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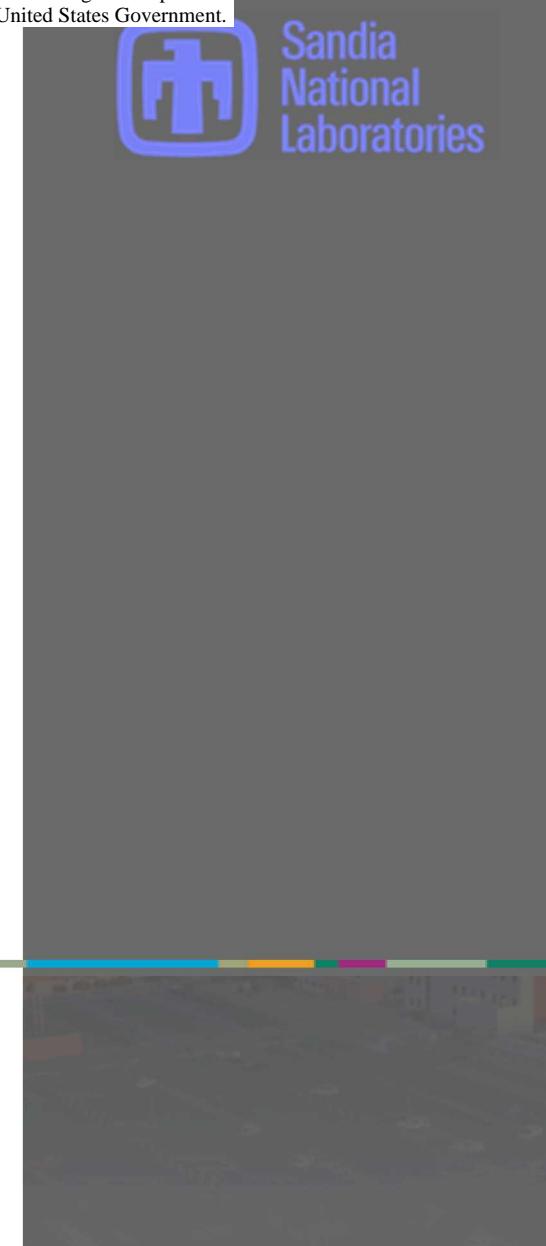
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# Helium Bubble Evolution in Implanted Erbium Deuteride Bulk and Multilayered Structures

PRESENTED BY

**Caitlin Taylor**, Sandia National Laboratories, ABQ, NM

co-authors are listed in the acknowledgements ☺



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# Outline

**Objectives:** 1) to understand the fundamental mechanisms behind accelerated He release in tritium storage materials, 2) and to understand how He interacts with interfaces in multilayered structures

## ➤ **Background**

- tritium storage and helium in metal tritides

## ➤ **Part I: Bulk ErD<sub>2</sub>**

- microstructural changes after He implantation into ErD<sub>2</sub>

## ➤ **Part II: Multilayers**

- development of an ErD<sub>2</sub>/Mo multilayered hydride structure and initial He implantation results

## ➤ **Summary**

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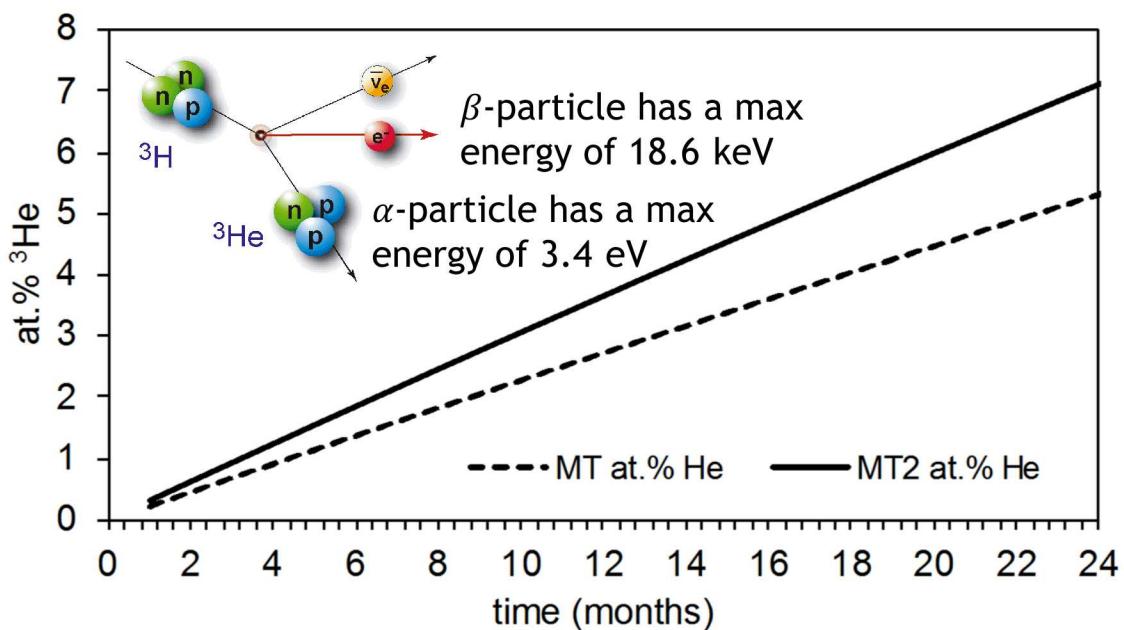
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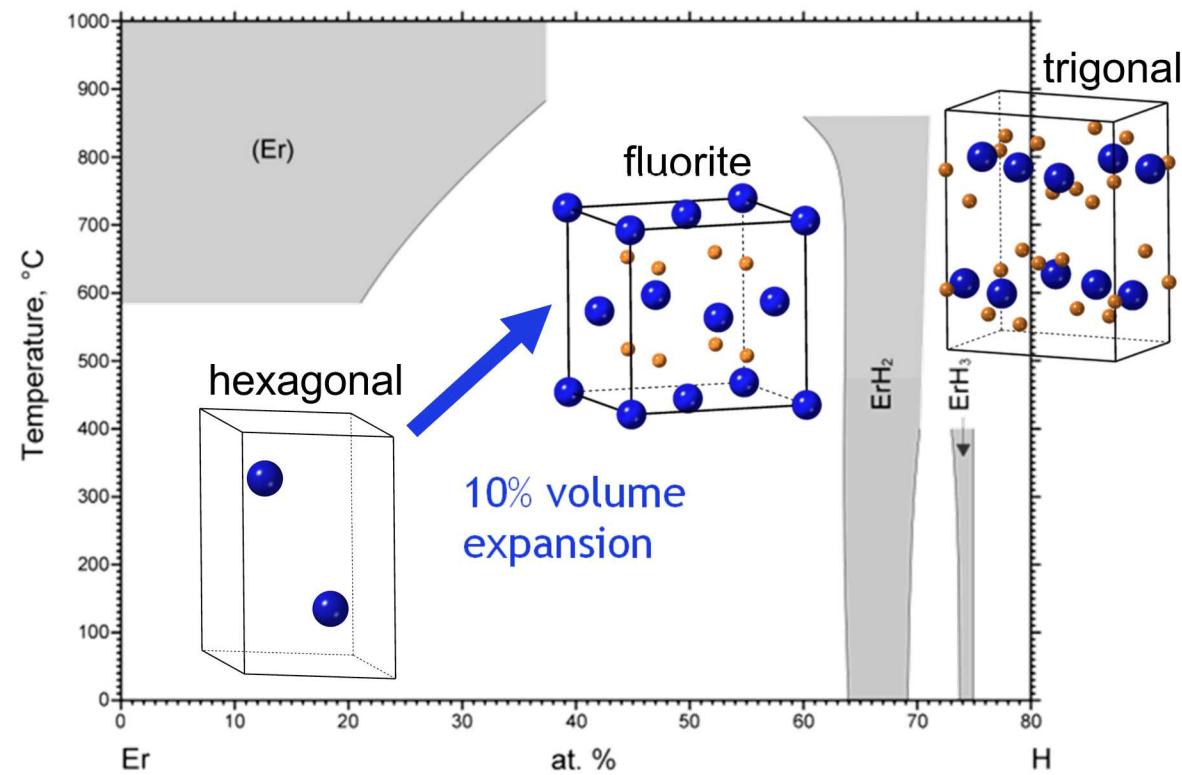
## ➤ **Summary**

# Metal Hydrides are Used for Tritium Storage

## Tritium Storage

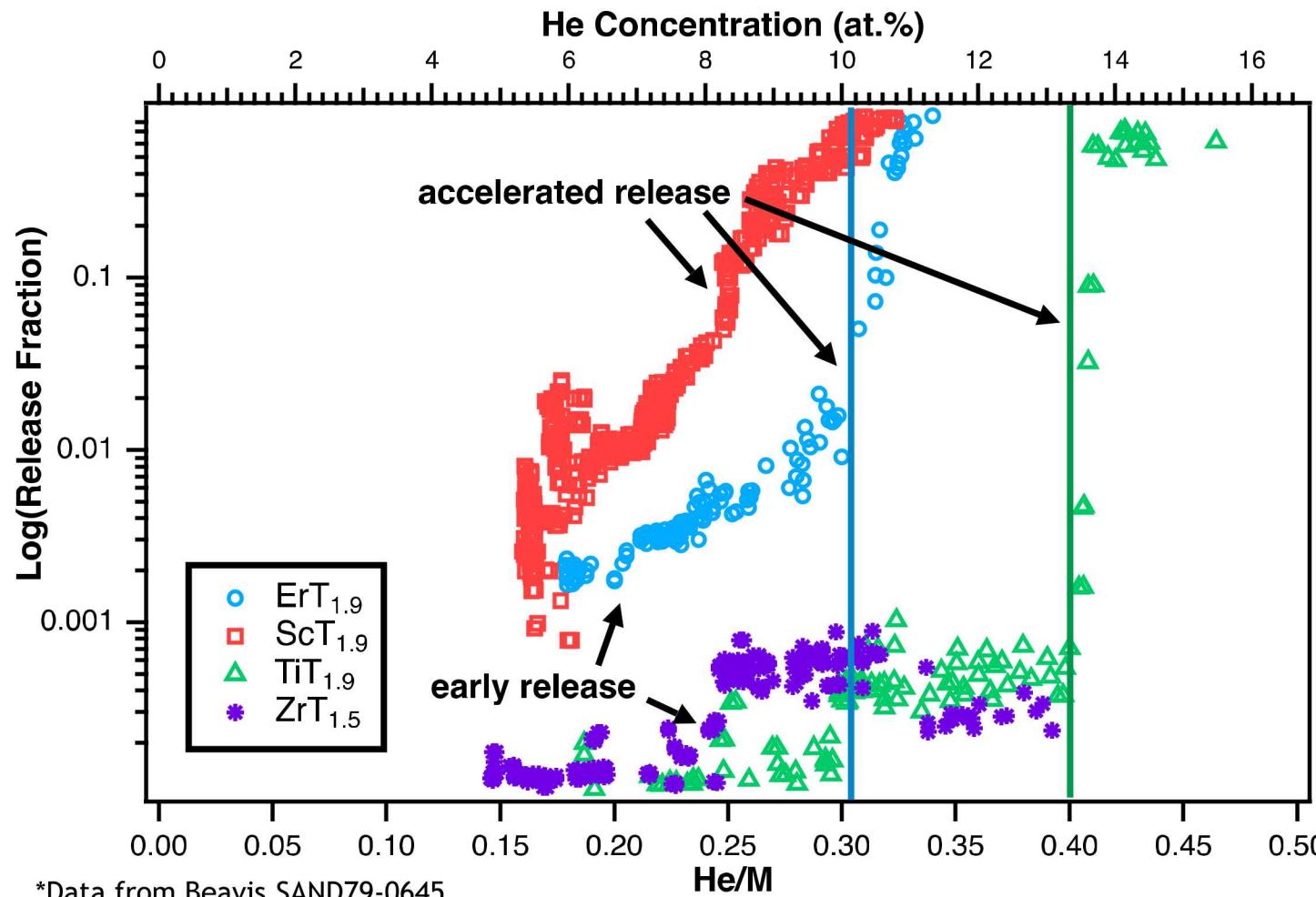


## Metal to Hydride Phase Transformation



- Transition metals (groups III & IV), lanthanides, and actinides are used because they form stable hydrides with a density  $\geq$  solid  $\text{H}_2$ .
- Phase diagrams can be quite complex. Most hydrides have several phases.

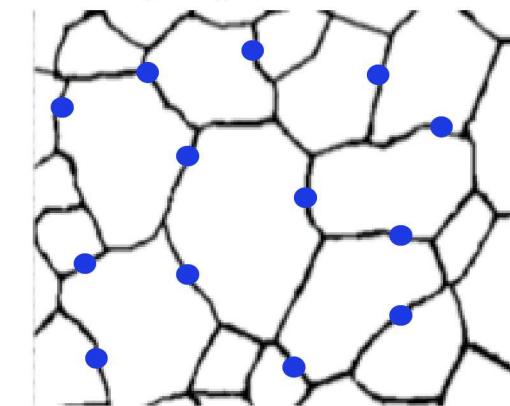
## Tritides Exhibit Accelerated He Release when ${}^3\text{He}$ Capacity is Reached



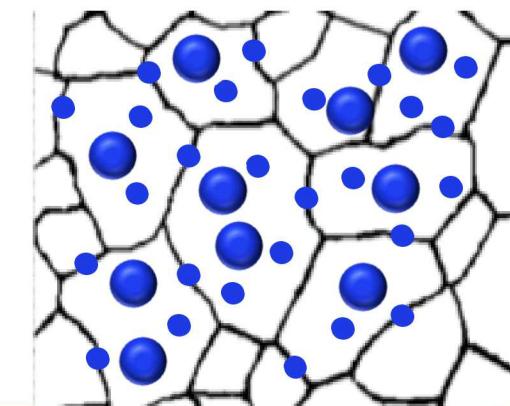
\*Data from Beavis SAND79-0645

- Accelerated release occurs when  $\{\text{He release rate}\} = \{\text{He generation rate}\}$
- Timescale is 0 months for ScT<sub>2</sub>, ~35 months for ErT<sub>2</sub>, ~48 months for TiT<sub>2</sub>

Early release occurs from  ${}^3\text{H}$  decay in grain boundaries

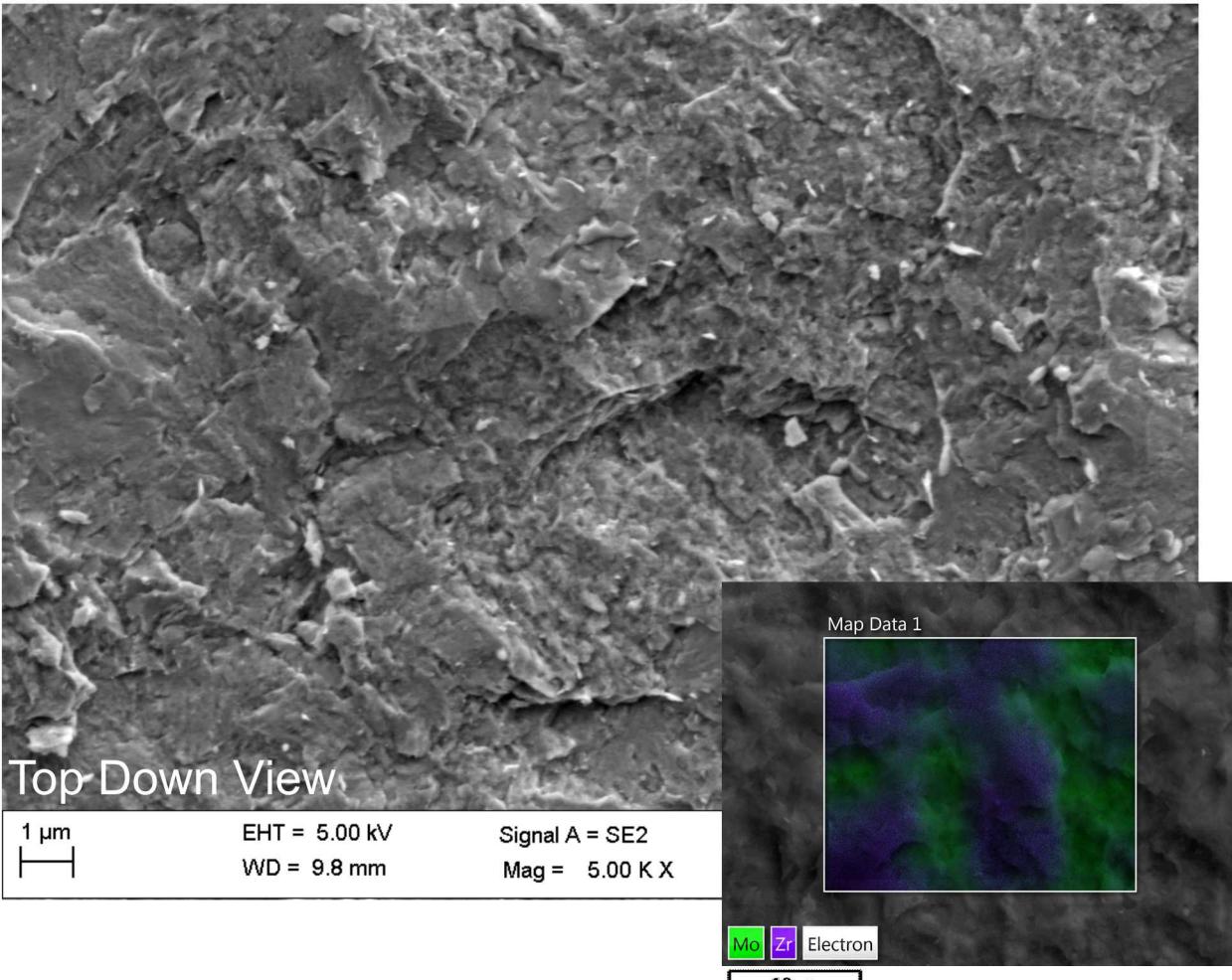


Based on Wolfer theory, accelerated release probably occurs when 1) lattice stresses are such that loop punching and therefore He bubble growth can no longer occur or 2) inter-bubble fracture causes cracking

