



SAND2011-6984P

Sandia National Laboratories Overview





Key Mission Areas

- Energy
- Homeland Security
- Defense Systems
- Nonproliferation
- Nuclear Weapons





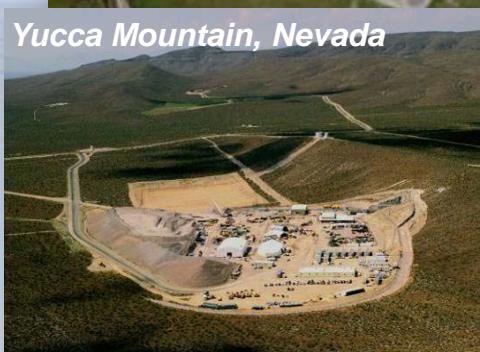
Sandia Locations



Livermore, California



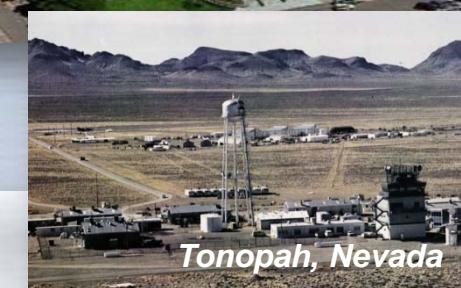
Albuquerque, New Mexico



Yucca Mountain, Nevada



Pantex, Texas



Tonopah, Nevada



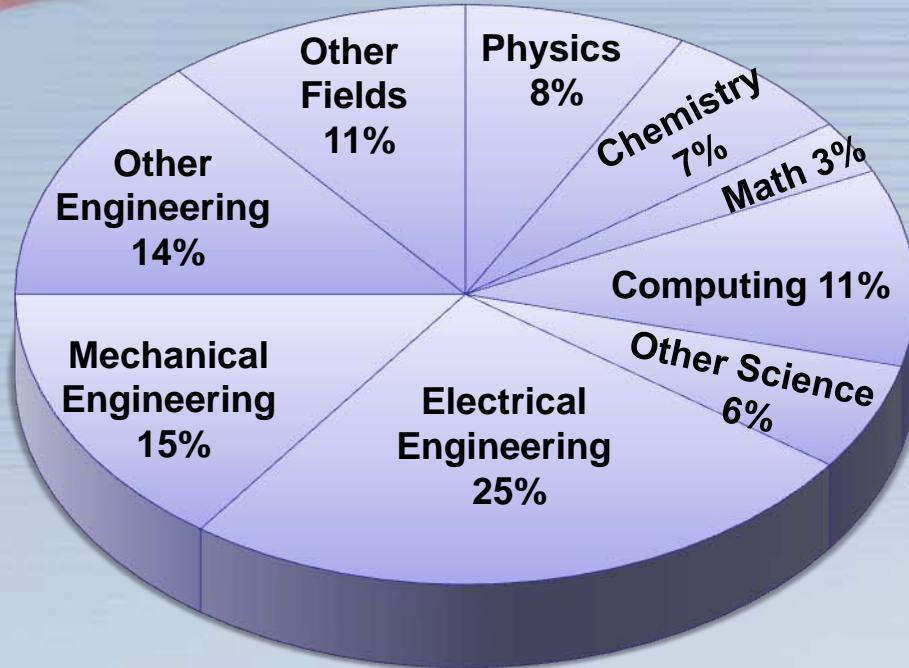
Kauai, Hawaii



WIPP, New Mexico



Sandia has a Workforce of Over 10,000 Employees



- 8,500 regular full-time employees
- 1,500 post docs, limited term, & contract employees
- 1,500 PhDs and 2,300 Masters
- \$2+ billion operating budget



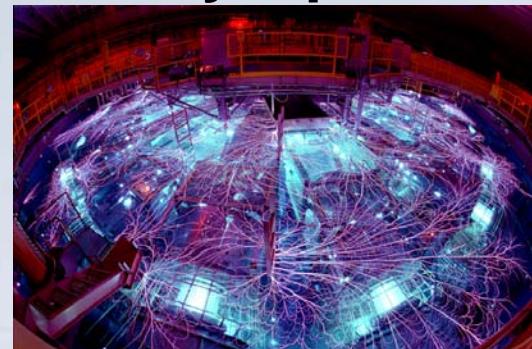
Work/life balance: eye candy





From Z (division) to Z (machine)

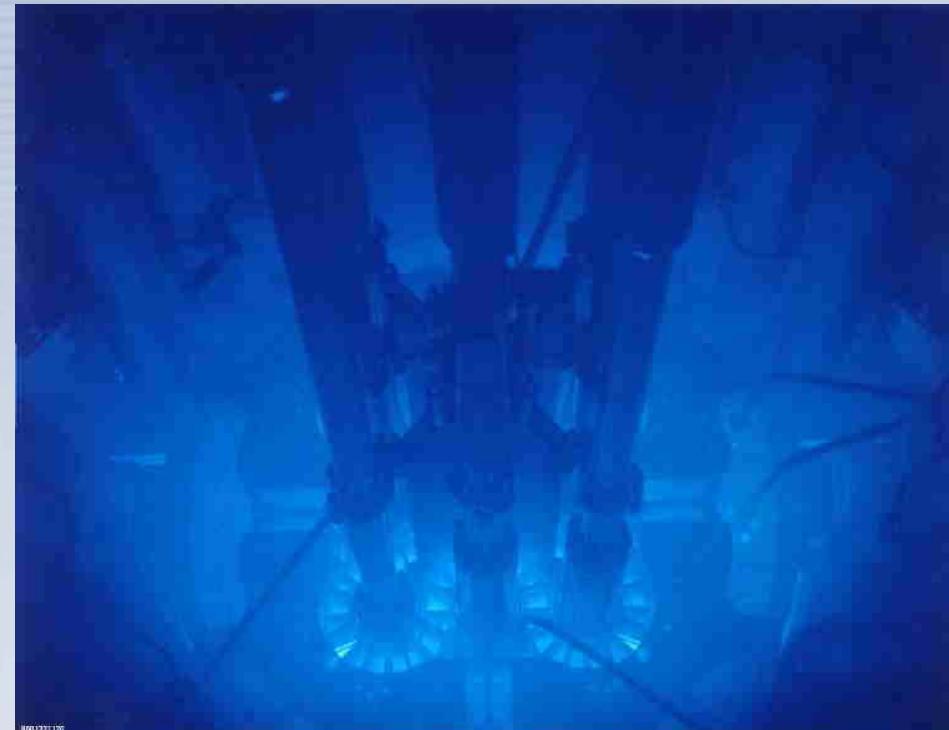
- 1945: Los Alamos Z Division moves to Albuquerque
- 1949: Sandia (Albuquerque) founded to turn physics packages into deployable weapons
- 1956: Livermore site opened
- 1974: technical advisor for WIPP
- 1997: started Z-pinch research for basic science and stockpile stewardship
- 2010: creation of Livermore Valley Open Campus





Some Sandia Opportunities for Nuclear Engineers

- Pulsed power
- Nonproliferation
- Advanced fuel cycles
- Materials development
- Homeland security
- Waste transport/storage





Fast Neutron Signatures for Uranium Hexafluoride Enrichment Measurements

Scott Kiff

Radiation and Nuclear Detection Systems

September 20, 2011



Gas centrifuge enrichment plant,
Piketon, OH

Sandia National Laboratories is a multi-program laboratory managed and operated by Sandia Corporation, a wholly owned subsidiary of Lockheed Martin Corporation, for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-AC04-94AL85000.



Special thanks to...

- **Pete Marleau, Erik Brubaker**
 - Technical expertise
- **Mark Gerling**
 - Neutron Scatter Camera experiments, modeling
- **Michael Streicher (summer intern, Purdue)**
 - Neutron Scatter Camera experiments, modeling



A Rough Presentation Outline

- Introduction to the problem
- How I got involved
- Using fast neutrons
- Initial calculations
- Some measurements
- Future plans
- Other possibilities
- Discussion

THE SENSATIONAL *New* **NUCLIOMETER***

Detection MODEL DR-299

THE MOST VERSATILE URANIUM DETECTOR EVER BUILT

The new DR-299 is truly a tremendous step forward in uranium detection. Not only will the instrument detect many deposits previously missed by other, less-sensitive instruments; it can be used to check individual specimens, to make field assay estimations and in airborne survey work (see model DR-299-A below).

CHECK THESE 16 AMAZING FEATURES!

1. Reads directly in % of ore (with sample holder provided).
2. Probe and cable for use as Geiger counter.
3. Much more sensitive than scintillation counters using $1'' \times 1\frac{1}{2}''$ crystal.
4. Extreme sensitivity (.01 Mr/Hr full-scale).
5. 4 time constants.
6. 6 operating ranges.
7. Zero adjustment.
8. Simplicity of operation.
9. Standardization of parts.
10. Exceptional versatility.
11. Rugged & compact. (Not affected by heat and vibration)
12. Stability of circuit.
13. Low-cost operation.
14. Grain leather carrying case and shoulder strap.
15. Radium calibration source.
16. 90-day parts warranty.

