

*FNS Pathways Core Group
UCLA -- Jan. 24-25, 2011*

Plasma Facing Components

- Divertors
- First Wall
- In-vessel Components
 - RF launchers, armor
 - RF mirrors
 - large probes (mat'ls, calorimetry, ..)

Presented by Dennis Youchison (Sandia)

*prepared by Richard Nygren (Sandia)
based on past input from many people*



Plasma Facing Components (FNS Pathway)

Outline

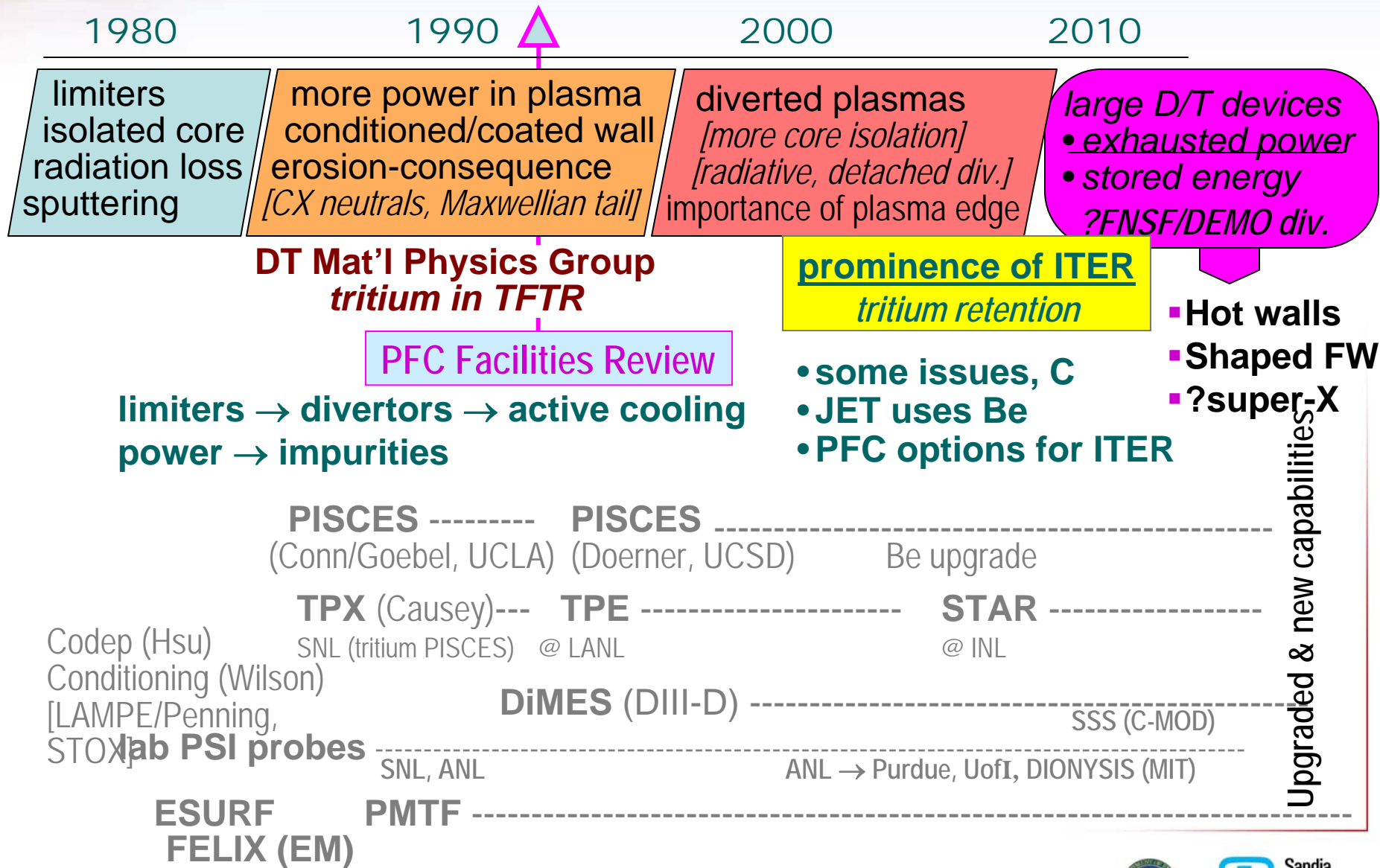
1. Status of US PFC program
2. Views of gaps and issues
3. Importance of sequence in the FNS Pathway
4. Brief overviews:
 - liquid surface PFCs
 - modeling
5. Example of R&D program for W PFCs
 - nature of the “bits” or elements in the FNS Pathway

Main theme today:

What kinds of
elements and
sequences
will we use to
construct our
pathway?

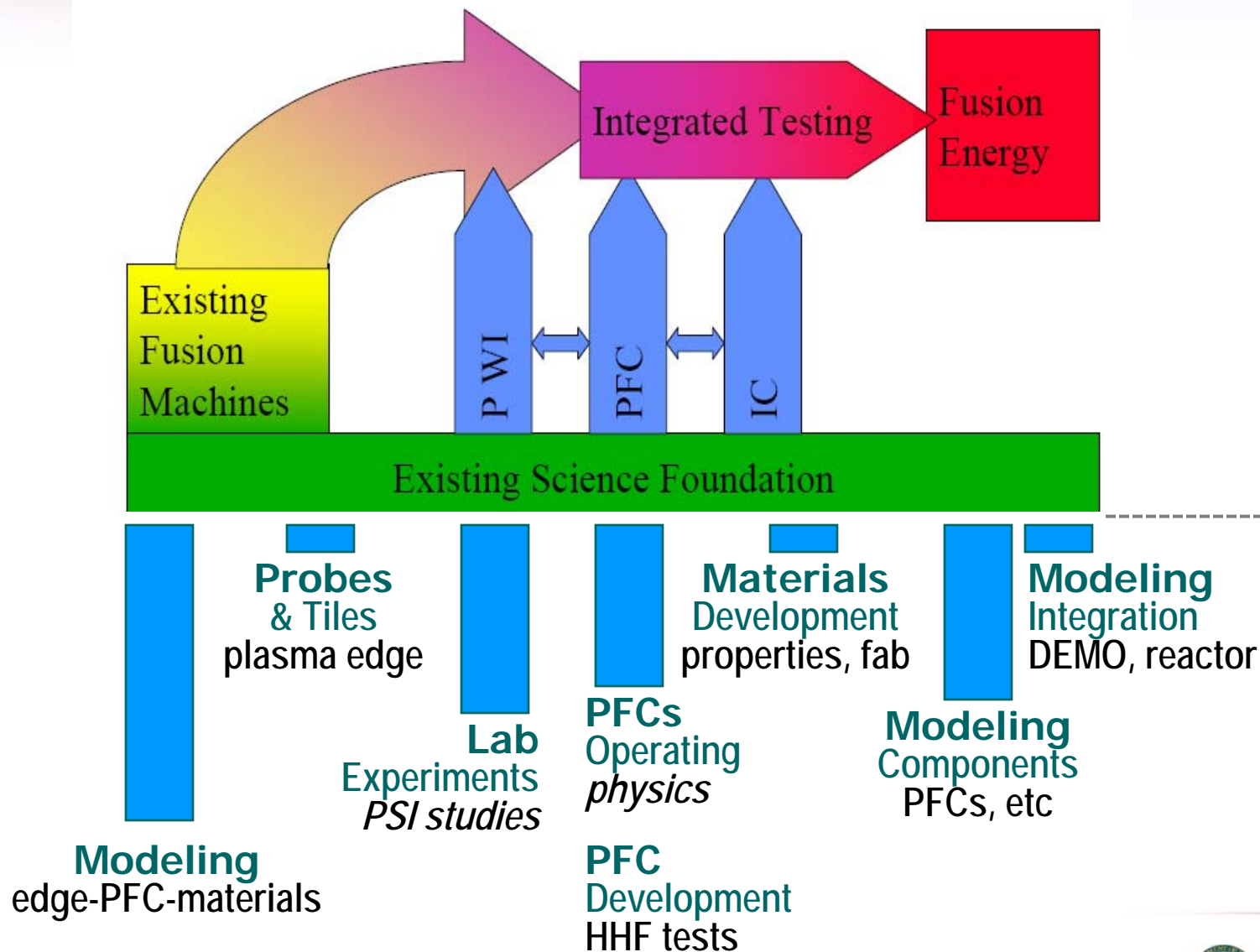
3. Importance of sequence in the FNS Pathway
5. Example of R&D program for W PFCs
(elements in the FNS Pathway)

Historical Perspective on PFC/PMI Development



PFC-PSI Development & Integration

PFC program elements

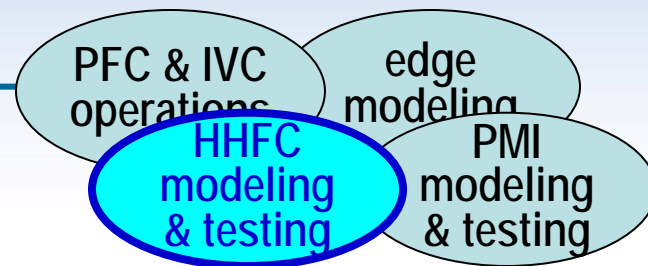


Upper portion of figure by Ulrickson from ReNeW

Program elements added by Nygren

Status of HHFC Program

Excluding PMI and edge programs



- #1** Support ITER PFC design & R&D and develop component fabrication processes, QA, and operation.
excellent relations with IPO, IP, DAs; US R&D & testing (for all DAs) ongoing; US role in design expanding; valuable insight into design/machine interfaces

gap: test capabilities (old & frail); design integration & interfaces; participation in divertor R&D

- 3.** Develop and prove robust PFCs for future confinement devices.

limited but sustained work on He cooled W and on liquid surfaces

gap: expanded test capabilities; stronger integrated modeling; test capabilities (He, liq. met.)