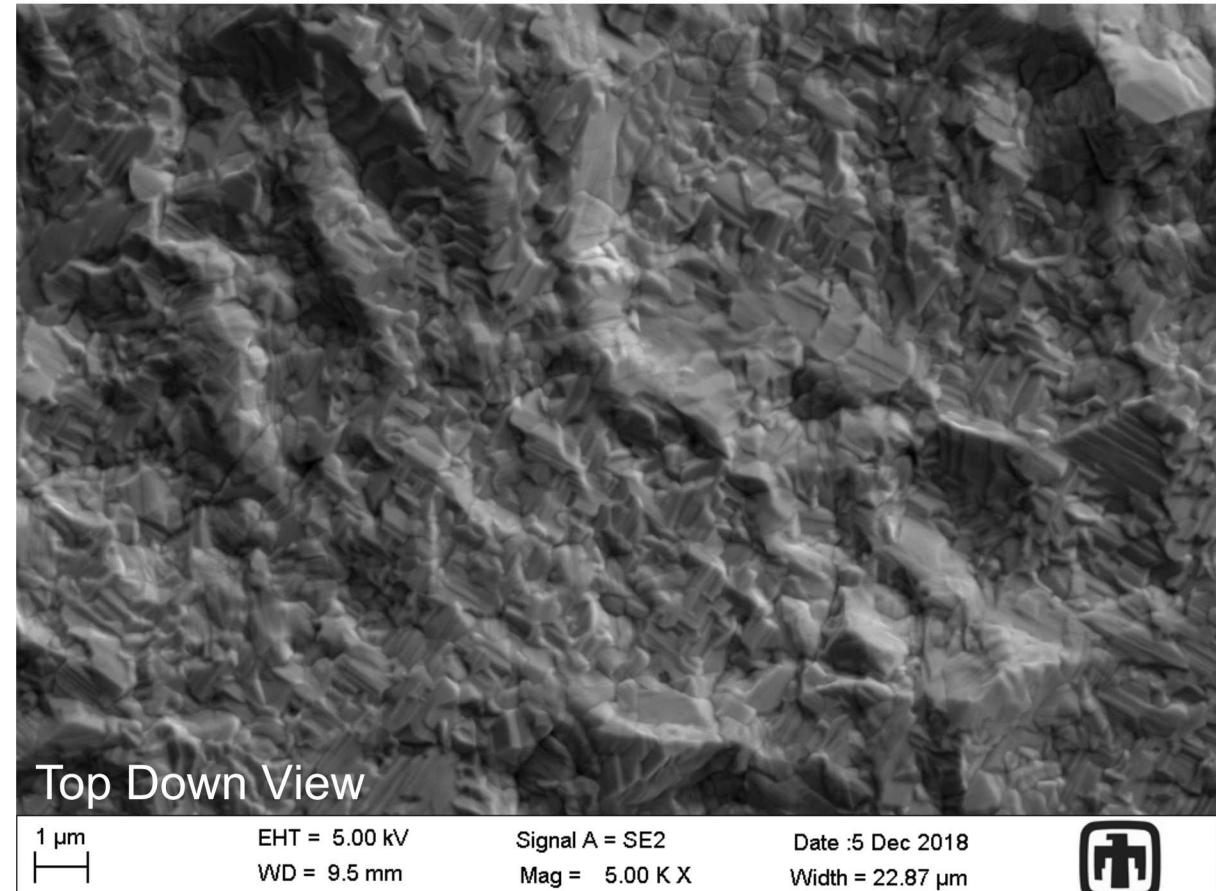


# Spalling can Occur with Age, Especially Near Accelerated Release

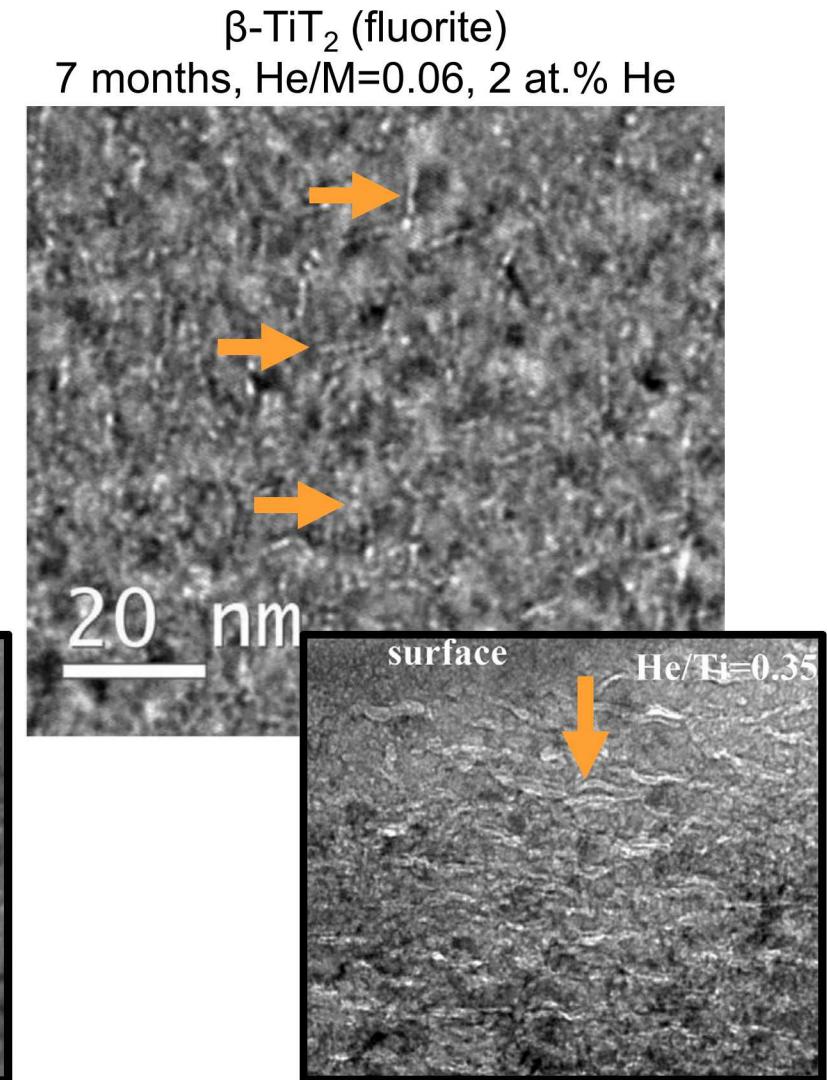
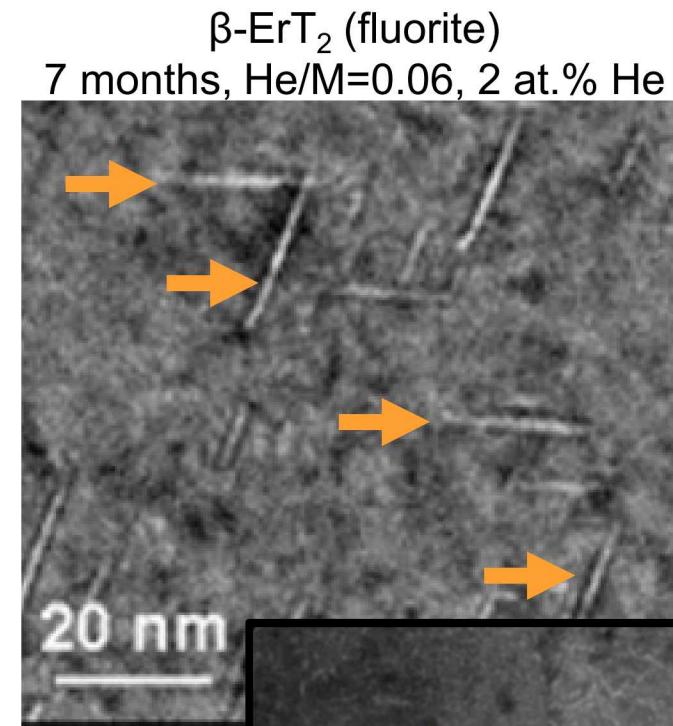
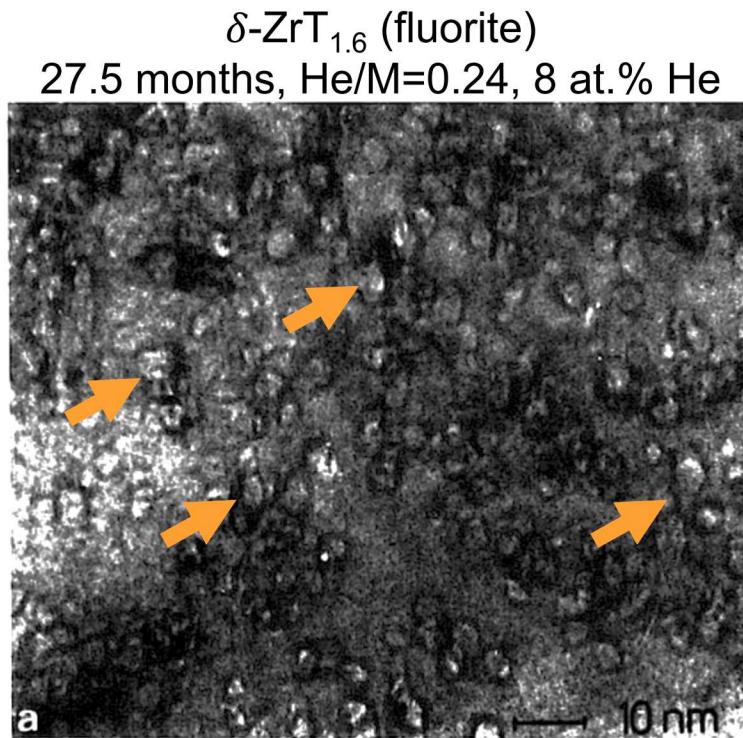
ZrT<sub>2</sub> Film in Accelerated Release (69 months)



ZrT<sub>2</sub> Film in Early Release (22 months)



# Nanometer Sized He Bubbles form During Early Release Stage



- Bubbles become faceted in  $\text{ZrT}_{1.6}$ , so far no evidence of platelet or crack formation.
- He platelets form in  $\text{ErT}_2$  and  $\text{TiT}_2$ , eventually connecting to form large cracks that appear to reach the surface in some cases.
- No publications have connected surface microstructure with bubble microstructure (SEM to TEM).

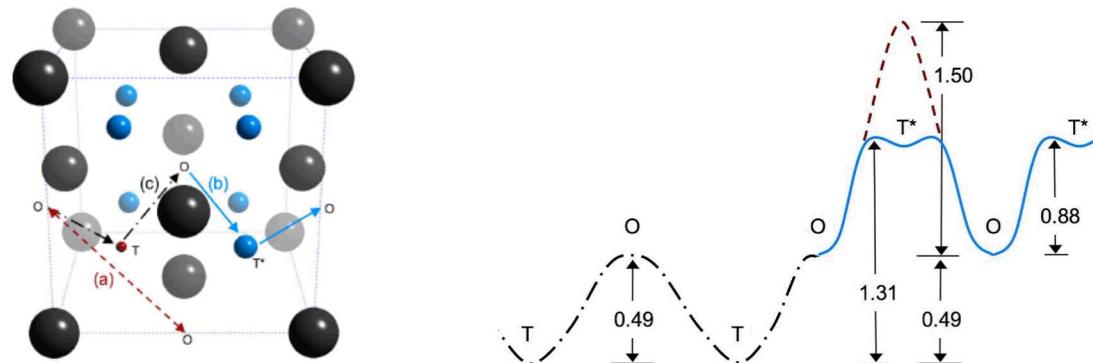
Zr work: Schober et al JNM 141-143 (1986) 453-457.

Er work: Bond et al, J. Appl. Phys 107 (2010) 083514 & Snow et al JNM 453 (2014) 296-306.

Ti work: Wang et al, JNM 509 (2018) 700-706

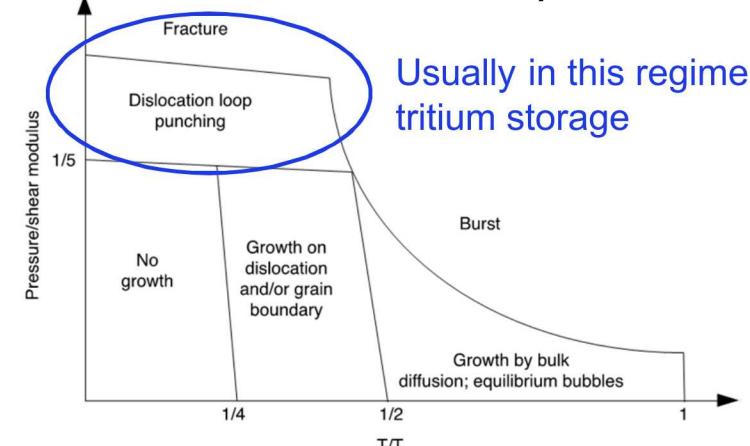
# He Bubble Nucleation, Diffusion and Growth in Metal Tritides

Helium Traps in T-site & Migrates through T-T diffusion in  $\beta$ -ErT<sub>2</sub>



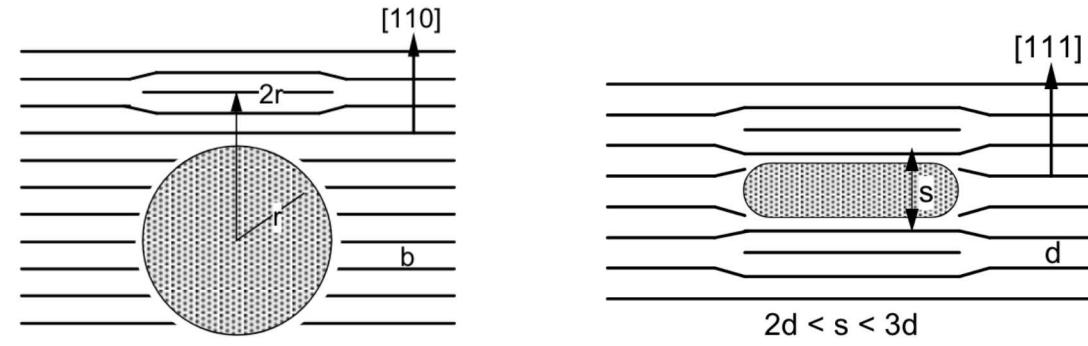
Wixom et al JAP 103 (2008) 123708

## Wolfer He Bubble “Growth Map” for Metals



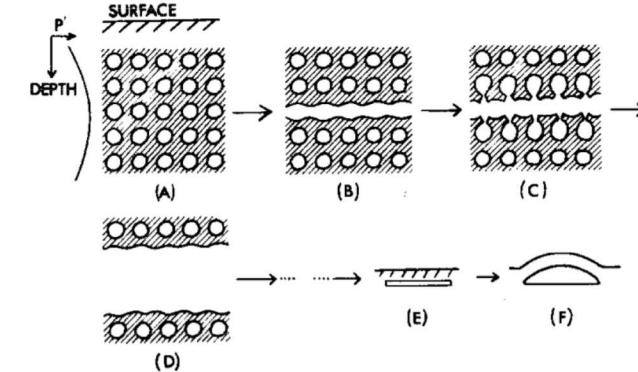
Wolfer Radiation Effects in Plutonium *Los Alamos Science* 26 (2000)

Spherical Bubbles Growth by “Dislocation Loop Punching” and Platelets can Grow by “Dislocation Dipole Expansion”



Cowgill, IHISM Workshop, St. Petersburg, Russia, 2-6 July 2007

## Inter-Bubble Fracture Accelerated Helium Release Mechanism

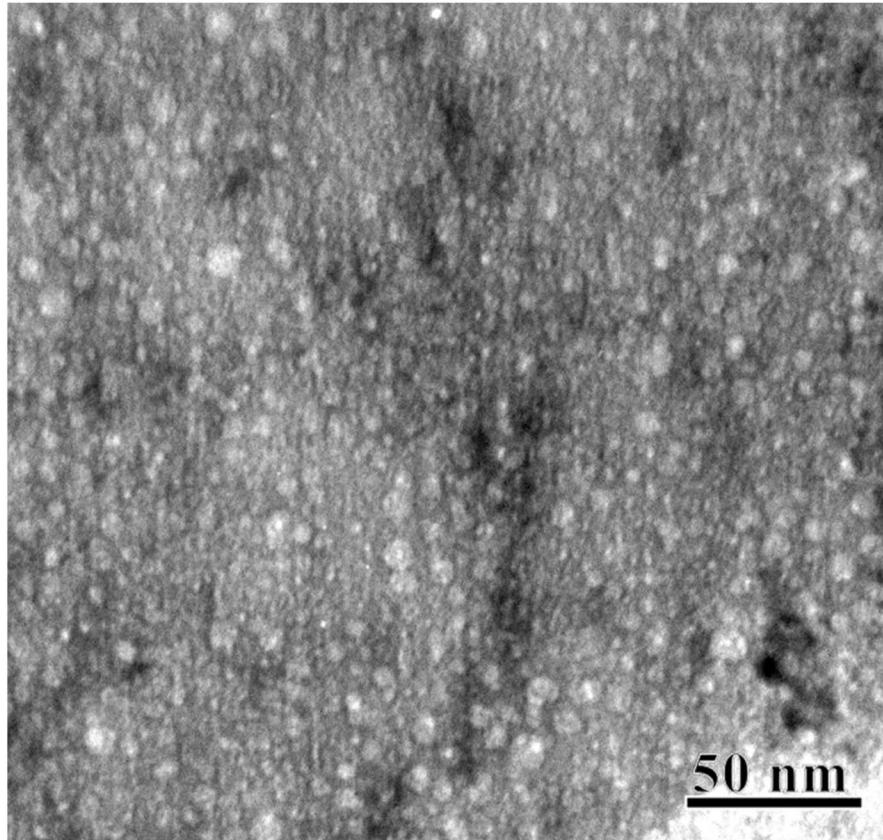


Evans JNM 68 (1977) 129-140 & Evans JNM 1978 (76 & 77) 228-234.

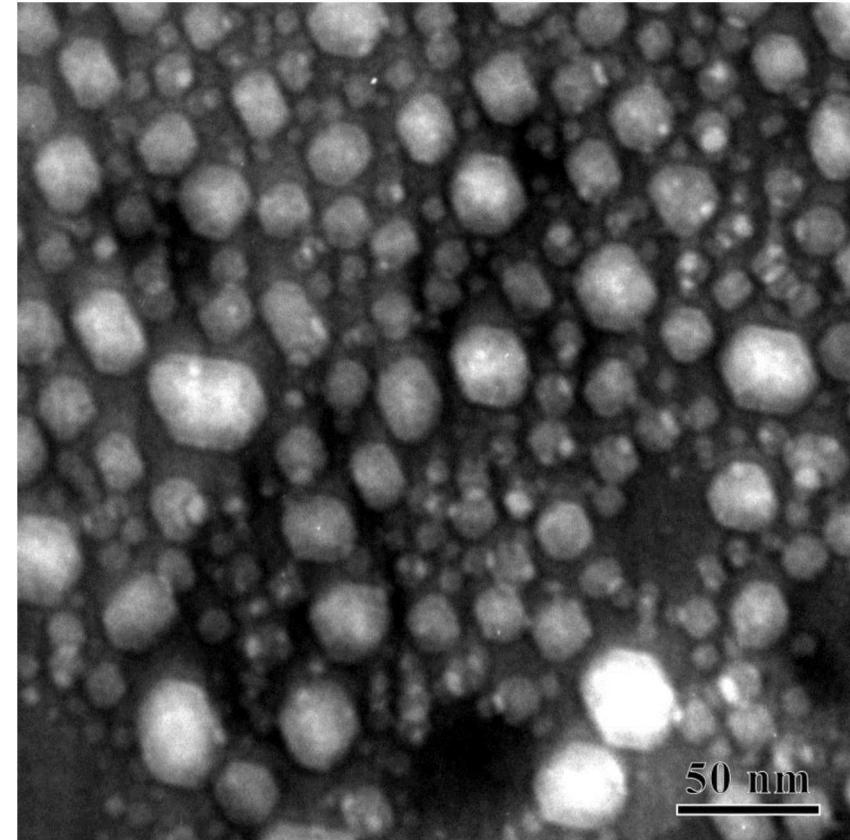
## He Bubble Growth Increases with Storage Temperature

In-situ TEM anneal of **tritium aged Pd- 5% Ni** alloy containing ~12 at.% He

In-situ annealing at  $400^{\circ}\text{C}$  ,  $T/T_M = 0.37$



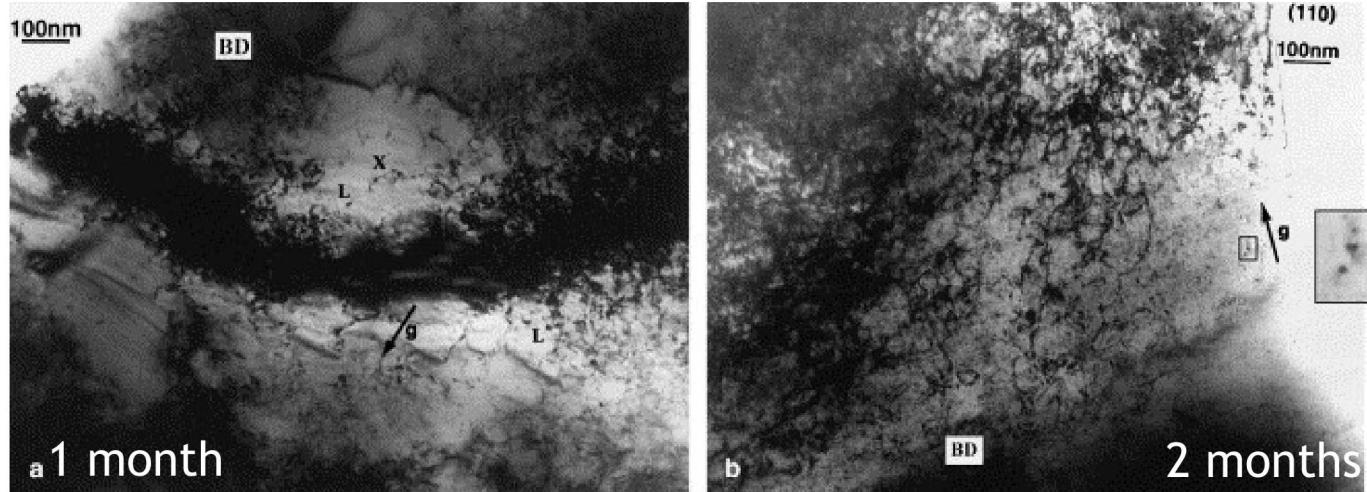
In-situ annealing at  $900^{\circ}\text{C}$  ,  $T/T_M = 0.64$



- Since bubble density is extremely high in tritides, growth occurs by 1) He diffusion to bubble or 2) absorption of nearby cavities (potentially by nanoscale inter-bubble fracture between cavities?). No cavity migration.