SPECIFICATIONS

• Area of detection sensitivity	• Batteries
—42 Sq. in.	1—67.5 Volt "B" (Eversoody #467 or equivalent)
• Physical dimensions	3—30 Volt "B" (Eversoody #413 or equivalent)
—4 1/2" wide, 12 1/2" long, 7" high.	3—Size D "A" cells (Steady and flashlight cells)
• Weight	• Sensitivity, Probe — .2, .2, 20 Mr/Hr
—9 1/2 lbs. complete	• Sensitivity, Instrument — .01 .03, .1 Mr/Hr
• Tubes	• Finish — Aluminum case finished in dark grey baked-enamel lacquer.
19—Victoreen 1885	
6—Victoreen 26306	
1—#34411 voltage	
ampere meter	
3—Royston 533 AX	
2—Royston CX 526AX	
1—TUS (R.C.A. or other top quality)	
1—#P51 (General Electric or other)	

NO PRICE INCREASE

\$545.00 LIST

Complete with cable and probe, specimen holder, calibration standard, leather case and complete instructions. Shipping weight 13 lbs. \$570

AIRBORNE USE — MODEL DR-299-A

The extreme sensitivity of the new DR-299 makes it an excellent instrument for airborne work. After originally being discovered by aerial survey, the instrument can be used in the same manner to pinpoint the deposit, using the grid map technique.

A 10 foot cable and extra meter are provided, enabling the instrument itself to be placed in the luggage compartment of the plane, away from the radio-disturbing instrument panel. The extra meter is hung on the panel with the rest of the instruments. Price, complete with cable and probe, extra meter and cable, specimen holder, calibration standard, leather case and complete instructions. \$570

Photo of a man operating the Nucliometer in an airplane cockpit.



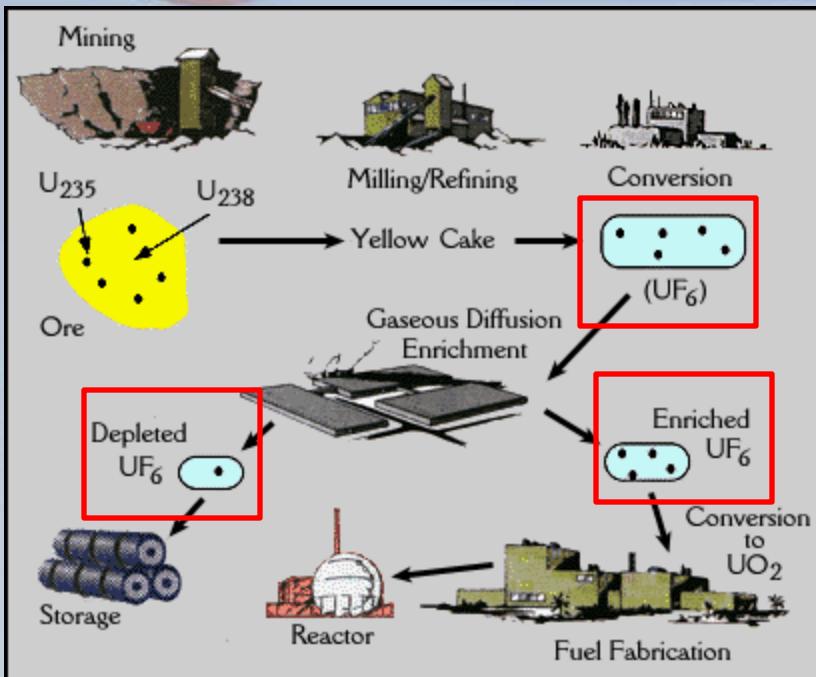
What is the problem?

- Enriched uranium can be used to construct a nuclear weapon
- It is important to verify the enrichment of uranium as it exits the processing stream to detect material diversion efforts
- Diversions may be possible with current NDA technology





A quick review: the front end of the fuel cycle

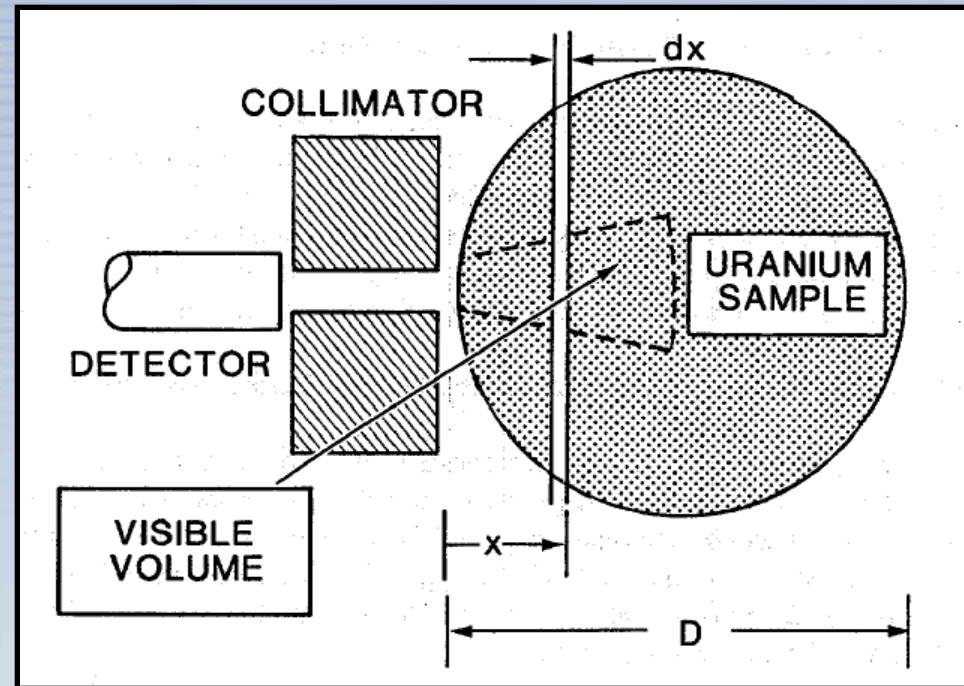


Material of construction	Minimum volume		Approximate tare weight (without valve protector)		Maximum enrichment wt % uranium-235	Shipping Limit	
	ft ³	liters	lb	kg		Maximum, UF ₆	Maximum, kg
Nickel	0.0053	0.15	1.75	0.79	100.00	1.0	0.45
Nickel	0.0254	0.72	4.2	1.91	100.00	4.9	2.22
Monel	0.284	8.04	55	25	100.00	55	24.95
Monel	0.284	8.04	55	25	100.00	55	24.95
Monel	1.319	37.35	120	54	12.5	255	115.67
Nickel	2.38	67.4	185	84	5.0	460	208.7
Monel	2.38	67.4	185	84	5.0	460	208.7
Steel	25.65	726.0	1,400	635	5.0 ^b	4,950	2,245
Steel	26.0	736.0	1,400	635	5.0 ^b	5,020	2,277
Steel	108.9	3,084	4,500	2,041	4.5 ^b	21,030	9,539
Steel	108.9	3,084	4,500	2,041	4.5 ^b	21,030	9,539
Steel	140.0	3,964	5,200	2,359	4.5 ^b	27,030	12,261
Steel	139.0	3,936	2,600	1,179	1.0 ^f	26,840 ^e	12,174 ^e
Steel	142.7	4,041	5,200	2,359	4.5 ^b	27,560	12,501
Steel	140.0	3,964	3,170	1,438	1.0 ^f	27,030	12,261
Steel	140.0	3,964	3,170	1,438	1.0 ^f	27,030	12,261



Current technology is good, but...

- **“Enrichment meter” measures gamma emissions from the uranium hexafluoride (UF_6)**
 - Gives local enrichment, not total mass (need a scale for masses)
 - Sensitive to variations in container wall thickness
 - Not sensitive to material beyond outer skin of UF_6



The enrichment meter principle. From Reilly et al., *Passive Nondestructive Assay of Nuclear Materials*, Fig. 7.3



How I got involved...

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journal homepage: www.elsevier.com/locate/nima

Elsevier

^{235}U enrichment or UF_6 mass determination on UF_6 cylinders with non-destructive analysis methods

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^a The European commission, Joint Research Centre, Institute for the Protection and Security of the Citizen, TP 800, Via Fermi, Ispra, Italy
^b Consultant, Dresden, Germany

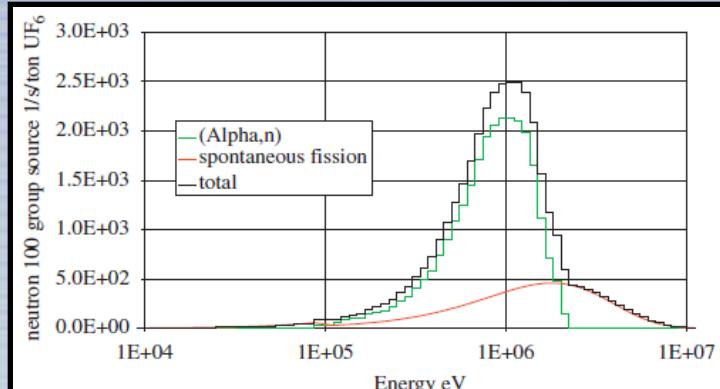


Fig. 4. Hundred group neutron spectra for natural UF_6 [7].

