

FNS Pathways Core Group
UCLA -- Jan. 24-25, 2011

Plasma Facing Components

- Divertors
- First Wall
- In-vessel Components
 - RF launchers, armor
 - RF mirrors
 - large probes (mat'l's, calorimetry, ..)

Presented by Dennis Youghison (Sandia)

*prepared by Richard Nygren (Sandia)
based on past input from many people*



Plasma Facing Components (FNS Pathway)

Outline

- 1. Status of US PFC program**
- 2. Views of gaps and issues**
- 3. Importance of sequence in the FNS Pathway**

4. Brief overviews:

- liquid surface PFCs
- modeling

5. Example of R&D program for W PFCs

- nature of the “bits” or elements in the FNS Pathway

Main theme today:

**What kinds of
elements and
sequences
will we use to
construct our
pathway?**

- 3. Importance of sequence in the FNS Pathway**
- 5. Example of R&D program for W PFCs
(elements in the FNS Pathway)**

Historical Perspective on PFC/PMI Development

1980

1990

2000

2010

limiters
isolated core
radiation loss
sputtering

more power in plasma
conditioned/coated wall
erosion-consequence
[CX neutrals, Maxwellian tail]

diverted plasmas
[more core isolation]
[radiative, detached div.]
importance of plasma edge

large D/T devices
• exhausted power
• stored energy
?FNSF/DEMO div.

DT Mat'l Physics Group
tritium in TFR

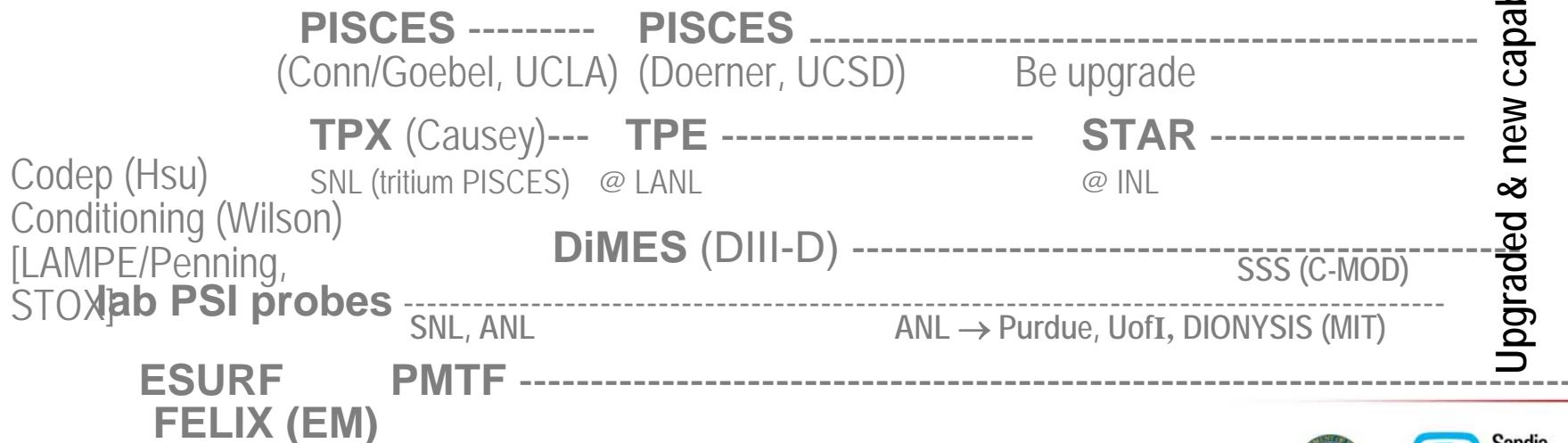
PFC Facilities Review

limiters → divertors → active cooling
power → impurities

prominence of ITER
tritium retention

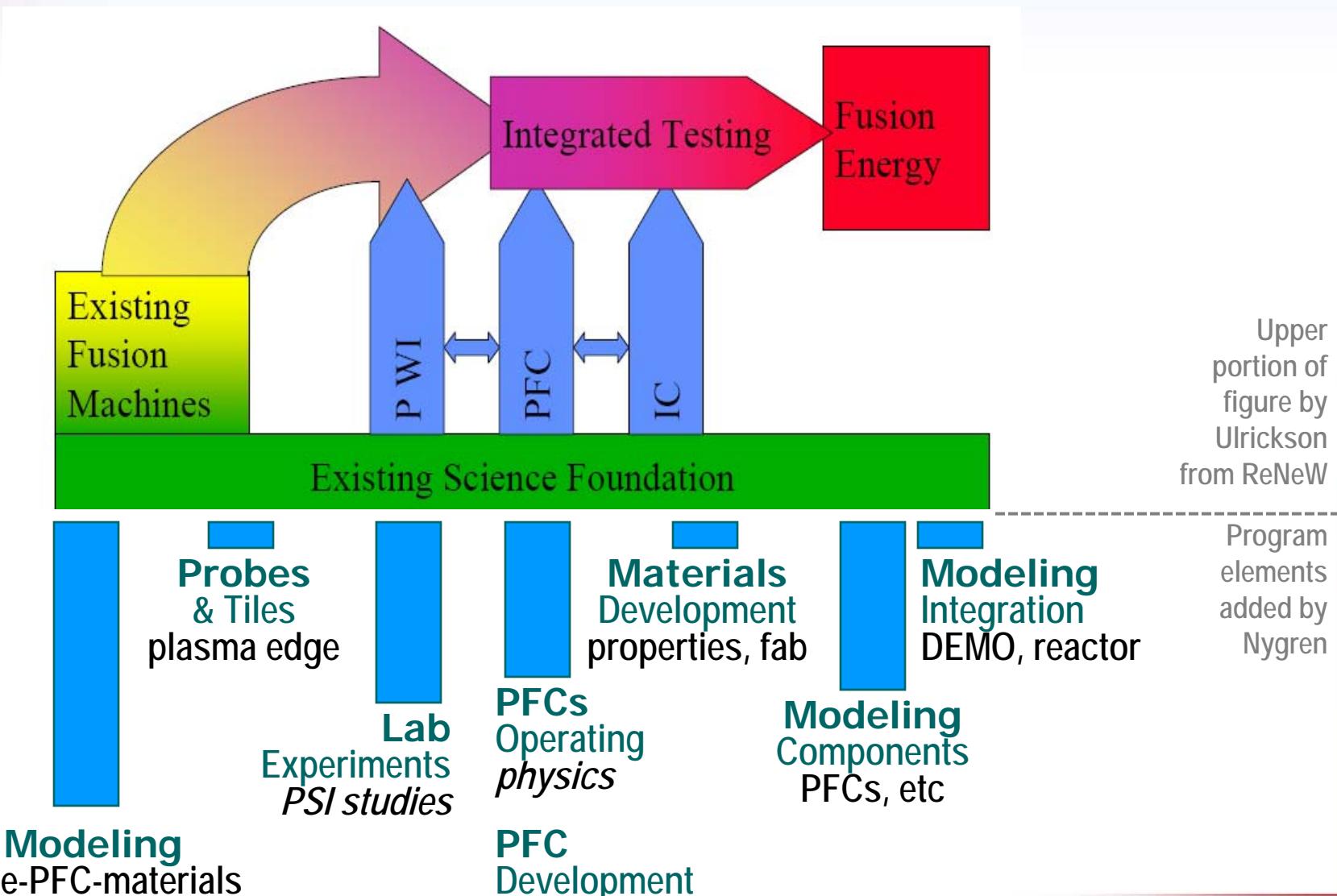
- some issues, C
- JET uses Be
- PFC options for ITER

- Hot walls
- Shaped FW
- ?super-X



PFC-PSI Development & Integration

PFC program elements



Status of HHFC Program

Excluding PMI and edge programs

PFC & IVC
operations

edge
modeling

HHFC
modeling
& testing

PMI
modeling
& testing

#1

Support ITER PFC design & R&D
and develop component fabrication
processes, QA, and operation.

*excellent relations with IPO, IP, DAs;
US R&D & testing (for all DAs) ongoing;
US role in design expanding; valuable
insight into design/machine interfaces*

gap: *test capabilities (old & frail);
design integration & interfaces;
participation in divertor R&D*

**③ Develop and prove robust PFCs for
future confinement devices.**

limited but sustained work on He cooled W and on liquid surfaces

gap: *expanded test capabilities; stronger integrated modeling;
test capabilities (He, liq. met.)*

**② Support physics missions of
existing, upgraded and new
US confinement experiments.**
*needs but limited R&D, some
testing for C-MOD and DIII-D
(NSTX Liquid Li divertor and DiMES
are really PMI)*
gap: *program organization;
integration with machines;
test capabilities (He, probes,
disruption simulation)*



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"Greenwald" Panel Report

Recommendation 4. nine major initiatives.