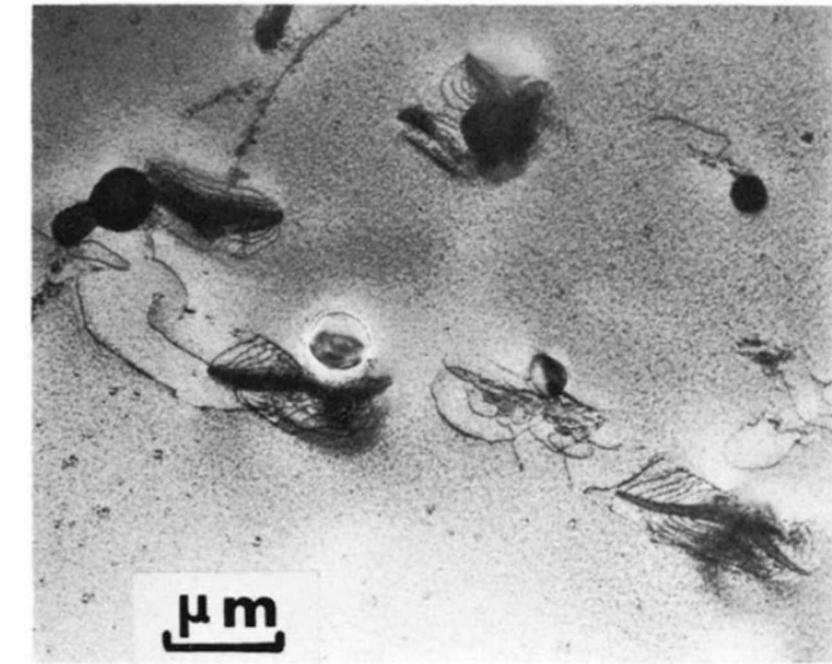
# Intrinsic Defects are Produced During the Hydriding Process

Dense Dislocation Network has been Imaged in PdT



Thiébaut et al JNM 277 (2000) 217-225

Dislocations Induced by Zr Hydride Nucleation in Zr metal

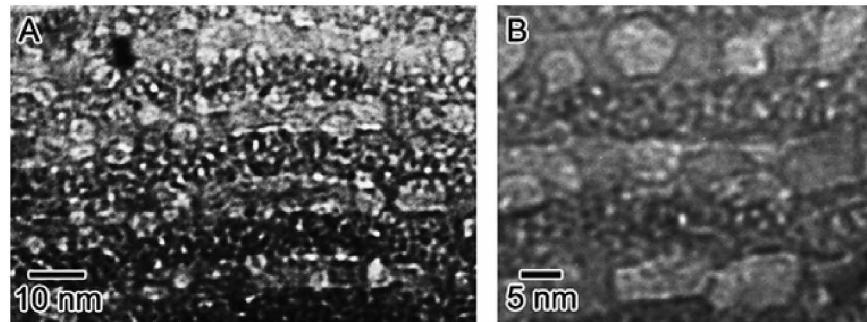


Carpenter JNM 48 (1973) 267-276

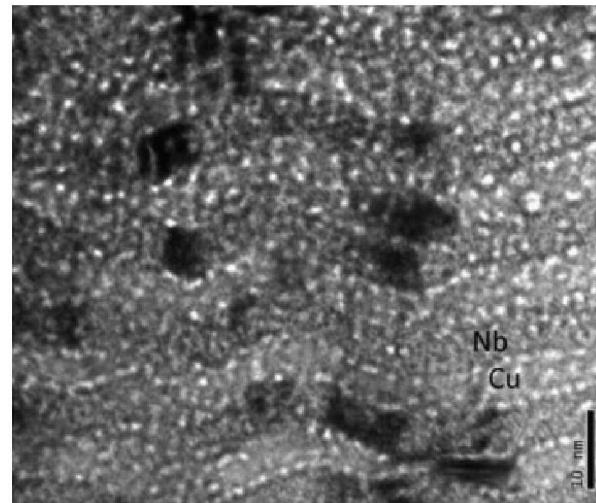
- Dislocations are created in the metal phase during hydride formation.
- Dislocation density continues to increase with hydriding/de-hydriding cycles.
- Hydriding-induced dislocations may act as trapping sites for He as metal tritides age.

# Engineered Nanostructures Have Been Shown to Trap He at Interfaces

## Multilayered Nanocomposites

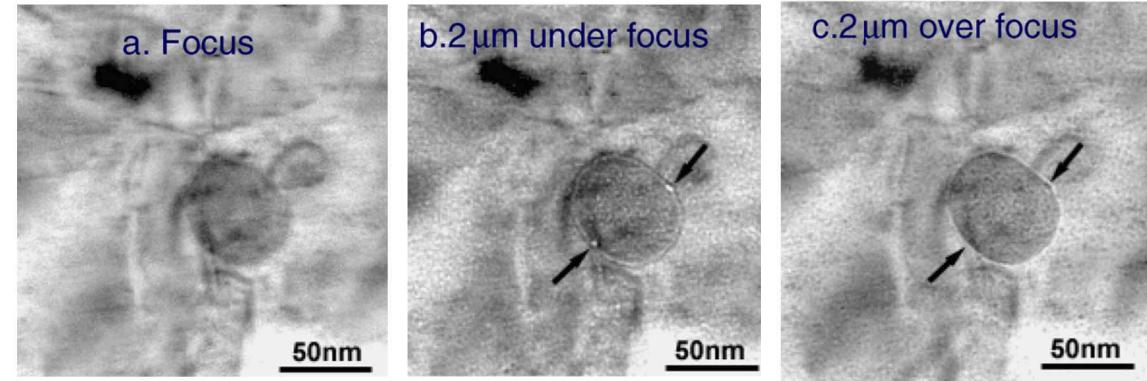


Hattar *et al.* Scripta Materialia 58 (2008) 541-544

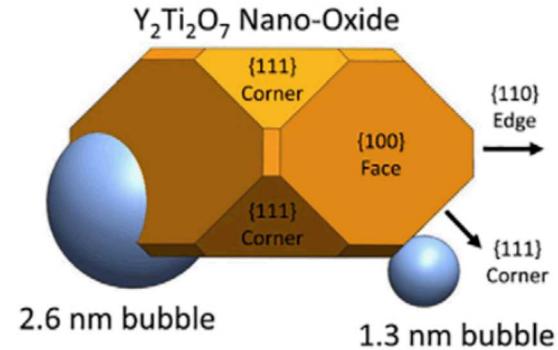


Demkowicz *et al* Current Opinion in Solid State and Materials Science 16 (2012) 102-108

## Nanostructured Ferritic Alloys (NFAs)



Yamamoto *et al* JNM 367-370 (2007) 399-410



Odette Scripta Materialia 143 (2018) 142-148

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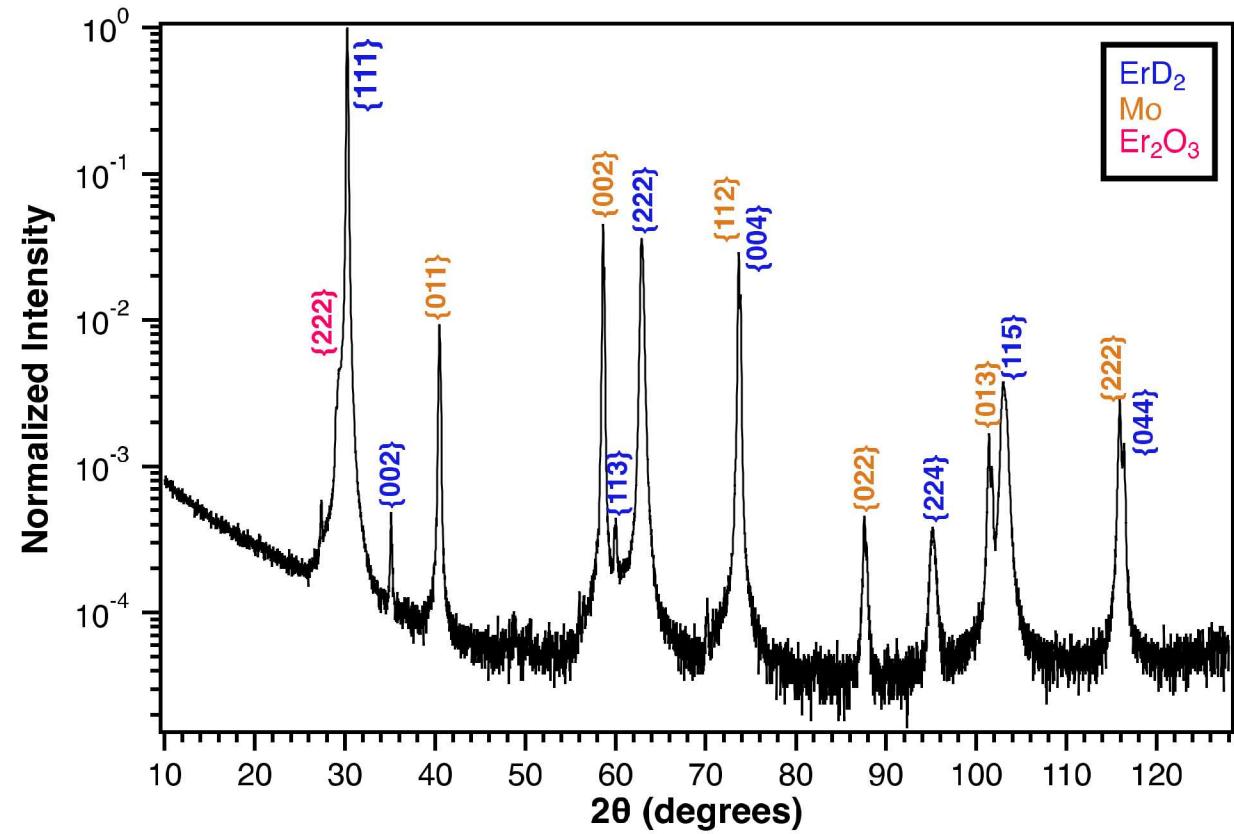
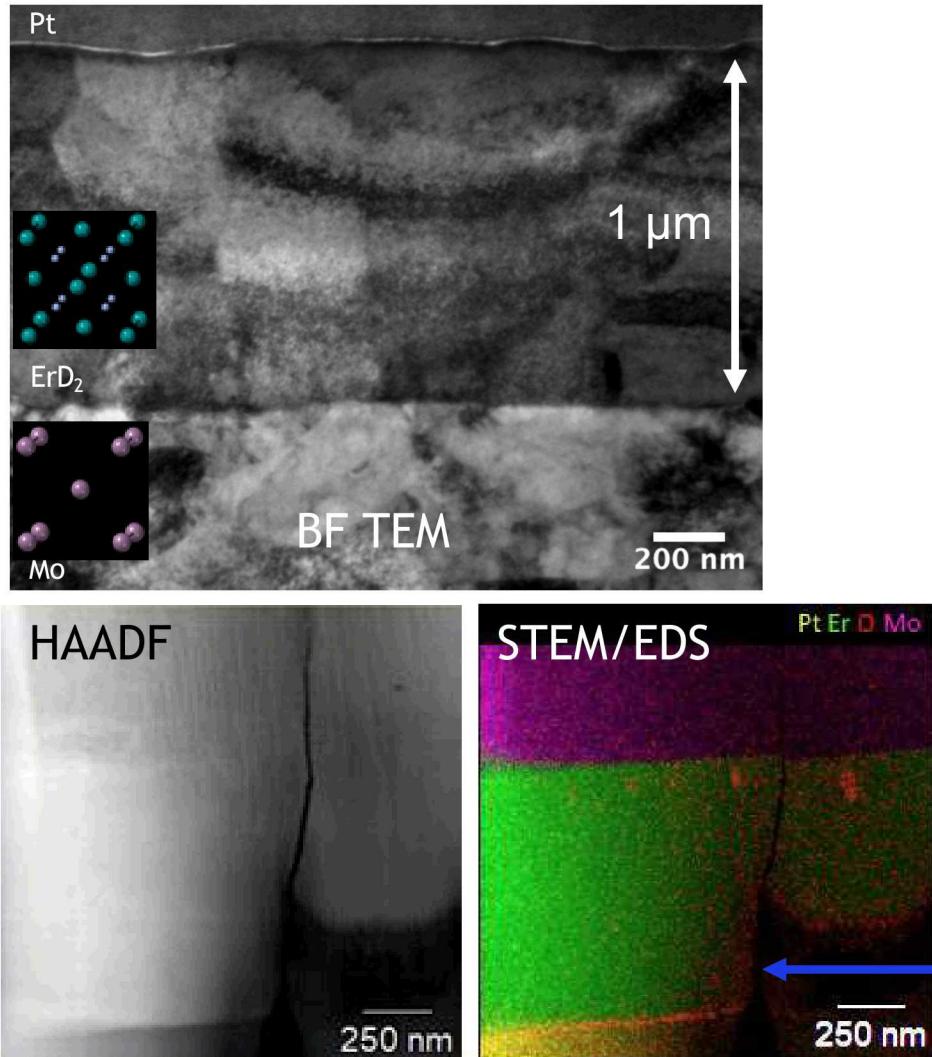
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## ➤ Summary

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## As-Deposited/Loaded ErD<sub>2</sub> Samples were Characterized with TEM & XRD

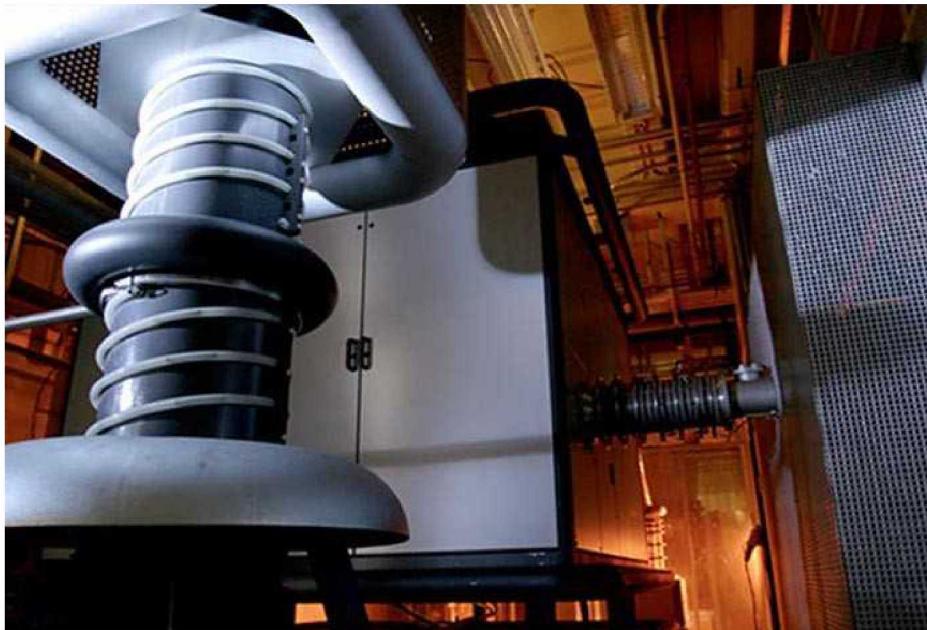
- 1  $\mu\text{m}$  thick Er films were evaporated onto polished Mo substrates and loaded with  $^2\text{H}$ , a surrogate for  $^3\text{H}$ .