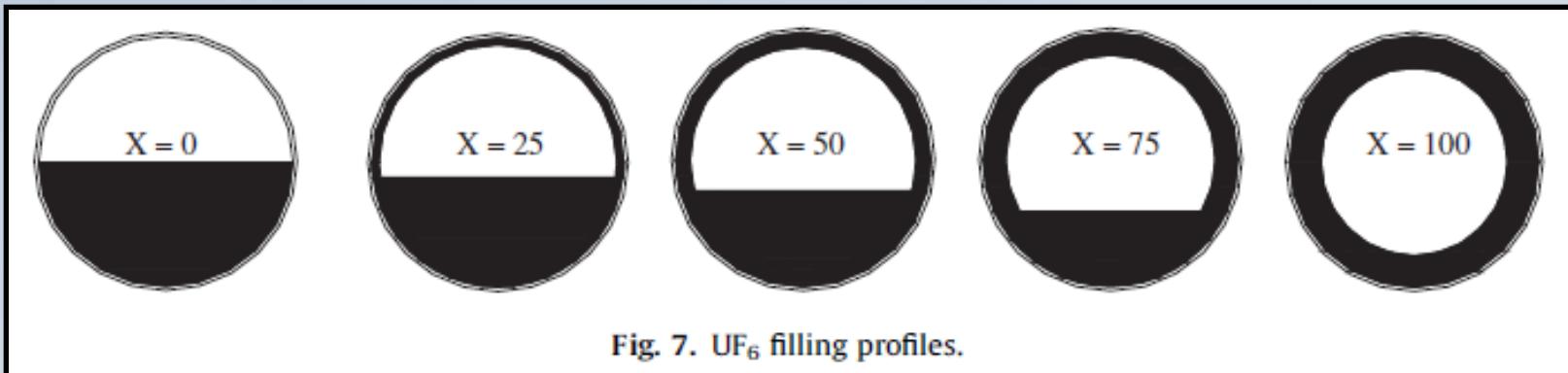
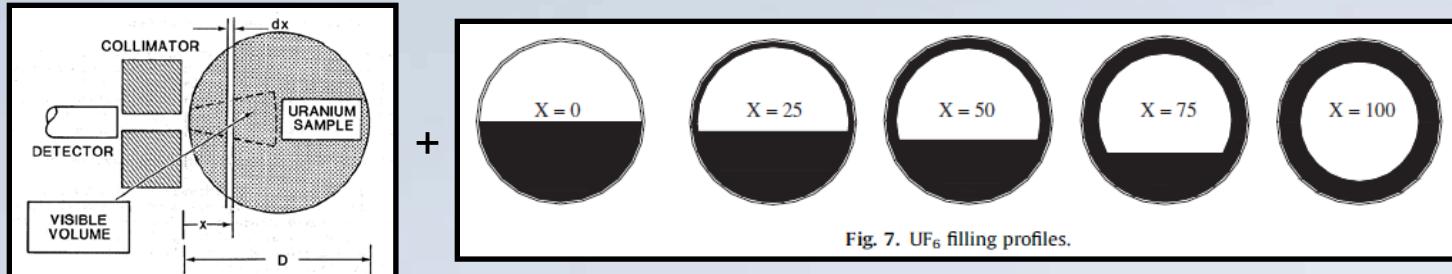


Fig. 7. UF_6 filling profiles.



Summarizing the problems

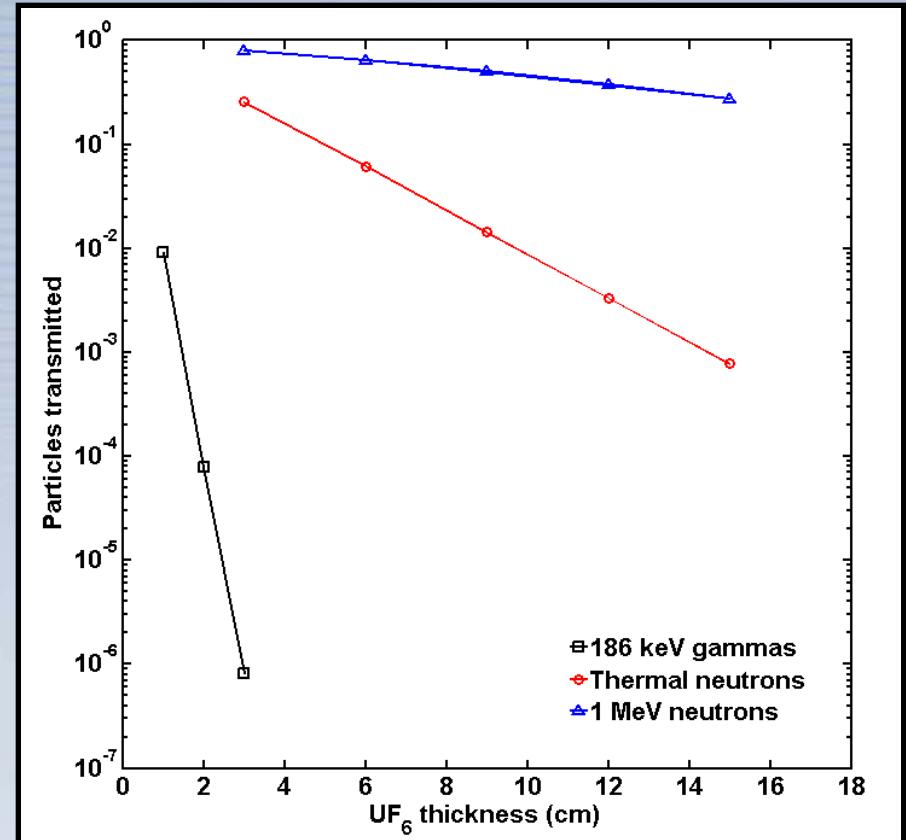
- **Typical NDA techniques measure enrichment, not isotope masses**
 - To obtain masses, a load cell (scale) measurement is necessary
- **The enrichment measurement relies upon weakly-penetrating particles**
 - Sensitive to container wall thickness
 - Sensitive to geometry
 - Cannot sample entire volume...what's in the center?





Sandia's concept: directly measure fast neutron emissions

- Fast neutrons generated by independent processes within the UF_6 can provide an independent enrichment measurement that samples the entire UF_6 volume
- Neutron imaging of the UF_6 distribution detects unexpected UF_6 geometries and applies necessary corrections
- Sandia has developed expertise in neutron imaging and spectroscopy that will enable success

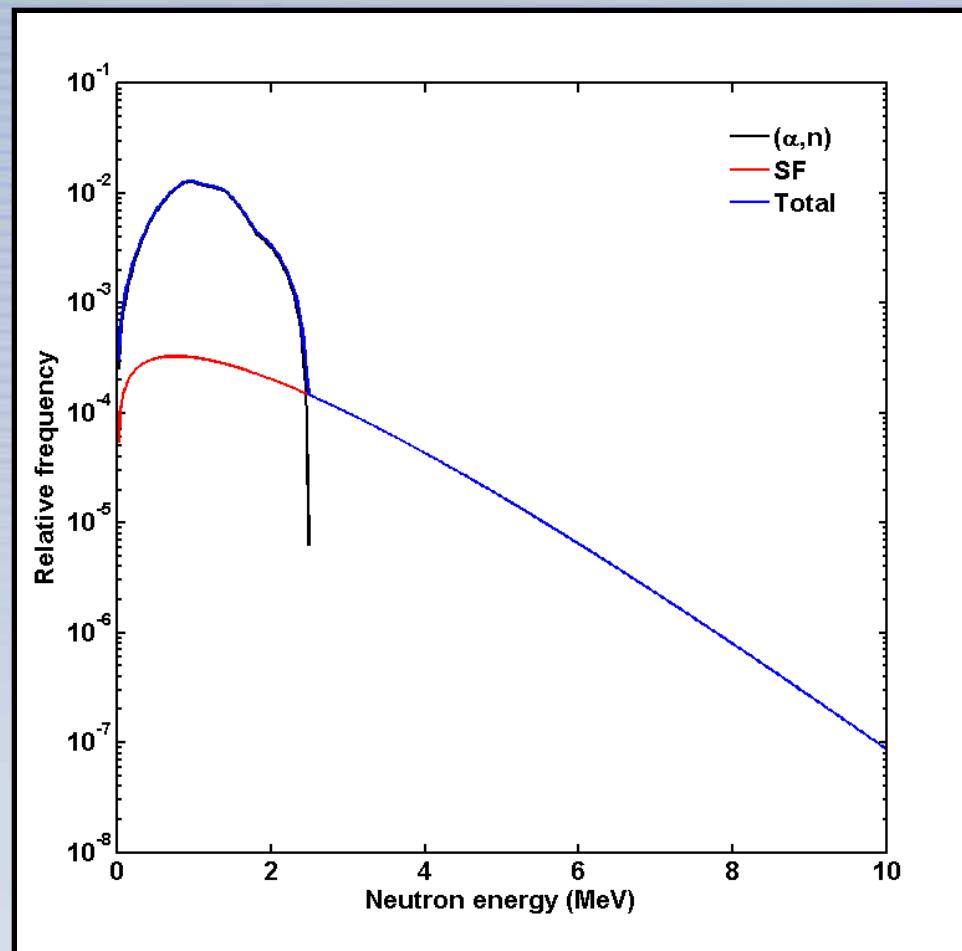


Transmission of particle beams through 5% enriched UF_6 (without container wall)



Neutron spectrometry can potentially be used to determine UF_6 enrichment and mass in a 30B

- ^{238}U : neutrons via spont. fission and (α, n) reaction on F atoms
- ^{234}U : neutrons via (α, n) reaction on F atoms
- The two processes have measurably different energy spectra
 - It should be possible to separate ^{234}U and ^{238}U contributions to the energy spectrum
 - Direct measurement of ^{234}U and ^{238}U masses
- ^{234}U content is proportional to ^{235}U content (proven by LANL for enrichment $\leq 5\%$)



SOURCES4C calculation of neutron spectrum for 5% enriched UF_6



Summarizing the concept...