- I-1. .. predictive plasma modeling and validation ..,**
- I-2. Extensions to ITER AT capabilities .. burning AT regimes**
- I-3. Integrated advanced burning physics ...facility .. dedicated**
- I-5. .. disruption-free concepts .. performance extension device ..**
- I-4. Integrated experiment for PWI/PFCs .. steady-state .. non-DT**
- I-6. .. advanced computer modeling and laboratory testing .. single-effects science for major fusion technology issues,**
- I-8. Component development/testing program ... multi-effect issues in critical technology .. breeding/blanket .. first wall**

- I-7 Materials qualification facility ... (IFMIF).**
- I-9. Component qualification facility.. high availability.. heat flux .. neutron fluence .. DT device (CTF).**

FNSTF Core Group Homework: List of Generic FNST R&D

Richard Nygren and Stan Milora 8 sep 2010

Refs:

1. Presentation by Rajesh Maingi at PMI Test Stand Workshop, ORNL, Aug 30 – Sep 2, 2010
2. Draft “Generic FNST R&D – VLT, August 2010” (developed by R. Maingi and M. Peng, Aug 2010)
3. Draft “ReNeW: Technology Content, Thrusts 1-18” (developed by R. Nygren, 4 june 2009)

All thrusts target elements in PWI, PFC, and IC

9. Unfolding the physics on the boundary layer plasma
 - SOL, divertor, private flux, pedestal; innovative divertors
 - Develop predictive capability, new diagnostics
10. Decode and advance the science and technology of plasma surface interactions
 - Well-controlled and well-diagnosed dedicated facilities
11. Improved power handling through modeling innovations
 - New solid and liquid concepts, innovative heat sinks
12. Demonstrate an integrated solution for plasma-material interfaces compatible with an optimized core plasma
 - Integrated test of PFC and IC components + PMI modeling
 - Low activation environment with improved diagnostics
13. ~ “Fusion nuclear science” (paraphrased)
14. ~ “Materials science” (paraphrased)

Several “lists” of R&D topics for FNST have been developed and distributed to a limited group. To start the FNSF Core Group, this list was developed from the references above. The flow follows that of an excellent presentation (Ref. 1) with pages 4, 6, 8, 9 and 11 from Ref. 2 added to give more specifics. Some final comments (p13) are also added based on Ref. 3.

T9: Unfolding the physics on the boundary layer plasma

- Develop and deploy new diagnostics in existing devices for comprehensive boundary layer plasma density, temperature, and neutral density in at least one plasma device.

T10: Decode and advance the science and technology of plasma surface interactions (in well-diagnosed dedicated facilities)

- Upgrade existing laboratory facilities capable of extending plasma parameters closer to conditions of interest, including the capability to handle tritium (first principles models for T_1 and T_2).
- Build large-size test stands for PFC tests and design validations.
- Develop and improve first principles models for plasma coupling for future fusion energy experimental data.
- Invest in surface material diagnostics and evolution.
- Develop and test new surface materials with improved performance margins (connect to Thrust 14)

From FNSP Summary of ReNeW Themes 9-14

Nygren, Milora, Peng

T11: Improved power handling through engineering innovations

- Design, fabricate and test refractory heat sinks with advanced cooling techniques for high temperature operation ($>600\text{C}$) and deploy liquid metal PFC experiments in plasma devices.
- Develop fabrication processes and better joining techniques using selected activation refractory alloys for both PFCs and internal components, e.g. RF launchers.
- Construct/upgrade new lab facilities for synergistic testing including cyclic high-heat-flux, irradiation/permeation, and liquid metal performance; and improve models of thermal performance, irradiation damage and tritium transport in PFCs.
- Provide improved PFCs for qualification on existing or new confinement experiments.
- Develop more robust PFCs for transient events with higher design margins and improved reliability and maintainability. Include engineering diagnostics to monitor PFC performance and provide data for lifetime prediction models.

T14: Materials Science and Technology Needed to Harness Fusion Power

- Improve the performance of existing and near-term materials, while also developing the next generation of materials with revolutionary properties.

T12: Demonstrate an integrated solution for plasma-material interfaces compatible with an optimized core plasma

- Develop design options for a new moderate-scale facility with a relevant boundary, to assess core-edge integration in fusion and fusion-like liquid metal coolants. Key desired features include high power density, sufficient pulse length and duty cycle, elevated wall temperature, as well as steady-state control of an optimized core plasma. Design for a hydrogen and deuterium fuel environment, to assure flexibility in changing boundary components as well as access for comprehensive measurements to fully characterize the boundary plasma and plasma-facing surfaces. The balance and sequencing of hydrogen and deuterium operation should be part of the design optimization. Develop an accurate cost and schedule for this facility, and construct it.
- Extend and validate transient heat flux control from Thrust 2, plasma control and sustainment from Thrust 5, boundary plasma models from Thrust 9, plasma-material interaction science from Thrust 10 and plasma-facing component technology from Thrust 11 with the new research from this facility, thereby demonstrating a viable solution to the very challenging core edge integration problem for DEMO.

From FNSP Summary of ReNeW Themes 9-14

Nygren, Milora, Peng

T13: Establish the Science and Technology for Fusion Power Extraction and Tritium Sustainability

Perform fundamental research to establish the scientific parameters necessary to address the issues. An example activity is the exploration of plasma chemistry, heat transfer, and magnetic field interactions in lithium-
ing liquid metal coolants

resistance to cracking.
length.

combined impact of typical of a fusion burning plasma as a test extraction experiments designs, and operating

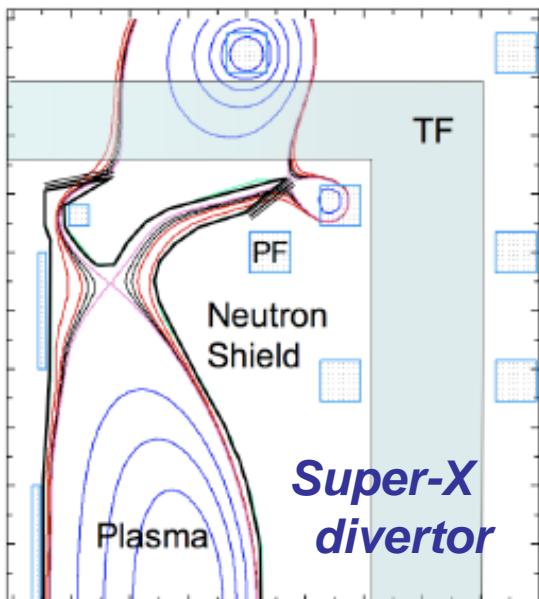
complete effect of fusion could be construction (FNSF) to perform from the effects of time in concert with all

models necessary to collect reliability and

Some concerns for HHFCs

Richard's Rules

1. Manageable heat loads based on
 - a) reasonable physics, including startup, shutdown and plasma transients and
 - b) sound engineering practices.
2. Credible integration of systems.
3. Recognition of credible development paths and recommendations on which path must be taken.



(for example, engineer's view of Super-X)

Idea is intriguing; need experiments, e.g., MAST.

Proof of physics will include information about how T_e and T_i decrease from X-point to divertor.

[Important for PSI effects along lengthy throat.]

What about requirements on plasma control, restoring position promptly, etc?

Will an adequate blanket fit with a super-X divertor?

[You do not know until you do detailed engineering.]

