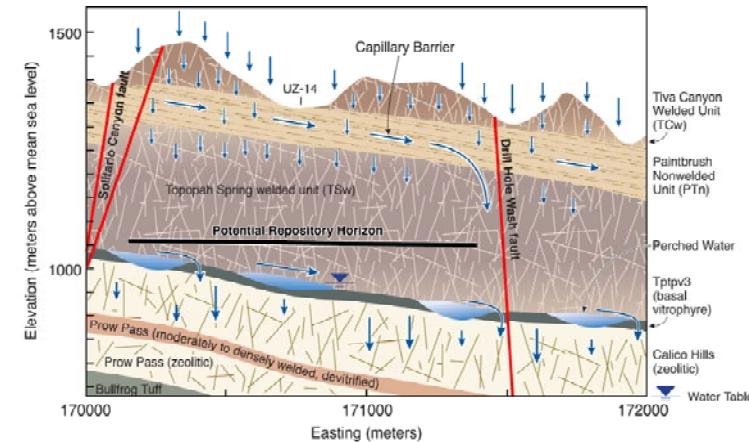
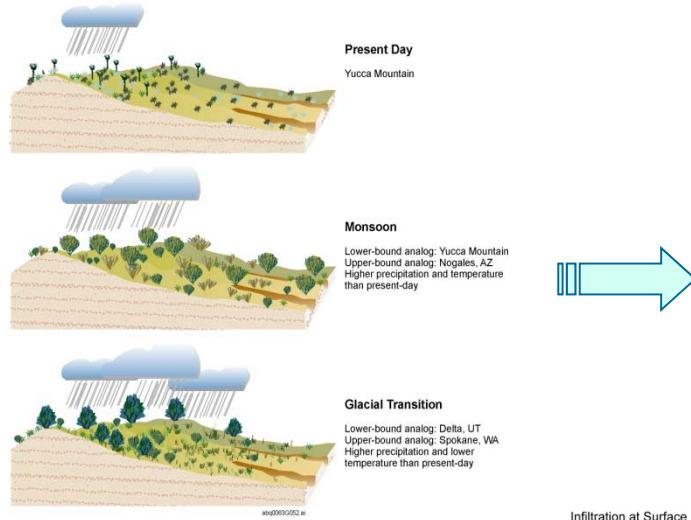
Undisturbed performance

Disturbed performance

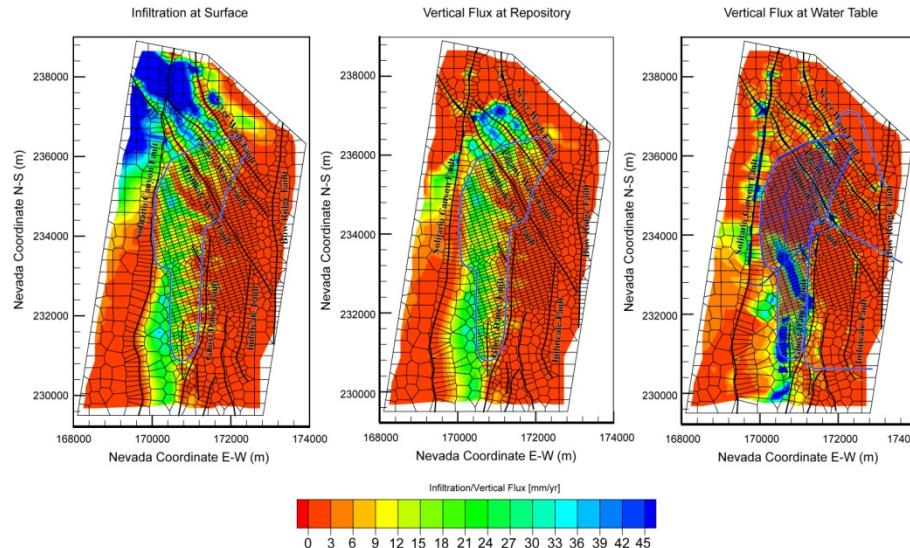
Steps in Iterative Performance Assessment

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Example: Groundwater Flow at Yucca Mountain

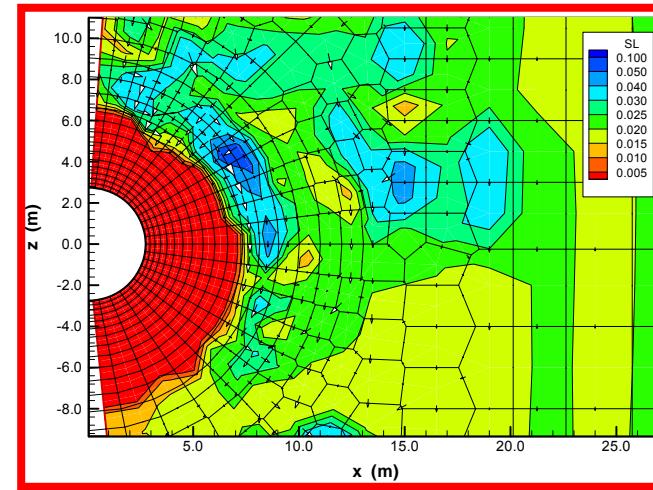
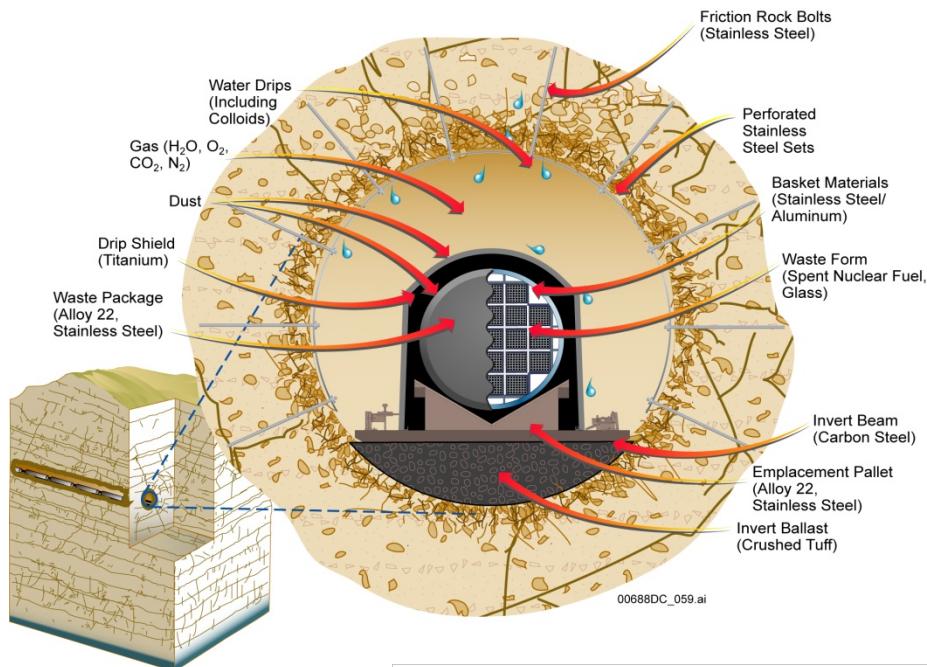


Field tests and models provide basis for understanding infiltration and flow in unsaturated rocks at Yucca Mountain

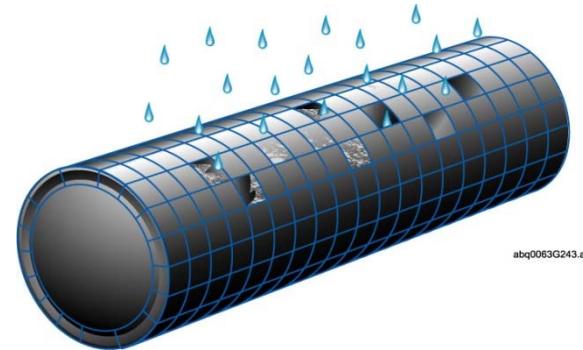
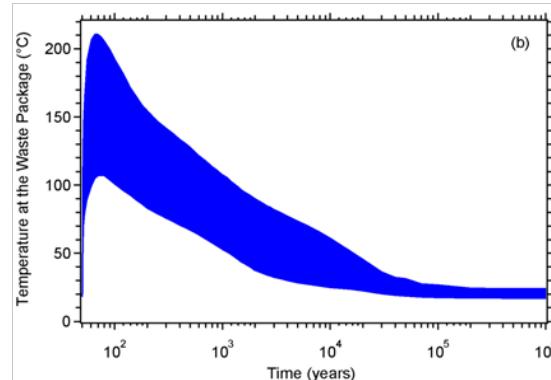