- 2.** Support physics missions of existing, upgraded and new US confinement experiments.

needs but limited R&D, some testing for C-MOD and DIII-D (NSTX Liquid Li divertor and DiMES are really PMI)

gap: program organization; integration with machines; test capabilities (He, probes, disruption simulation)



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nature of the “bits” or elements in the FNS Pathway

"Greenwald" Panel Report

Recommendation 4. nine major initiatives.

- I-1. .. predictive plasma modeling and validation ..**
- I-2. Extensions to ITER AT capabilities .. burning AT regimes**
- I-3. Integrated advanced burning physics ...facility .. dedicated**
- I-5. .. disruption-free concepts .. performance extension device ..**
- I-4. Integrated experiment for PWI/PFCs .. steady-state .. non-DT**
- I-6. .. advanced computer modeling and laboratory testing ..
single-effects science for major fusion technology issues,**
- I-8. Component development/testing program ... multi-effect
issues in critical technology .. breeding/blanket .. first wall**
- I-7 Materials qualification facility ... (IFMIF).**
- I-9. Component qualification facility.. high availability.. heat
flux .. neutron fluence .. DT device (CTF).**

FNSTF Core Group Homework: List of Generic FNST R&D

Richard Nygren and Stan Milora 8 sep 2010

- Refs:
1. Presentation by Rajesh Maingi at PMI Test Stand Workshop, ORNL, Aug 30 – Sep 2, 2010
 2. Draft “Generic FNST R&D – VLT, August 2010” (developed by R. Maingi and M. Peng, Aug 2010)
 3. Draft “ReNeW: Technology Content, Thrusts 1-18” (developed by R. Nygren, 4 June 2009)

All thrusts target elements in PWI, PFC, and IC

9. Unfolding the physics on the boundary layer plasma
 - SOL, divertor, private flux, pedestal; innovative divertors
 - Develop predictive capability, new diagnostics
10. Decode and advance the science and technology of plasma surface interactions
 - Well-controlled and well-diagnosed dedicated facilities
11. Improved power handling through being innovations
 - New solid and liquid concepts; innovative heat sinks
12. Demonstrate an integrated solution for plasma-material interfaces compatible with an optimized core plasma
 - Integrated test of PFC and IC components + PMI modeling
 - Low activation environment with improved diagnostics
13. ~ “Fusion nuclear science” (paraphrased)
14. ~ “Materials science” (paraphrased)

Several “lists” of R&D topics for FNST have been developed and distributed to a limited group. To start the FNSF Core Group, this list was developed from the references above. The flow follows that of an excellent presentation (Ref. 1) with pages 4, 6, 8, 9 and 11 from Ref. 2 added to give more specifics. Some final comments (p13) are also added based on Ref. 3.

**T9: Unfolding
the physics on**

From FNSP Summary of ReNeW Themes 9-14
Nygren, Milora, Peng

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From FNSP Summary of ReNeW Themes 9-14
Nygren, Milora, Peng



T14: Materials Science and Technology Needed to Harness Fusion Power

- Improve the performance of existing and near-term materials, while also developing the next generation of materials with revolutionary properties.

T13: Establish the Science and Technology for Fusion

Power Extraction and Tritium Sustainability

Nygren, Milora, Peng

T12: Demonstrate an integrated solution for plasma-material interfaces compatible with an optimized core plasma

Form fundamental research to establish the scientific parameters necessary to address the issues. An example activity is the exploration of plasma chemistry, heat transfer, and magnetic field interactions in lithium-cooled liquid metal coolants.

- **Develop design options for a new moderate-scale facility with a DEMO relevant boundary, to assess core-edge interaction issues and solutions.** Key desired features include high power density, sufficient pulse length and duty cycle, elevated wall temperature, as well as steady-state control of an optimized core plasma. Design for a hydrogen and deuterium fuel environment, to assure flexibility in changing boundary components as well as access for comprehensive measurements to fully characterize the boundary plasma and plasma-facing surfaces. The balance and sequencing of hydrogen and deuterium operation should be part of the design optimization. **Develop an accurate cost and schedule for this facility, and construct it.**
- Extend and validate transient heat flux control from Thrust 2, plasma control and sustainment from Thrust 5, boundary plasma models from Thrust 9, plasma-material interaction science from Thrust 10 and plasma-facing component technology from Thrust 11 with the new research from this facility, thereby demonstrating a viable solution to the very challenging core edge integration problem for DEMO.

combined impact of typical of a fusion burning plasma as a test extraction experiments designs, and operating

complete effect of fusion could be construction (FNSF) to perform from the effects of me in concert with all

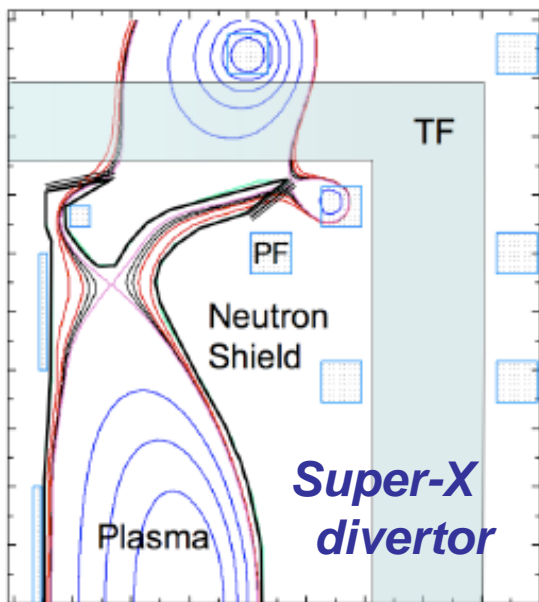
models necessary to collect reliability and



Some concerns for HHFCs

Richard's Rules

1. Manageable heat loads based on
 - a) reasonable physics, including startup, shutdown and plasma transients and
 - b) sound engineering practices.
2. Credible integration of systems.
3. Recognition of credible development paths and recommendations on which path must be taken.



(for example, engineer's view of Super-X)

Idea is intriguing; need experiments, e.g., MAST.

Proof of physics will include information about how T_e and T_i decrease from X-point to divertor.

[Important for PSI effects along lengthy throat.]

What about requirements on plasma control, restoring position promptly, etc?

Will an adequate blanket fit with a super-X divertor?

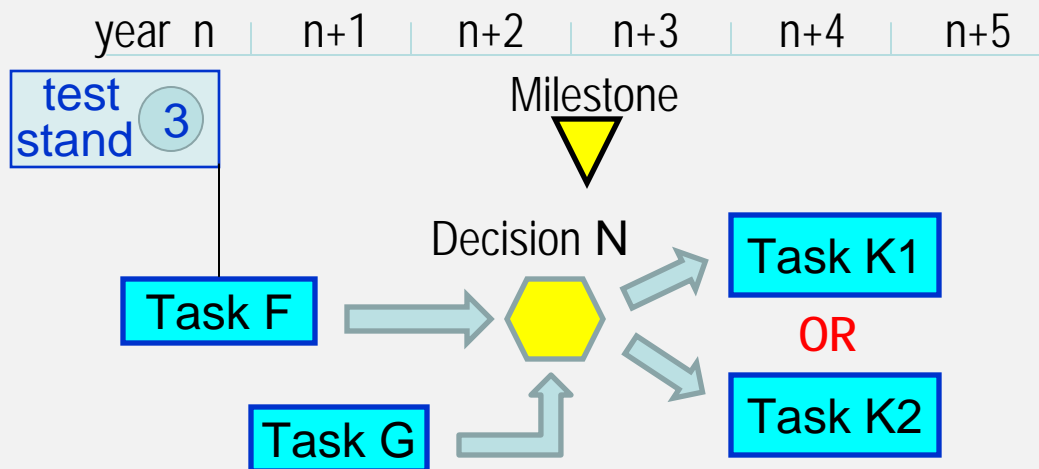
[You do not know until you do detailed engineering.]

We want to address the high level gaps and issues with some suite of options for tasks, sequences and required facilities.

At what level of detail should we be working now?

What supporting detail is needed in the future to confirm the credibility of the pathway analysis?

Main point:
What kinds of elements and sequences will we use to construct our pathway?