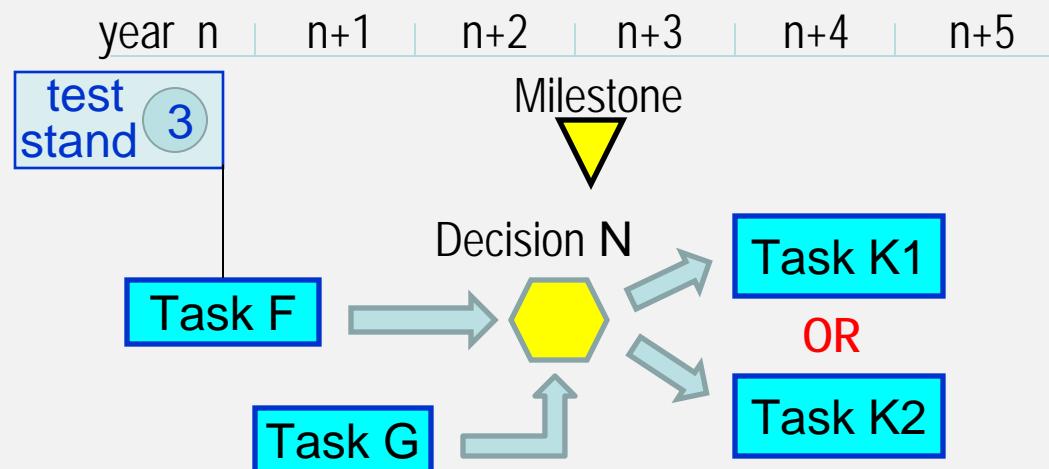
We want to address the high level gaps and issues with some suite of options for tasks, sequences and required facilities.

At what level of detail should we be working now?

What supporting detail is needed in the future to confirm the credibility of the pathway analysis?

Main point:

What kinds of elements and sequences will we use to construct our pathway?



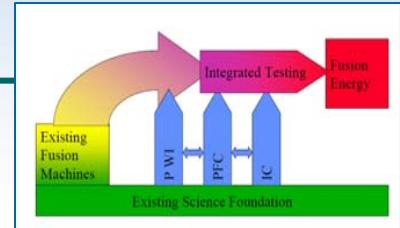


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PFC-PSI Development & Integration

Look at development of water-cooled PFC technology
heat sink technology, big impact on device operation



New PFCs & ICs
Operating
Physics

observe/measure
performance

Probes
& Tiles
plasma edge

Lab
Experiments
PSI studies

Modeling
edge-PFC-materials

PFC
Development
HHF tests

Materials
Development
properties, fab

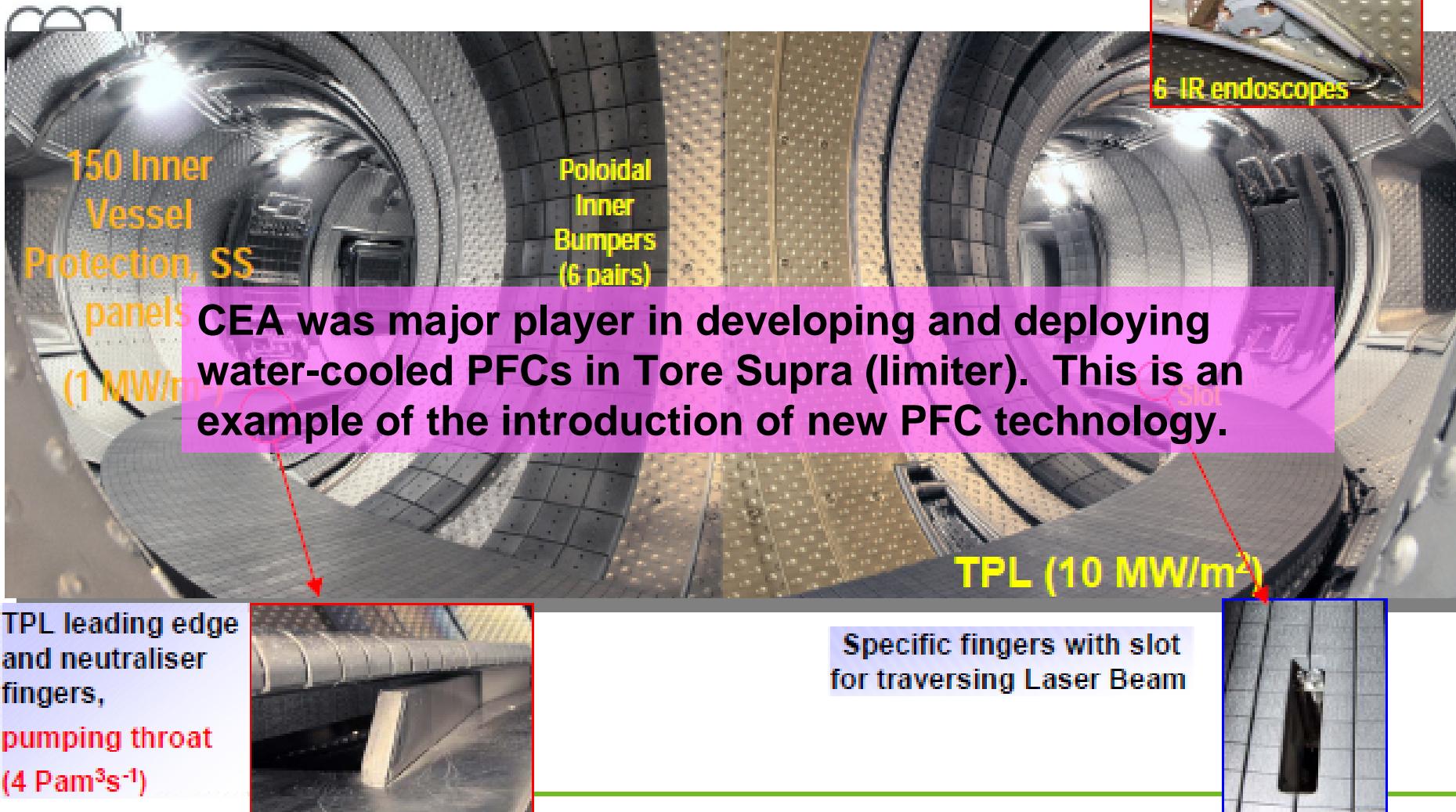
Modeling
Materials
alloys, damage

Modeling
Integration
FNSF, DEMO, reactor

benchmark
confirmations

FUSION
ENERGY

Tore Supra CIEL configuration (since 2002)



Development of water-cooled PFCs

Tore Supra water cooled PFCs

- modular limiters in 1990s failed
- very good history working closely with Plansee on fabrication
- yet still had quality problems
- rebuilt PFCs - CIEL completed 2002

Basic process cycle in development

✓ robust actively-cooled
HHFCs

- materials development
- component development

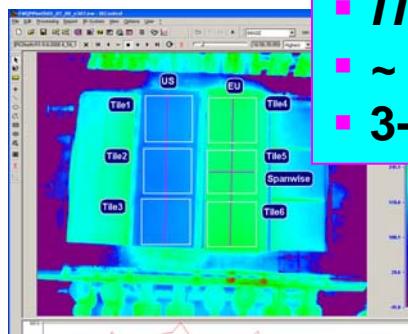
- design confirmation [modeling, testing]
- fabrication dev.
- QA & acceptance

Fundamental Point

HHFC R&D is challenging & time-consuming.

It requires strong coordination with confinement projects on interfaces and with industrial suppliers on fabrication development, QA and acceptance.

FWQM Testing Status
US & EU Mockups
Date: End of May 8, 2008
Cycles Completed: 3447



- ~25y - fusion-specific water-cooled heat sinks
- ~15y - ITER PFC R&D
- ~10y detailed R&D
- *ITER design changing*
- ~ 4y FWQ mockups
- 3-5y final design to fab

Development of He-cooled PFCs

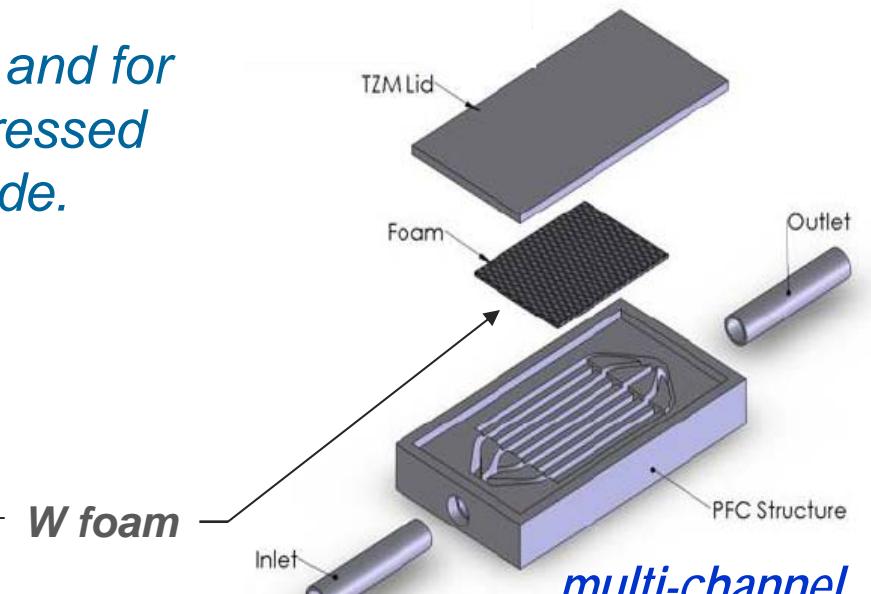
Start: Deploy He-cooled probes or guards for longer pulse operation.
Room temperature He is easier adaption of the technology (high density, mat = SS)

A strong well integrated HHFC program (near term) could enable new PFCs and IVCs for longer shots, higher power or hot walls in upgrades of current devices.