- **Spectral information**

- The high-energy portion of the spectrum is purely from fission, and the magnitude is a function of the total ^{238}U mass
- The low-energy portion of the spectrum is mostly from (α, n) on F, and the magnitude is a function of the total $^{234}\text{U} + ^{238}\text{U}$ mass
- The sum of these components indicates the total sample mass
- The ratio of these components is a function of uranium enrichment

- **Imaging information**

- Imaging the material distribution may allow for geometry corrections to be applied (if necessary)
- Imaging the total volume can provide confidence that diversions are not occurring (“smuggling” with an inner volume)



Advantages of this technique

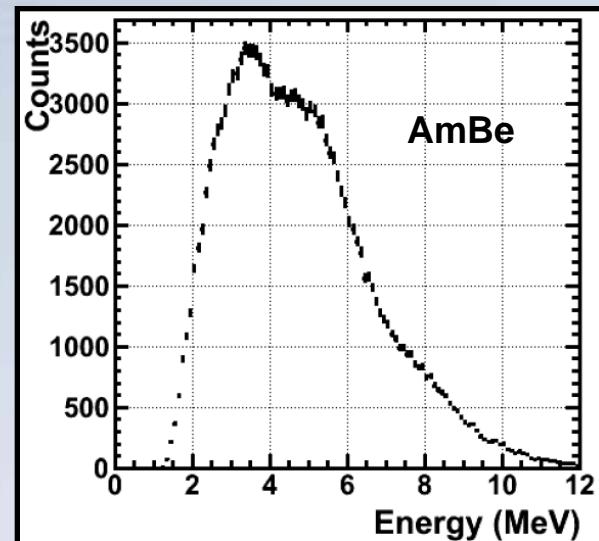
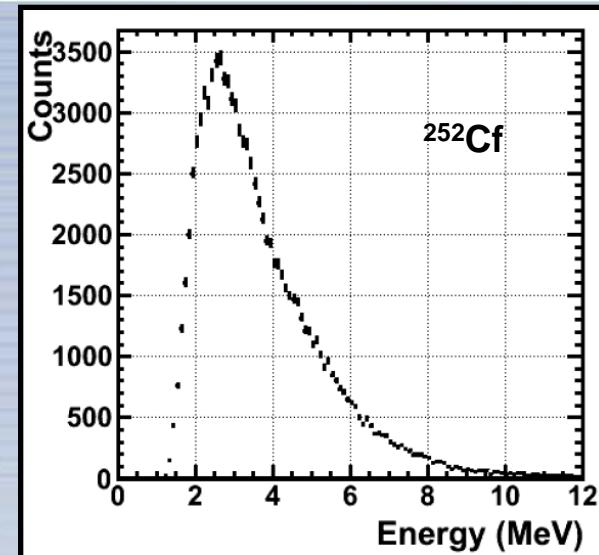
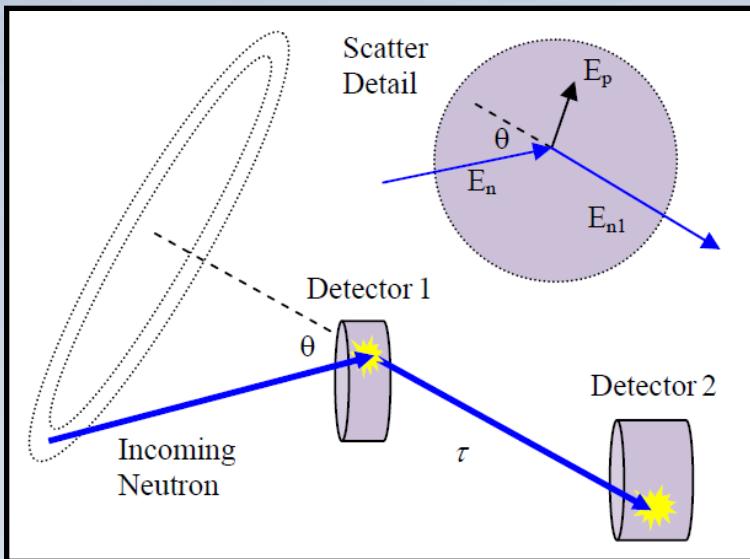
- **Independent verification of isotope ratios and masses**
- **Highly-penetrating particles are less sensitive to geometry perturbations**
- **No load cell necessary**
- **Imaging can be used to:**
 - Map material distribution
 - Reject natural backgrounds
 - Reject neutrons from nearby cylinders





Neutron spectrometry measurements can be performed with the Neutron Scatter Camera

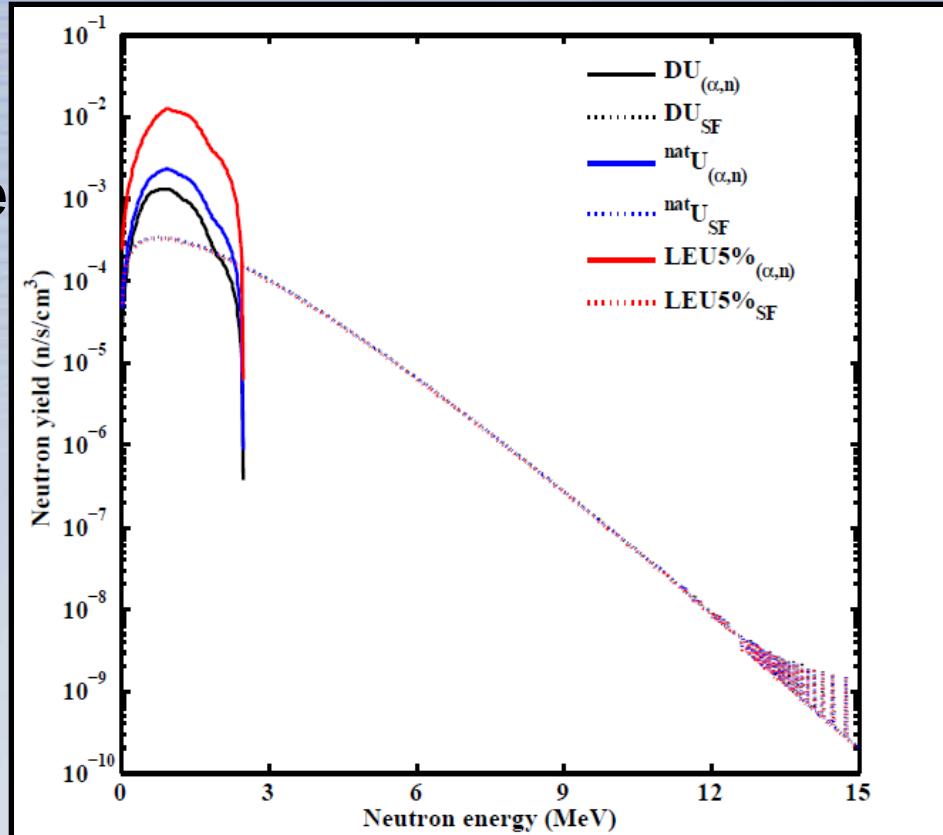
- The Neutron Scatter Camera is a mature system developed at Sandia for large-area search
 - Multi-element system
 - Liquid scintillator for n/γ discrimination
 - Imaging capabilities (interaction cell locations, measured energies)
 - Spectrometry (deposited energy, time-of-flight)





Initial calculations: emitted neutron spectra using SOURCES4C

- Calculate spectra for different enrichments to examine the dependence
- Use SOURCES4C
 - Input: isotopics, density, energy bin boundaries
 - Output: (α, n) , spontaneous fission rates (n/cm^3s)
- No detector response or transport physics in cylinder at this point