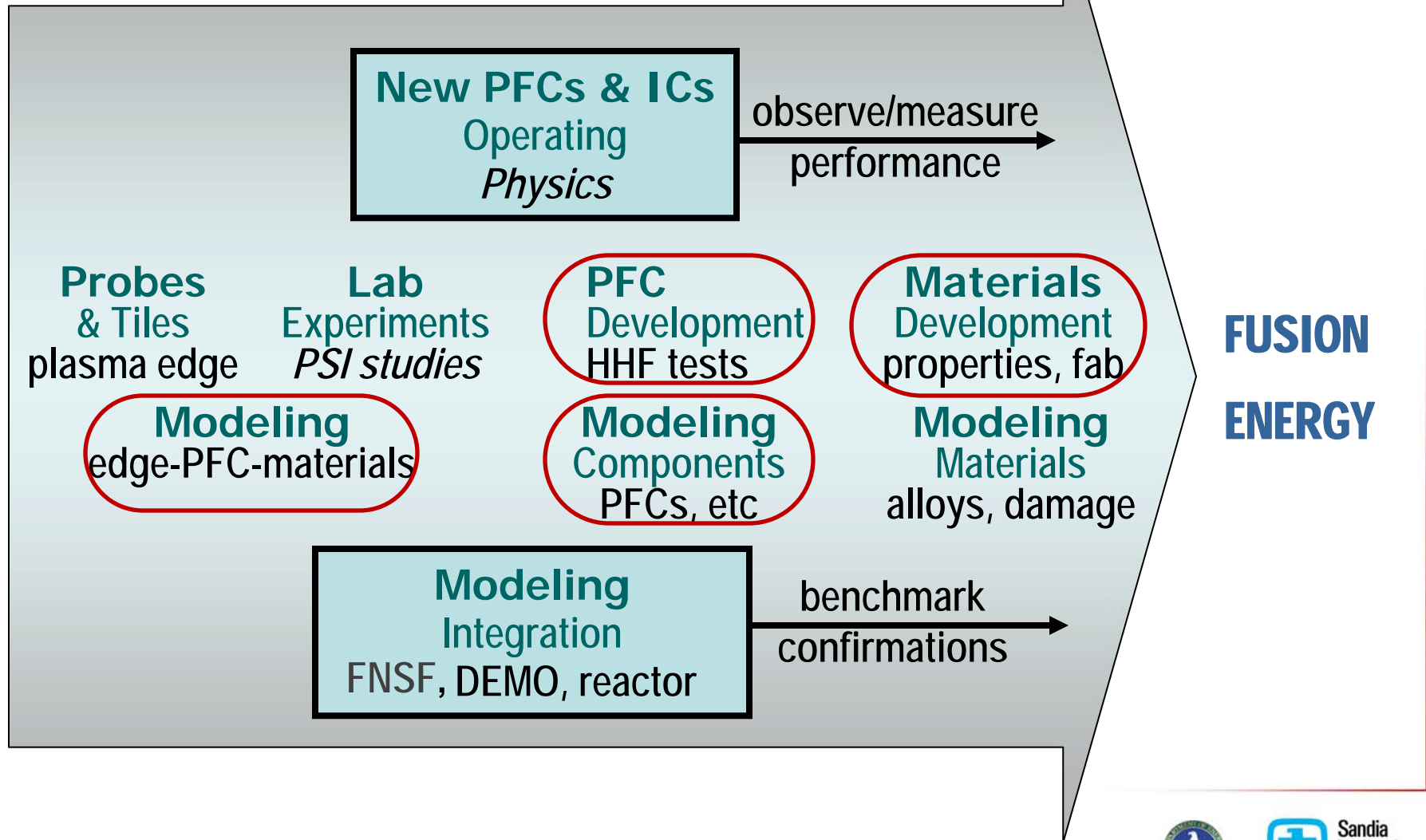
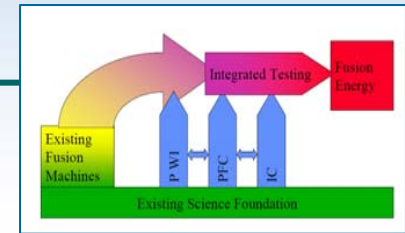


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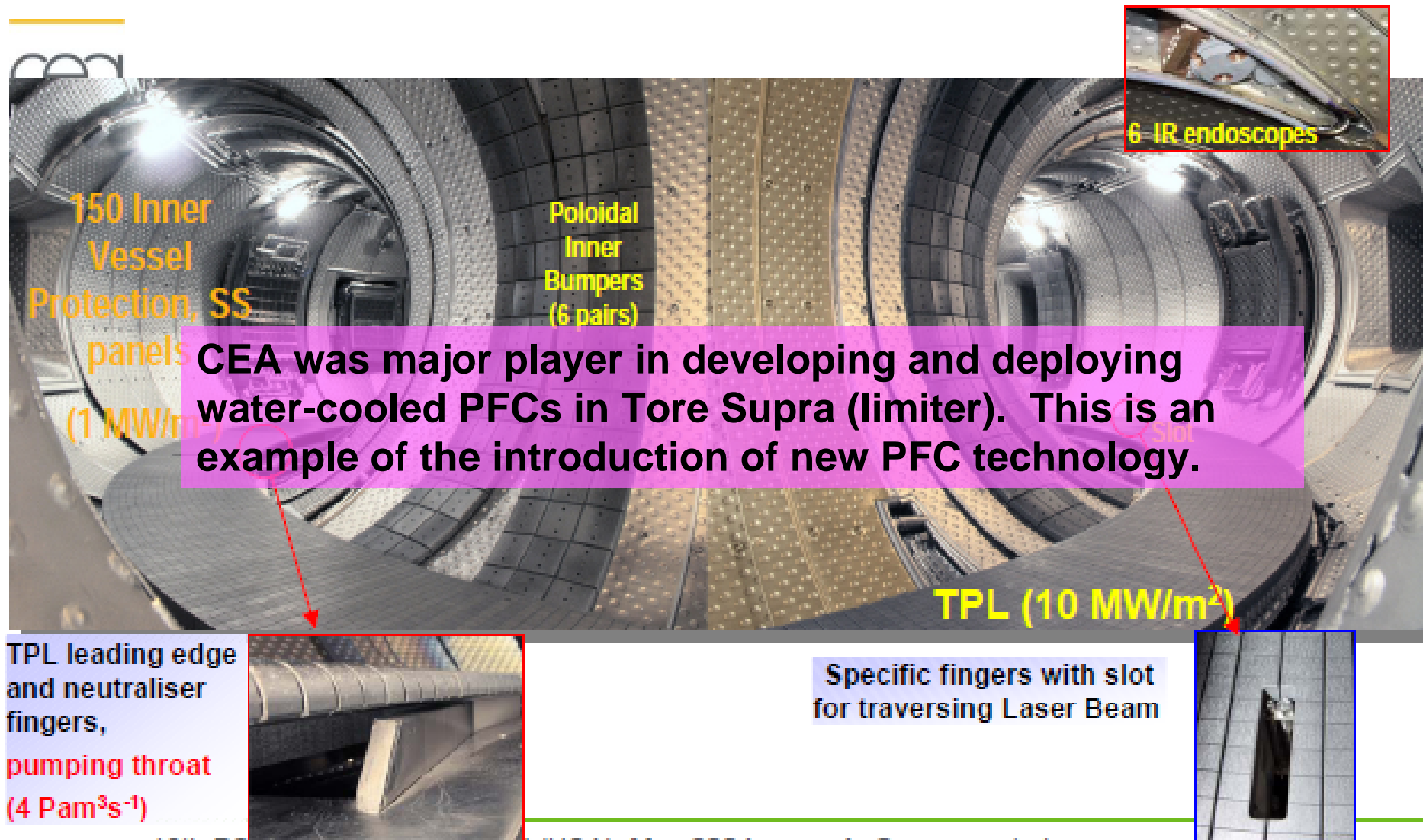
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PFC-PSI Development & Integration

Look at development of water-cooled PFC technology
heat sink technology, big impact on device operation



Tore Supra CIEL configuration (since 2002)



Development of water-cooled PFCs

Tore Supra water cooled PFCs

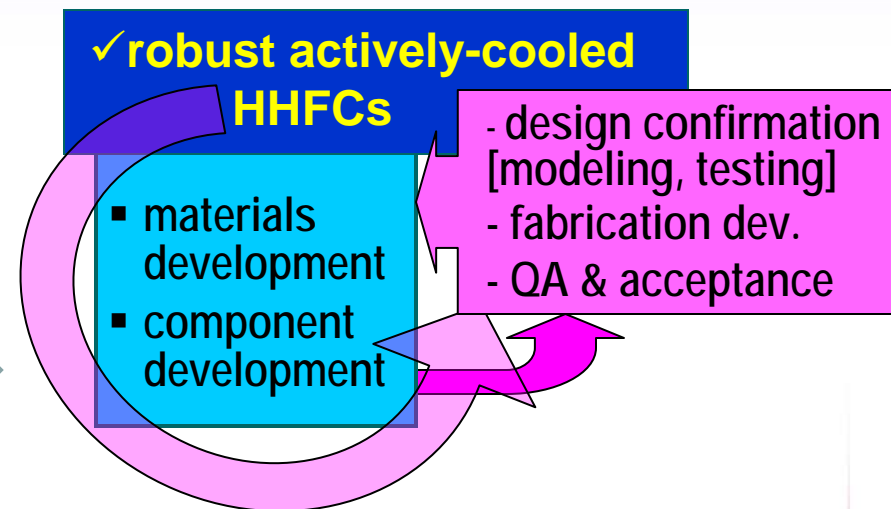
- modular limiters in 1990s failed
- very good history working closely with Plansee on fabrication
- yet still had quality problems
- rebuilt PFCs - CIEL completed 2002

Basic process cycle in development

Fundamental Point

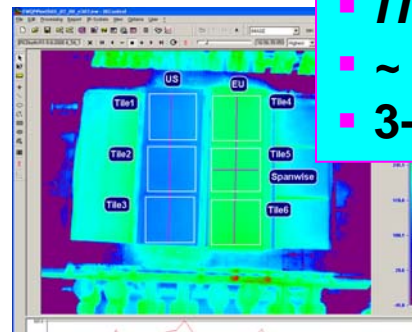
HHFC R&D is challenging & time-consuming.