The technologies for heat pipes and for helium cooling have both progressed significantly in the last decade.



US He-cooled PFC target >20MW/m²



*multi-channel
He-cooled plate*

New PFCs as Enabling Tech.

PFC-1 Enabling Tech.

HHF In-Vessel Comp'ts (IVCs)

- RF launchers, armor
- RF mirrors
- Probes (mat'l, calorimeter, ..)

HHFC, divertor targets

- Strike point targets
- Li pumping/edge
- Pre-prototype deployment

“Hot wall” PFCs

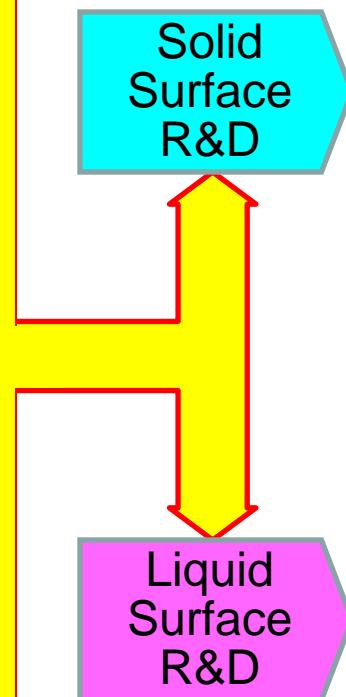
(heaters tough, as better)

- Protoype wall panel
- Hot divertor tiles
- Full wall/div deployment

Liquid Surface PFCs

- Pumping panels, LLD/LTX
- Divertor targets (heat load)
- Liq. walls (plumbing, MHD)

.. enabling R&D on current devices and upgrades that must precede an FNSF.



PFC-2 for FNFS/DEMO

Solid high temp. PFCs

- Divertor (integrated system)
- FW integral with blanket
- Probes (mat., calorimeter, ..)

HHF IVCs - 2

? AND/OR ?

PFC-3 for FNFS/DEMO

Liquid Surface PFCs

- Strike point targets
- Li pumping/edge
- Pre-prototype deployment

HHF IVCs - 3

PFC-1 Enabling Tech.

R&D on solid surface PFCs and liquid surface

PFCs are somewhat independent pathways.

HHF In-Vessel Comp'ts (IVCs)

- RF launcher

- RF mirrors

- Probes (mat'l, calorimeter, ..)

HHFC, divertor targets

- Strike point targets

- Li pumping/edge

- Pre-prototype deployment

The next section gives a brief overview of liquid surfaces and modeling (short to save time).

“Hot wall” PFCs

(heaters tough, as better)

- Prototype wall panel

- Hot divertor tiles

- Full wall/div deployment

Liquid Surface PFCs

- Pumping panels, LLD/LTX

- Divertor targets (heat load)

- Liq. walls (plumbing, MHD)

Solid Surface R&D

Liquid Surface R&D

Solid high temp. PFCs

- Divertor (integrated system)
- FW integral with blanket
- Probes (mat'l, calorimeter, ..)

HHF IVCs - 2

Liquid Surface PFCs

- Strike point targets
- Li pumping/edge
- Pre-prototype deployment

HHF IVCs - 3



Outline

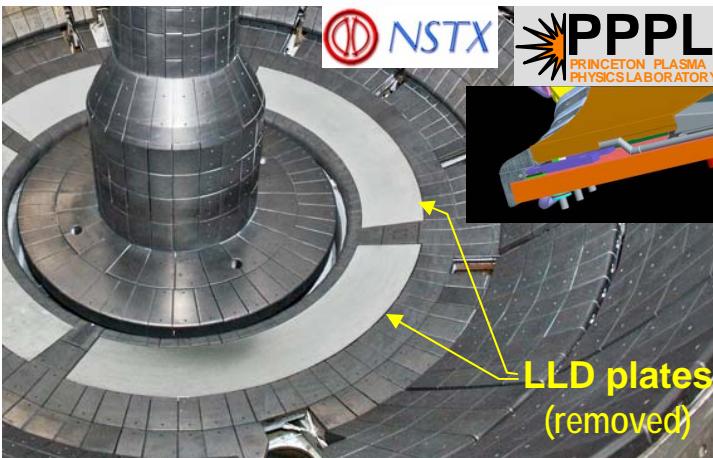
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Liquid Surfaces *Divertor/FW*

Experiments in
T-11M, CDXU/LTX



Li limiter in FTU



Application

- First wall $q'' < 5 \text{ MW/m}^2$
- Divertor $q'' > 5 \text{ MW/m}^2$

Material

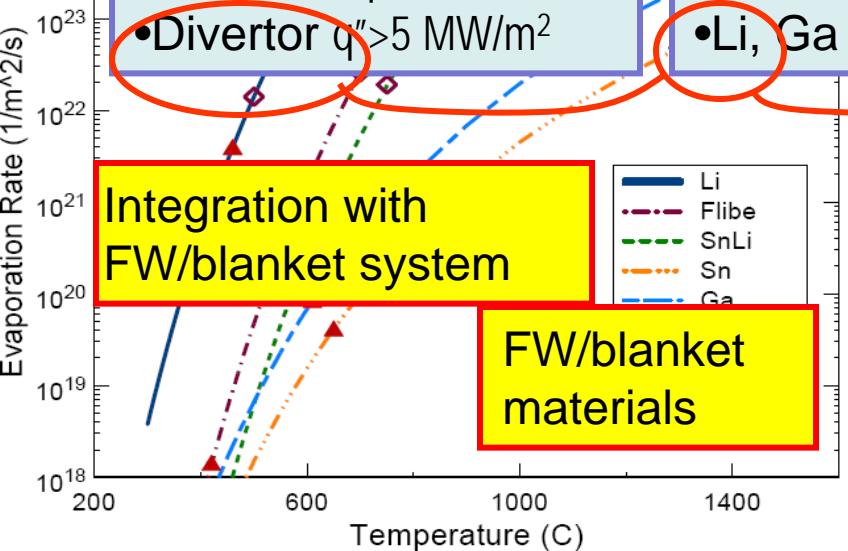
- FLiNaBe
- Li, Ga

Function

- Cooling
- Pumping

Speed/cooling

- fast, self cooled
- slow, on heat sink



$T_{\text{Li-surface}} - T_{\text{coolant}}$ too large
Excessive evaporation!
(unless there is some new regime)

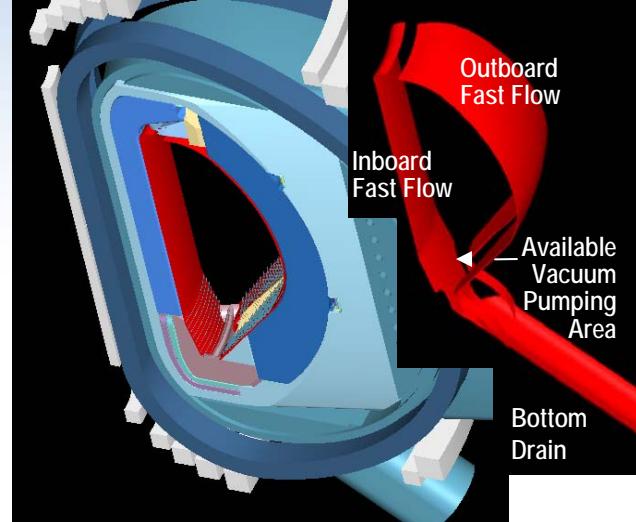
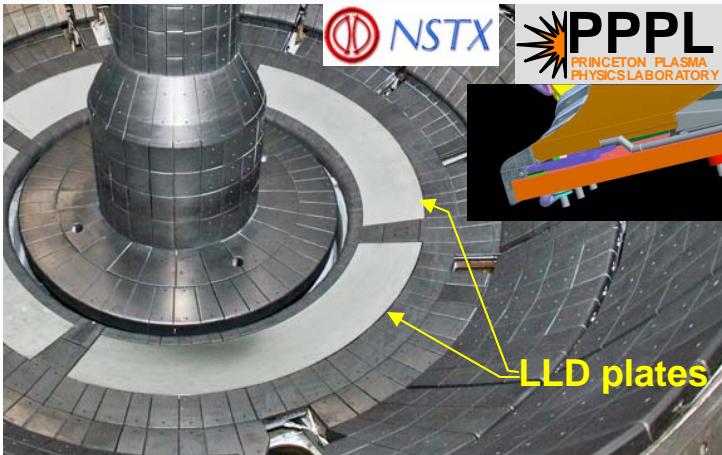