- Very little oxide in XRD pattern

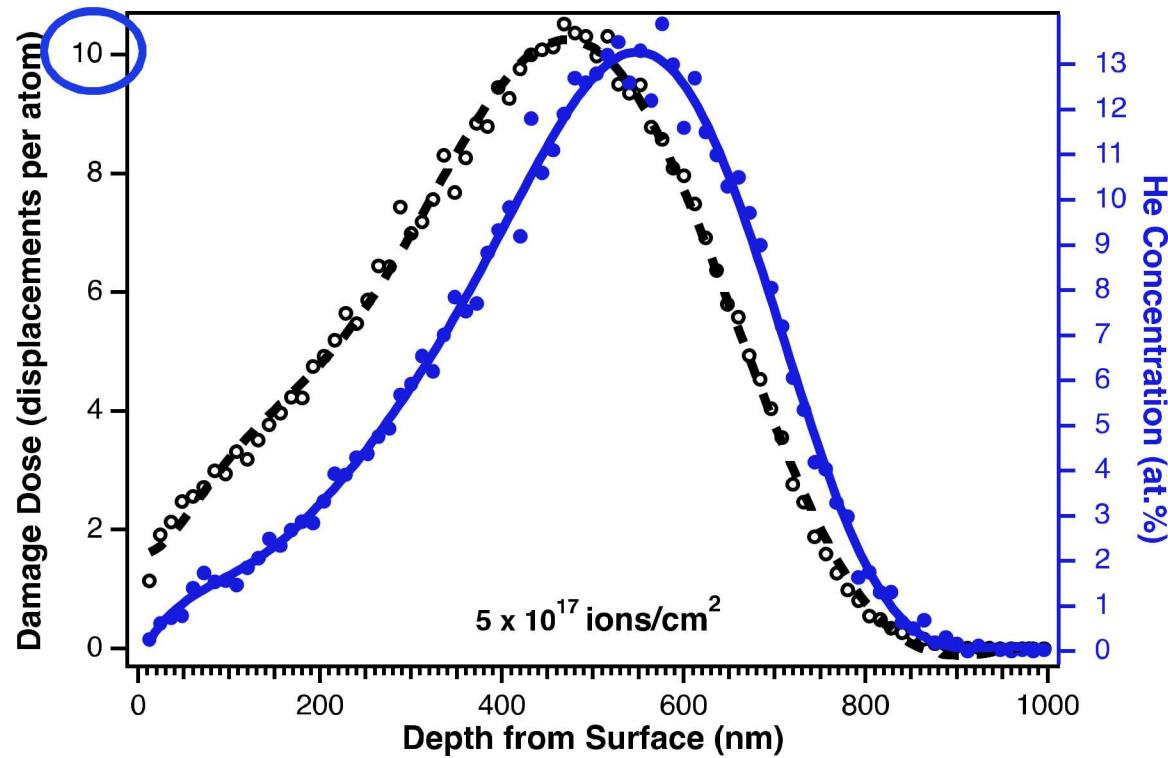
# Samples were He Implanted to Simulate Long-Term Tritium Decay

Danfysik Implanter @ LANL



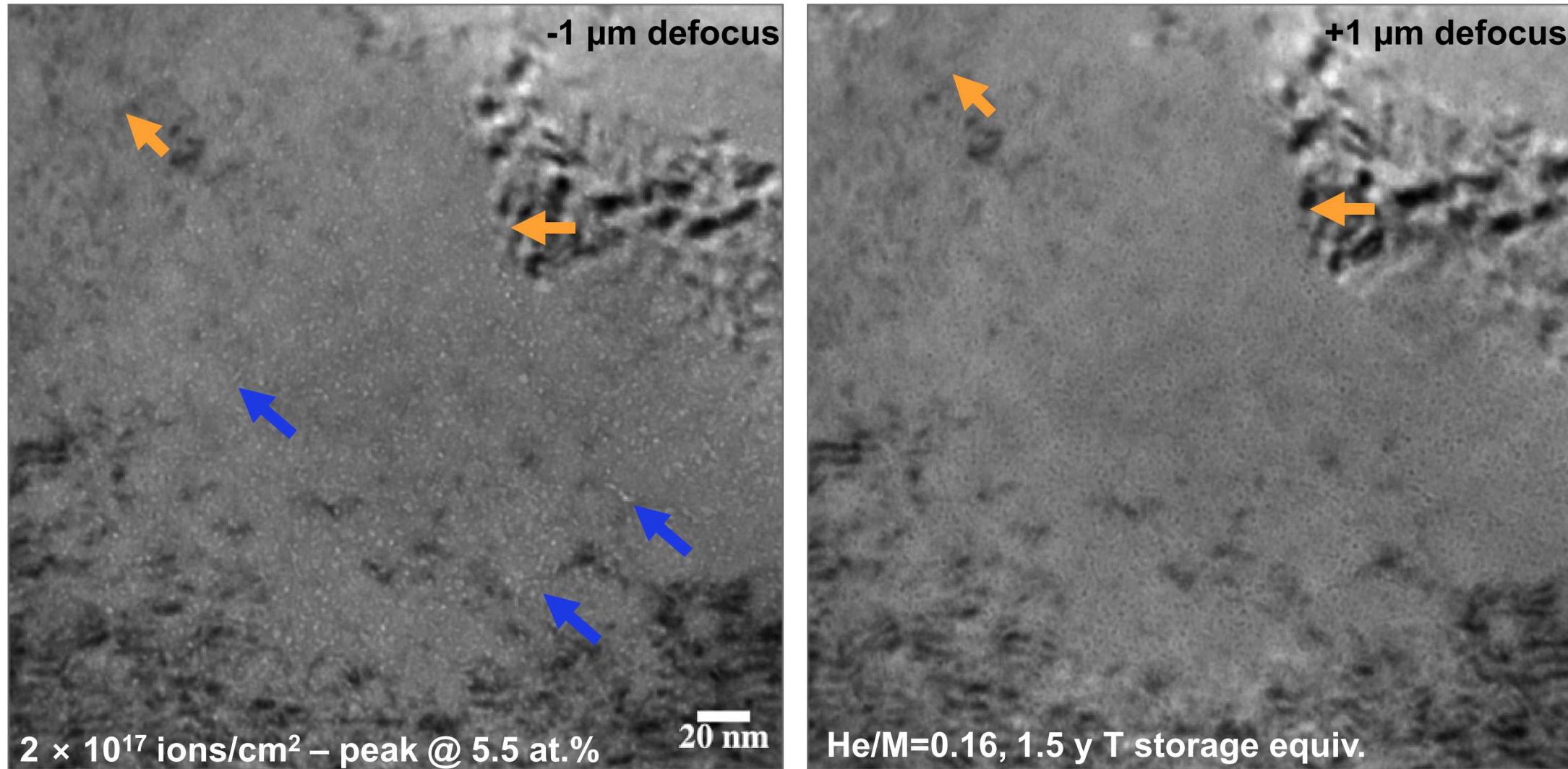
LANL IBML Website: <https://www.lanl.gov/science-innovation/science-facilities/ion-beam-materials-lab/>

Bulk Implantations were done using 120 keV <sup>4</sup>He



- He implantation is used to reach “accelerated release” level He concentrations (>10 at.% He) in a day, rather than several years. Removes radioactive contamination hazards.
- Implantation is not an exact surrogate for <sup>3</sup>H β-decay, which causes zero atomic displacement damage and continuously removes H from the lattice.

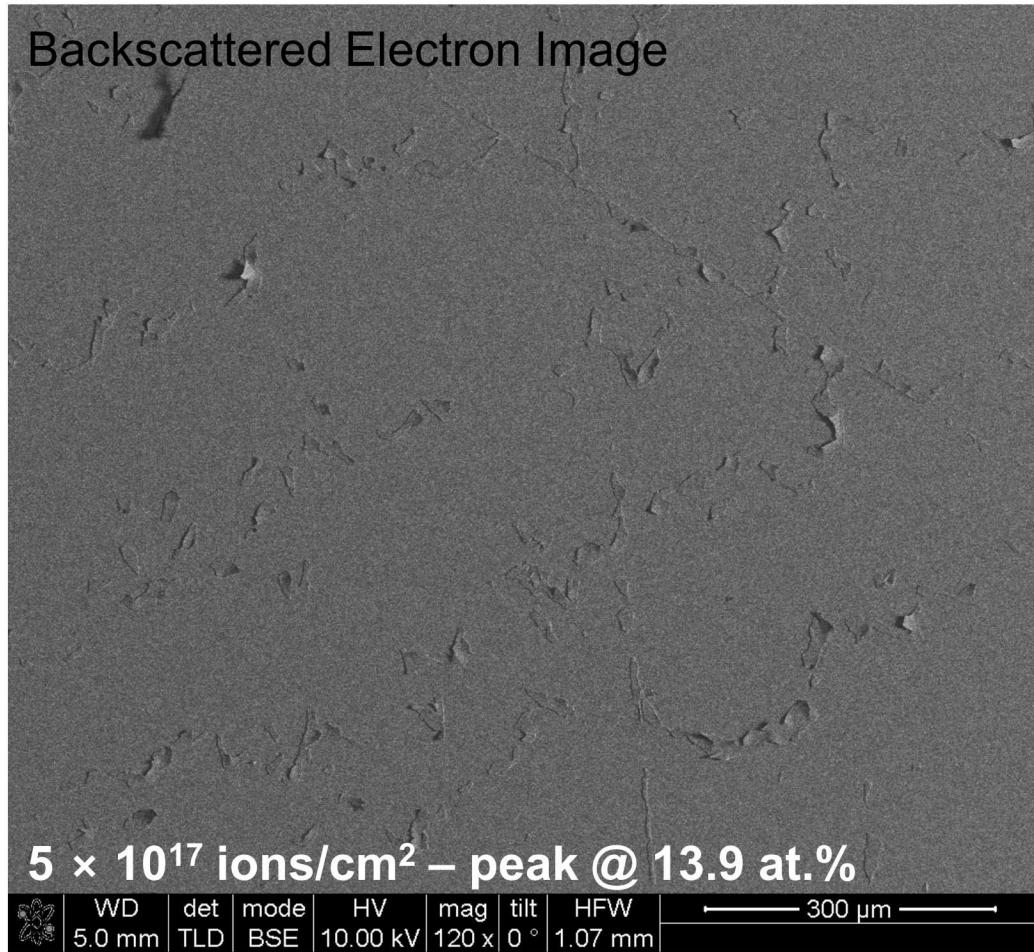
## Nanometer Sized He Bubbles were Observed After 1-3 at.% He



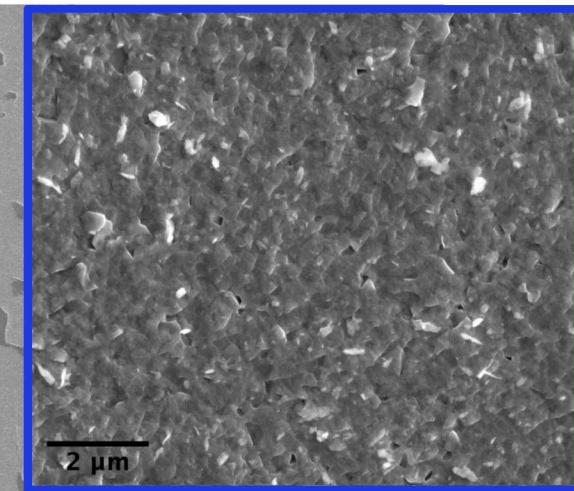
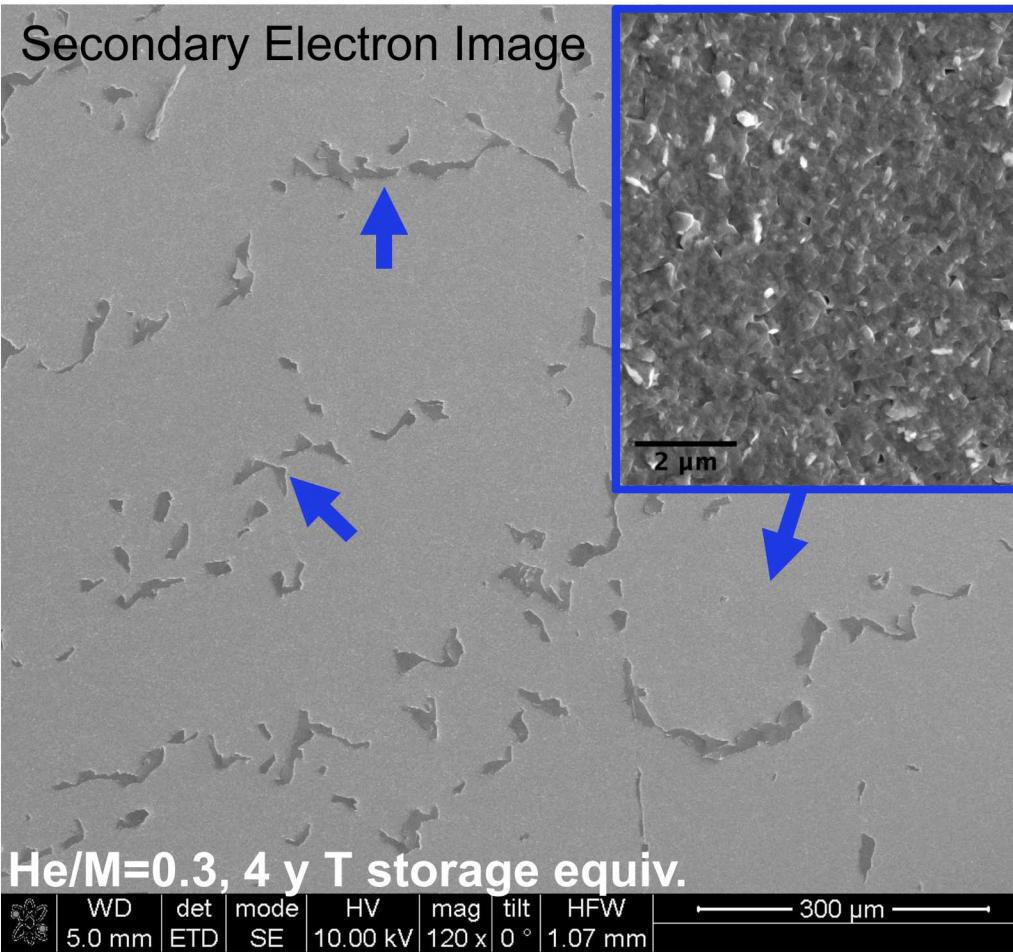
- He bubbles were found starting at 1.7 at.% (low density). Grain boundary nucleation & some evidence of linkage.

# Significant Surface Spalling Occurred after Implantation to High Dose

Backscattered Electron Image



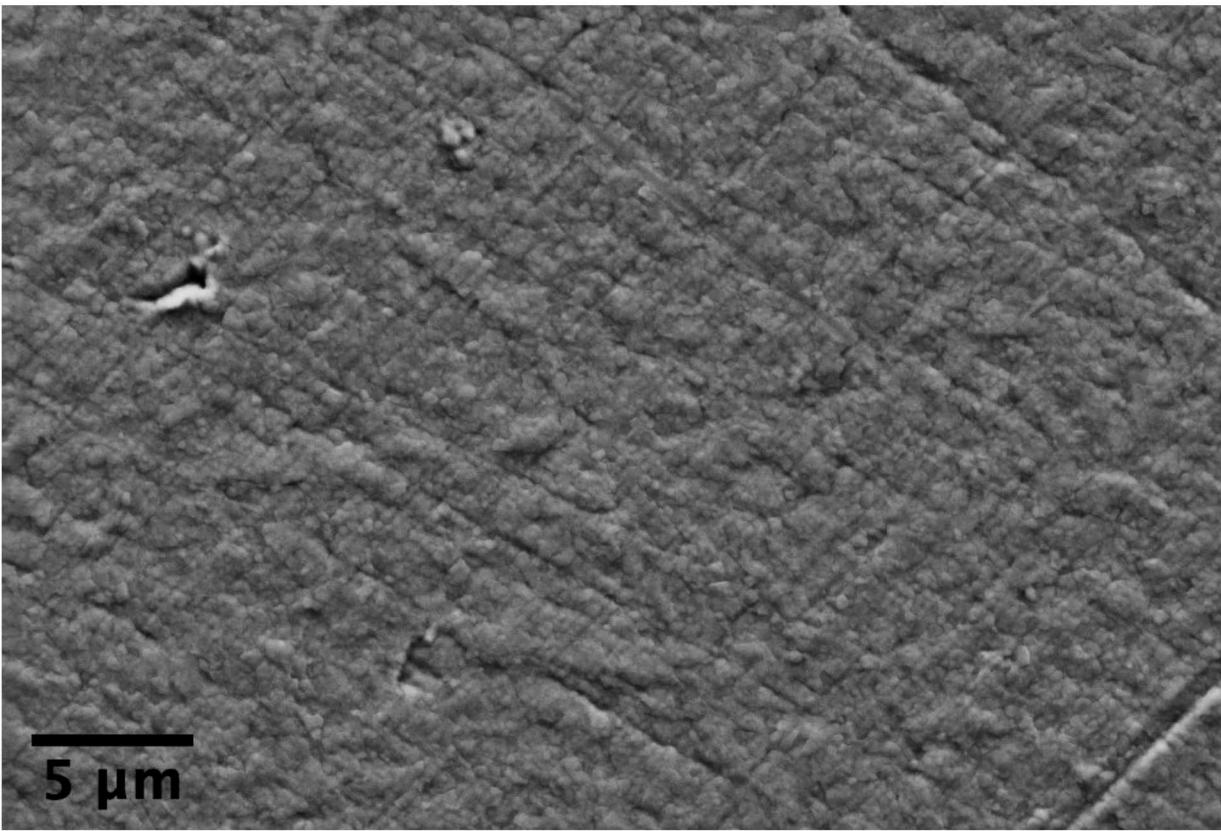
Secondary Electron Image



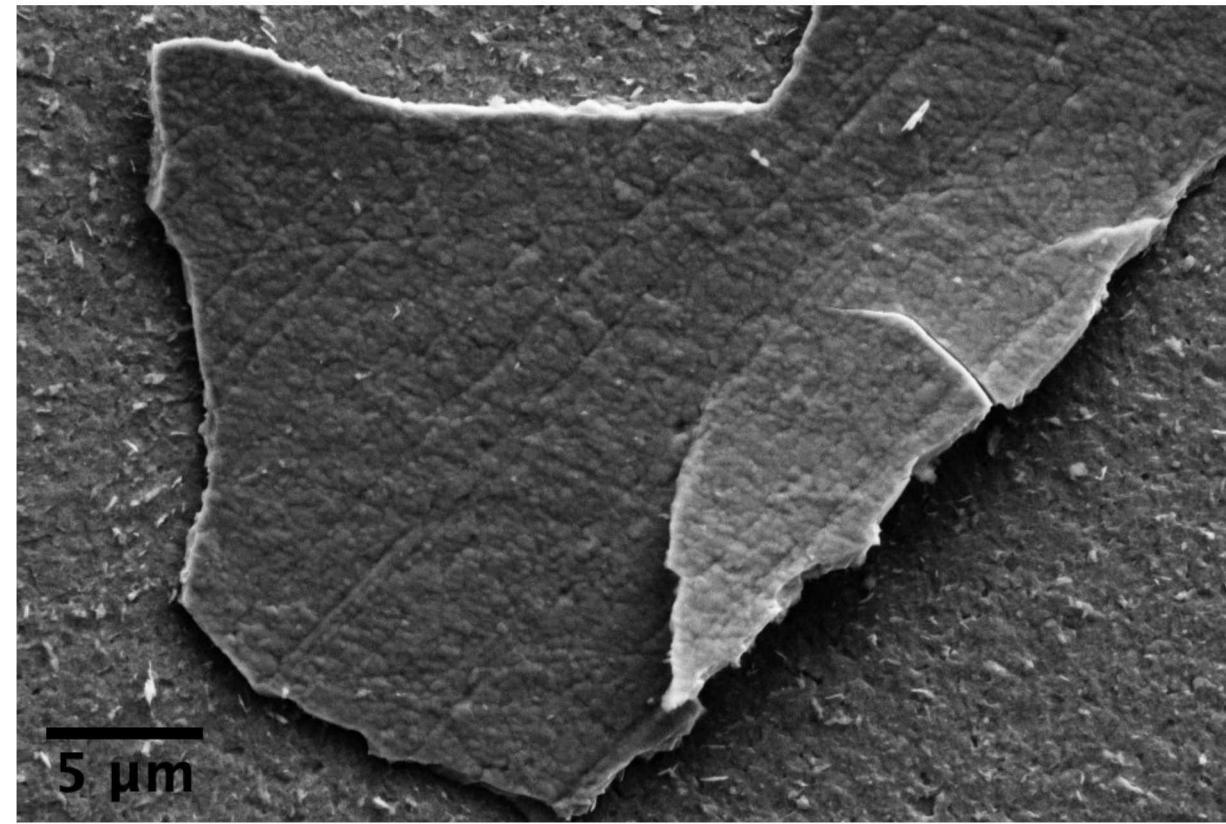
- SE image shows that most of the surface spalled off, leaving a few un-spalled areas. Lack of contrast in BSE image shows that spalling occurred within the 1 μm film thickness, not at the substrate.
- Fracture surface does not appear to show intergranular fracture, no obvious failure mechanism.