It requires strong coordination with confinement projects on interfaces and with industrial suppliers on fabrication development, QA and acceptance.



- ~25y - fusion-specific water-cooled heat sinks
- ~15y - ITER PFC R&D
- ~10y detailed R&D
- *ITER design changing*
- ~ 4y FWQ mockups
- 3-5y final design to fab

FWQM Testing Status
US & EU Mockups
Date: End of May 8, 2008
Cycles Completed: 3447



Development of He-cooled PFCs

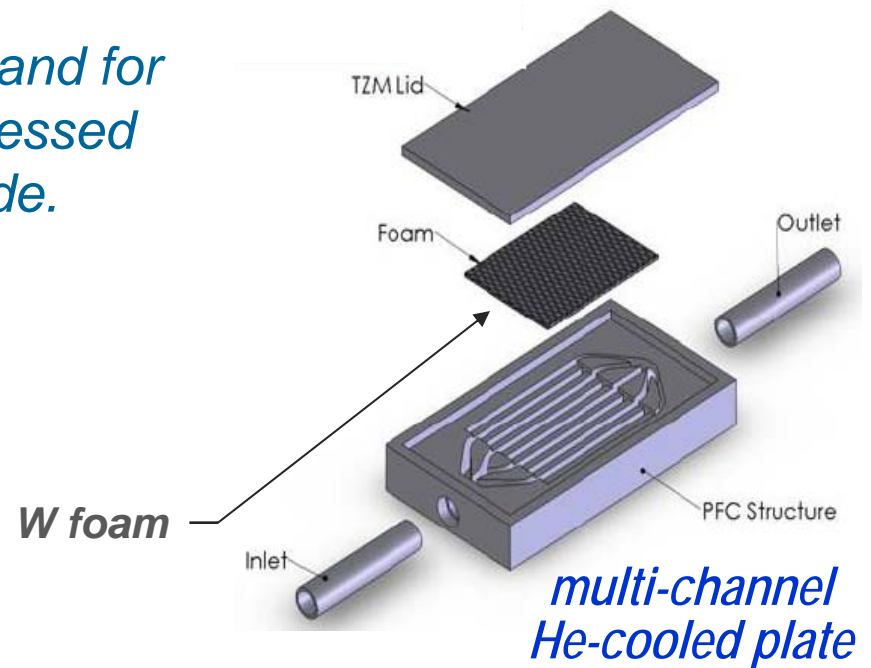
Start: Deploy He-cooled probes or guards for longer pulse operation.
Room temperature He is easier adaption of the technology (high density, mat = SS)

A strong well integrated HHFC program (near term) could enable new PFCs and IVCs for longer shots, higher power or hot walls in upgrades of current devices.

The technologies for heat pipes and for helium cooling have both progressed significantly in the last decade.



US He-cooled PFC target $>20\text{MW}/\text{m}^2$



New PFCs as Enabling Tech.

.. enabling R&D on current devices and upgrades that must precede an FNSF.

PFC-1 Enabling Tech.

HHF In-Vessel Comp'ts (IVCs)

- RF launchers, armor
- RF mirrors
- Probes (mat'l, calorimeter, ..)

HHFC, divertor targets

- Strike point targets
- Li pumping/edge
- Pre-prototype deployment

“Hot wall” PFCs

(heaters tough, as better)

- Prototype wall panel
- Hot divertor tiles
- Full wall/div deployment

Liquid Surface PFCs

- Pumping panels, LLD/LTX
- Divertor targets (heat load)
- Liq. walls (plumbing, MHD)

Solid
Surface
R&D

Liquid
Surface
R&D

PFC-2 for FNFS/DEMO

Solid high temp. PFCs

- Divertor (integrated system)
- FW integral with blanket
- Probes (mat., calorimeter, ..)

HHF IVCs - 2

? AND/OR ?

PFC-3 for FNFS/DEMO

Liquid Surface PFCs

- Strike point targets
- Li pumping/edge
- Pre-prototype deployment

HHF IVCs - 3

R&D on solid surface PFCs and liquid surface PFCs are somewhat independent pathways.

Both are important because we do not have a confirmed solution for FNSF and DEMO PFCs.

Solid
Surface
R&D

Solid high temp. PFCs

- Divertor (integrated system)
- FW integral with blanket
- Probes (mat'l, calorimeter, ..)

The next section gives a brief overview of liquid surfaces and modeling (short to save time).

The last section uses development of W PFCs as an example to illustrate some questions about pathways.

Liquid
Surface
R&D

Liquid Surface PFCs

- Strike point targets
- Li pumping/edge
- Pre-prototype deployment

HHF IVCs - 3

PFC-1 Enabling Tech.

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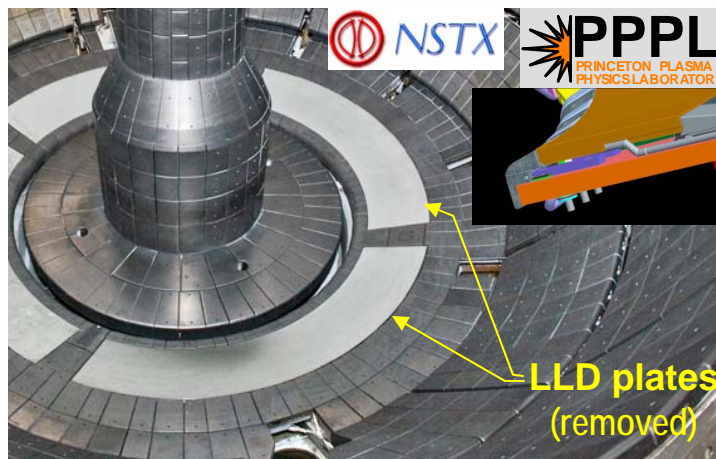
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Liquid Surfaces *Divertor/FW*

Experiments in
T-11M, CDXU/LTX



Li limiter in FTU

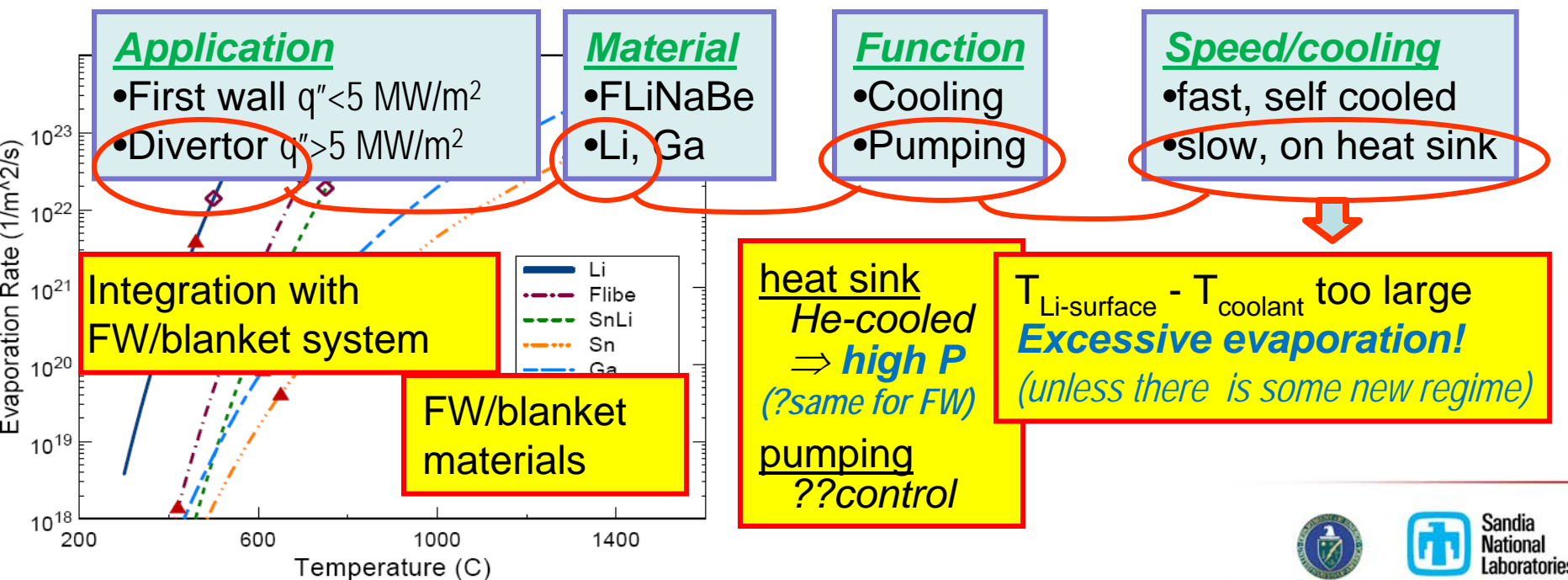


Liquid surfaces

- Li limiter (T-11M, FTU, CDXU/LTX)
- Liquid Li divertor (NSTX)
mostly tests for pumping

Other ideas proposed

- capillary-supplied Li
- flowing Li (modified edge)
- Sn, Ga, ...