Liquid Surfaces *Divertor/FW*

Experiments in
T-11M, CDXU/LTX



Li limiter in FTU



CLIFF (Flowing FLiBe)

APEX design study
Prof. Abdou, UCLA, leader

Application

- First wall $q'' < 5 \text{ MW/m}^2$
- Divertor $q'' < 5 \text{ MW/m}^2$

Material

- FLiNaBe
- Li, Ga

Function

- Cooling
- Pumping

Speed/cooling

- fast, self cooled
- slow, or heat sink

Integration with
FW/blanket system

FW/blanket
materials

coolant system
LM MHD flows
pumping power
draining
corrosion
pumping ??Ga
exhaust ducts

High speed flow
difficult to achieve,
predict and control

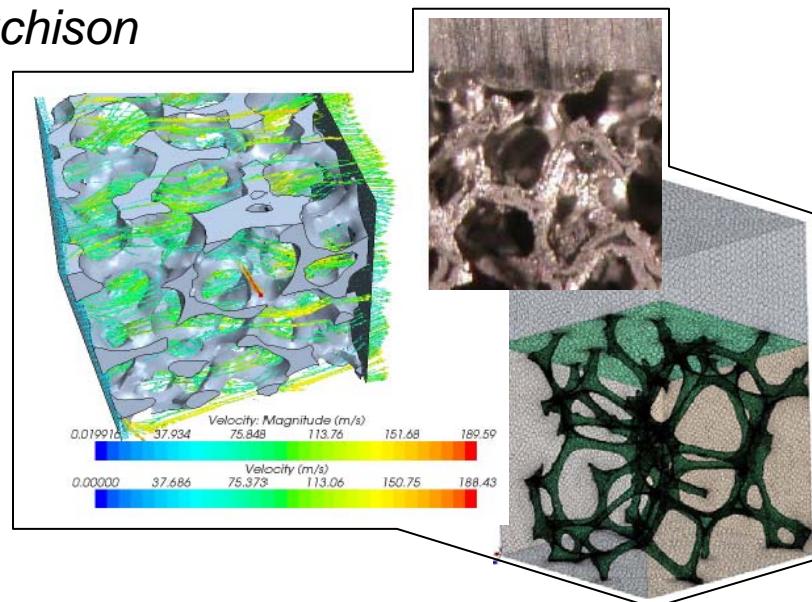
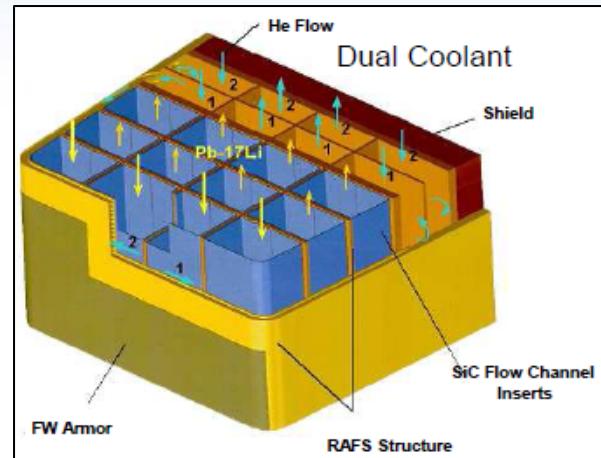
US Activities in modeling PFCs

PFCs

- All Metal ITER (IAEA paper) - Brooks *et al.*
- APEX studies – Abdou and APEX Team
- ARIES design studies – ARIES Team
- ITER TBM and TITAN activities – UCLA/GA/others
- Integrated Blanket Systems – Ying, Narula, Wang
- Thermal performance models of mockups
 - Youchison, Sharafat, Raffray
- Response to EM loads – Ulrickson, Youchison
- He flow through porous media (CFD)
 - Youchison

PSI – *covered elsewhere*

- Tritium retention in neutron damaged W
 - Wampler *et al.*
- Conditions for formation and erosion of tungsten “fuzz” – Doerner *et al.*
- Erosion and redeposition – Brooks *et al.*
- Erosion and vapor shielding of W PFCs
 - Hassanein
- Hydrogen on the surface of W – Kolasinski
- Hydrogen permeation in W – Causey *et al.*
- Other work

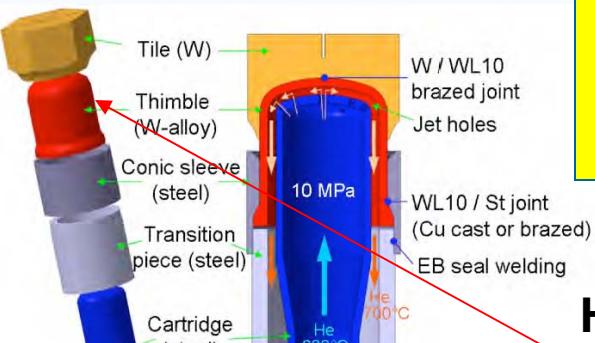
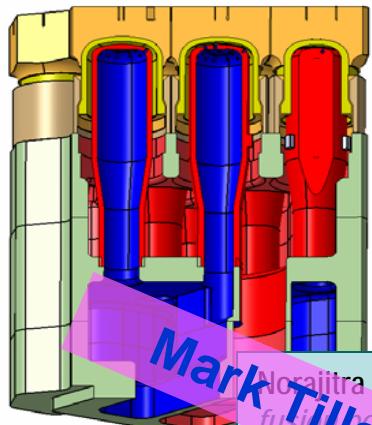




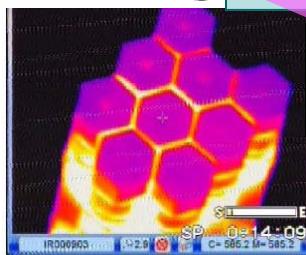
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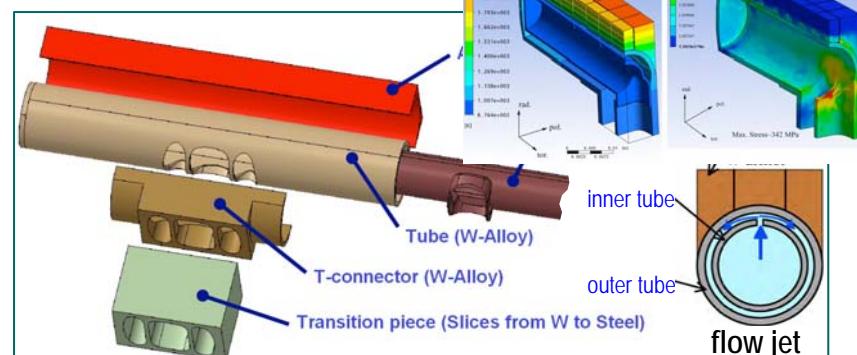
W PFC research – He-cooled W, e.g., HEJM in by EU



Norajitra et al., *Divertor conceptual designs for a fusion power plant*, FED 83 (7-9) (2008) 893-902.



IR imaging during testing,
P. Norajitra et al., FZK

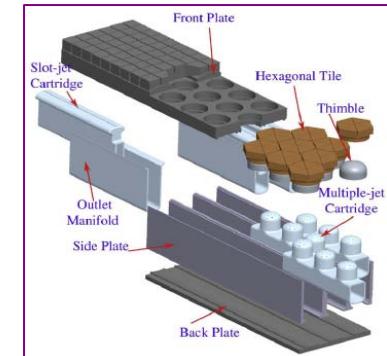


T-tube in ARIES-CS divertor - nice idea

HEMJ – component dev. aspects

- design iterations, high heat flux tests
- fabrication development
- commercial mat'l's + innovations

HEMJ Thimbles



- Deep drawing is suitable for mass production. Grain orientation of the W follows the contour.
- Dry machining does not contaminate surfaces for subsequent brazing and takes 1/4 work time.
- Finishing with a CBN grinding wheel (rather than EDM) eliminates fine surface cracking.