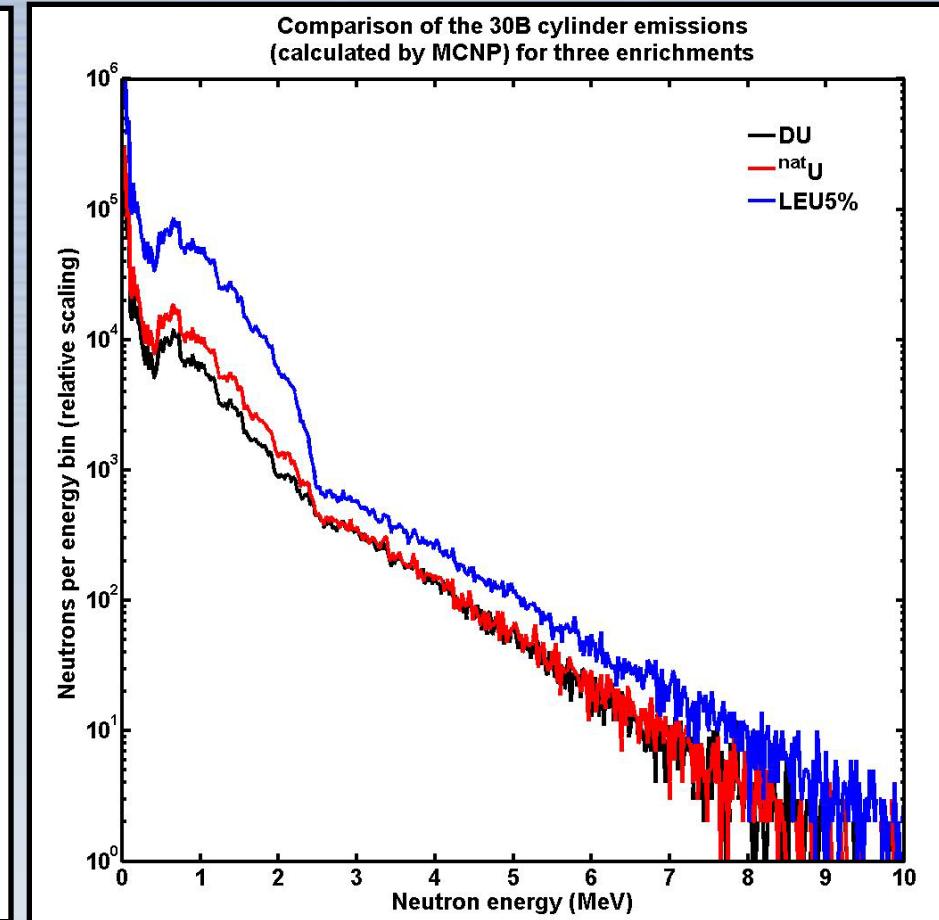
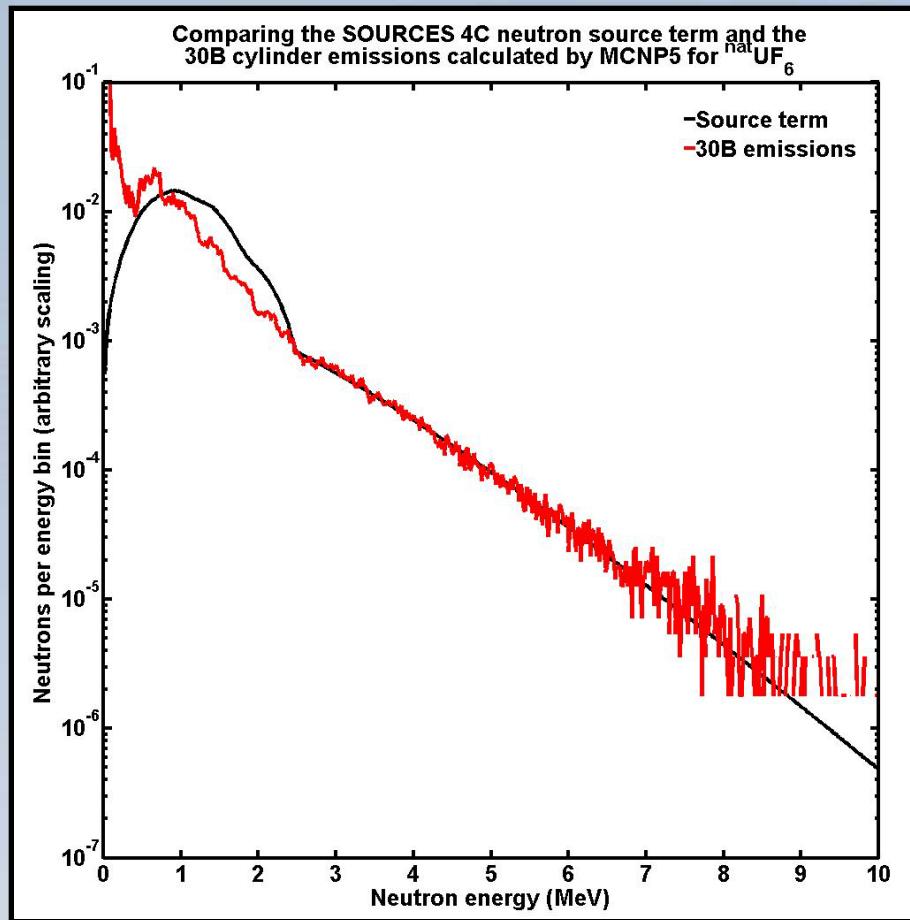


Calculations of the cylinder emissions imply manageable spectral perturbations

- The source term is perturbed in a large mass of UF_6 .
 - Scattering
 - Induced fission
 - Absorption
- A 30B cylinder was modeled in MCNP5.
 - Enrichments: DU, ${}^{\text{nat}}\text{U}$, 5% enriched ${}^{235}\text{U}$
 - Maximum fill mass
- Spectra appear to maintain enough structure for the measurement concept to work.

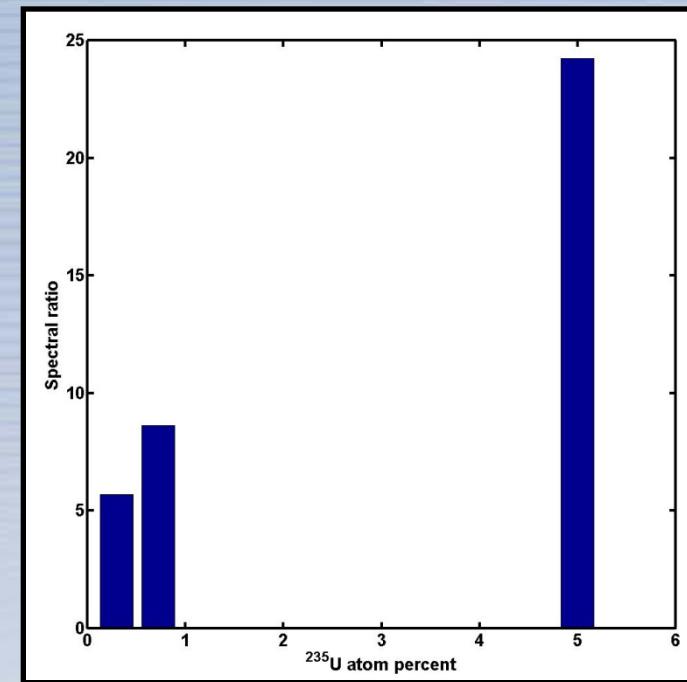
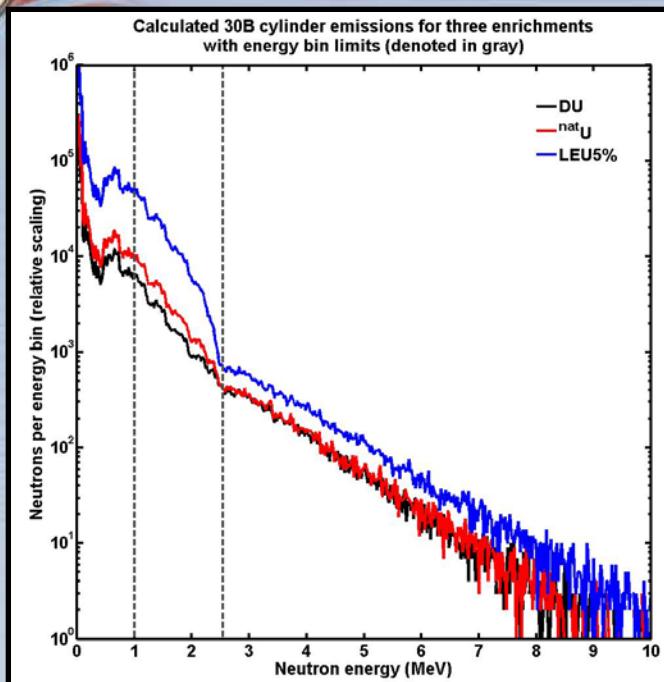