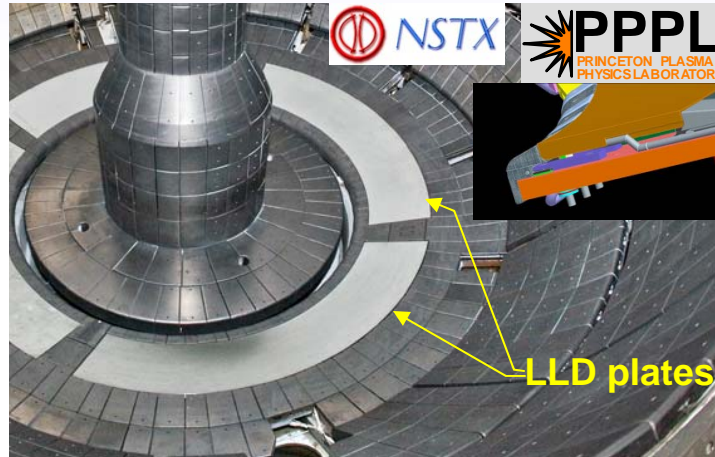


Liquid Surfaces *Divertor/FW*

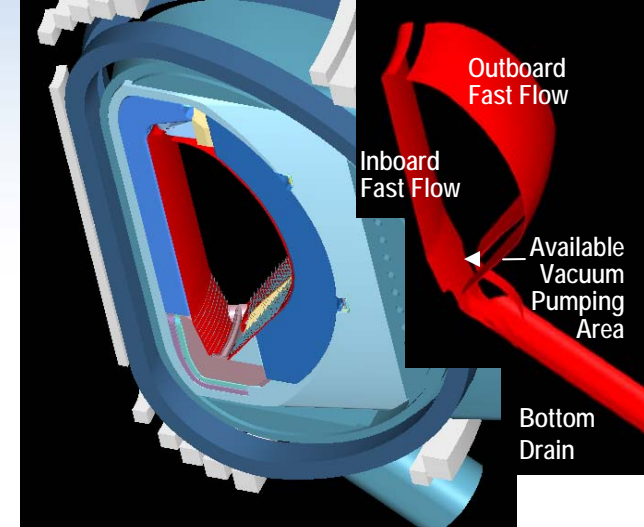
Experiments in
T-11M, CDXU/LTX



Li limiter in FTU



LLD plates



CLIFF (Flowing FLiBe)

APEX design study
Prof. Abdou, UCLA, leader

Application

- First wall $q'' < 5 \text{ MW/m}^2$
- Divertor $q'' < 5 \text{ MW/m}^2$

Material

- FLiNaBe
- Li, Ga

Function

- Cooling
- Pumping

Speed/cooling

- fast, self cooled
- slow, or heat sink

Integration with
FW/blanket system

FW/blanket
materials

coolant system
LM MHD flows
pumping power
draining
corrosion
pumping ??Ga
exhaust ducts

High speed flow
difficult to achieve,
predict and control

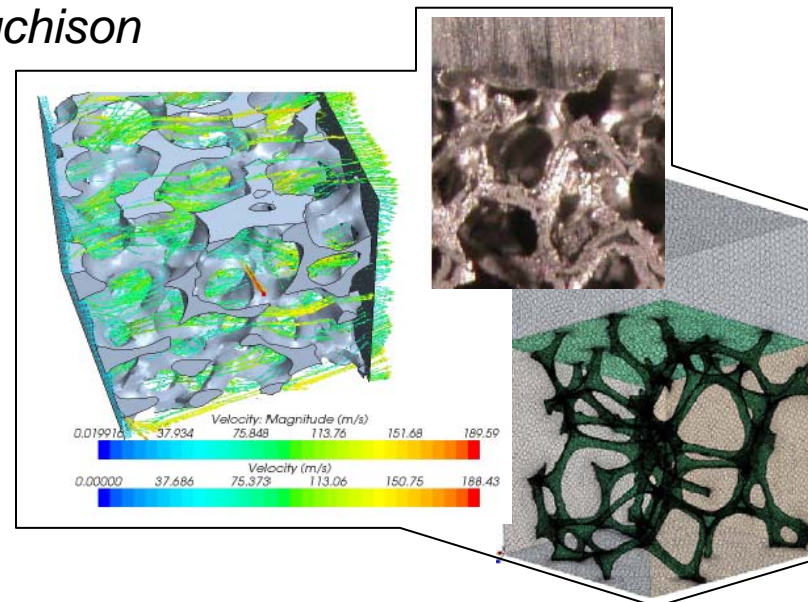
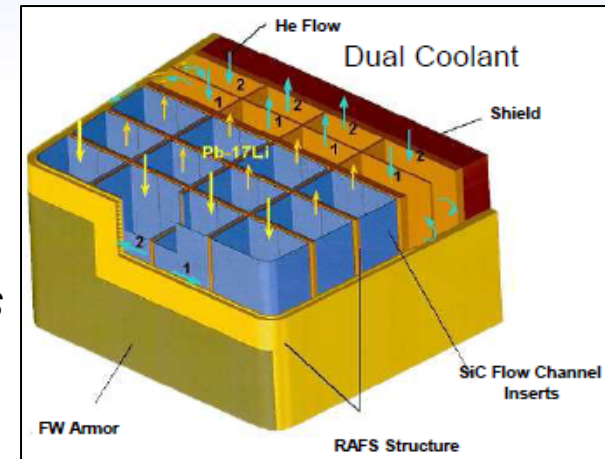
US Activities in modeling PFCs

PFCs

- All Metal ITER (IAEA paper) - *Brooks et al.*
- **APEX studies** – *Abdou and APEX Team*
- **ARIES design studies** – *ARIES Team*
- **ITER TBM and TITAN activities** – *UCLA/GA/others*
- **Integrated Blanket Systems** – *Ying, Narula, Wang*
- **Thermal performance models of mockups**
– *Youchison, Sharafat, Raffray*
- **Response to EM loads** – *Ulrickson, Youchison*
- **He flow through porous media (CFD)**
– *Youchison*

PSI – covered elsewhere

- Tritium retention in neutron damaged W
– *Wampler et al.*
- Conditions for formation and erosion of tungsten “fuzz” – *Doerner et al.*
- Erosion and redeposition – *Brooks et al.*
- Erosion and vapor shielding of W PFCs
– *Hassanein*
- Hydrogen on the surface of W – *Kolasinski*
- Hydrogen permeation in W – *Causey et al.*
- Other work

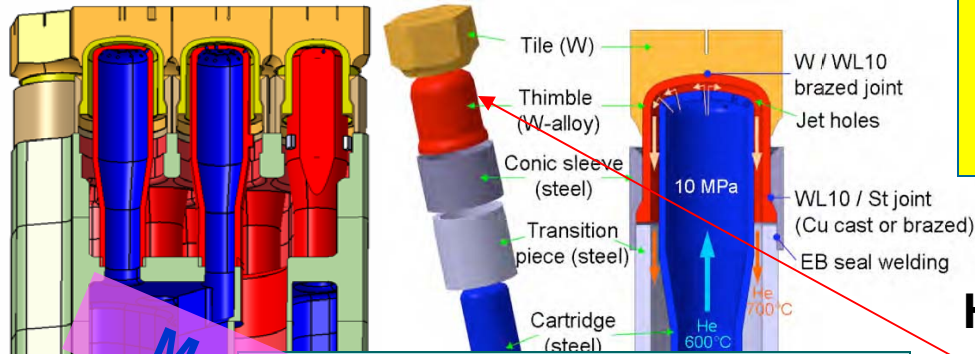




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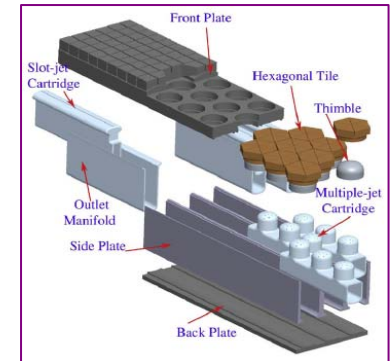
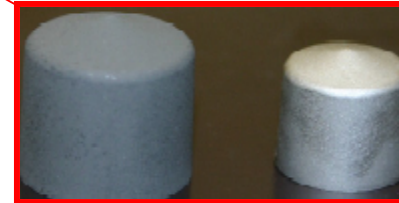
W PFC research – He-cooled W, e.g., HEJM in by EU



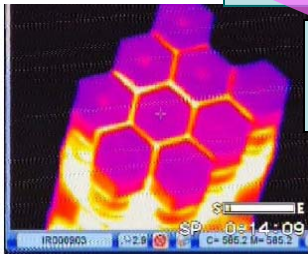
HEMJ – component dev. aspects

- design iterations, high heat flux tests
- fabrication development
- commercial mat'ls + innovations

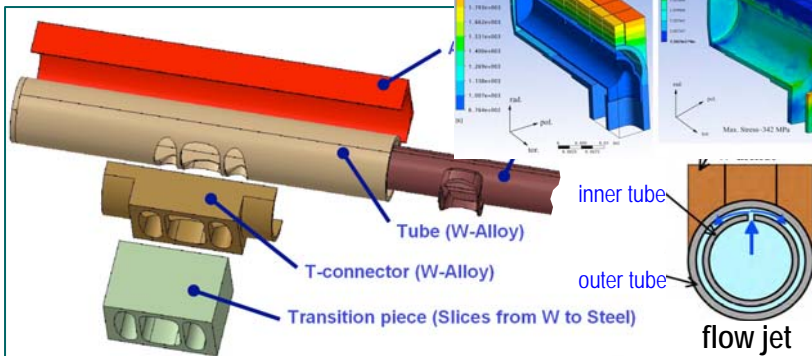
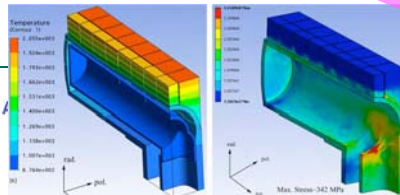
HEMJ Thimbles



Mark Tillack will present more information on these designs.