Reiser et al., *Development of a He-cooled Divertor: Technological Studies of Tungsten Machining*, SOFT08

PFC-PSI Development & Integration for W PFC

Nature of inquiry

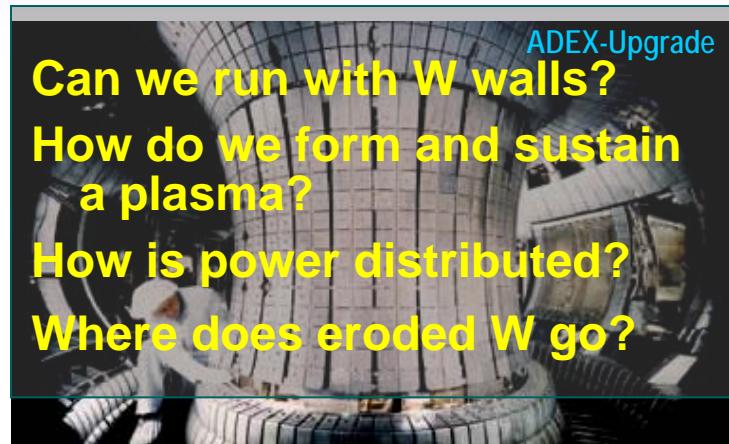
Physics in all metal devices

- **ASDEX-U** (divertor) →
- **Alcator C-MOD** (divertor)
- **Others: TRIAM-U** (limiter)
- **JET** with “ITER-like” divertor

R&D on W PFCs

- **JET ITER-like divertor**
(W monoblocks & coatings)
- **Performance of Mo & W tiles in C-MOD** →
- **Coatings in ASDEX-U**
- **W limiters in TEXTOR, HT-7**
- **Fabrication development**
- **Probe/tiles measurements** *PSI*
 - **W erosion in DIII-D and C-MOD** →
 - **D or He damage in TRIAM, LHD**

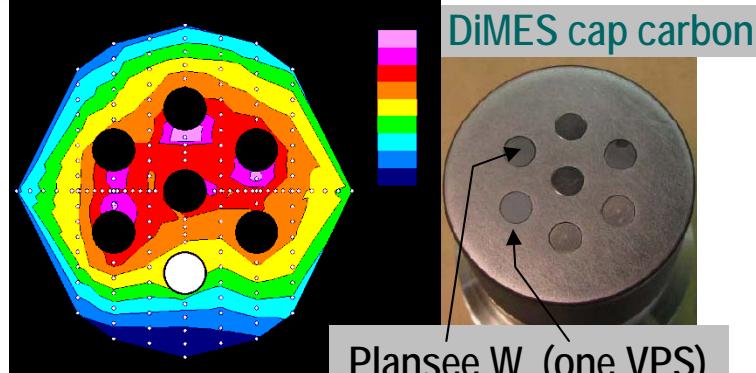
PSI R&D in labs (PISCES, NAGDIS, IBL, TPE ...) →



US Activities with W PFCs - erosion

W redeposition, DiMES div. probe

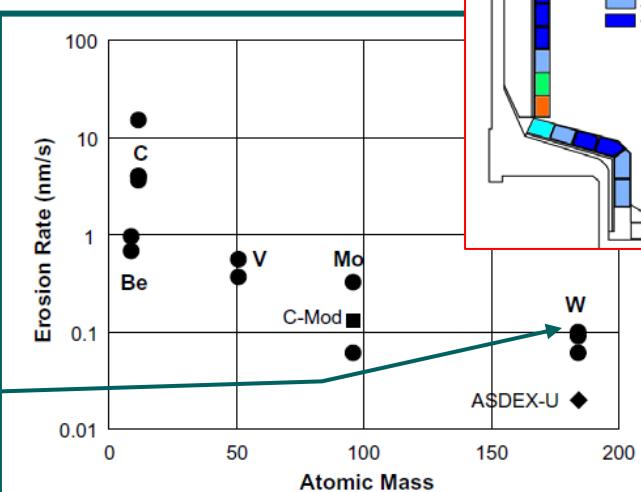
Courtesy of DIII-D (General Atomics) & DiMES Team



- RBS measurements (pre/post)
- W deposits mainly near source
- Average net W erosion rate 0.13 nm/s

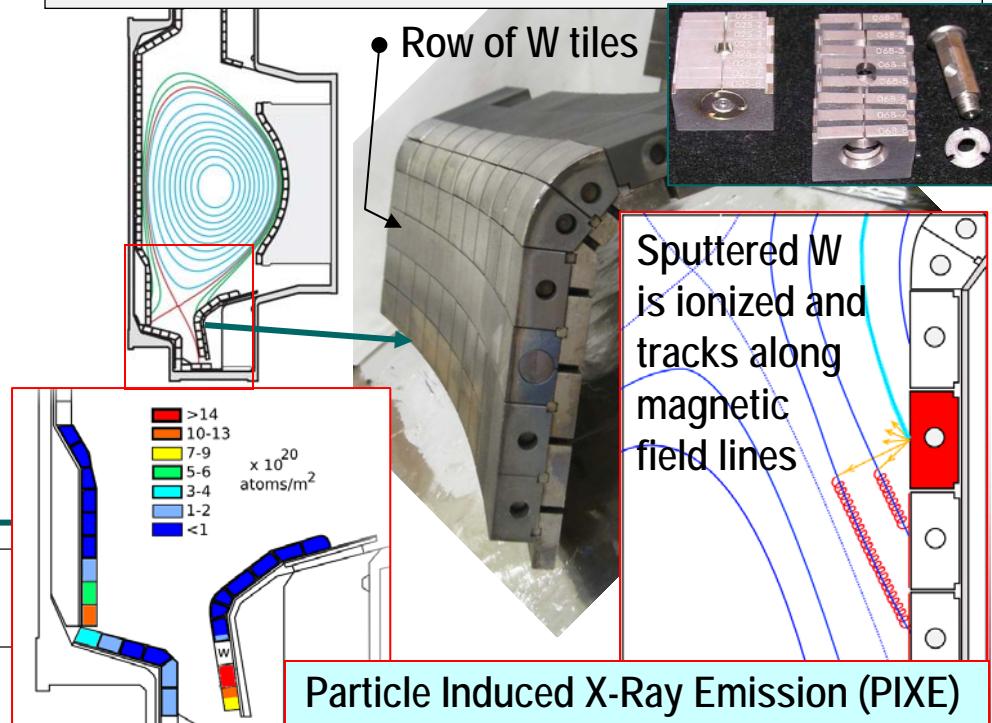
Fit with earlier measurements

W. Wampler,
Nucl. Instr. & Meth. in Phys. Res. B
219–220 (2004)



Section of Alcator C-Mod outer divertor

Courtesy of MIT Plasma Fusion Science Center



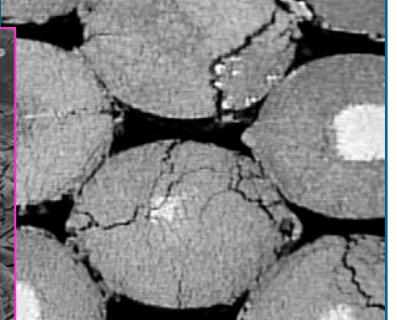
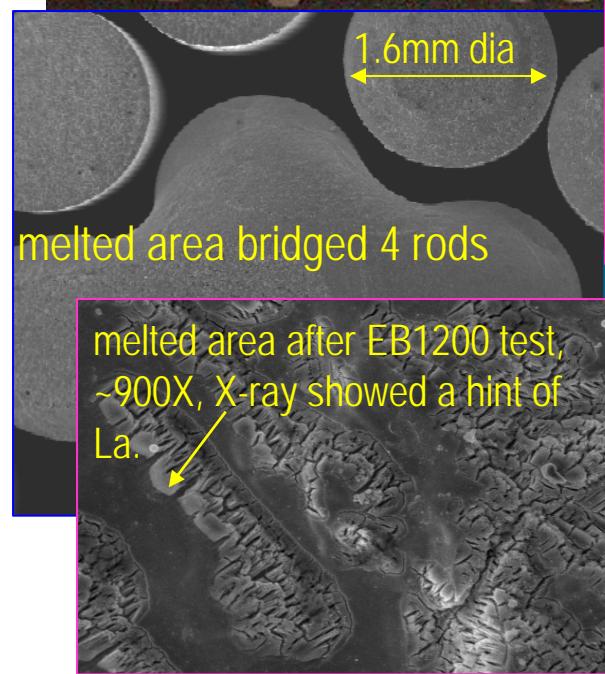
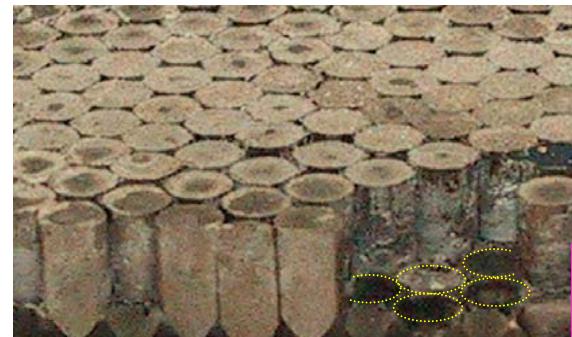
US Activities with W PFCs – R&D on W tiles