Results of the MCNP5 calculations





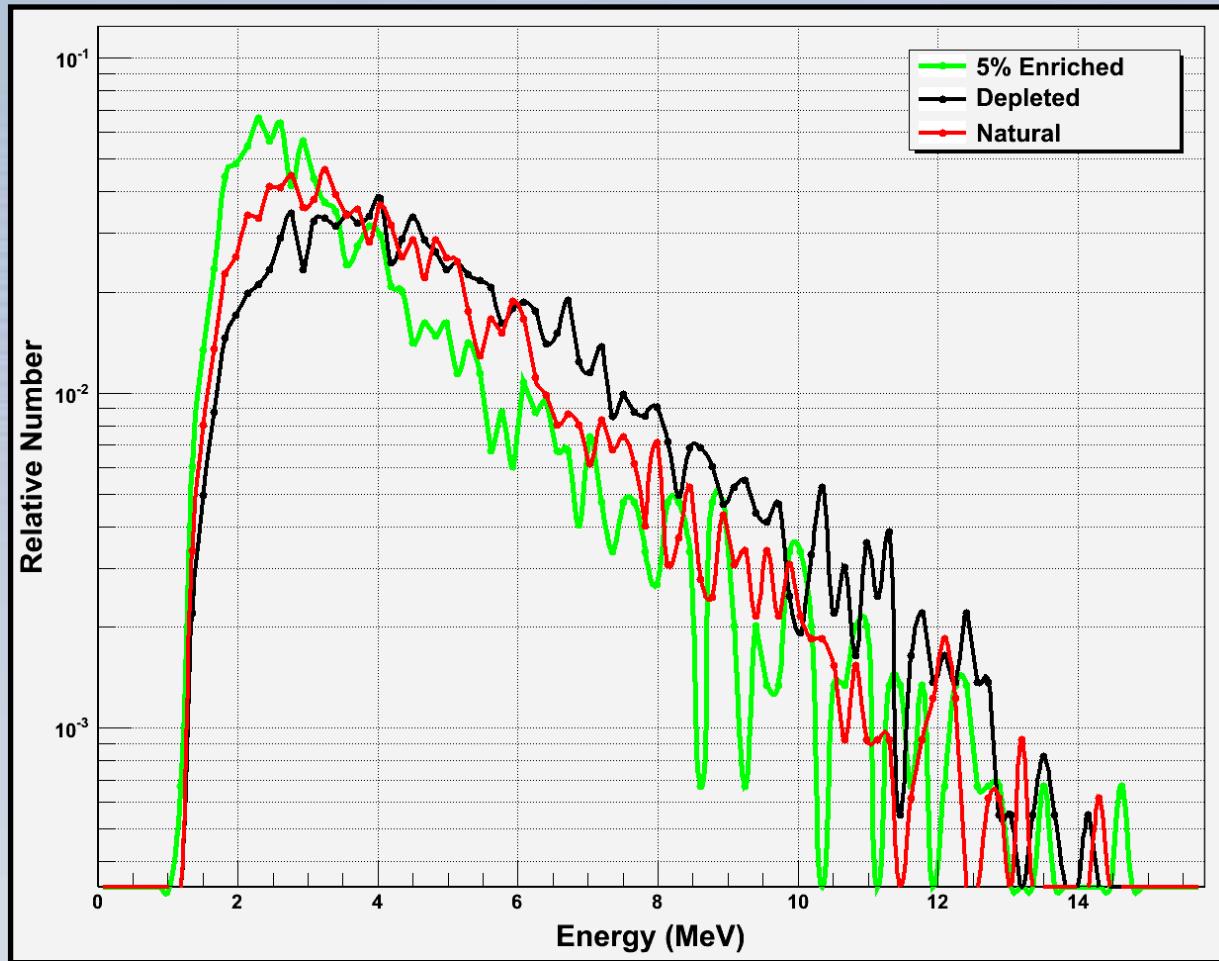
The enrichment can be inferred from the neutron energy spectrum



- The ratio of neutrons in the (α, n) and S.F. regions is a function of enrichment
 - Cut data at the end of the (α, n) spectrum (~2.54 MeV)
 - A realistic detector will have a detection threshold (choose 1 MeV)
- For the simulated data, the ratio is a monotonic function of enrichment



The (simulated) ^{30}B emissions with full detector response

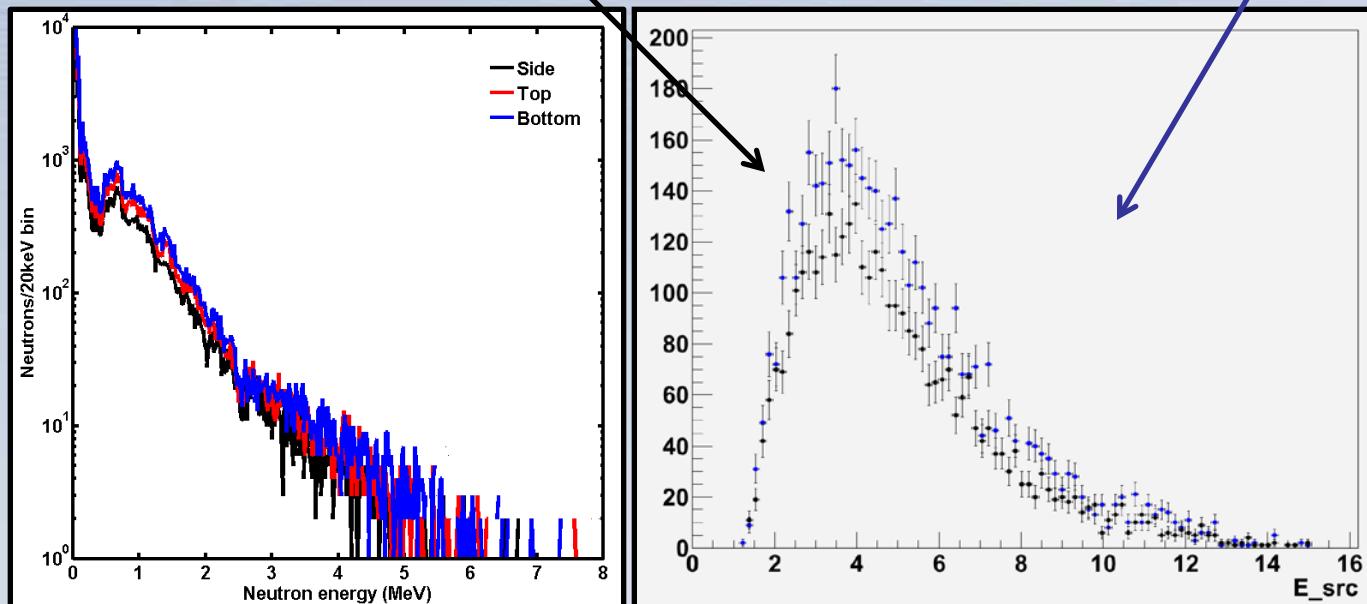
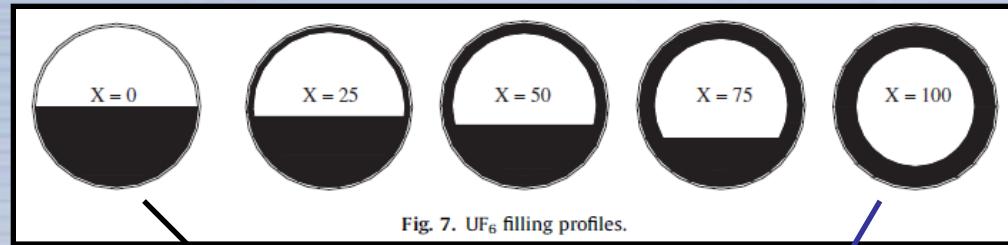


These spectra are normalized to more easily compare the distributions



Challenge: filling profile differences may require imaging

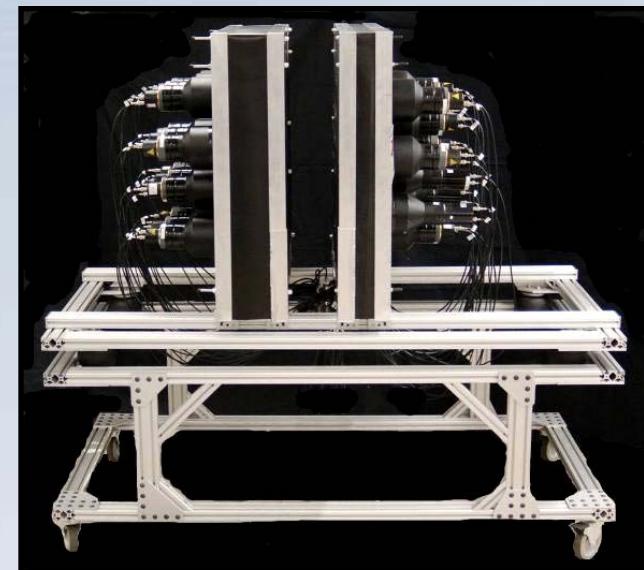
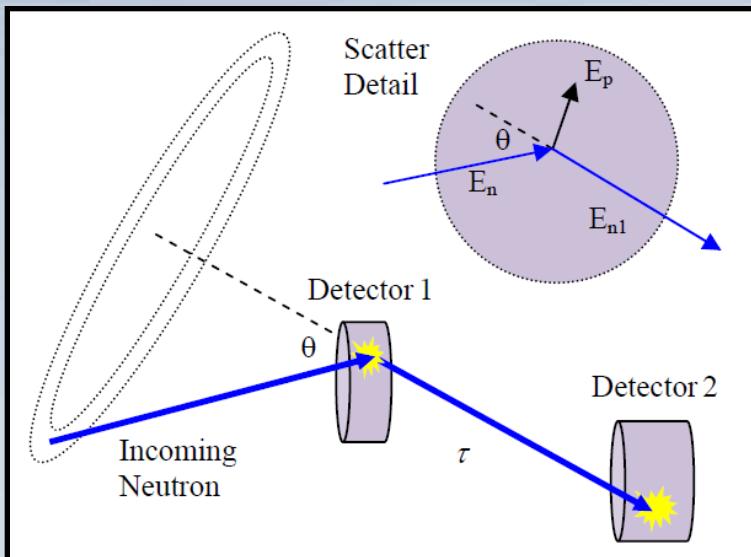
- Imaging may be necessary to couple with spectrometry