## No Evidence of Blister Formation in Un-Spalled Regions

Unirradiated Surface

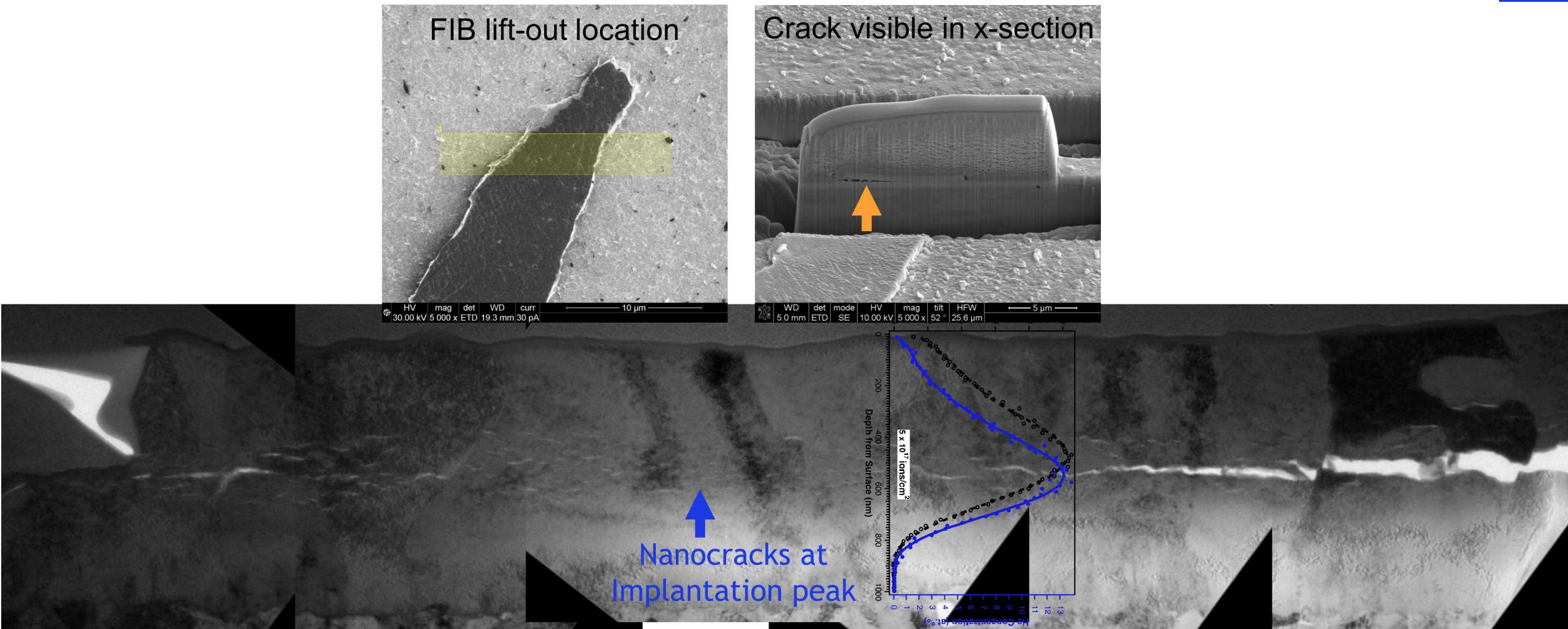


Spalled Surface ( $5 \times 10^{17}$  ions/cm<sup>2</sup>)



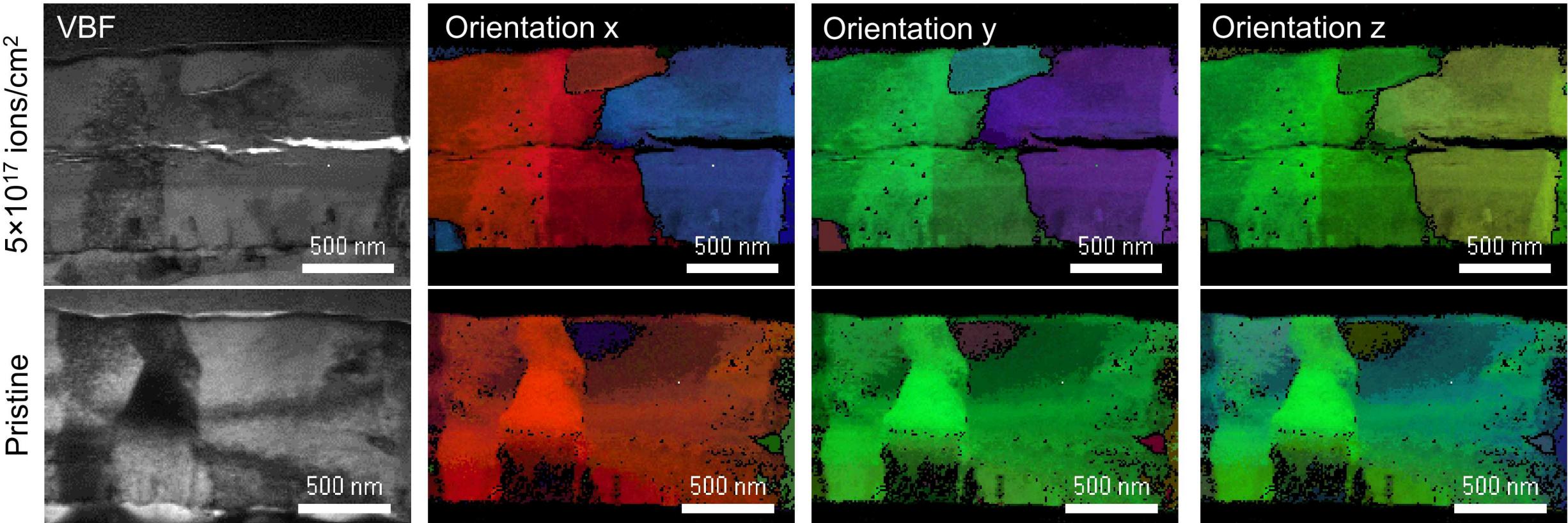
- Un-spalled surfaces appear nearly identical to the unirradiated sample surface, with the exception of cracks and flaked regions.
- Edges of un-spalled areas appear very jagged, not round as one would expect if blisters had burst to form them.

## Nanosized Cracks were Observed in the Un-Spalled Areas



- Nanocracks appear after ~7.5 at.% He. Stress between cracks causes linkage at the implantation peak, forming larger cracks that cause spalling.

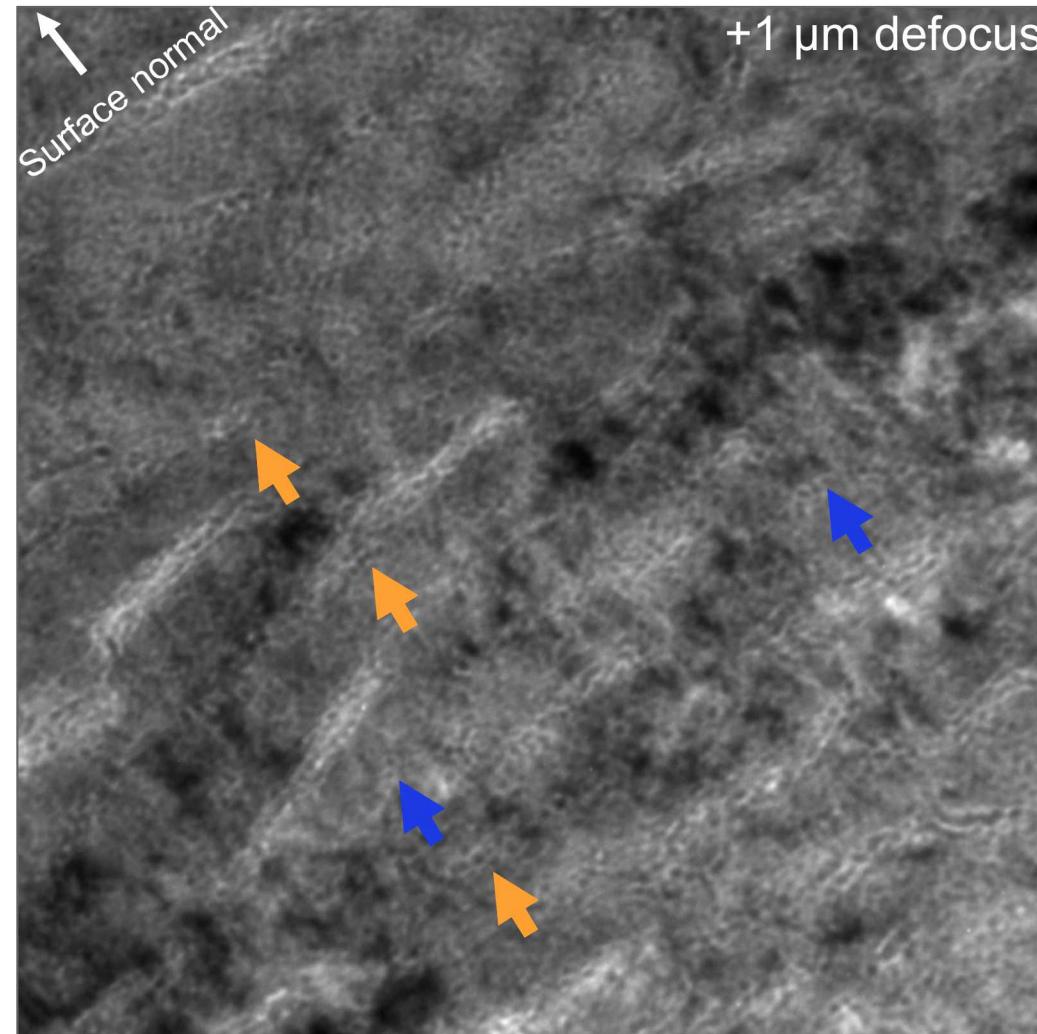
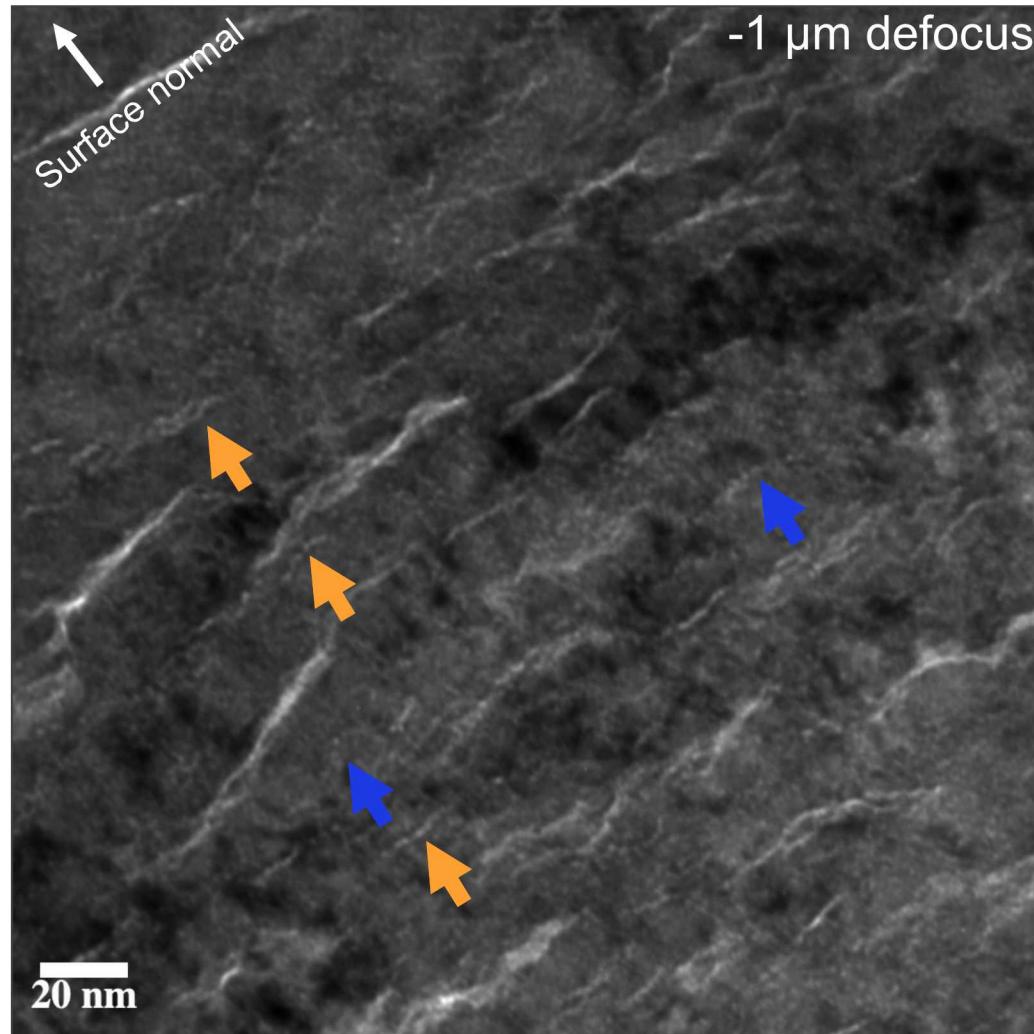
## Precession Electron Diffraction Confirmed Intragranular Cracking



- Crack clearly initiates inside the grain and propagates intragranularly.
- Small misorientations (<15°) were measured near the near the crack region in some samples but were believed to be due to bending in the FIB lift-out, as they are also present in the pristine (unirradiated) sample.



## Bubbles Appear to Form Chains that Lead to Nano-crack Formation



- Nano-cracks are spaced 4-15 nm apart and not always exactly parallel with the surface. In some cases, the bubble chains appear to be forming along **interfaces or defects**.