Alcator C-MOD tokamak at MIT's
Plasma Fusion Science Center



Sandia-Boeing
ITER W rod mockup

- rods in PS Cu bed
- bed HIPPed to CuCrZr
- method did not work.

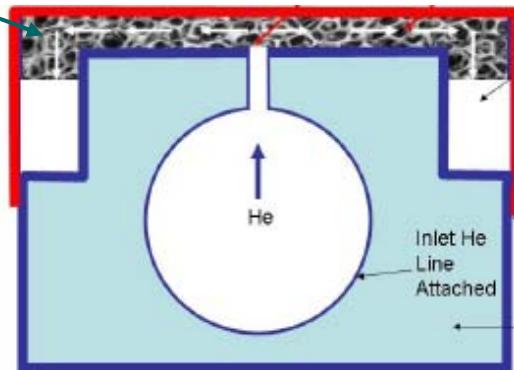
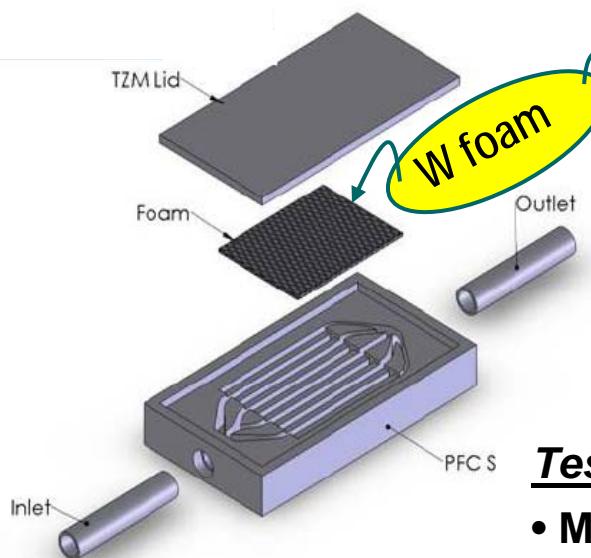


US Activities - He-cooled W mockups

Sandia tested He-cooled heat sinks since the 1990's.

In more recent tests:

- dual-channel He-cooled W heat sink, **Thermacore***, **34.6 MW/m²** (study channel-to-channel flow instability)
- 15-mm-diameter tubes with porous medium, integral structure by **Ultramet*** with CVD W, **22.4 MW/m²**
**done under DOE grants for Small Business Innovative Research*



Test in coming year

- Multiple channel (4)
- Flat surface
- All refractory
- Short flow paths
- 600 C inlet temps

Investigate:

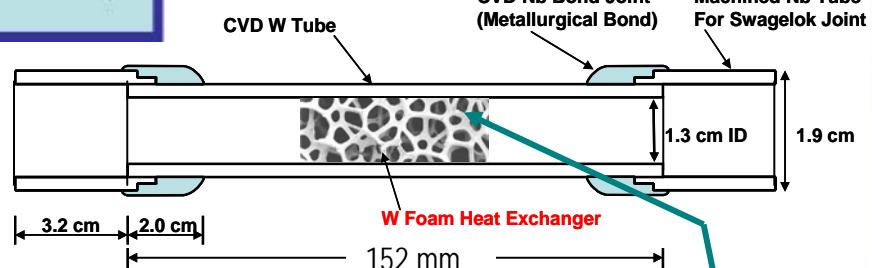
- Larger heated areas
- Flow instabilities



EB1200
(1.2MW)

E-beams in Sandia's
Plasma Materials Test Facility

new He
loop



carbonization of open-celled
foam then W CVD coating

Basic Elements for W PFC Development*

Development of W-based materials, including production, fab, joining and neutron damage

Investigation of PMI issues

Measurements of the plasma edge

Development and deployment of PFCs (confirmation of design, goals for performance, QA, high heat flux testing, ..)

Experiments with large areas of W PFCs; operation with hot walls.

Modeling of materials and the evolution and effects of damage

Modeling of the plasma edge

Benchmarking of edge and materials models

Modeling of component performance and integrated testing

Benchmarking of predictive performance models that integrate plasma edge, materials evolution and component performance

***Making Tungsten Work**

R Nygren ICFRM-14 (2009)

We can list basic tasks.

This is one way to summarize activities in a pathway.

But a list has no sequence implied.

And the tasks here cut across several of our topical areas.

Basic Elements for W PFC Development

Topical Area (Phys/PSI/PFC/MAT)

Development of W-based materials, including production, fab, joining and neutron damage

PFC *MAT*

Investigation of PMI issues

PSI

Measurements of the plasma edge

Physics *PSI*

Development and deployment of PFCs (confirmation of design, goals for performance, QA, high heat flux testing, ..)

PFC *MAT*

Experiments with large areas of W PFCs; operation with hot walls.

Physics *PSI* *PFC*

Modeling of materials and the evolution and effects of damage

PSI *MAT*

Modeling of the plasma edge

Physics *PSI*

Benchmarking of edge and materials models

Physics *PSI* *MAT*

Modeling of component performance and integrated testing

PSI *PFC*

Benchmarking of predictive performance models that integrate plasma edge, materials evolution and component performance

Physics *PSI* *PFC* *MAT*



Materials Issues for W PFCs

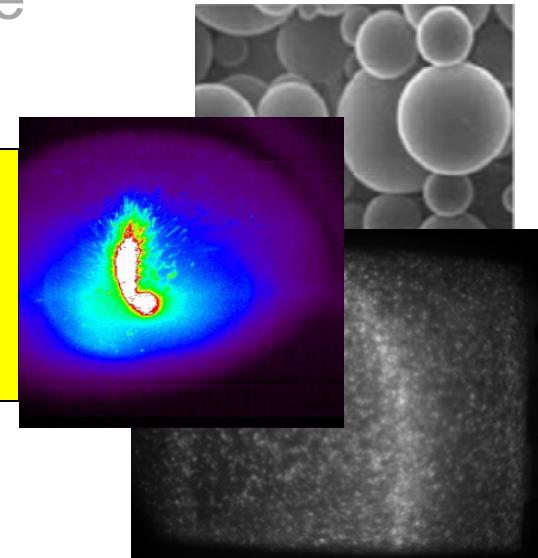
What improvements can we deliver?

- Lower DBTT
- Better machinability
- Mitigation of some neutron & ion damage
- ⋮
- ⋮ PSI , oxidation, ...
- *Reduced cost*
- *Weight*

Data for

- **Alloy selection**
- **Safety analyses, credible accidents, "off normal" events (strong drivers)**

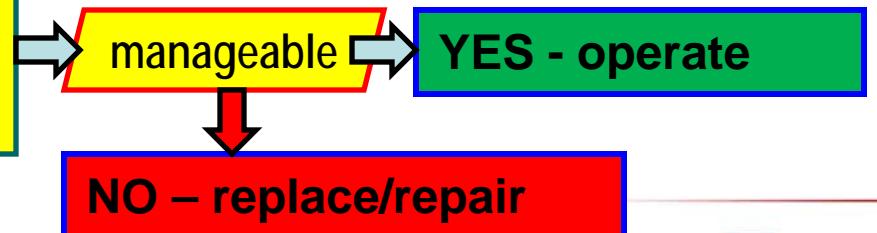
Rudikov (General Atomics),
Dust Study: loose dust mobilized
(? electrical charging) and prevented
discharge. *MIT PFC Meeting, 2009*



What are intrinsic limitations?