IR imaging during testing,
P. Norajitra et al., FED 83 (7-9) (2008) 893-902.



- Deep drawing is suitable for mass production. Grain orientation of the W follows the contour.
- Dry machining does not contaminate surfaces for subsequent brazing and takes 1/4 work time.
- Finishing with a CBN grinding wheel (rather than EDM) eliminates fine surface cracking.

Reiser et al., Development of a He-cooled Divertor: Technological Studies of Tungsten Machining, SOFT08

T-tube in ARIES-CS divertor - nice idea

PFC-PSI Development & Integration for W PFC

Nature of inquiry

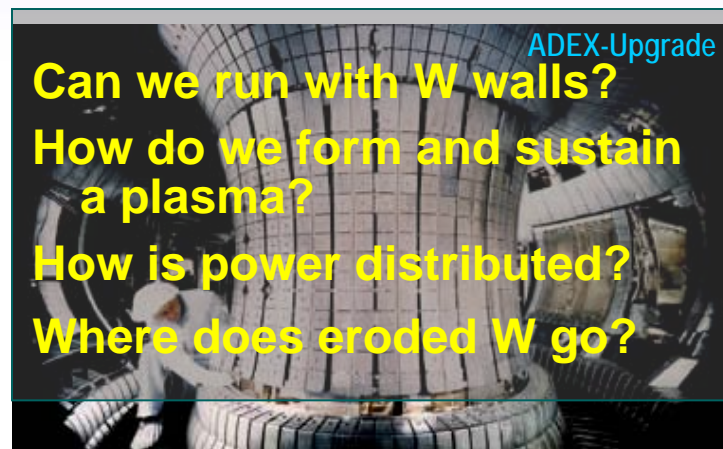
Physics in all metal devices

- ASDEX-U (divertor) →
- Alcator C-MOD (divertor)
- Others: TRIAM-U (limiter)
- JET with “ITER-like” divertor

R&D on W PFCs

- JET ITER-like divertor
(W monoblocks & coatings)
 - Performance of Mo & W tiles in C-MOD →
 - Coatings in ASDEX-U
 - W limiters in TEXTOR, HT-7
 - Fabrication development
-
- Probe/tiles measurements
- W erosion in DIII-D and C-MOD →
- D or He damage in TRIAM, LHD

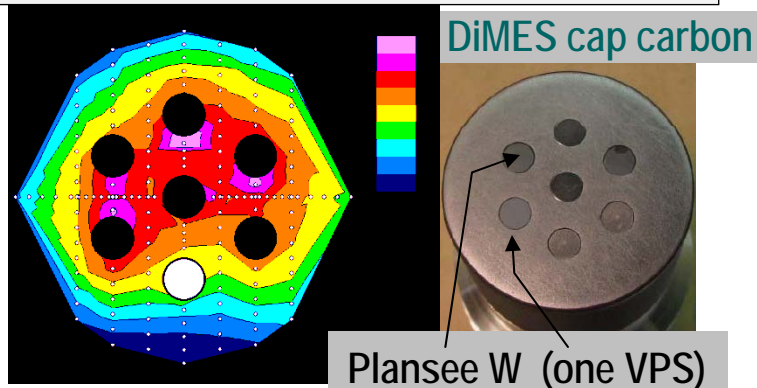
PSI R&D in labs (PISCES, NAGDIS, IBL, TPE ...) →



US Activities with W PFCs - erosion

W redeposition, DiMES div. probe

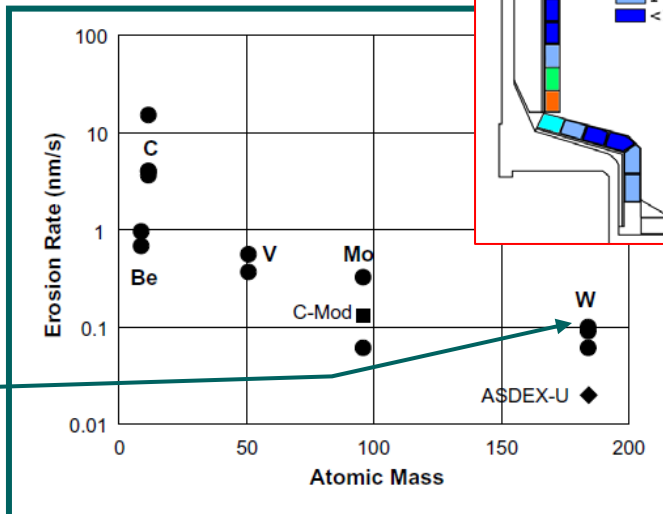
Courtesy of DIII-D (General Atomics) & DiMES Team



- RBS measurements (pre/post)
- W deposits mainly near source
- Average net W erosion rate 0.13 nm/s

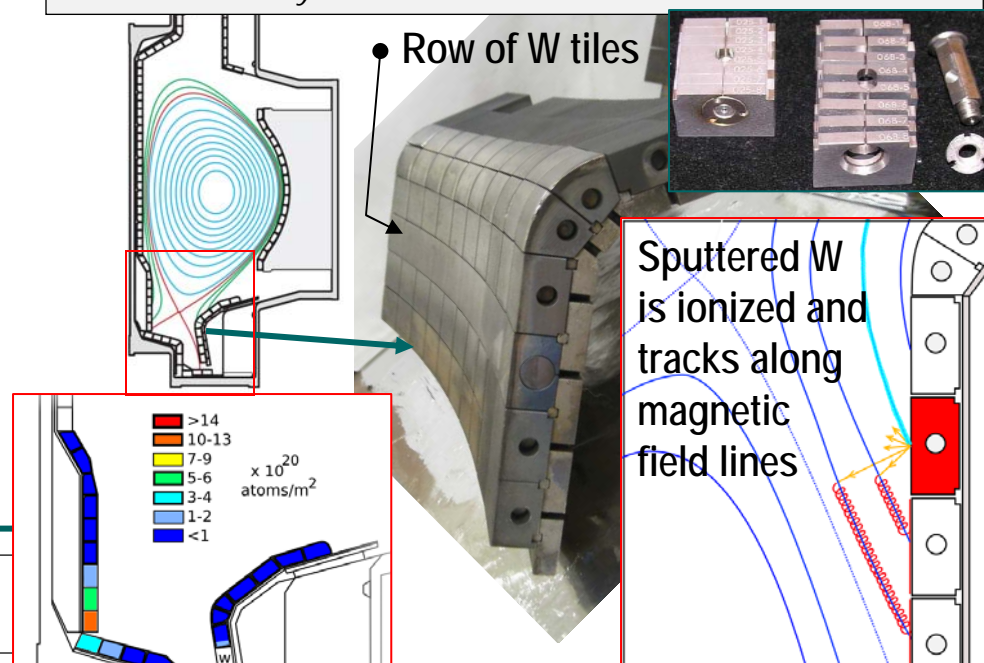
Fit with earlier measurements

W. Wampler,
*Nucl. Instr. & Meth.
in Phys. Res. B*
219-220 (2004)



Section of Alcator C-Mod outer divertor

Courtesy of MIT Plasma Fusion Science Center



Particle Induced X-Ray Emission (PIXE)
Analysis for Measuring W Migration,
Barnard, Lipschultz, Whyte, Saint
PFC Mtg MIT July 2009

Extensive studies in ASDEX-Upgrade

All Metal ITER
(IAEA paper) J. Brooks et al.