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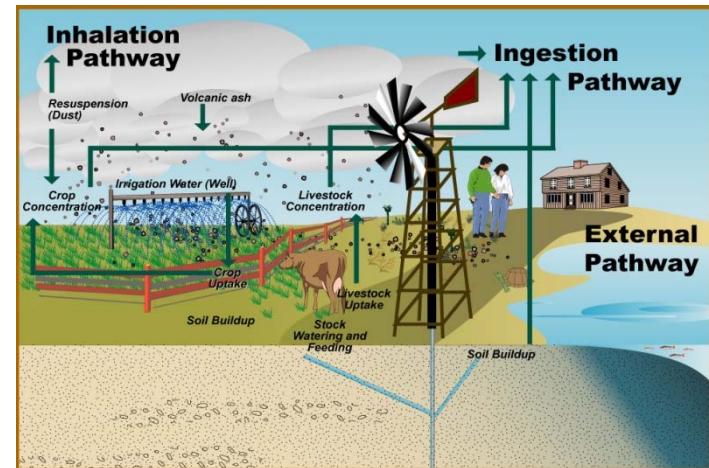
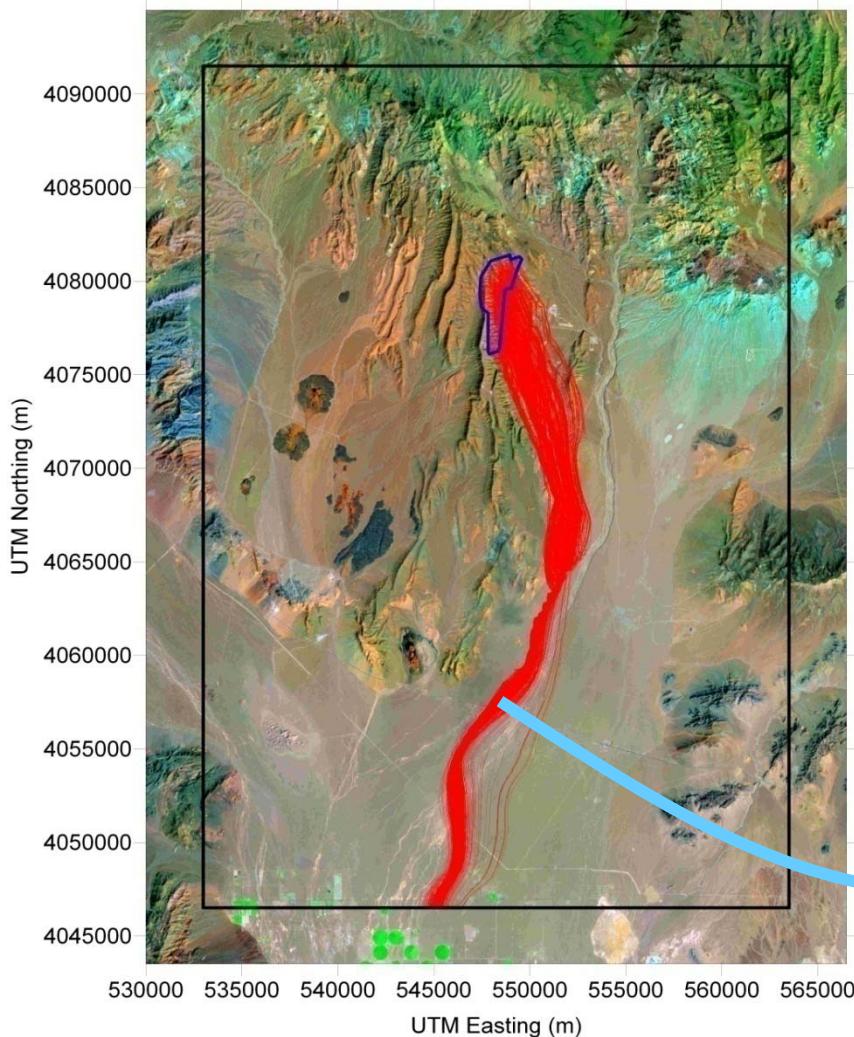
Example: Engineered Features at Yucca Mountain



Material testing and models characterize performance of the engineered barriers



Example: Estimating Dose to Hypothetical Future Humans



Modeled groundwater flow paths and hypothetical exposure pathways

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Characterizing Epistemic Uncertainty

Epistemic Uncertainty in

- Parameters (model inputs)
- Models

Parameter uncertainty is commonly represented by a probability space

Assign probability distribution to each uncertain input e_i

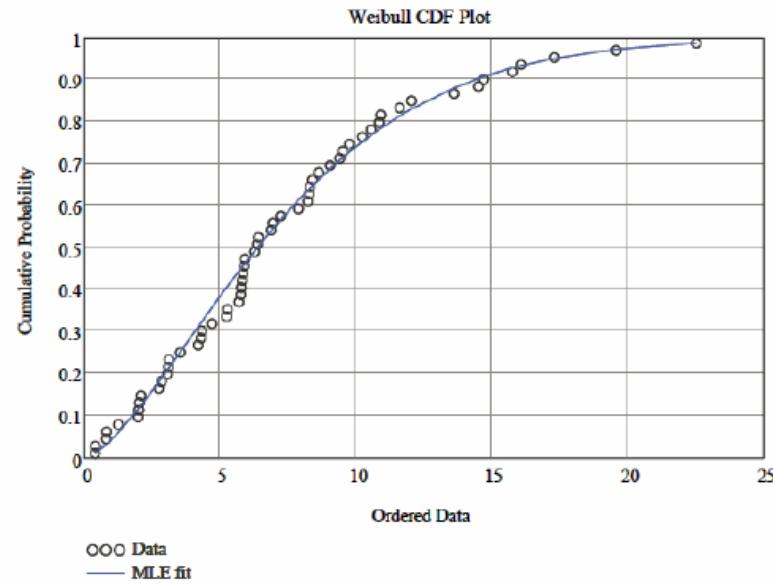
$$\mathcal{E} = \{ \mathbf{e} : \mathbf{e} = [e_1, e_2, \dots, e_N] \}$$