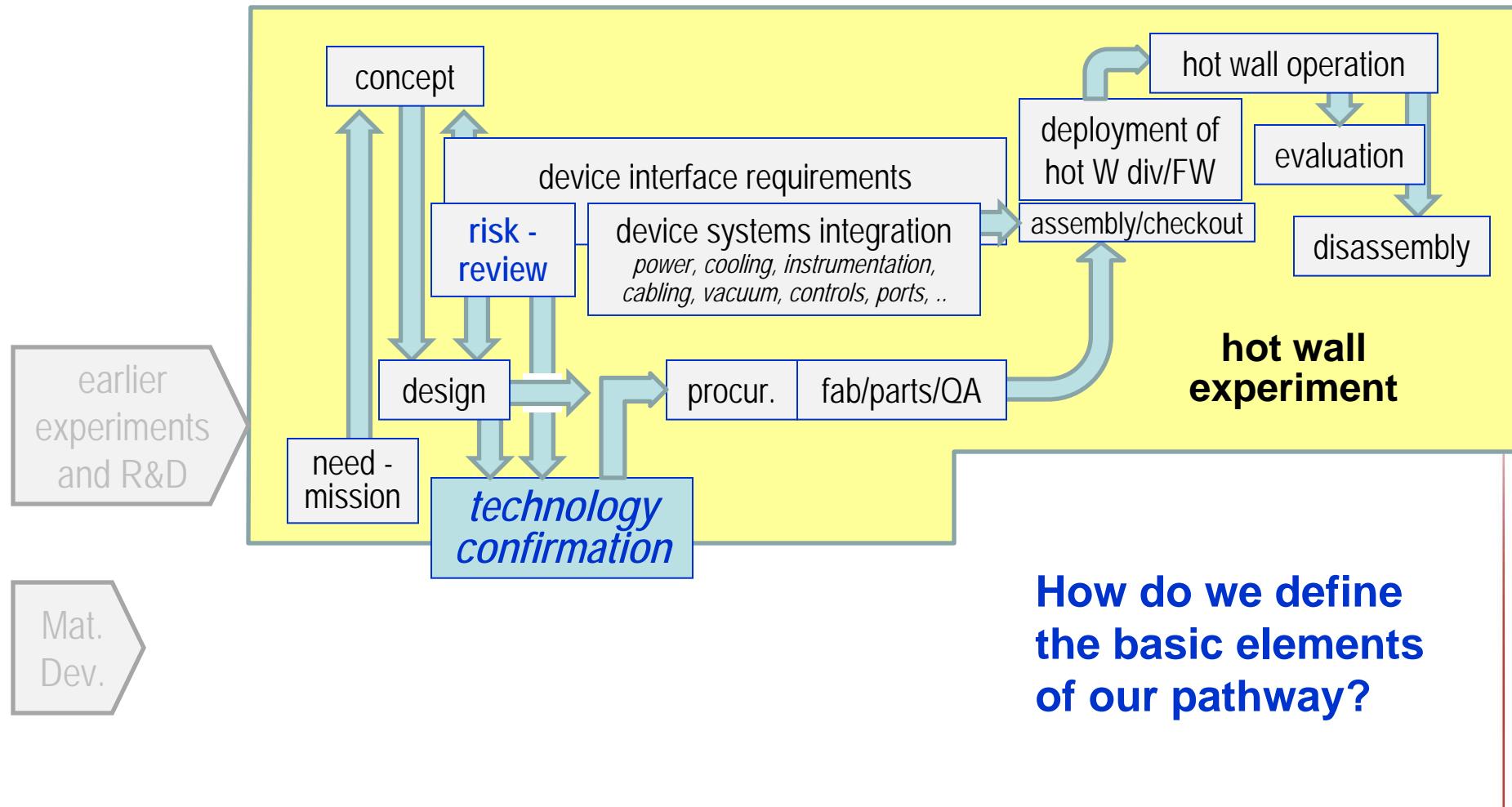
- Defects frozen in cascades "black spots"
- **Cracking**
- ?Other

- *base material*
- *evolving structure*
- *melt layer*



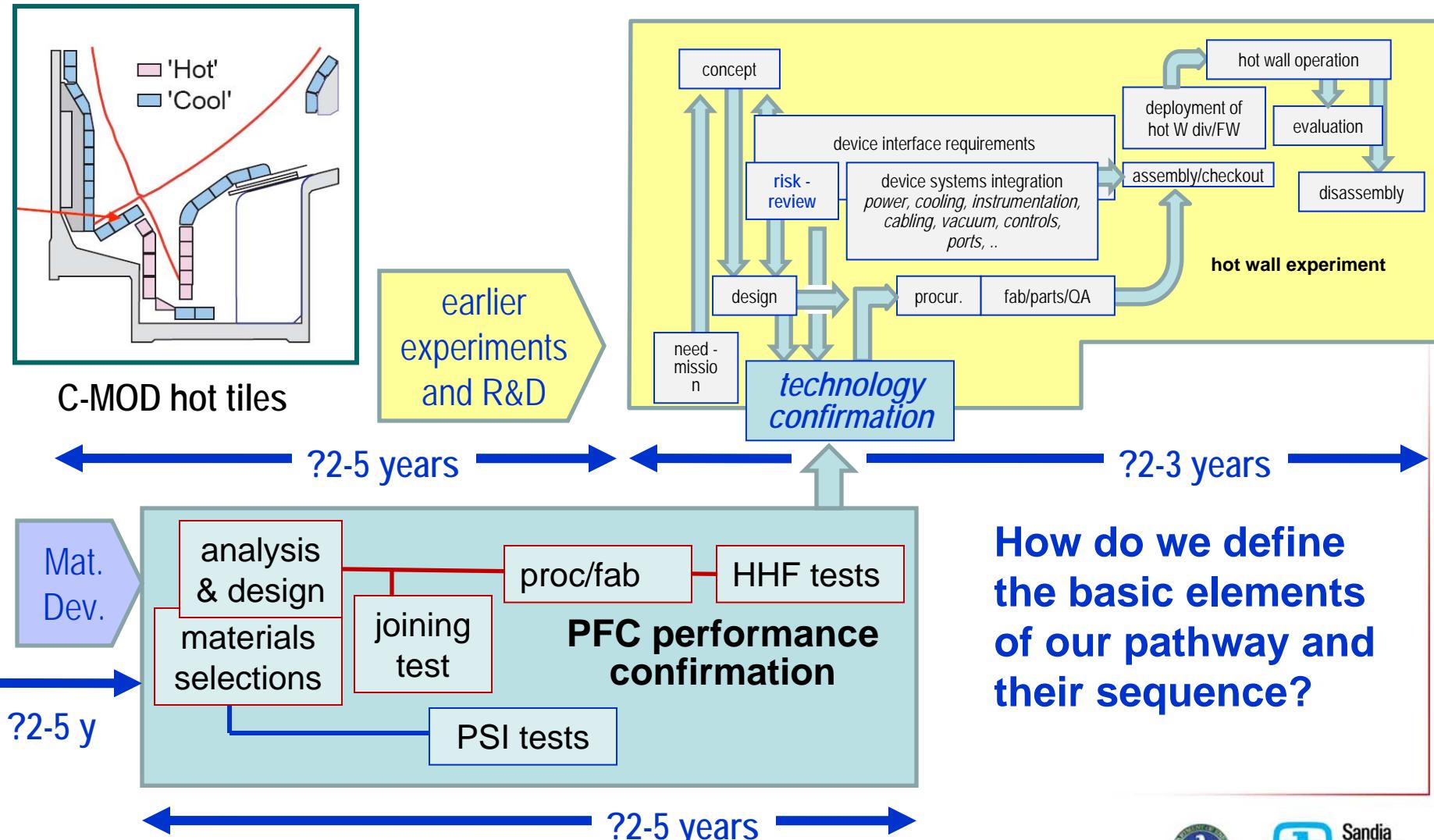
Hot Wall Experiment – large area of hot PFCs

gas-heated refractory heat sinks, tungsten armor
(control of wall temperature separate from power, no electrical heaters)



Hot Wall Experiment – large area of hot PFCs

gas-heated refractory heat sinks, tungsten armor
(control of wall temperature separate from power, no electrical heaters)





How do we define the basic elements of our pathway and their sequence?

E ND