Experiments to characterize the Neutron Scatter Camera

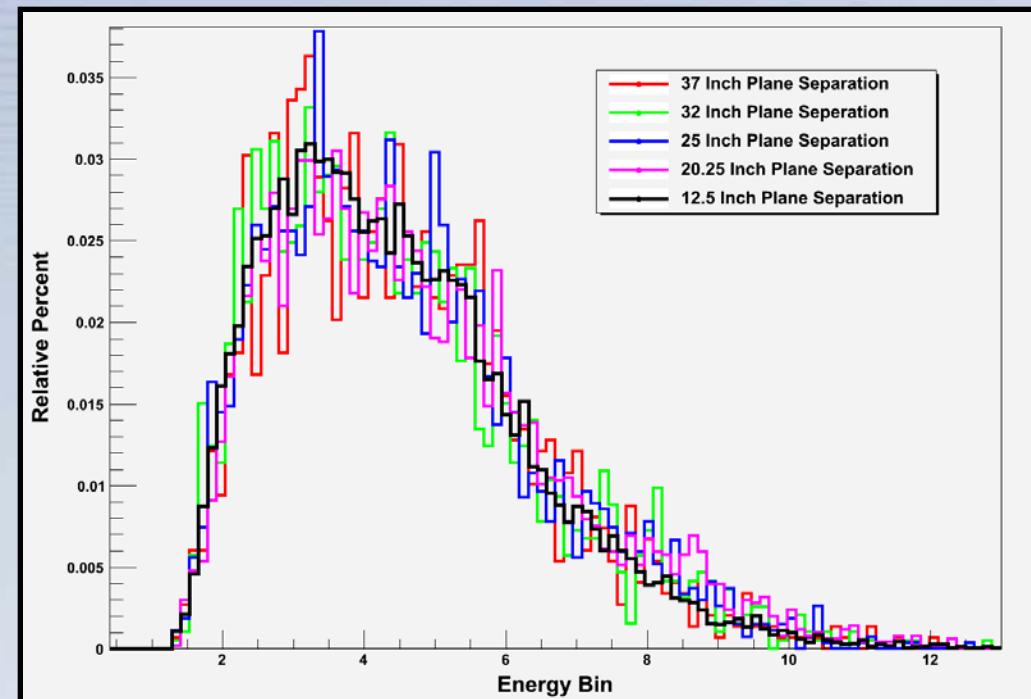
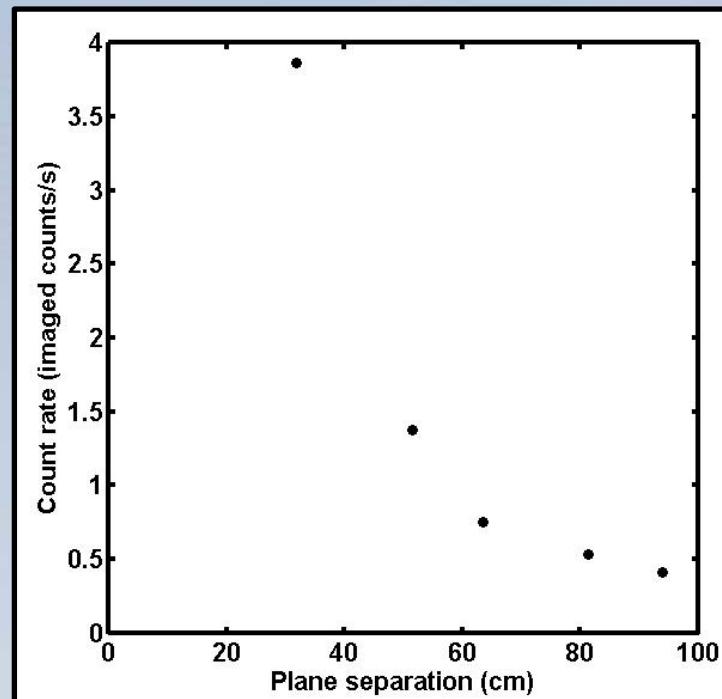
- We want to understand the tradeoff between energy resolution and double scatter rates as a function of plane spacing for the Neutron Scatter Camera
 - Close planes = high rate, degraded spectrum
 - Extended planes = low rate, best spectrum





Experiment: rates vs. plane spacing

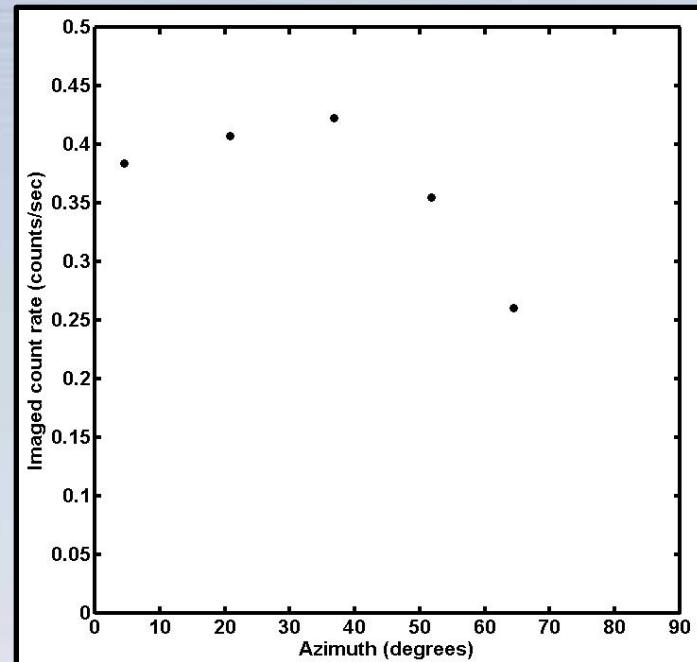
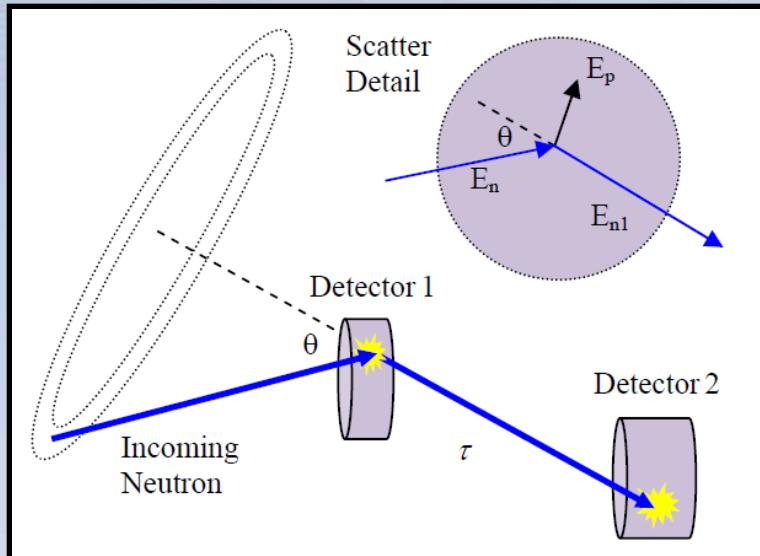
- Used an AmBe source in the B942 high bay
- DTRA camera with 4 rear cells missing (12+8)





Experiments: rates vs. camera angle

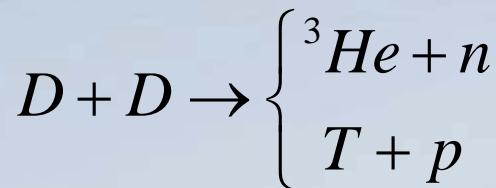
- The rate of imaged events will vary with the angle of the Neutron Scatter Camera relative to the source
 - Angles between cell pairs changes
 - Alters the energy partitioning between cells for identical neutrons