Model uncertainty is commonly addressed qualitatively

Comparison of alternative models

Use of a consensus model

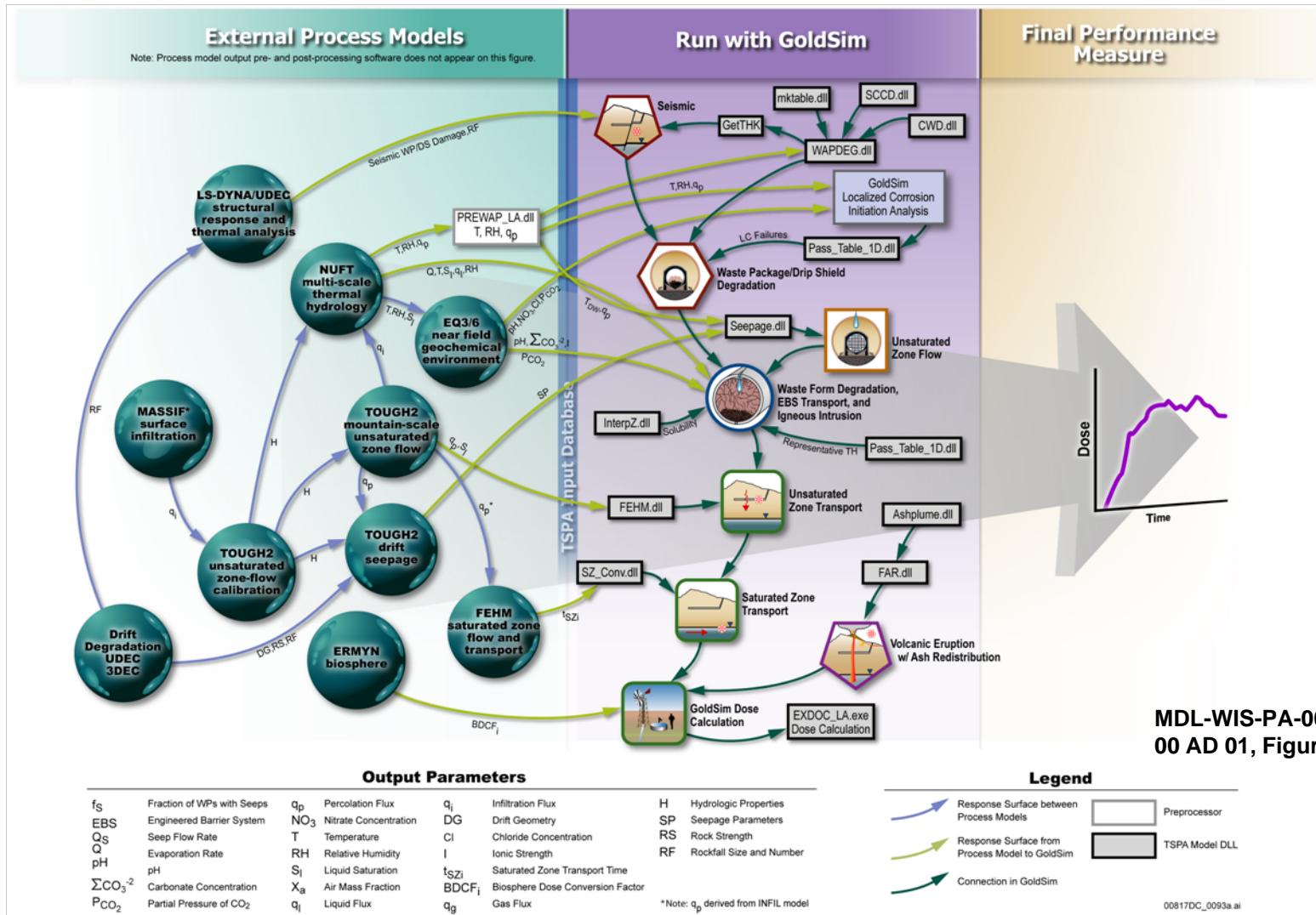
Provide rationale for models that are selected



Steps in Iterative Performance Assessment

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Example: Yucca Mountain TSPA



MDL-WIS-PA-000005 REV 00 AD 01, Figure 3-2[a]

Mathematical Structure

- Two (probability) spaces for inputs

Aleatory uncertainty $\mathcal{A} = \{\mathbf{a} : \mathbf{a} = [nEW, nED, \dots]\}$

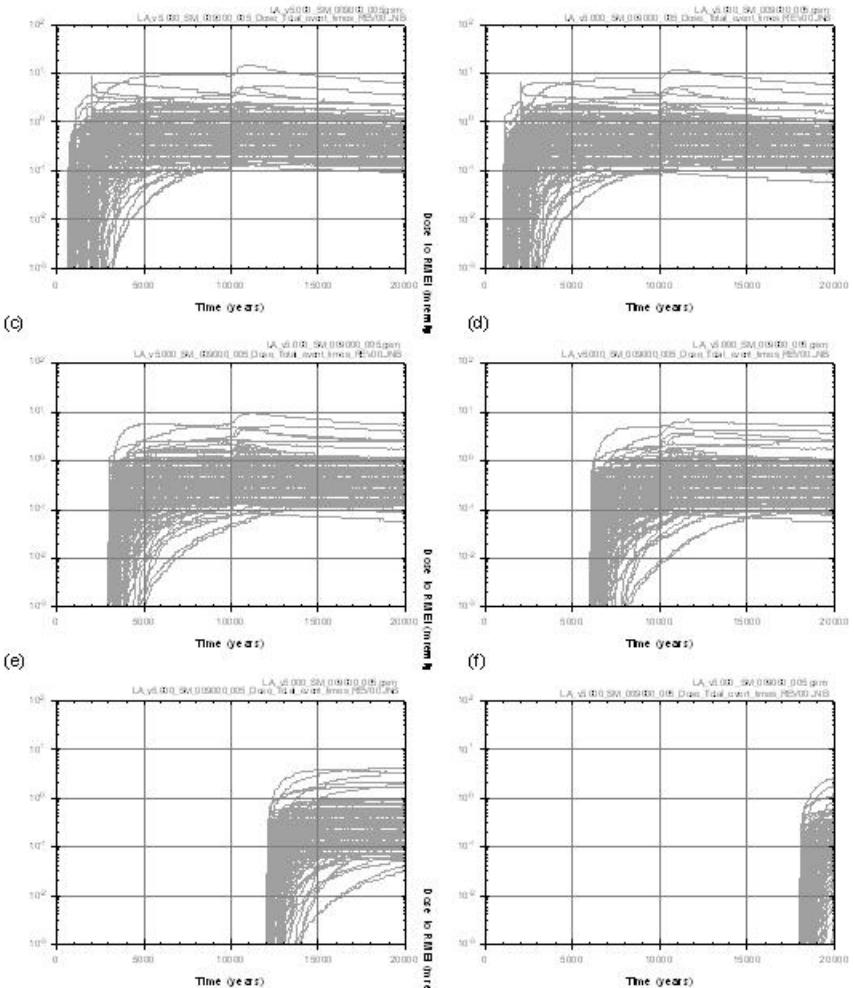
Epistemic uncertainty $\mathcal{E} = \{\mathbf{e} : \mathbf{e} = [e_1, e_2, \dots, e_N]\}$

- Notionally, a function $D(\tau | \mathbf{a}, \mathbf{e})$ (dose) to be evaluated
- Example: mean value of $D(\tau | \mathbf{a}, \mathbf{e})$