US Activities with W PFCs – R&D on W tiles

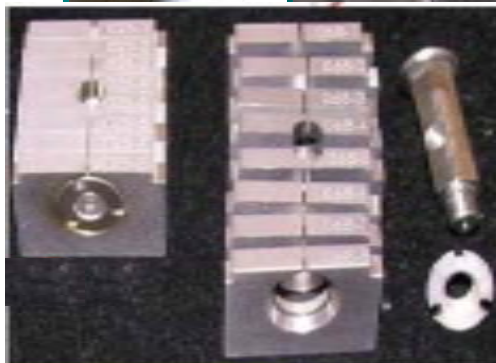
Alcator C-MOD tokamak at MIT's
Plasma Fusion Science Center



C-MOD divertor
with Mo tiles

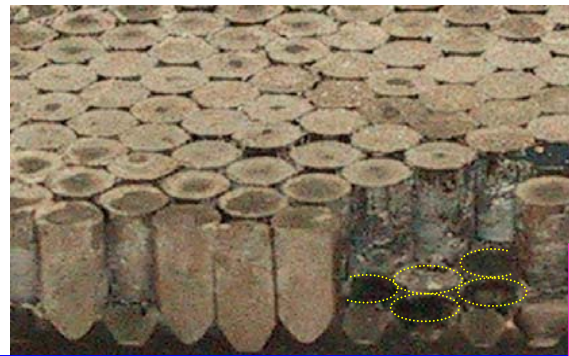


Row of W tiles

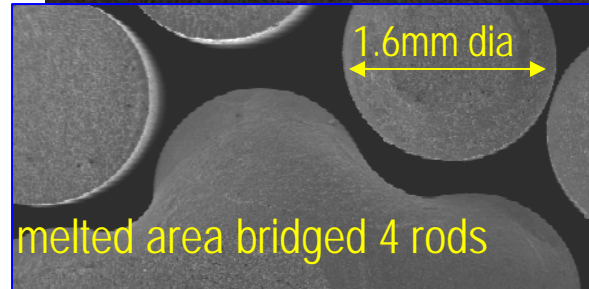


W lamellar
plate tiles

Sandia-Boeing
ITER W rod mockup

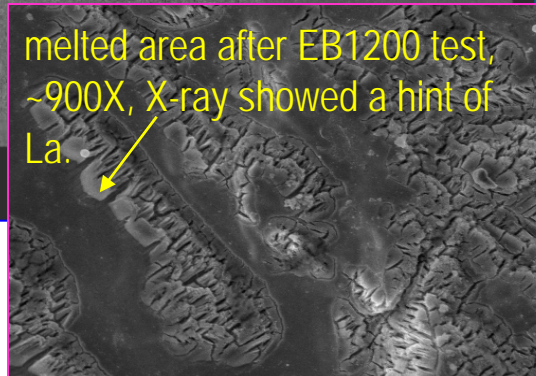


- rods in PS Cu bed
- bed HIPped to CuCrZr
- method did not work.

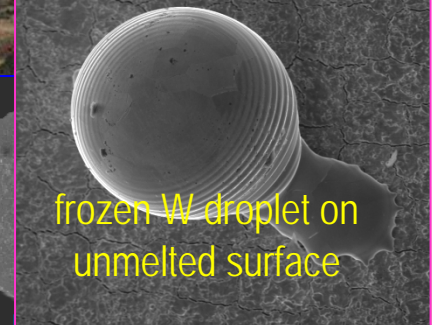


1.6mm dia

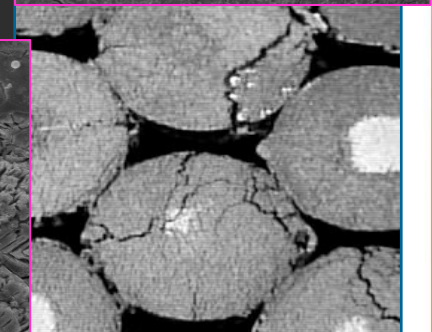
melted area bridged 4 rods



melted area after EB1200 test,
~900X, X-ray showed a hint of
La.



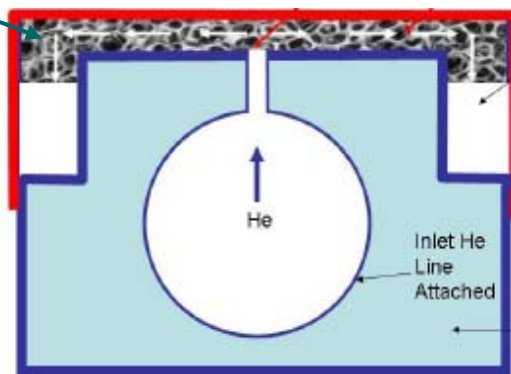
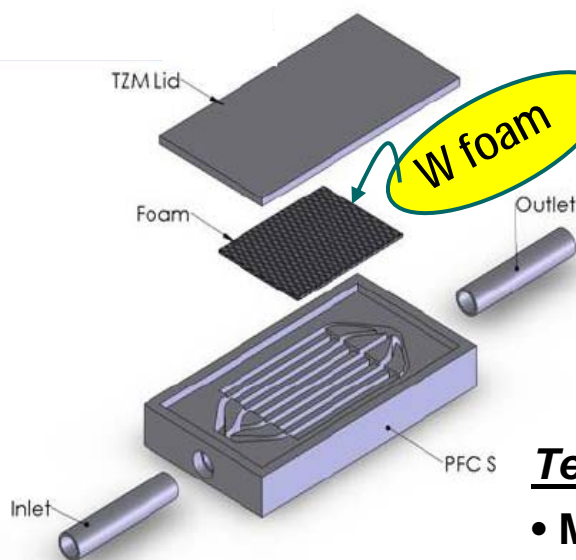
frozen W droplet on
unmelted surface



US Activities - He-cooled W mockups

Sandia tested He-cooled heat sinks since the 1990's.
In more recent tests:

- dual-channel He-cooled W heat sink, **Thermacore***, **34.6 MW/m²** (*study channel-to-channel flow instability*)
- 15-mm-diameter tubes with porous medium, integral structure by **Ultramet*** with CVD W, **22.4 MW/m²**
**done under DOE grants for Small Business Innovative Research*

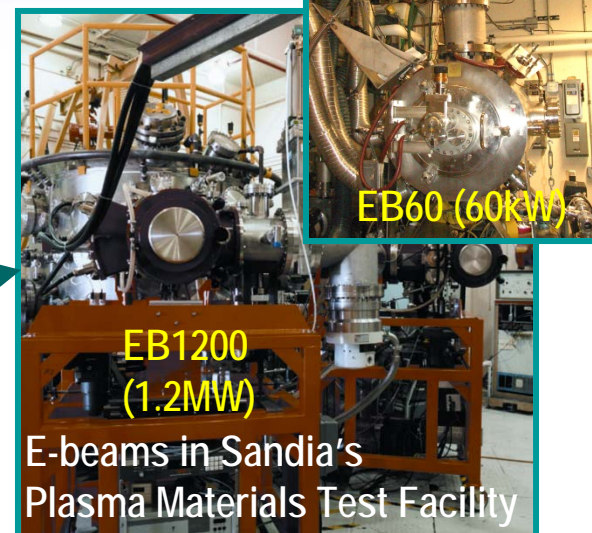


Test in coming year

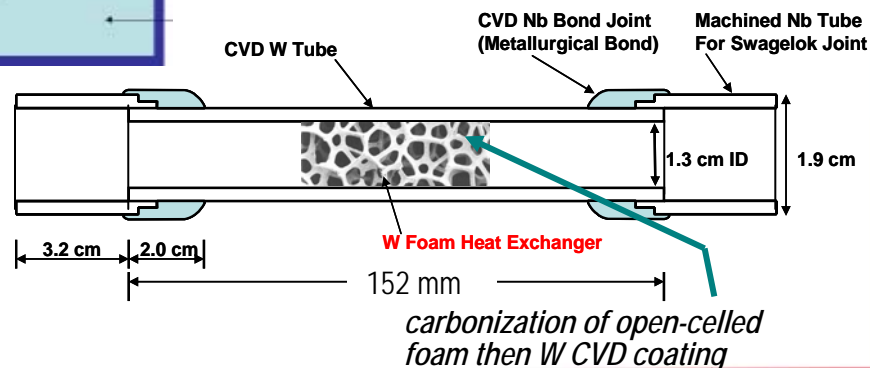
- Multiple channel (4)
- Flat surface
- All refractory
- Short flow paths
- 600 C inlet temps

Investigate:

- Larger heated areas
- Flow instabilities



new He loop



Basic Elements for W PFC Development*

Development of W-based materials, including production, fab, joining and neutron damage

Investigation of PMI issues

Measurements of the plasma edge

Development and deployment of PFCs
(confirmation of design, goals for performance, QA, high heat flux testing, ..)

Experiments with large areas of W PFCs;
operation with hot walls.

Modeling of materials and the evolution and effects of damage

Modeling of the plasma edge

Benchmarking of edge and materials models

Modeling of component performance and integrated testing

Benchmarking of predictive performance models that integrate plasma edge, materials evolution and component performance

*Making Tungsten Work

R Nygren ICFRM-14 (2009)

We can list basic tasks.

This is one way to summarize activities in a pathway.

But a list has no sequence implied.

And the tasks here cut across several of our topical areas.

Basic Elements for W PFC Development

Topical Area (Phys/PSI/PFC/MAT)

Development of W-based materials, including production, fab, joining and neutron damage

PFC MAT

Investigation of PMI issues

PSI

Measurements of the plasma edge

Physics PSI

Development and deployment of PFCs
(confirmation of design, goals for performance, QA, high heat flux testing, ..)

PFC MAT

Experiments with large areas of W PFCs;
operation with hot walls.

Physics PSI PFC

Modeling of materials and the evolution and effects of damage

PSI

MAT

Modeling of the plasma edge

Physics PSI

Benchmarking of edge and materials models

Physics PSI

MAT

Modeling of component performance and integrated testing

PSI

PFC

Benchmarking of predictive performance models that integrate plasma edge, materials evolution and component performance

Physics PSI

PFC MAT



Materials Issues for W PFCs

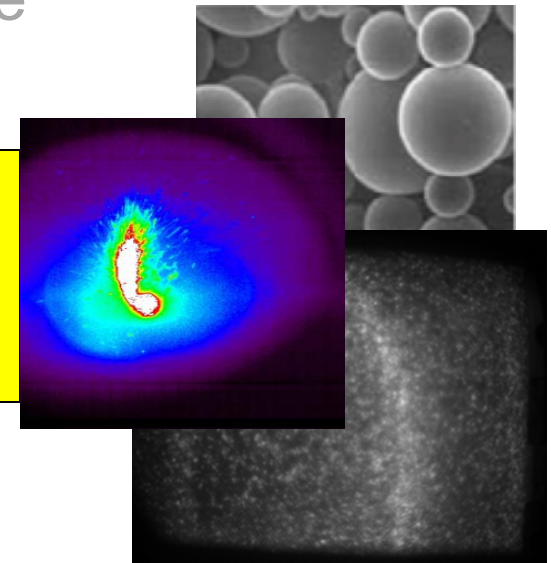
What improvements can we deliver?