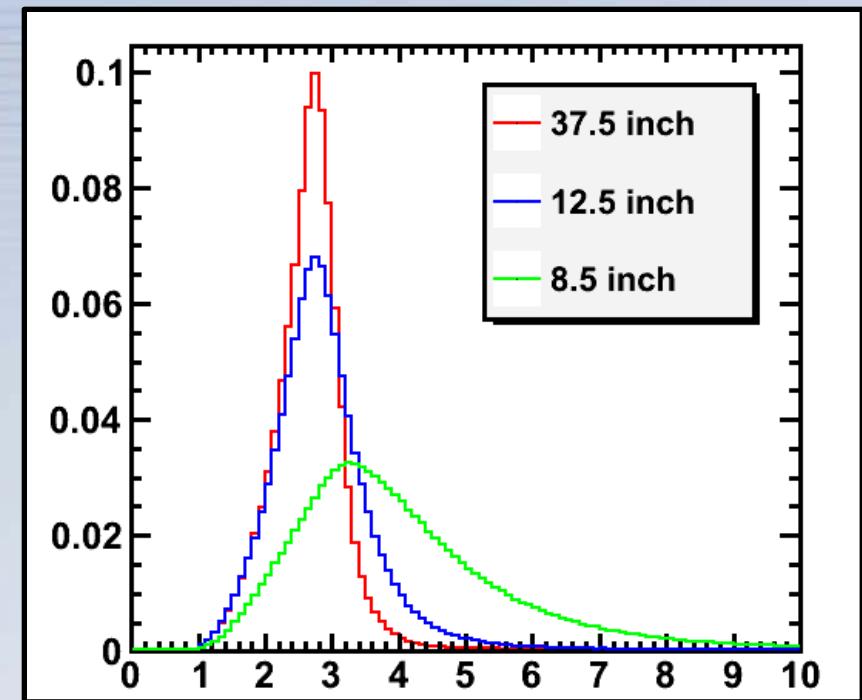
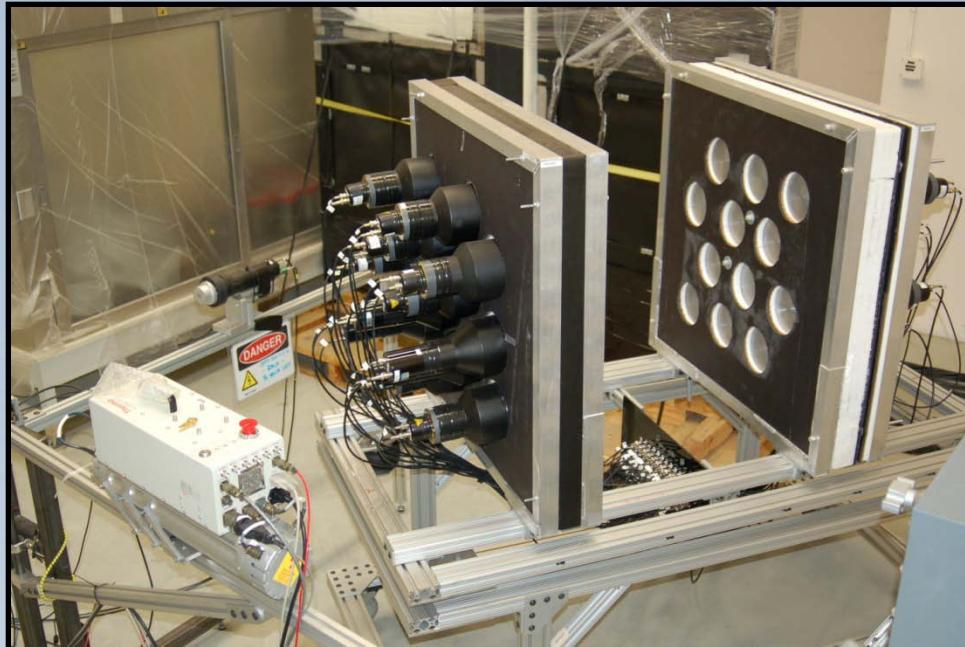
Quantifying the change in energy resolution

- The AmBe experiments are fine for rate studies, but it is difficult to quantify the energy resolution vs. plane spacing
- Use a D+D neutron generator
 - Monoenergetic neutrons of an appropriate energy: 2.45 MeV
 - Over time, also build in 14.1 MeV neutrons





D+D source experiments





Planning: field measurements

- It is important to measure filled 30B cylinders with the Neutron Scatter Camera to collect data with
 - The appropriate (α, n) source term
 - A complex source with multiplication, scattering
 - Appropriate rates (neutrons and gammas)
 - Realistic backgrounds
- Paducah Gaseous Diffusion Plant may be the place





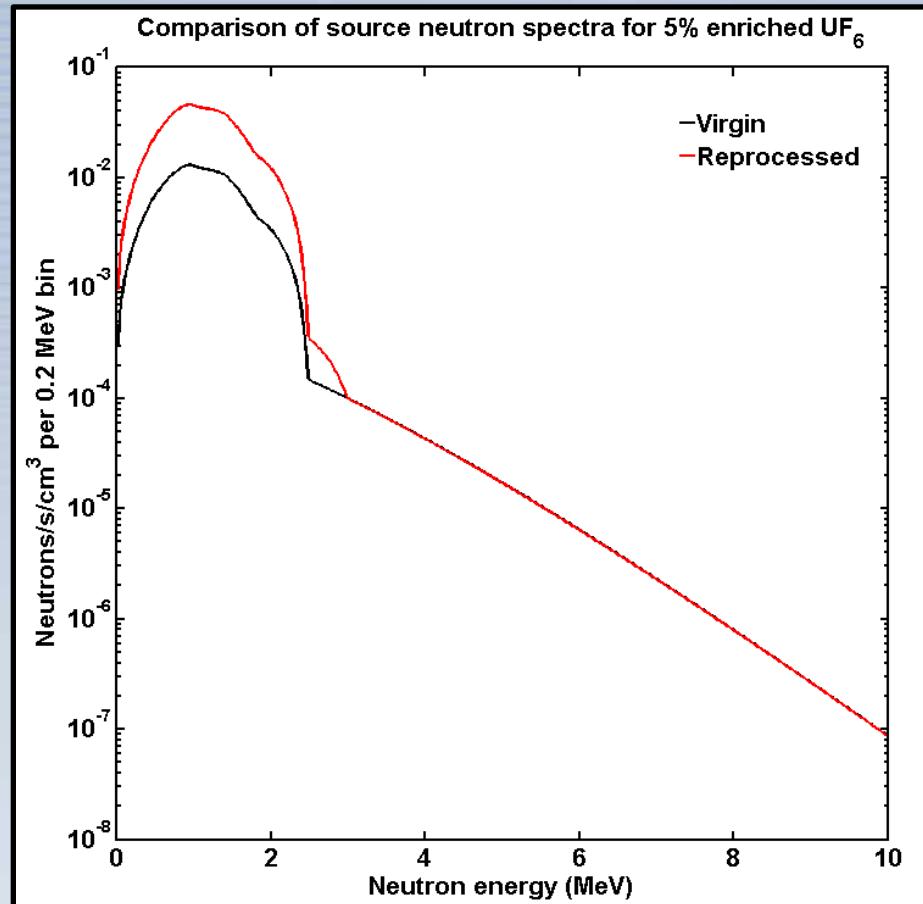
Discussion and Summary

- **Direct measurement of neutron signatures for UF_6 material accountancy appears to be a useful safeguards technique**
 - Two physical processes create neutrons with different energy spectra
 - Simulations indicate enrichment can be extracted from emitted neutrons, even after full transport
- **Advantages:**
 - Sensitive to entire cylinder volume (detect diversion attempts)
 - Imaging suppresses backgrounds from nearby cylinders, allows one to map material distribution within a cylinder (if important)
- **Accomplishments:**
 - Simulations show spectrum changes with enrichment (transport, detector response included)
 - Basic lab experiments to characterize Neutron Scatter Camera



Challenge: virgin vs. reprocessed UF_6

Isotopes	Amount (%)	Neutron source 1/s/ton UF_6		Relative yield (%)	
		Spontaneous fission	$\text{UF}_6(\alpha, n)$		
Depleted U					
^{232}U	0	0	0	0	0
^{234}U	0.00234	0	7.85E+03	28.9	
^{235}U	0.3	0	1.86E+03	6.9	
^{236}U	0	0	0	0	
^{238}U	99.6977	9.96E+03	7.53E+03	64.3	
Natural U					
^{232}U	0	0	0	0	
^{234}U	0.0056	0	1.88E+04	51.3	
^{235}U	0.718	0	4.45E+02	1.2	
^{236}U	0	0	0	0	
^{238}U	99.2764	9.92E+03	7.49E+03	47.5	
LEU 5%					
^{232}U	0	0	0	0	
^{234}U	0.039	0	1.31E+05	86.9	
^{235}U	5	0	3.10E+03	2.1	
^{236}U	0	0	0	0	
^{238}U	94.961	9.50E+03	7.17E+03	11.1	
LEU 5% repr.					
^{232}U	7.60E-07	0	1.66E+04	3.26	
^{234}U	0.133	0	4.46E+05	87.48	
^{235}U	5	0	3.10E+03	0.61	
^{236}U	1.55	0	2.76E+04	5.41	
^{238}U	93.317	9.50E+03	7.04E+03	3.24	





Considering PuO₂ holdup accountancy

- Consider PuO₂ holdup in a reprocessing facility
 - Gloveboxes
 - Pipes, components
- Geometry is variable, materials self-absorbing
 - Gamma-based measurements are not accurate
- Can spectrometry and imaging be used to locate and quantify material?



La Hague reprocessing facility



Plutonium vs. uranium measurements

- Consider PuO_2 . How do the proposed measurements compare to UF_6 ?
- **PuO_2 advantages:**
 - Spectrum: (α, n) on O produces higher-energy neutrons
 - Rates: There are $>1000x$ more neutrons emitted from PuO_2 per cm^3
- **PuO_2 disadvantage:**
 - Isotopes are messy...f(initial enrichment, burnup, cooling time)



Considering the PuO_2 neutron spectrum

- Each isotopic mix produces unique (α, n) and SF neutron spectra
- Ideal: use spectrum to estimate each isotope's individual contributions and mass
- Reality: not enough information; must use (α, n) , SF components only (for pure neutron measurements)