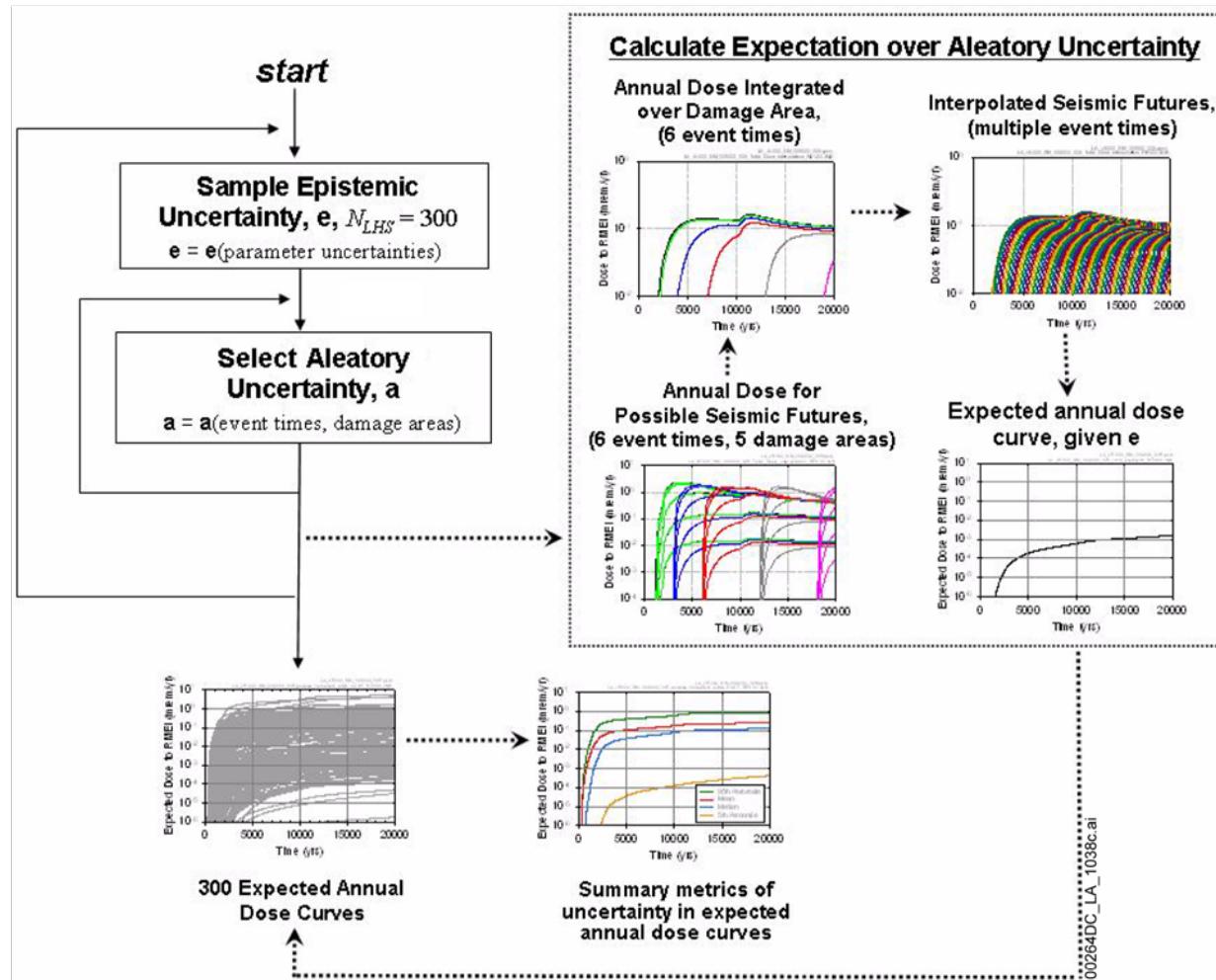
$$\begin{aligned}\bar{\bar{D}}(\tau) &= E_E \left[E_A \left(D(\tau | \mathbf{a}, \mathbf{e}_M) | \mathbf{e}_A \right) \right] \\ &= \int_{\mathcal{E}} \left[\int_{\mathcal{A}} D(\tau | \mathbf{a}, \mathbf{e}) d_A(\mathbf{a} | \mathbf{e}) dA \right] d_E(\mathbf{e}) dE \\ &\simeq \int_{\mathcal{E}} \left[\int_{\mathcal{A}} \left\{ \sum_{MC} \left[\boxed{D_{MC}(\tau | \mathbf{a}, \mathbf{e})} \right] \right\} d_A(\mathbf{a} | \mathbf{e}) dA \right] d_E(\mathbf{e}) dE\end{aligned}$$

Example: Calculation of Dose Yucca Mountain Seismic Ground Motion Scenario

- Sample values for epistemic uncertain inputs (parameters) e – 300 sample elements
- Select a few representative values for aleatory uncertain inputs a – seismic event time, level of damage
- For each combination (a, e) calculate dose over time
$$D(\tau | a, e)$$



Example: Calculation of Expected Dose Yucca Mountain Seismic Ground Motion Scenario



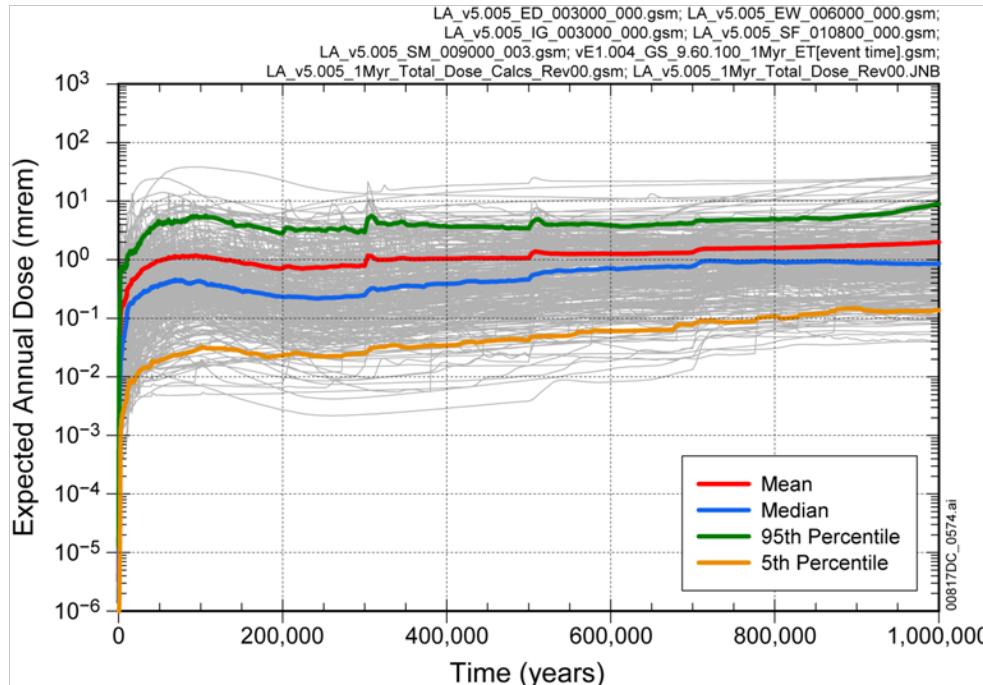
SAR Figure 2.4-8

Steps in Iterative Performance Assessment

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Analysis of Results

Example: Yucca Mountain Total Expected Dose

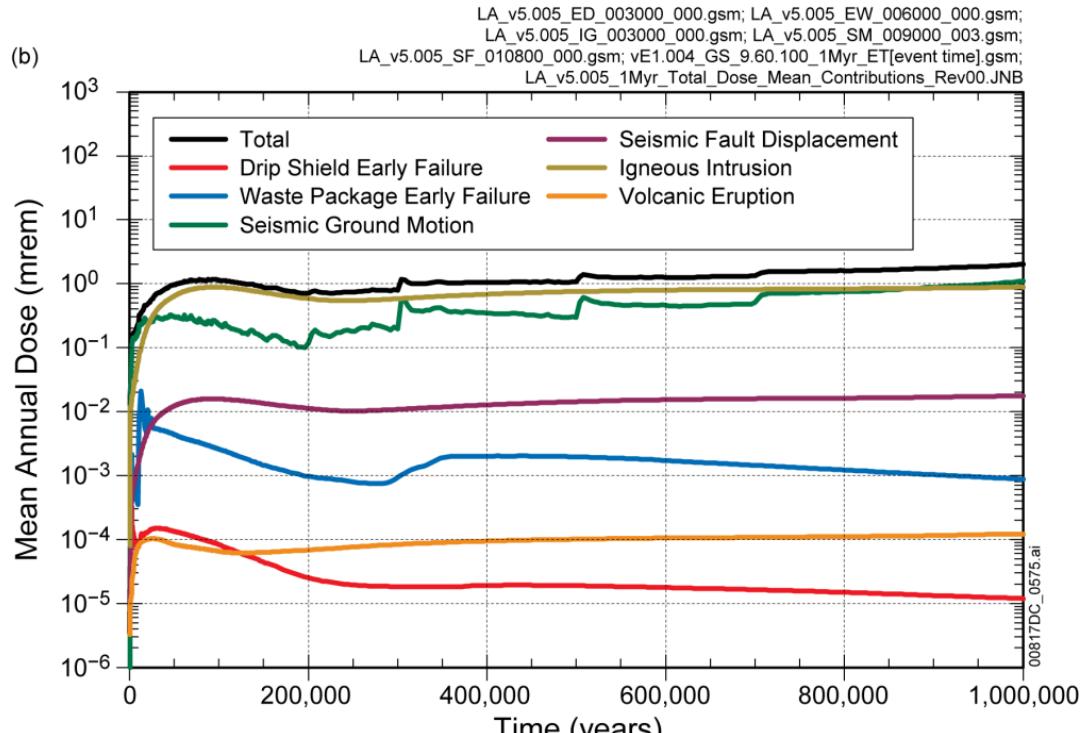


MDL-WIS-PA-000005 REV 00 AD 01, Figure 8.1-2[a]b

Four questions:

1. What determines the shape of these curves?
2. What determines the magnitude of total mean dose?
3. What determines the uncertainty in total expected dose?
4. Are these results stable?