## ➤ **Background**

- tritium storage and helium in metal tritides

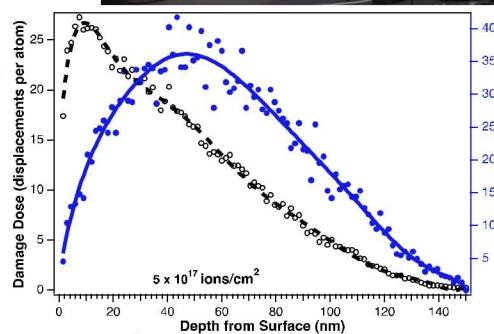
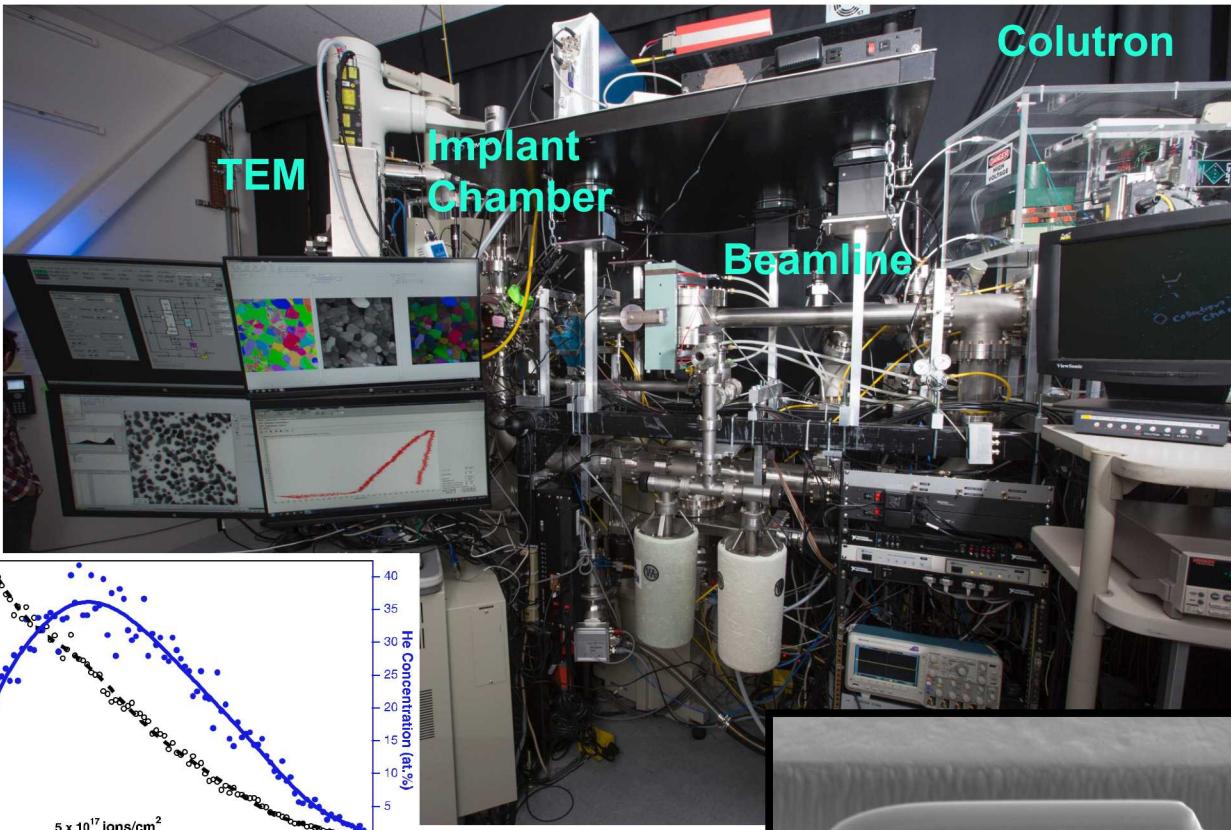
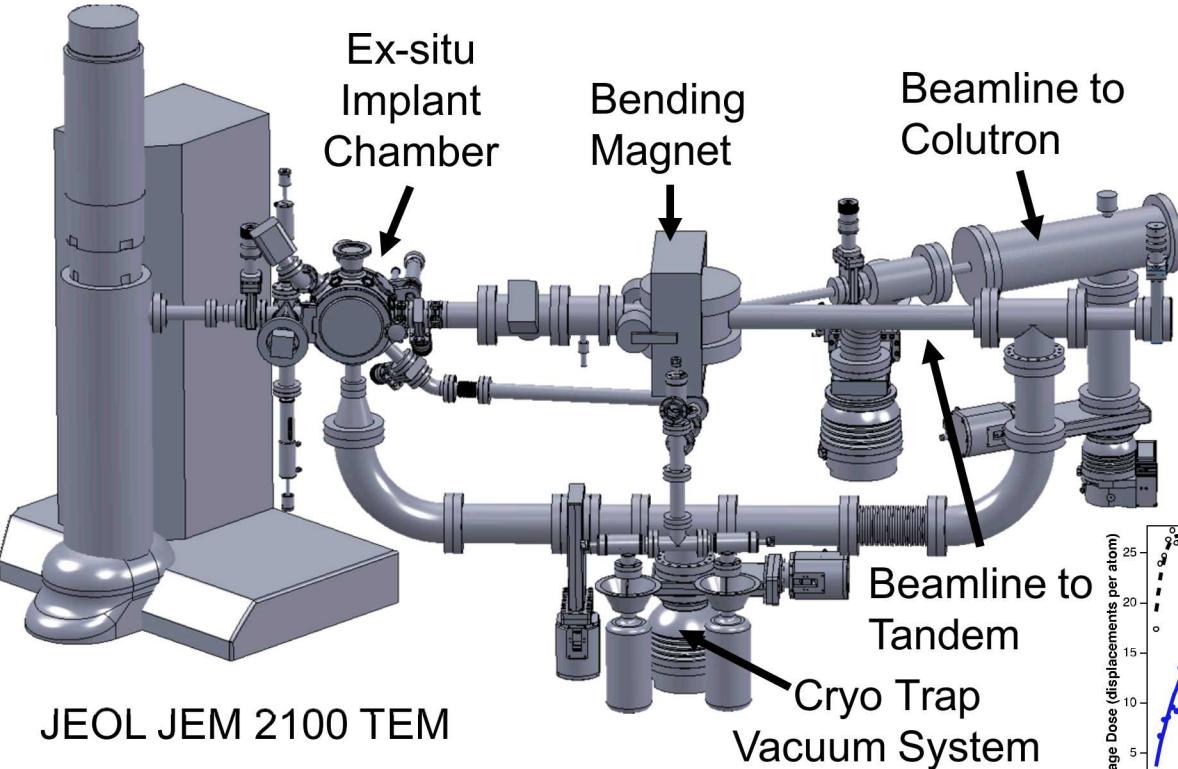
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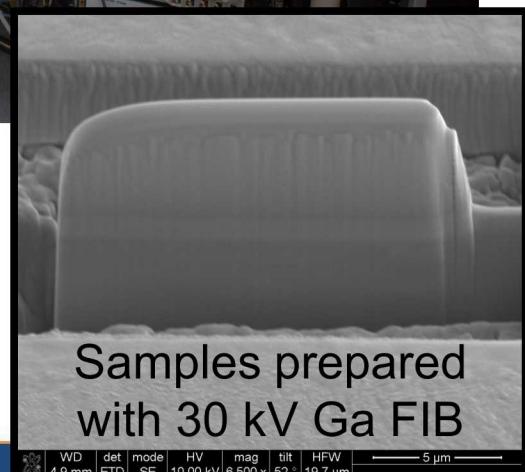
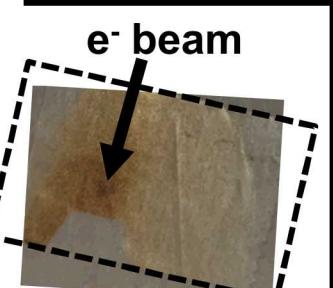
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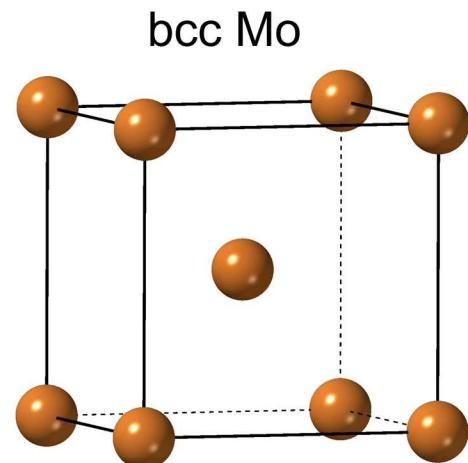
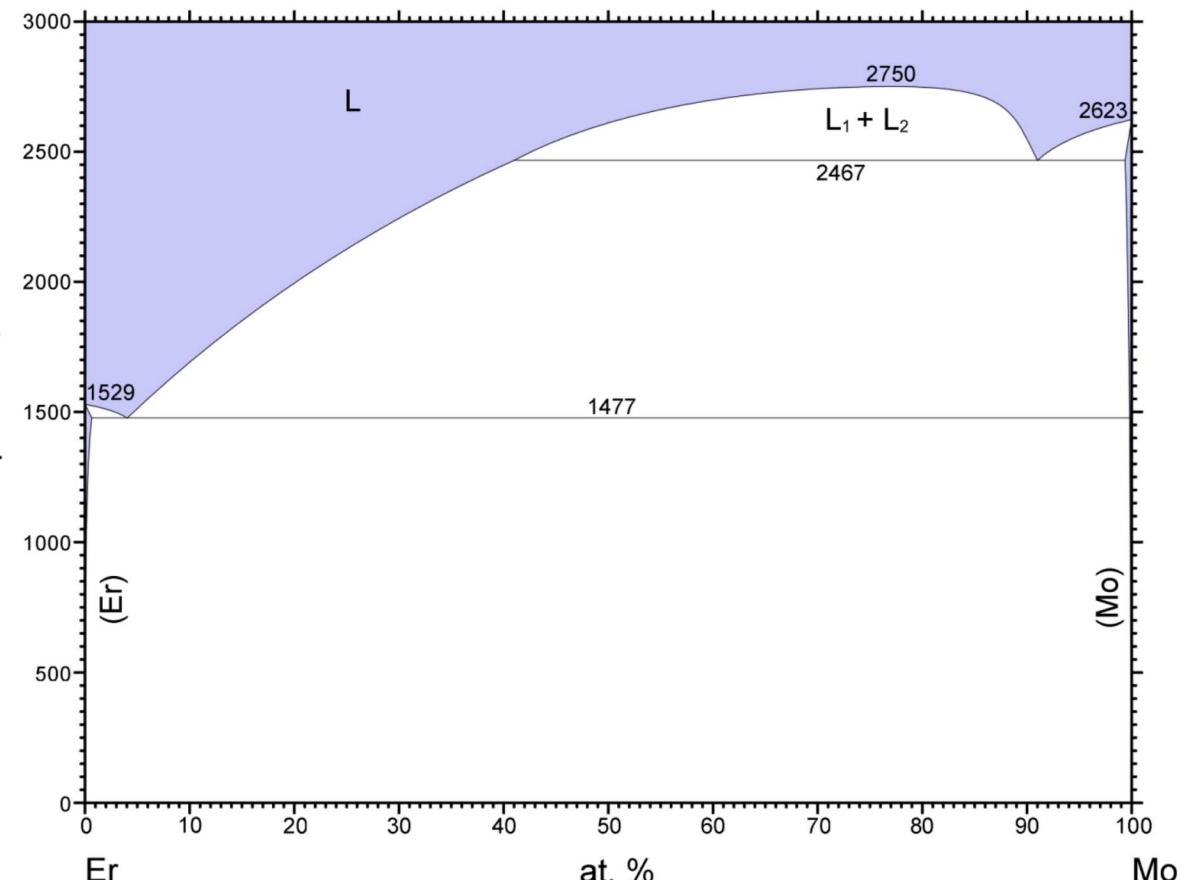
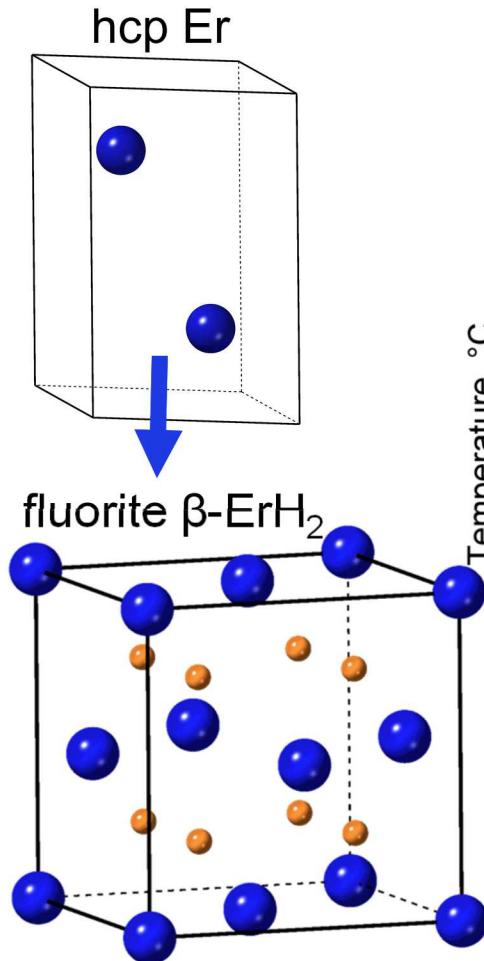
In-situ He Implantation Using Sandia's In-situ Ion Irradiation TEM (I<sup>3</sup>TEM)

10 keV He Beam  
Spot in Low Mag,  
+30° tilt in x



WD 4.9 mm | det ETD | mode SE | HV 10.00 kV | mag 6500 x | tilt 52° | HFW 19.7 μm | 5 μm

## All Multilayered Structures were Prepared with Er and Mo

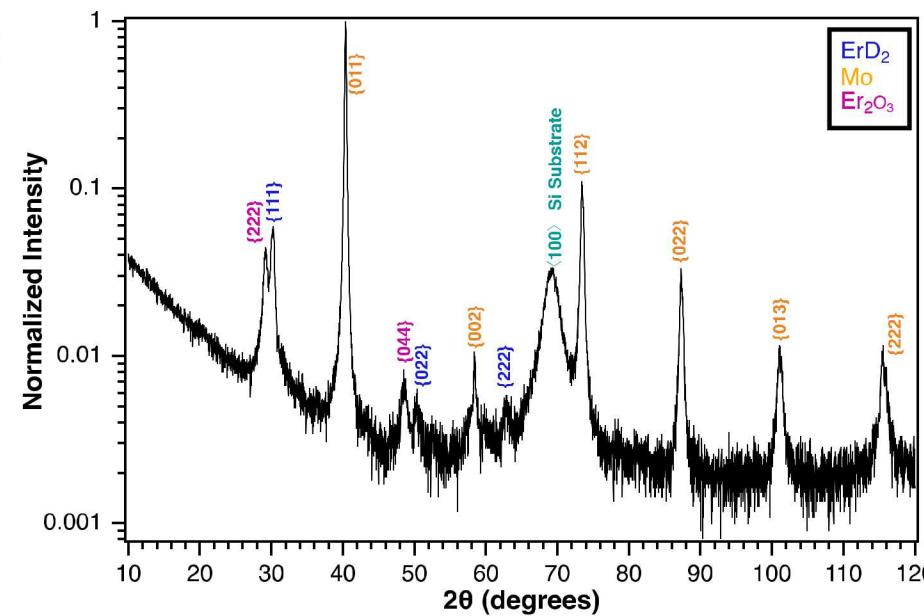
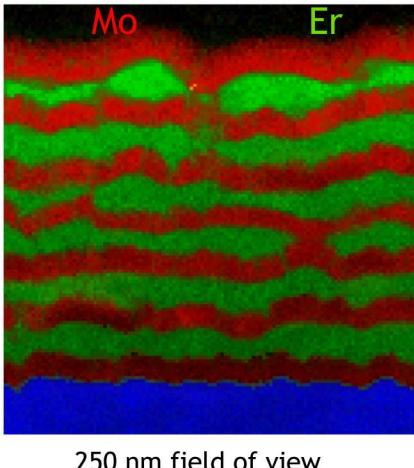
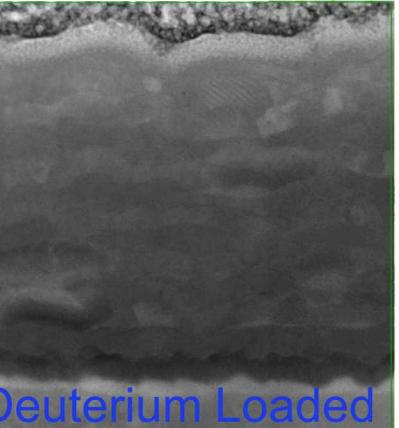
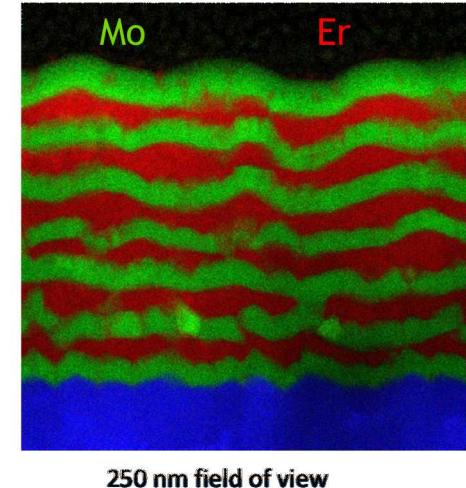
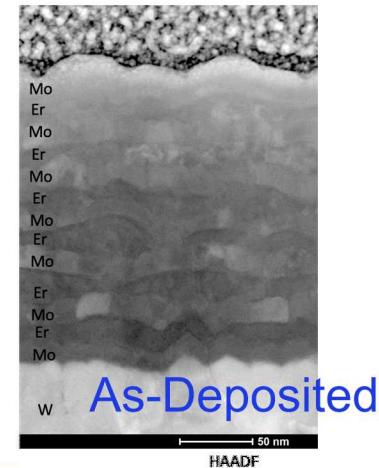


- Er and Mo are fully immiscible up to 1477°C and remain immiscible after hydriding.
- Hydriding induces a 10% volume expansion, so significant residual stress is expected in the multilayered structure.
- Big Question: Can we initiate the hydride reaction in a multilayered structure?

# Multilayer $\text{ErD}_2/\text{Mo}$ Films were Deposited and Successfully Hydrided

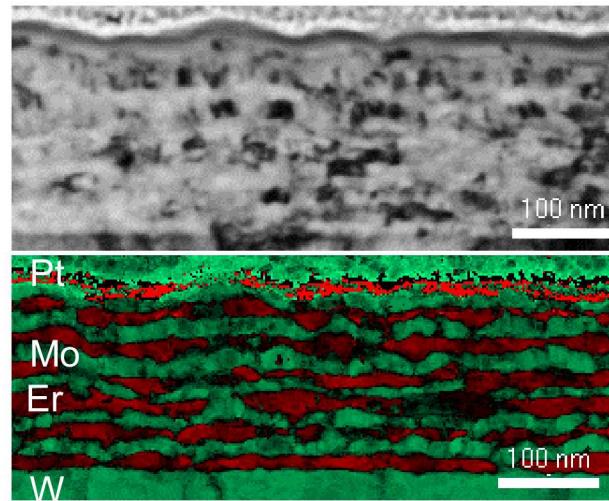


- W adhesion layer was sputter deposited at room temperature onto a Si substrate. 13 Er/Mo layers were electron beam evaporated.
- Samples were diced into 1 x 1 cm squares & successfully loaded with deuterium, as evidenced by the absence of hcp-Er & presence of fcc- $\text{ErD}_2$  in the XRD pattern.
- STEM/EDS maps of as-deposited/loaded structures showed no diffusion of adhesion layer, less oxide than previous films.



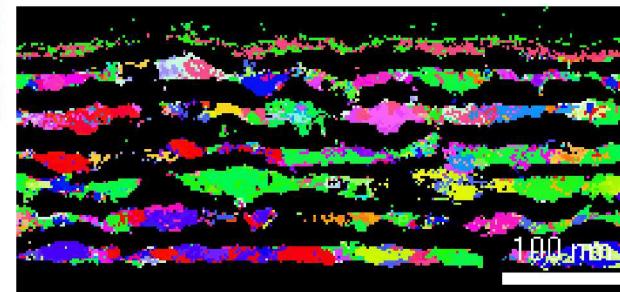
# Grain Structure Changes were Characterized After Hydriding

## Er/Mo (as-deposited) Multilayers



Max Er grain size: **31 nm**  
Max Mo grain size: **33 nm**

**Er** z orientation



**Mo** z-orientation

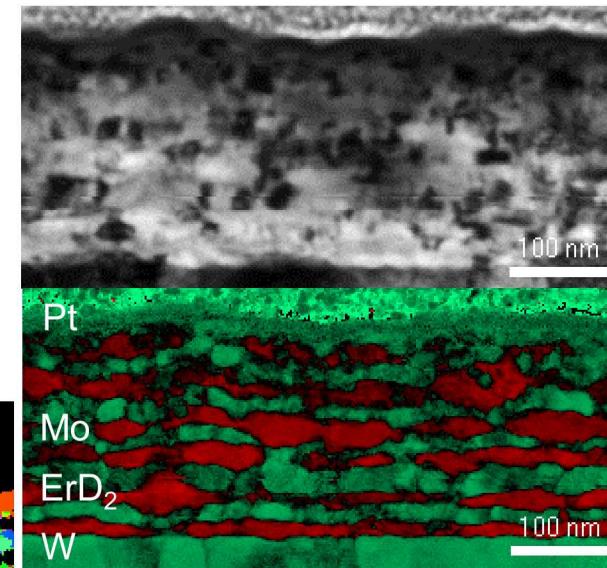


### PED parameters

2 nm step size  
Smallest condenser  
Spot 5,  $\alpha = 3$   
0.1° precession angle

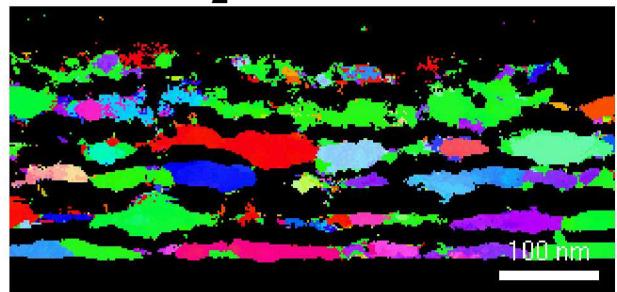
\*images were cropped  
for grain size analysis