- Lower DBTT
- Better machinability
- Mitigation of some neutron & ion damage
- ⋮ PSI, oxidation, ...
- *Reduced cost*
- *Weight*

Rudikov (General Atomics),
Dust Study: loose dust mobilized
(? electrical charging) and prevented
discharge. *MIT PFC Meeting, 2009*

Data for

- Alloy selection
- Safety analyses,
credible accidents,
"off normal" events
(strong drivers)



What are intrinsic limitations?

- Defects frozen in cascades "black spots"

▪ **Cracking**

- ?Other

- *base material*
- *evolving structure*
- *melt layer*

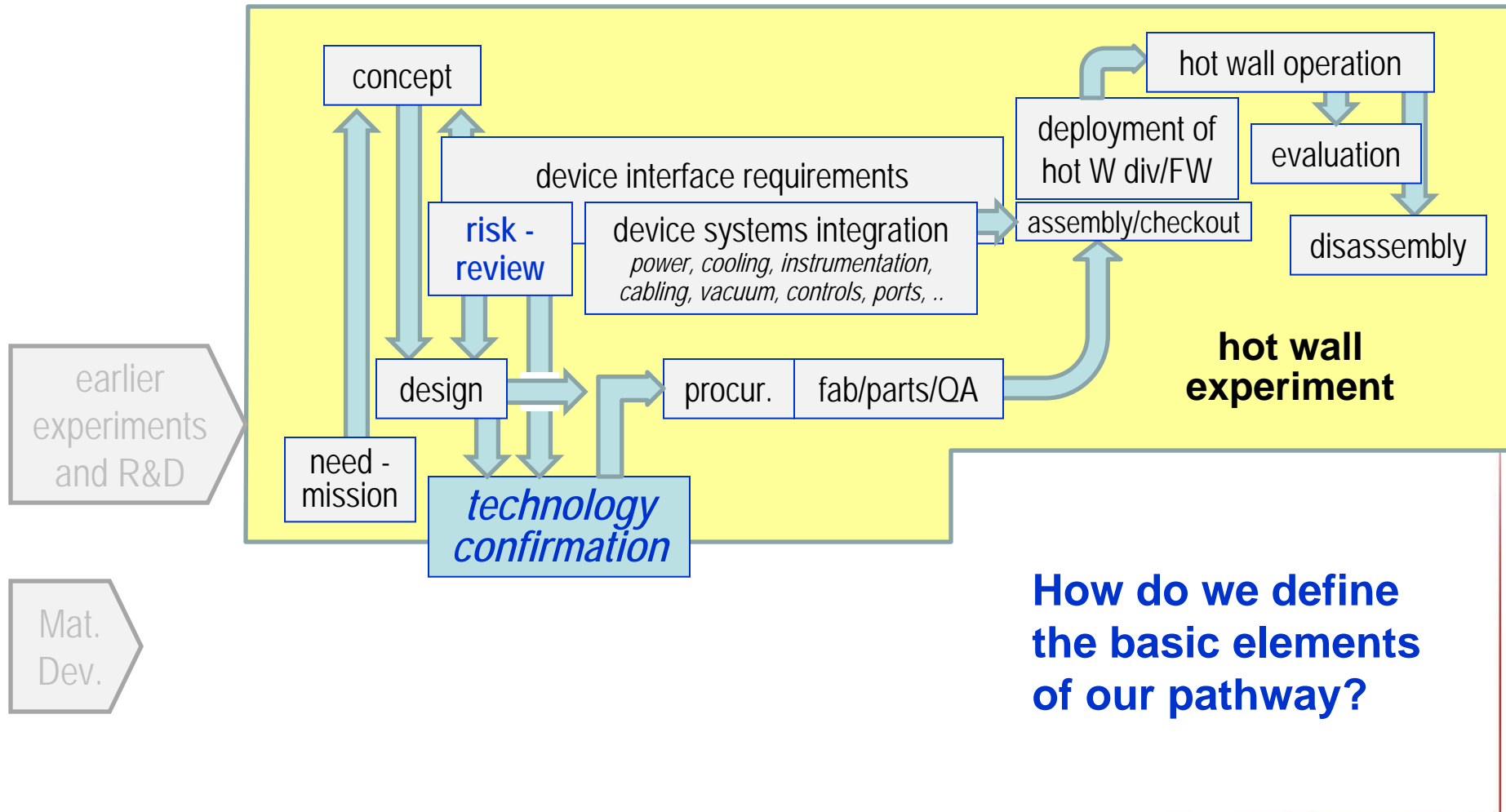
manageable

YES - operate

NO – replace/repair

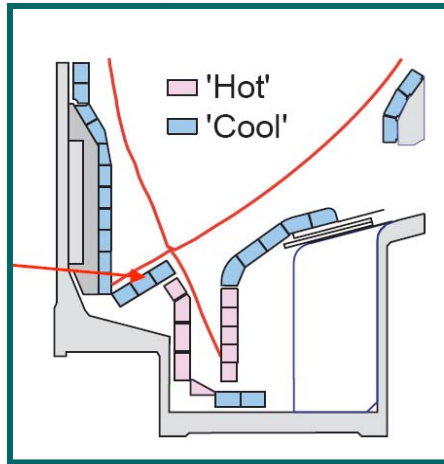
Hot Wall Experiment – large area of hot PFCs

gas-heated refractory heat sinks, tungsten armor
(control of wall temperature separate from power, no electrical heaters)



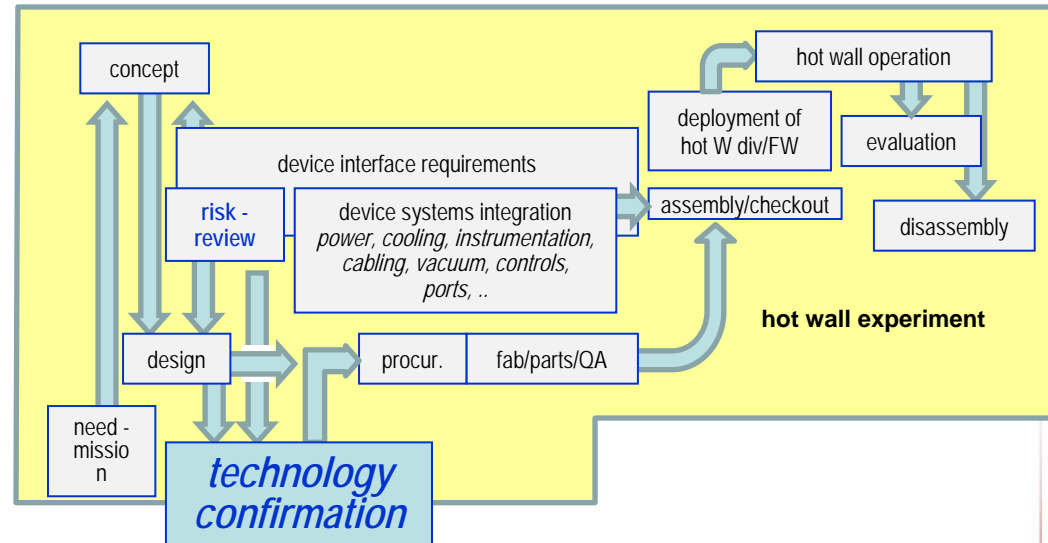
Hot Wall Experiment – large area of hot PFCs

*gas-heated refractory heat sinks, tungsten armor
(control of wall temperature separate from power, no electrical heaters)*



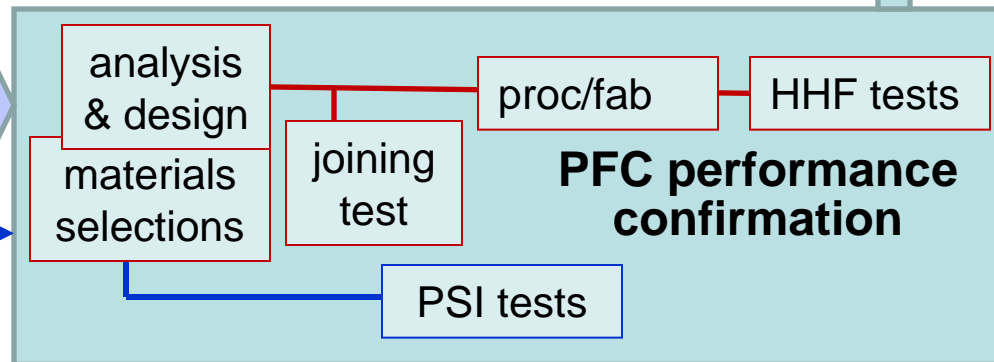
C-MOD hot tiles

earlier
experiments
and R&D



?2-5 years


?2-3 years



?2-5 y

?2-5 years

**How do we define
the basic elements
of our pathway and
their sequence?**



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of our pathway and their sequence?**

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