Modeling Cases Contributing to Total Mean Annual Dose



MDL-WIS-PA-000005 REV 00 AD 01, Figure 8.1-3[a]

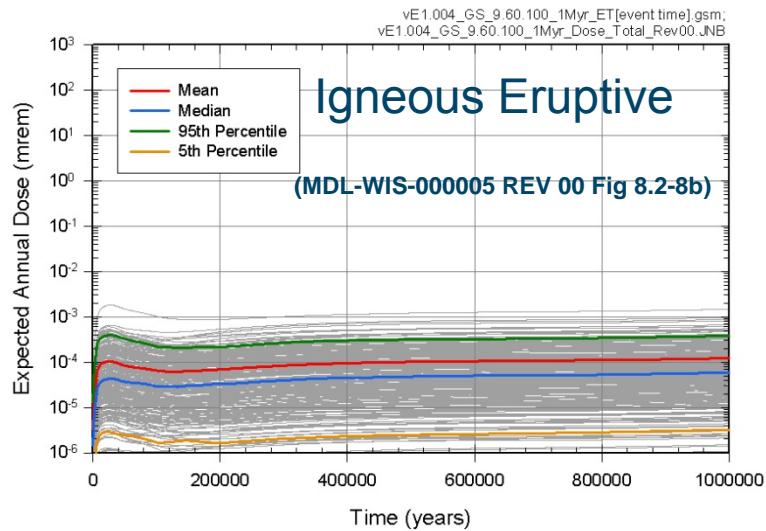
In order of importance:

Igneous Intrusion and
Seismic Ground Motion
(includes effects of
nominal processes)