## ErD<sub>2</sub>/Mo (as-loaded) Multilayers

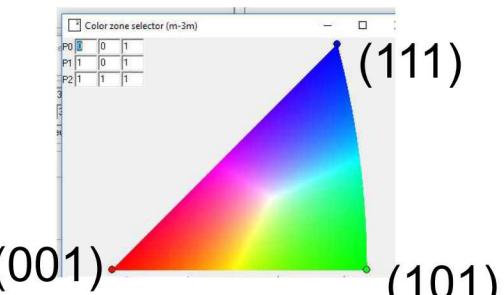


Max ErD<sub>2</sub> grain size: **62 nm**  
Max Mo grain size: **37 nm**

**ErD<sub>2</sub>** z orientation



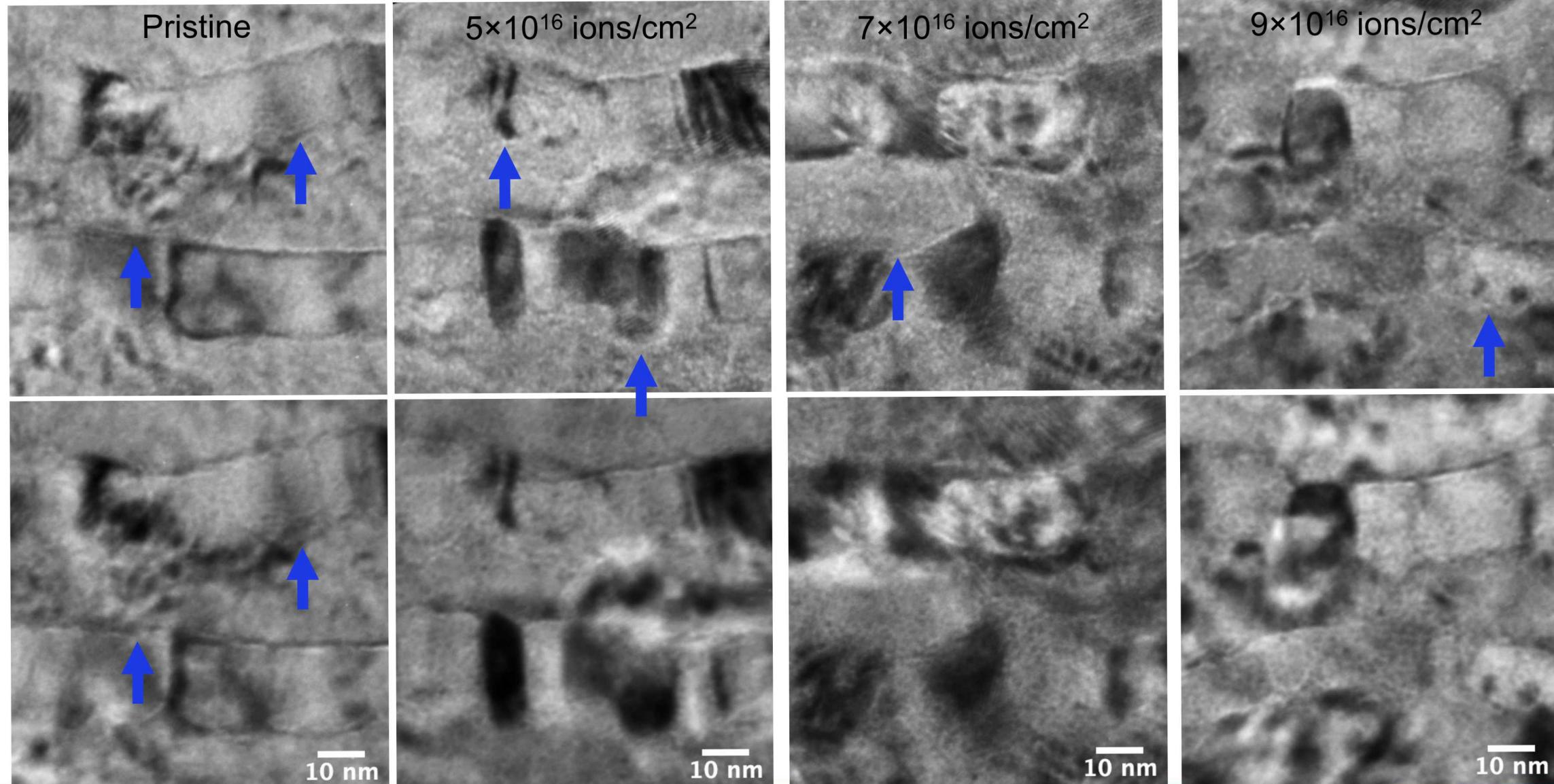
**Mo** z-orientation



Hydride Reaction Causes 10% Volume Swelling

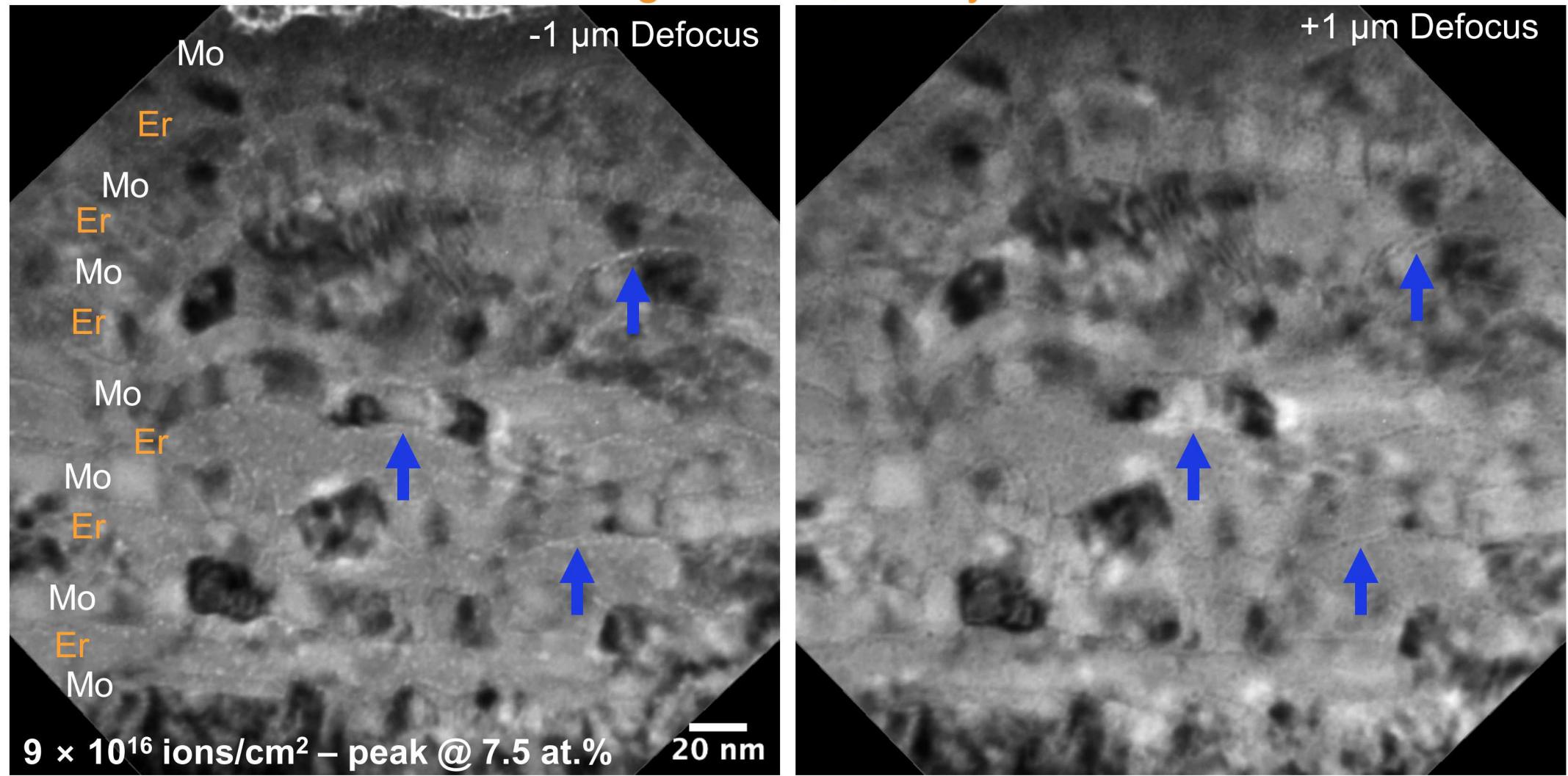
## He Bubbles were Identified at Interfaces and in Matrix

underfocus



overfocus

## Bubble Distribution is Inhomogeneous, Likely due to Oxide



- Layers are not uniform due to hydriding induced swelling, so PED was used to identify them.
- Helium bubbles likely inside the ErD<sub>2</sub> layers because of interstitial oxygen or Er<sub>2</sub>O<sub>3</sub> precipitates

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## Summary

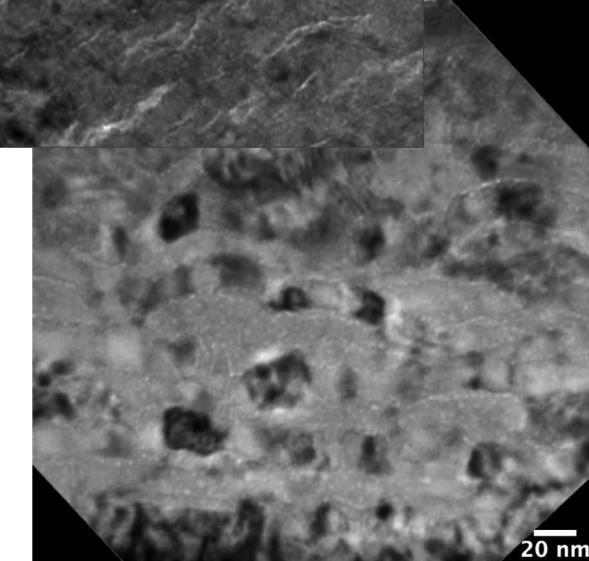
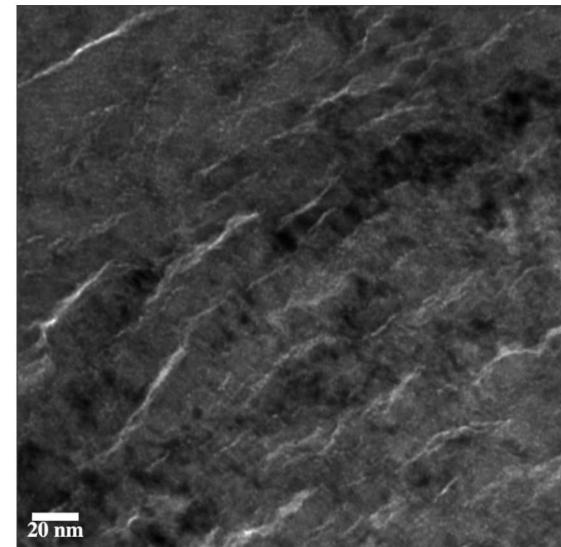
**Objectives:** 1) to understand the fundamental mechanisms behind accelerated He release in tritium storage materials, 2) and to understand how He interacts with interfaces in multilayered structures

### Helium implantation into $\text{ErD}_2$

- Intragranular nano-cracks were identified parallel to the surface in un-spalled areas of the implanted film.
- As stresses between the cracks increase, the cracks eventually link, forming larger cracks that cause spallation.
- A similar release mechanism may be occurring in  $\text{ErT}_2$ , where stresses between He platelets may cause linkage.

### Multilayered $\text{ErD}_2/\text{Mo}$ nanocomposites

- Helium was found to trap at multilayered interfaces.
- Helium is being trapped inside the  $\text{ErD}_2$  layers, likely at oxide interfaces.



## Acknowledgements

- STEM/EDS: Paul Kotula (SNL), Bethany Matthews (PNNL), Bruce Arey (PNNL)
- Film Deposition: Ron Goeke (SNL), Clark Snow (SNL)
- I<sup>3</sup>TEM Facility: Khalid Hattar (SNL)
- XRD: Dale Zschiesche (SNL)
- Bulk He Implantation: Yong Wang (LANL)
- EELS (in progress): Joshua Sugar (SNL-CA)

The Following CINT Facilities were Utilized for this Work:

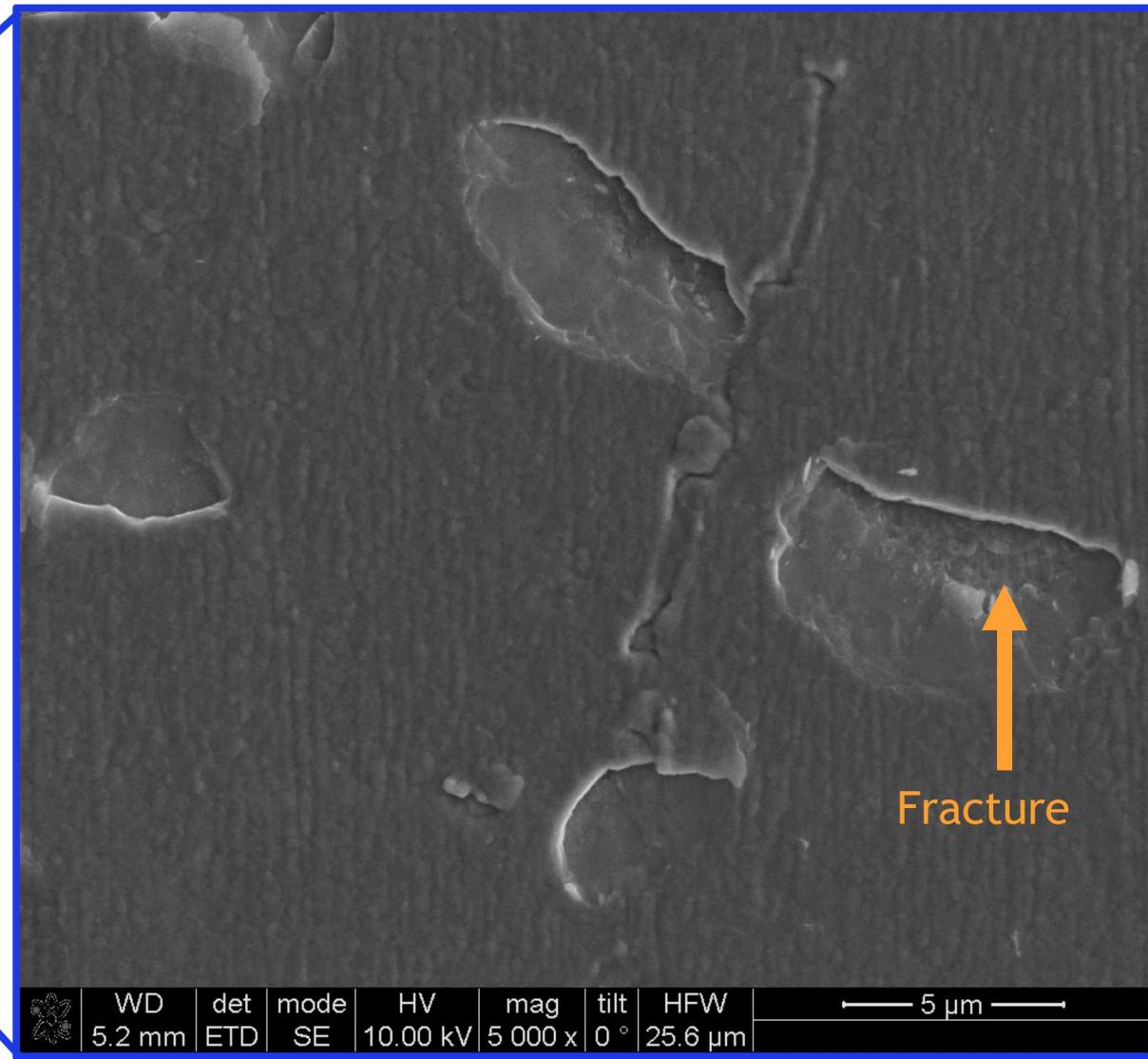
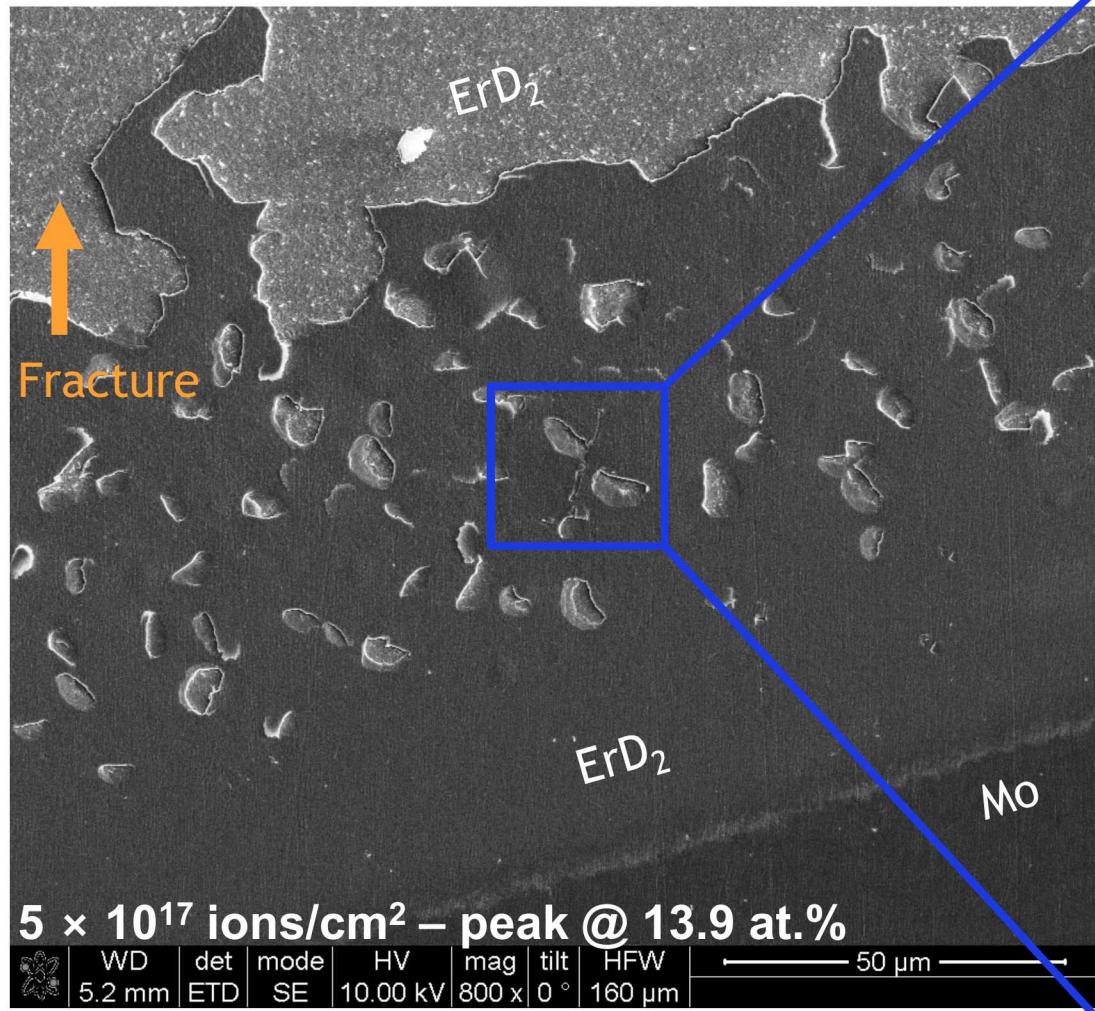
- In Situ Ion Irradiation TEM (SNL-NM)
- FEI Nova Focused Ion Beam (SNL-NM)
- LANL Ion Beam Laboratory (LANL)