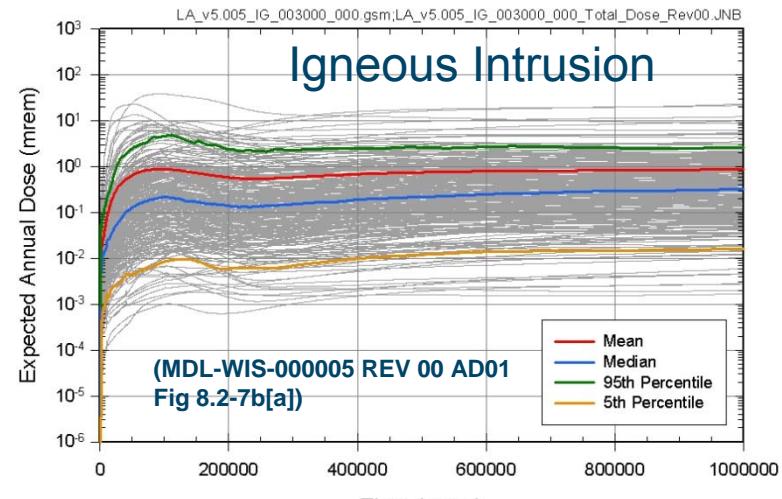
Seismic Fault Displacement

Early Failure, Volcanic
Eruption

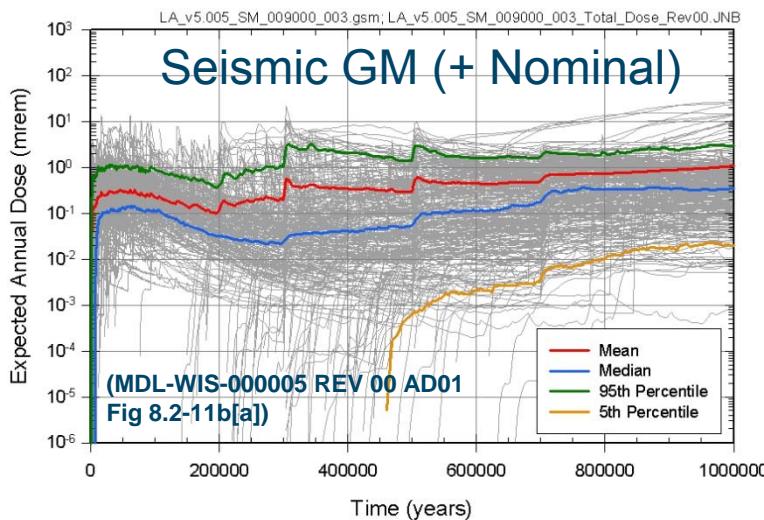
Construction of Total Expected Dose



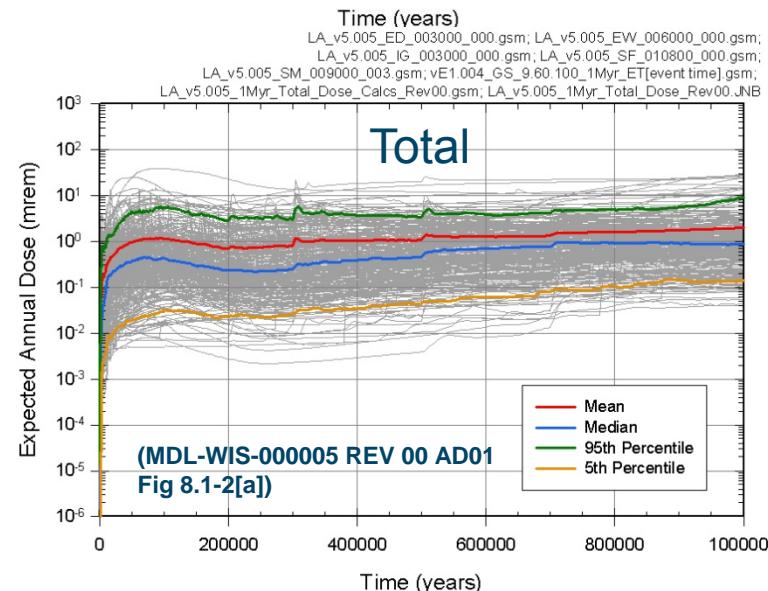
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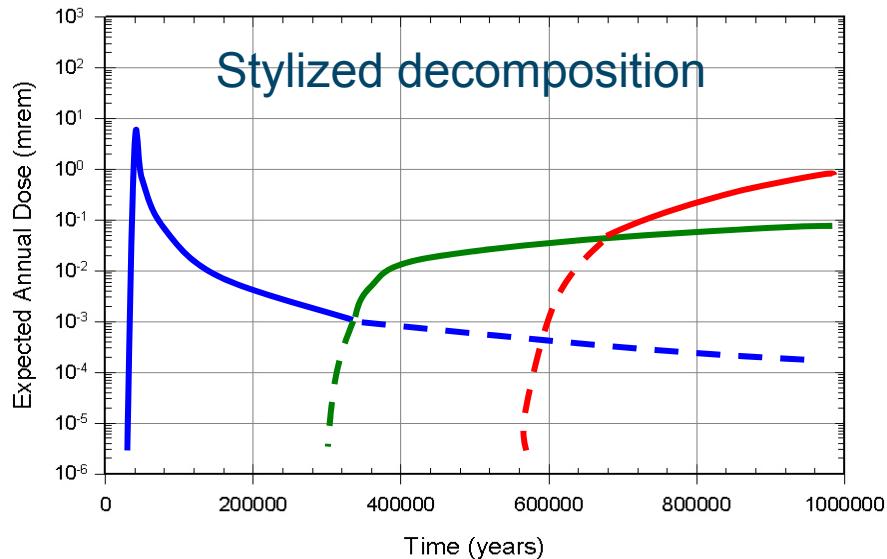
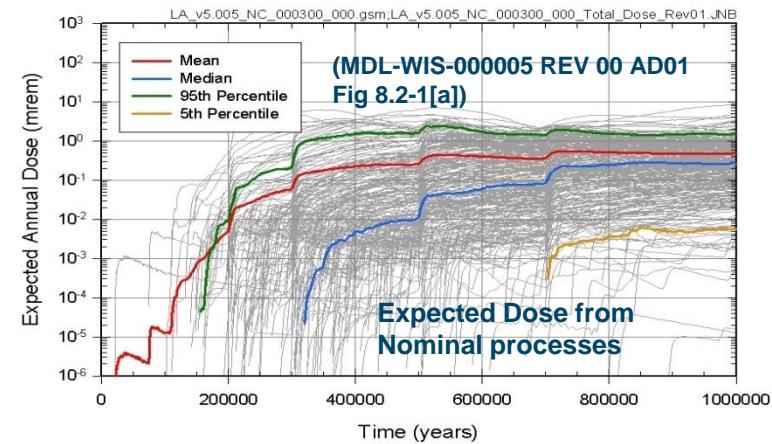
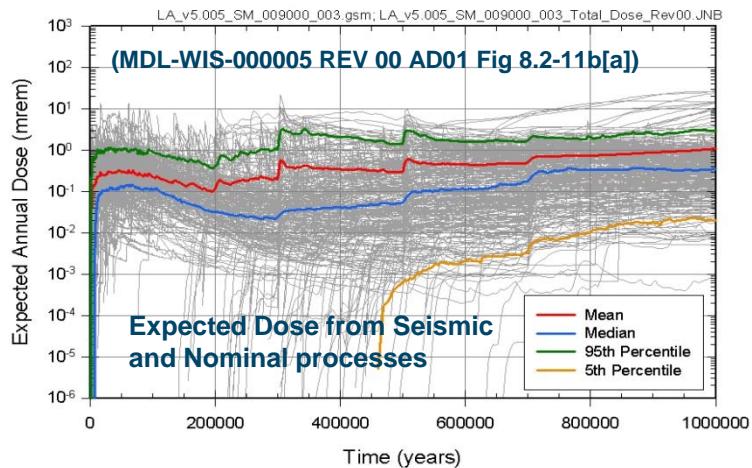
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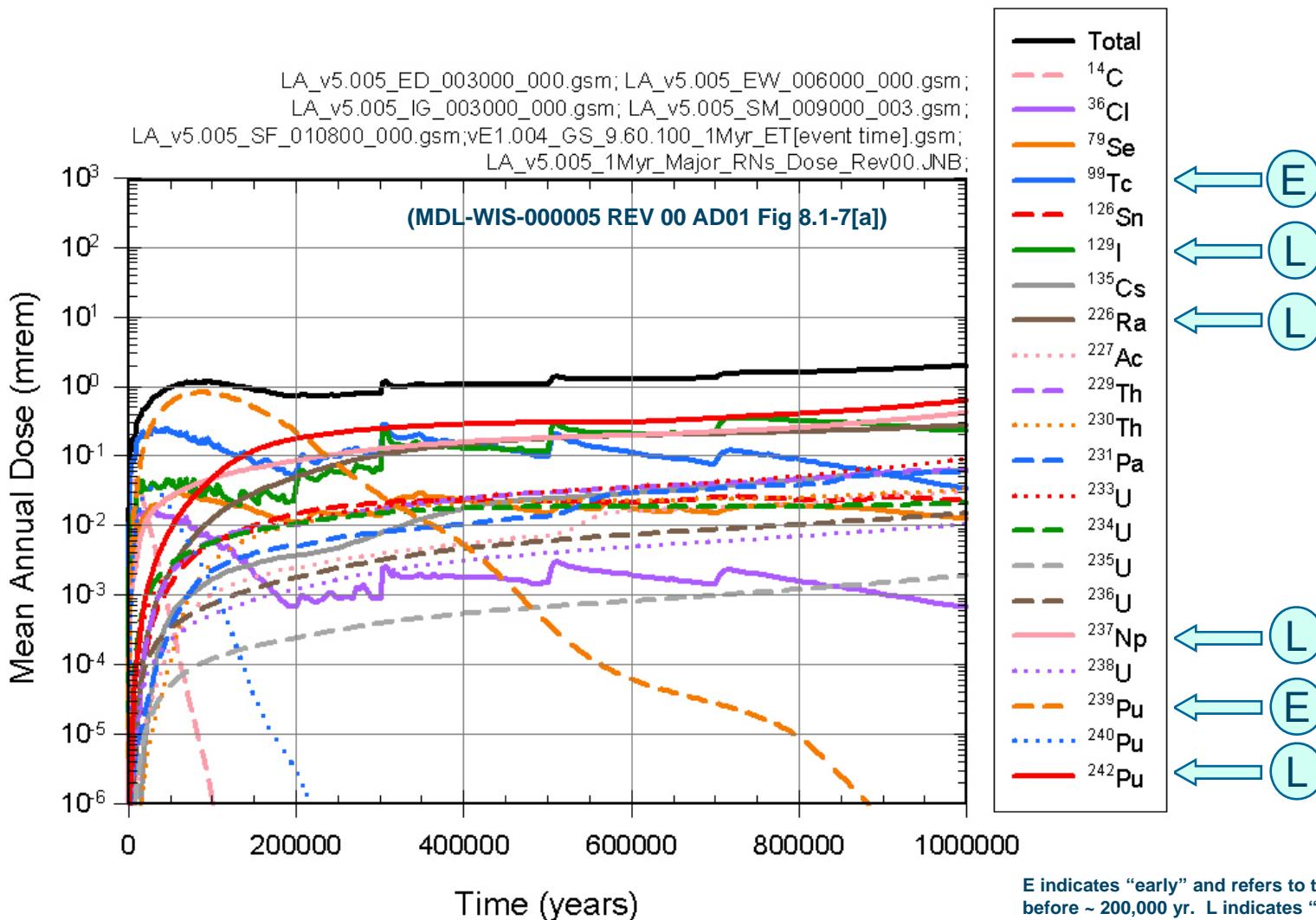


Decomposition of Seismic Ground Motion Dose

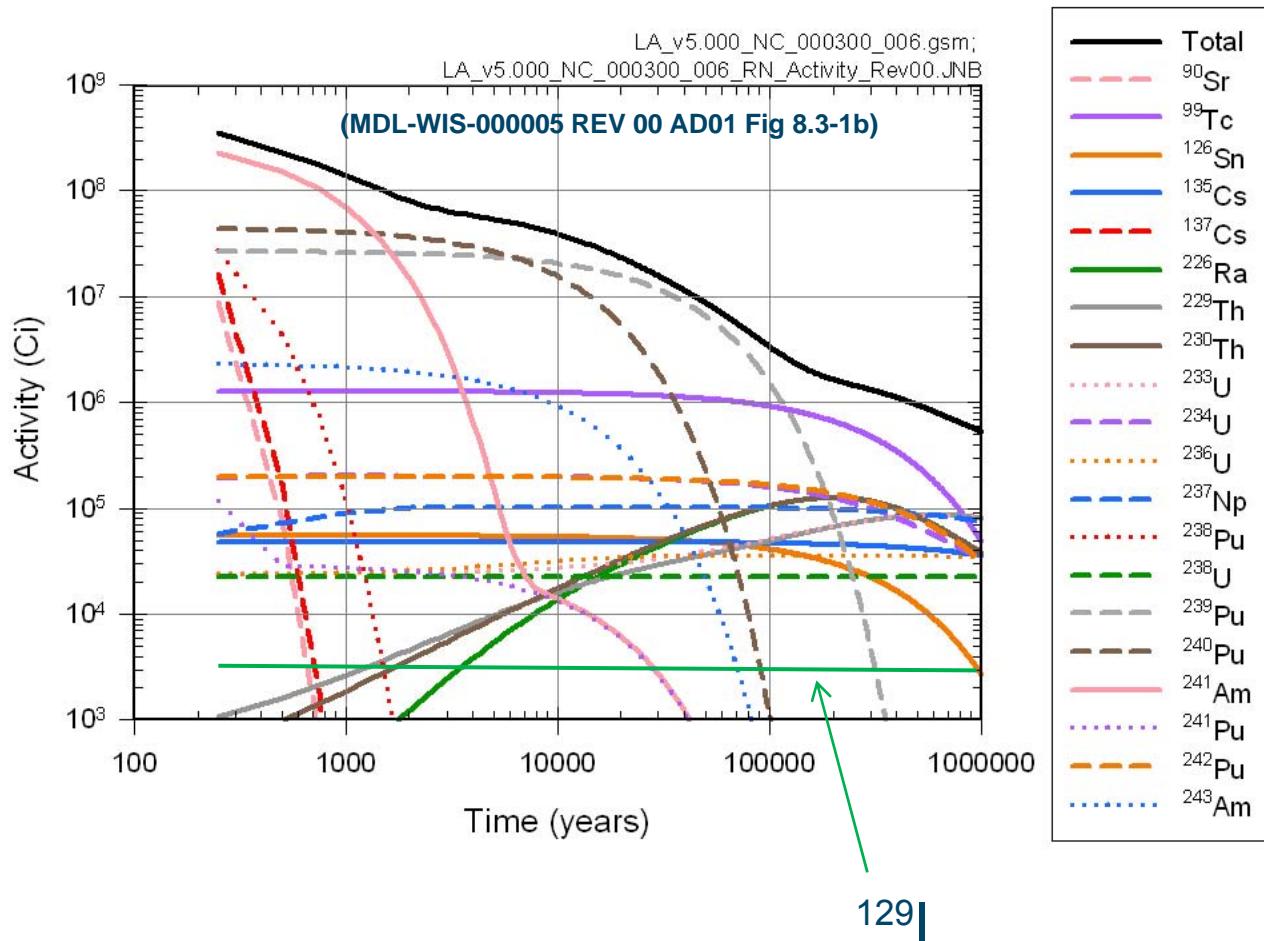


- From seismic damage to CDSP WP (diffusion)
- From SCC failure of CSNF WP (diffusion)
- From general corrosion failure of both WPs (advection)

Radionuclides Important to Mean Dose



Radionuclide Inventory



Early (in order of total activity):

^{241}Am , ^{239}Pu , ^{240}Pu

Late (in order of total activity):

^{99}Tc , ^{237}Np

Note that activity in inventory does not necessarily correlate with importance to mean dose

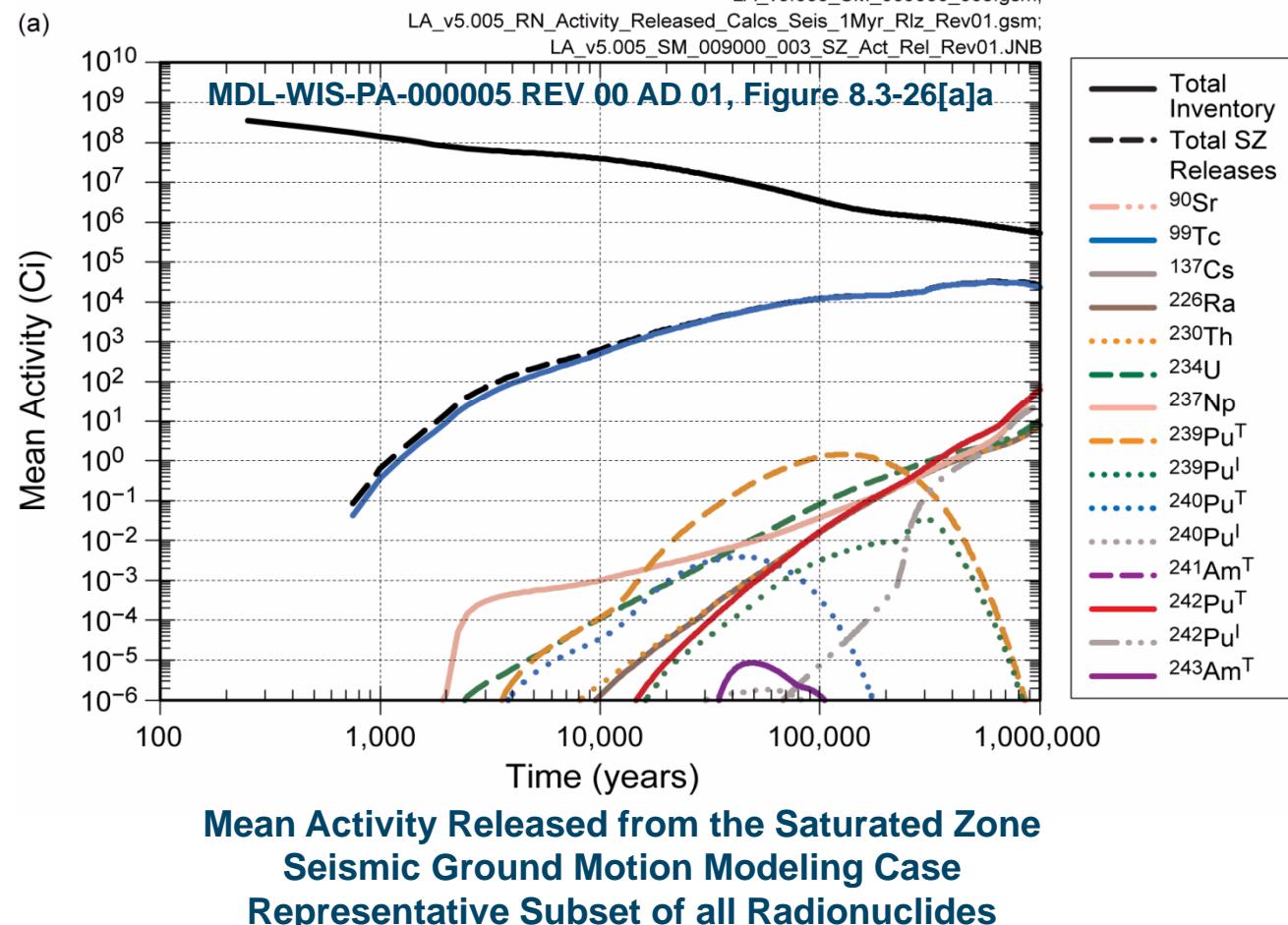
Example Barrier Capability Illustration

Seismic GM + Nominal Processes

At 1M yr, total mean activity released from SZ is about 5 % of total inventory

Short-lived species (e.g., Sr-90, Cs-137) are fully contained

Maximum releases of intermediate-lived species (e.g, Pu-239) are a small fraction of the total activity and occur before 1,000,000 yr



Features and Processes Contributing to Repository Performance

- Precipitation \rightarrow infiltration \rightarrow seepage into repository drifts

Climate	Precipitation (mm/yr)	Infiltration (mm/yr)	Seepage (mm/yr)
Present-day ¹	150	4	0.04
Post-10k yr ²	-	22	8.6

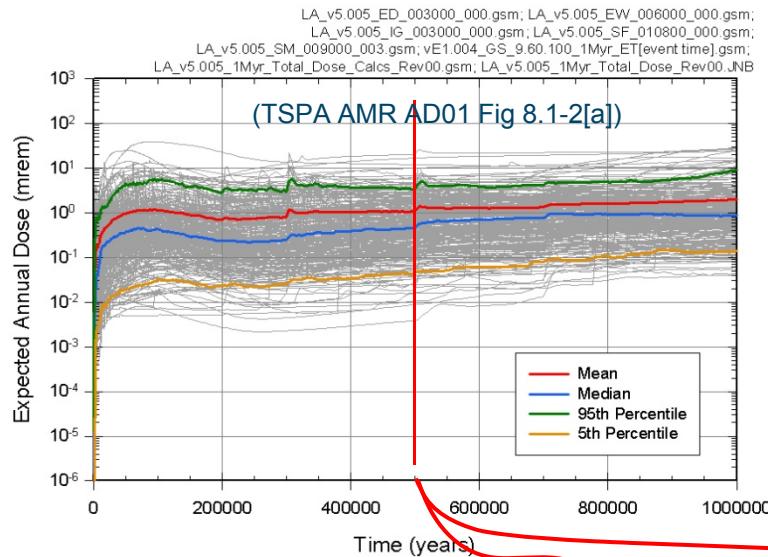
1) Nominal scenario, 10th percentile infiltration scenario, spatial averages, seepage converted from m³/WP/yr
2) Seismic + nominal, 10th percentile infiltration scenario, spatial averages, seepage converted from m³/WP/yr

- Low likelihood of advection through WP outer barrier
 - WP outer barrier failure generally consists of stress corrosion cracking
 - Low likelihood of igneous events, rupture, general corrosion failures
 - Limited water available interior to WPs
- Iron oxyhydroxides from degraded WP materials sorb actinides, buffer water chemistry away from acidic conditions
- Travel times preclude transport of relatively short-lived radionuclides (e.g. ²⁴⁰Pu), reduce concentrations of long-lived radionuclides

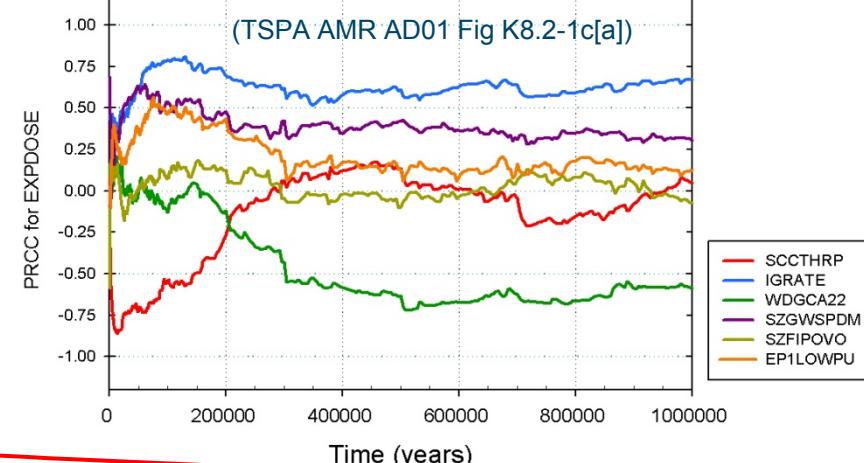
Sensitivity Analyses

- Inputs to TSPA-LA model are uncertain → model output is uncertain
- Monte Carlo analysis – sample from probability space describing inputs, for each sample element generate output
- Symbolically, for a vector \mathbf{x}_i of sampled values for inputs, obtain a vector $\mathbf{y}_i = f(\mathbf{x}_i)$ of values for the outputs
- Sensitivity analyses examine the relationship between \mathbf{x}_i and \mathbf{y}_i
- Explain which uncertain inputs cause uncertainty in output
- Correlation methods, graphical methods, global measures such as sample standard variation
- A. Saltelli et al – several textbooks on methods

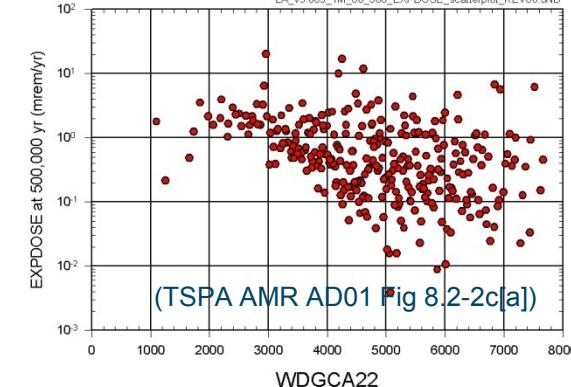
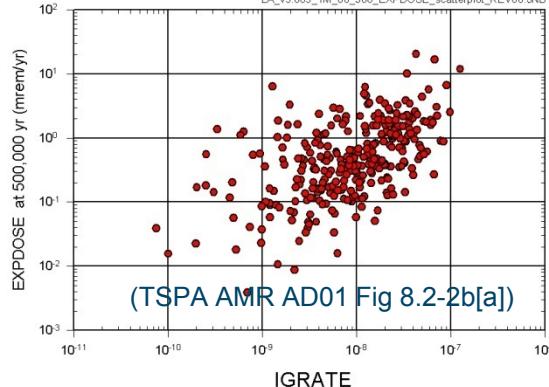
Uncertainty in Total Expected Dose



LA_v5.005_ED_003000_000.Total_Dose_Rev00.JNB; LA_v5.005_EW_006000_000_Rev00.JNB;
LA_v5.005_SM_009000_003.Total_Dose_Rev00.JNB; LA_v5.005_SF_010800_000.Total_Dose_Rev00.JNB;
LA_v5.005_1Myr_00_300_EXPDOSE.mview; LA_v5.005_1Myr_00_300_EXPDOSE_PRCC_HT_REV00.JNB



LA_v5.005_ED_003000_000.Total_Dose_Rev00.JNB; LA_v5.005_EW_006000_000_Rev00.JNB;
LA_v5.005_SM_009000_003.Total_Dose_Rev00.JNB; LA_v5.005_SF_010800_000.Total_Dose_Rev00.JNB;
LA_v5.005_1Myr_00_300_EXPDOSE.xls; LA_v5.005_1Myr_00_300_EXPDOSE.mview;
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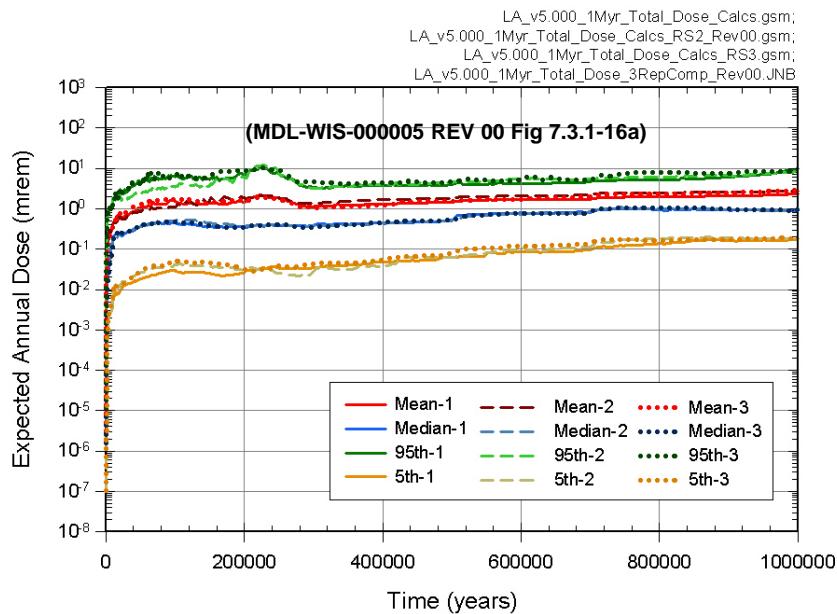


SCCTHRP – Stress threshold for SCC initiation

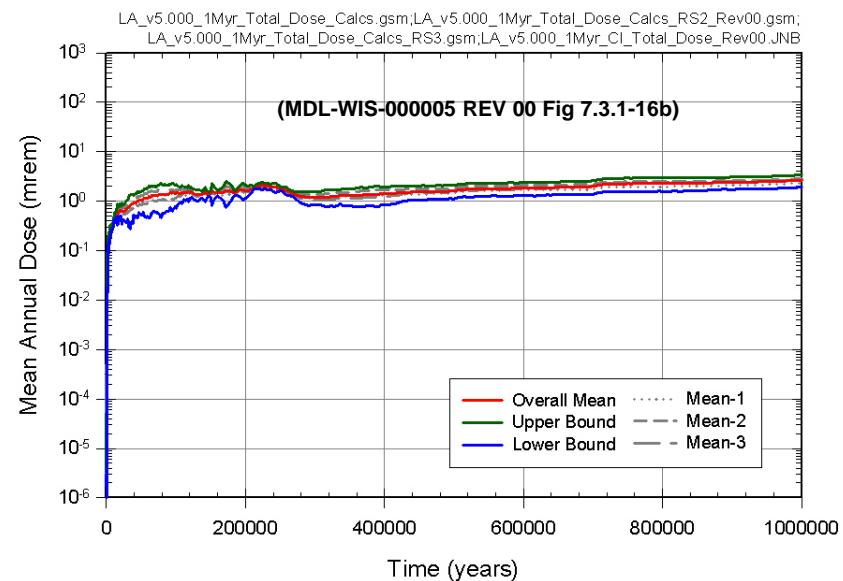
IGRATE – Frequency of igneous events

WDGCA22 – Temperature dependence in A22 corrosion rate

Stability of Total Dose



Replicated sampling demonstrates that sample size is sufficient



Confidence interval illustrates precision of estimate of total mean dose

Summary

- Basic Structure for Safety Assessments
 - Quantitative Risk Assessment
 - What can happen? How likely?
 - What are the consequences?
 - How certain are the answers?
- Iterative Process
 - Screen Features, Events and Processes
 - Develop Model
 - Characterize Uncertainty
 - Construct System Model
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