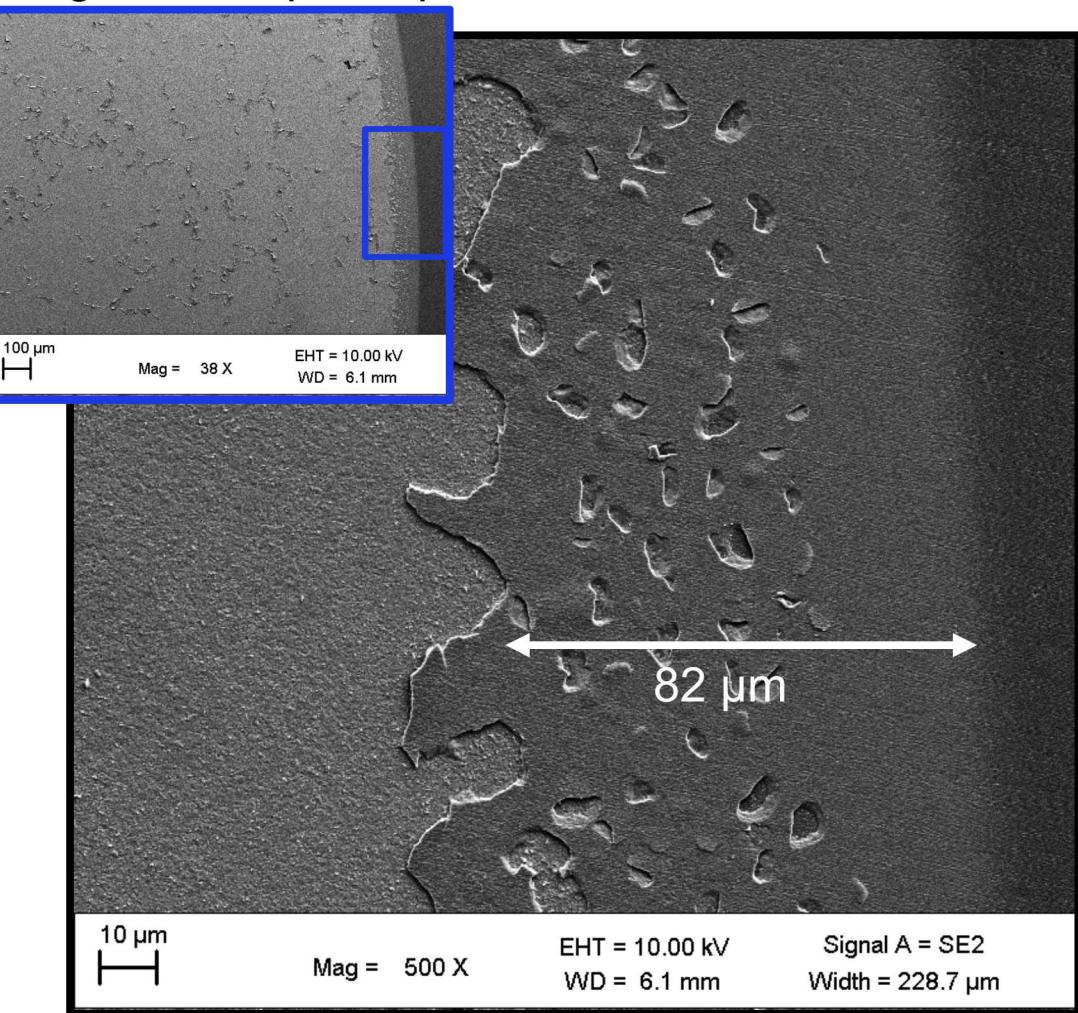
# Extra Slides

## Blister Microstructure Present at Film Perimeter

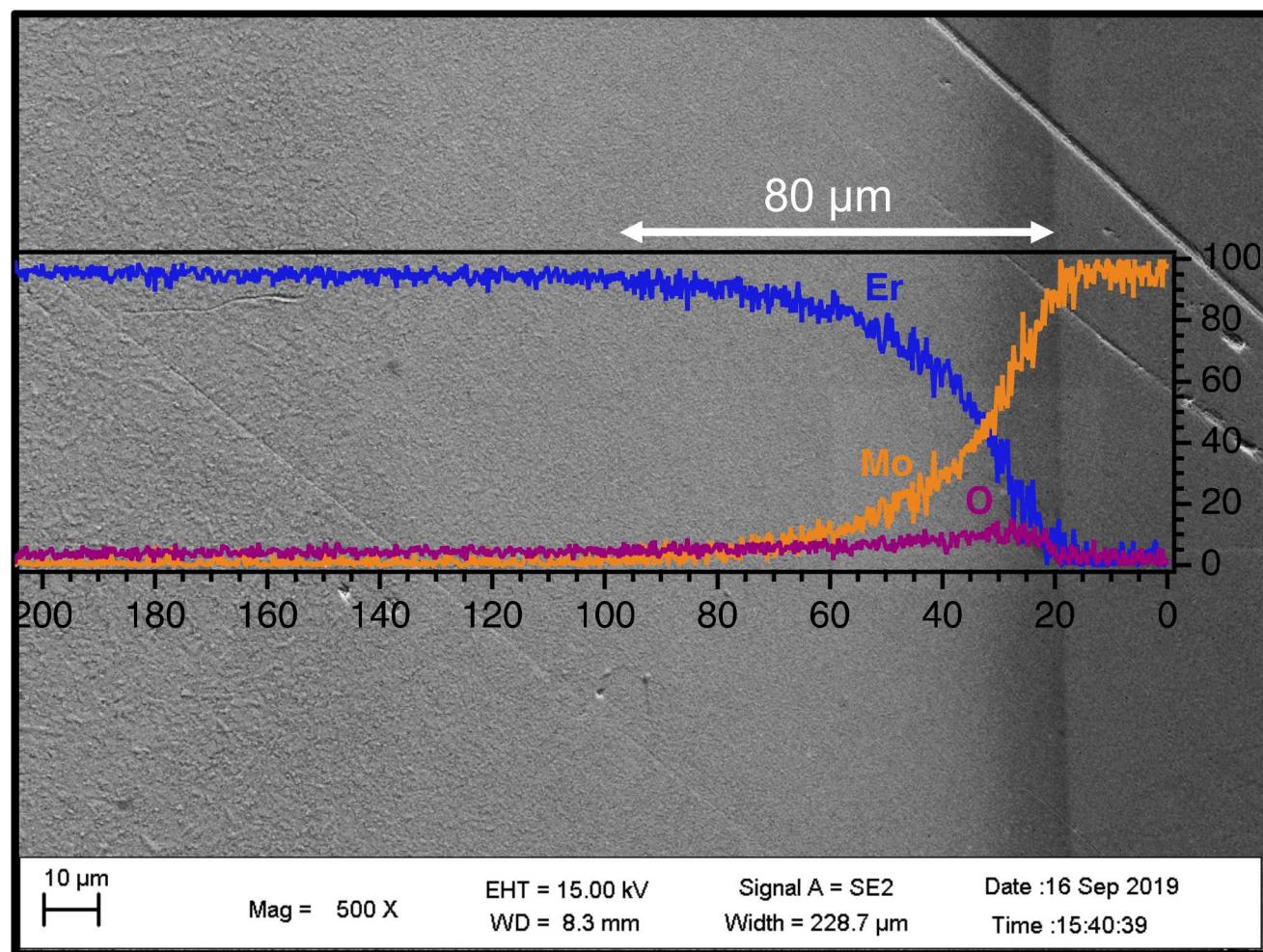


# Minor Chemistry Changes Result in Different He Effects

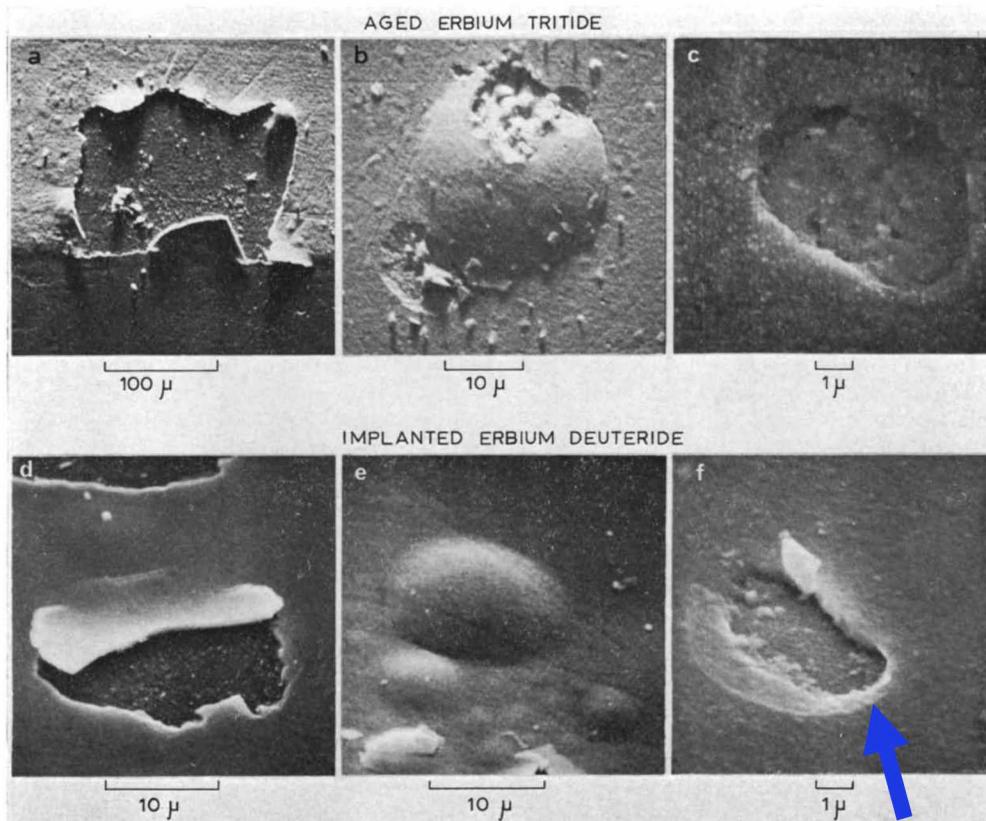
Edge of Sample Implanted with  $5 \times 10^{17}$  ions/cm<sup>2</sup>



Edge of Pristine Sample has Thinner Er Layer and More Oxide



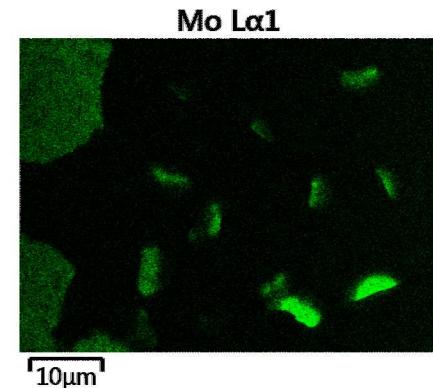
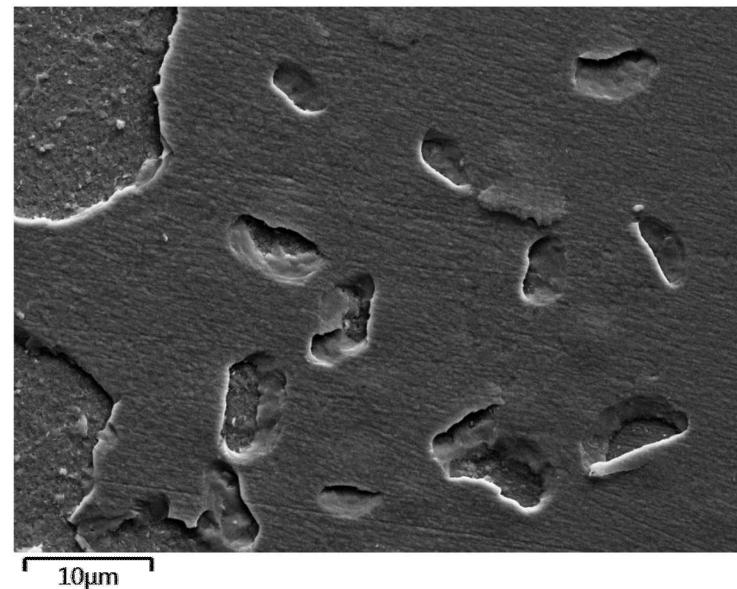
## Literature Reports Blistered Microstructure after He Implantation



Blewer and Maurin JNM 44 (1972) 260-278

Extends to substrate

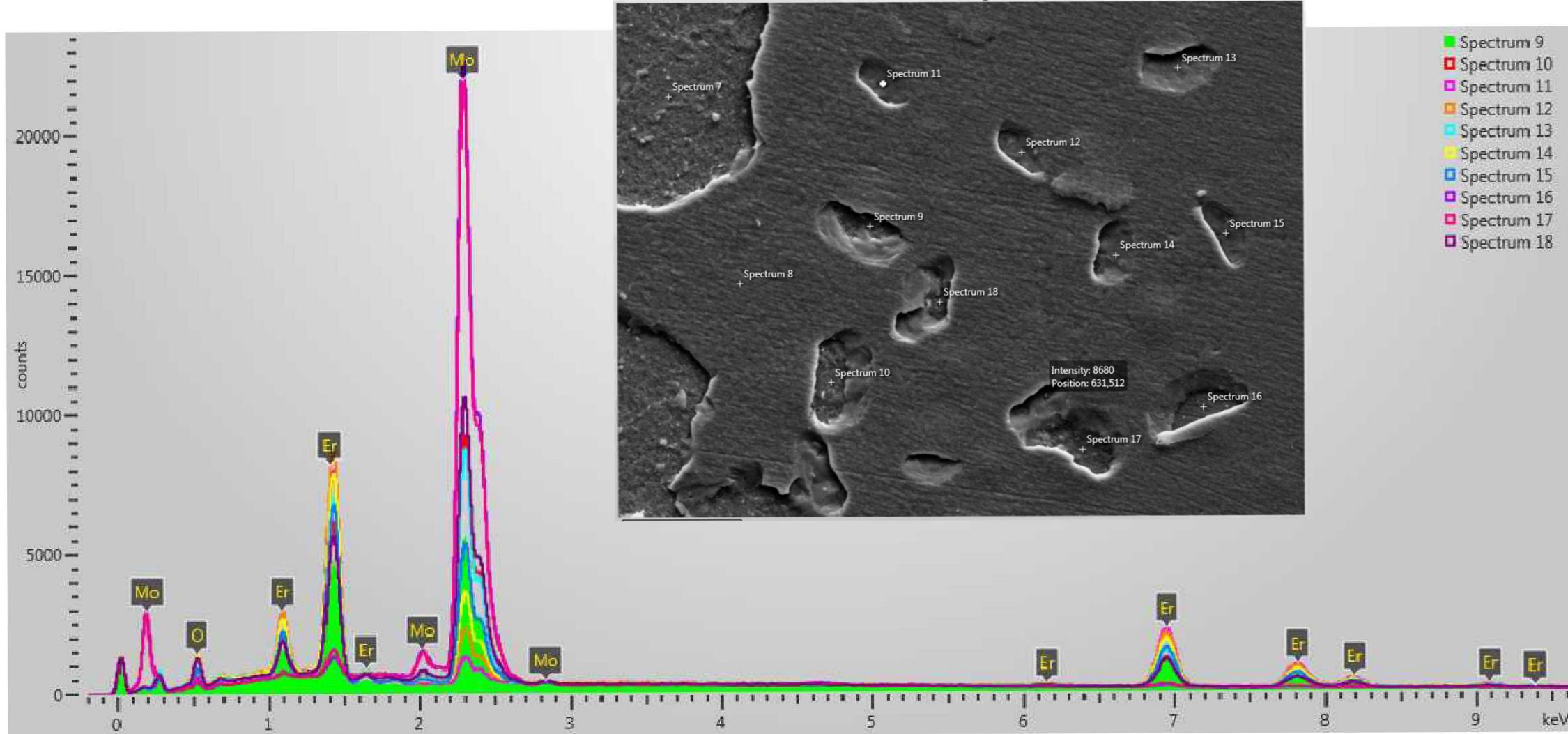
Similar Features in this Work



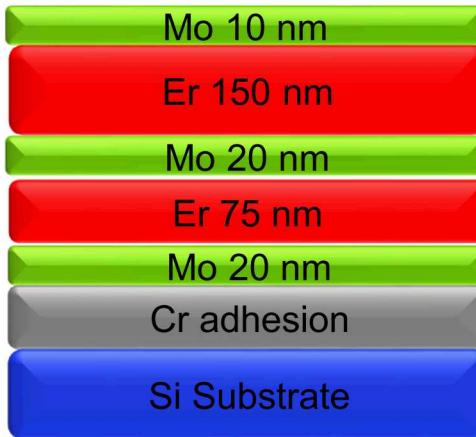
Some Extend to Substrate

- Previous results using 120 keV <sup>4</sup>He produced blisters and craters in the material, similar to the microstructures present on the perimeter of the samples implanted in this work.
- This microstructure was similar to the aged ErT<sub>2</sub> images available in literature.

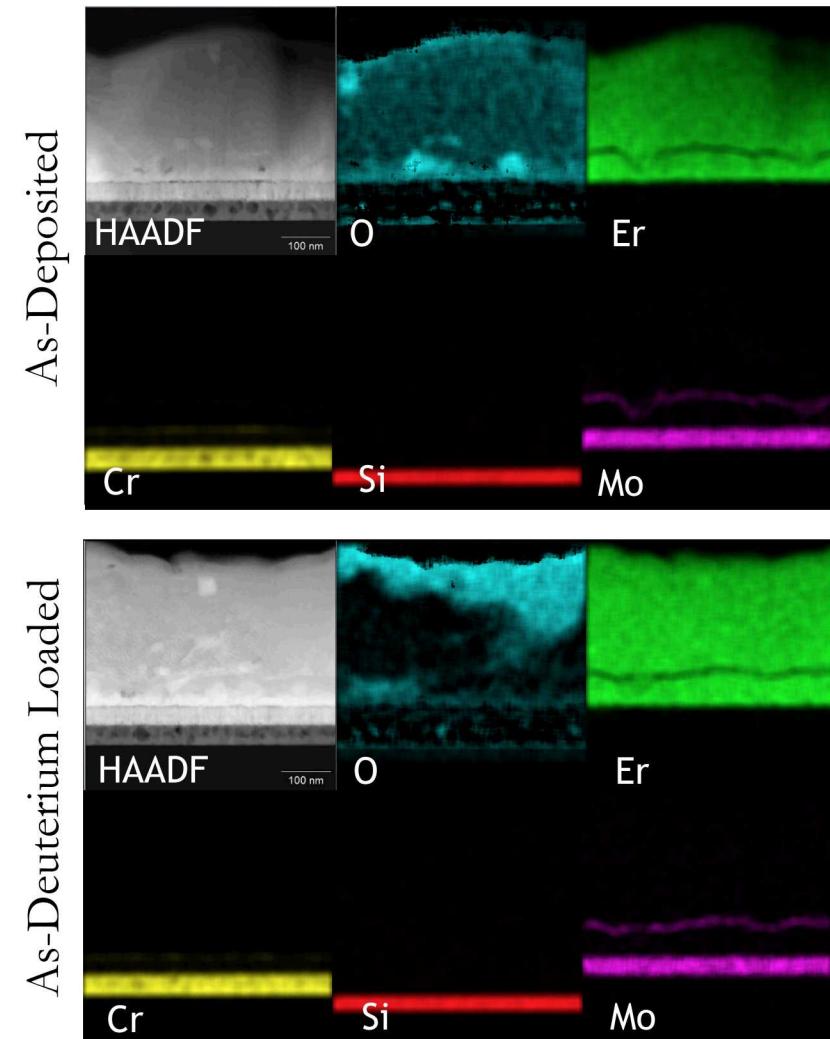
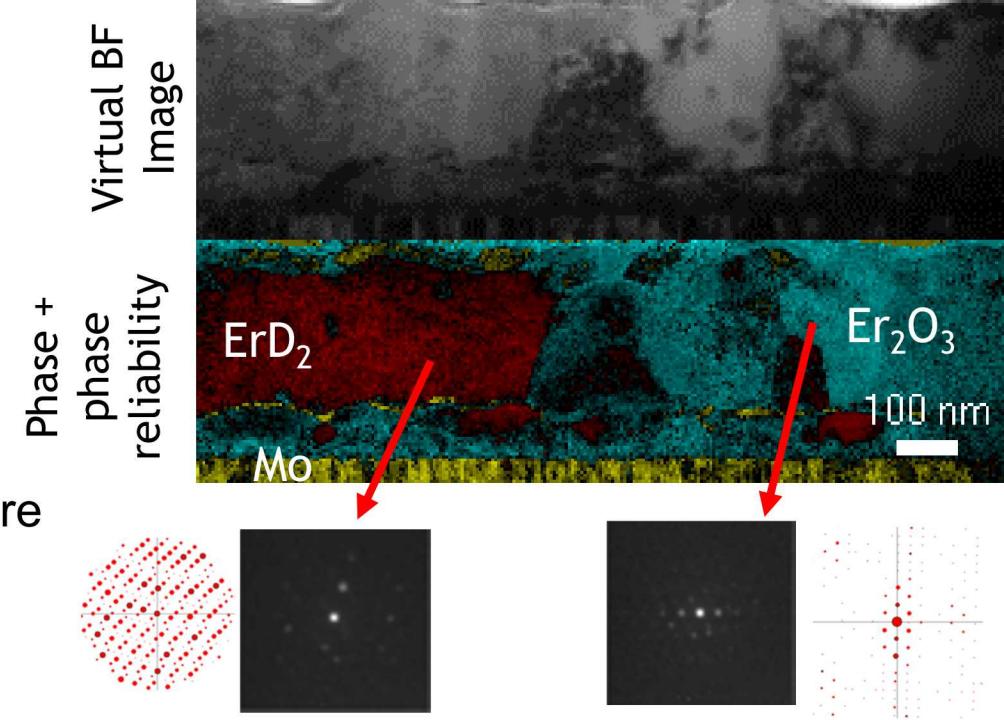
# EDS Spectra from All Holes in $\text{ErD}_2$ Implanted with $5 \times 10^{17} \text{ He}/\text{cm}^2$



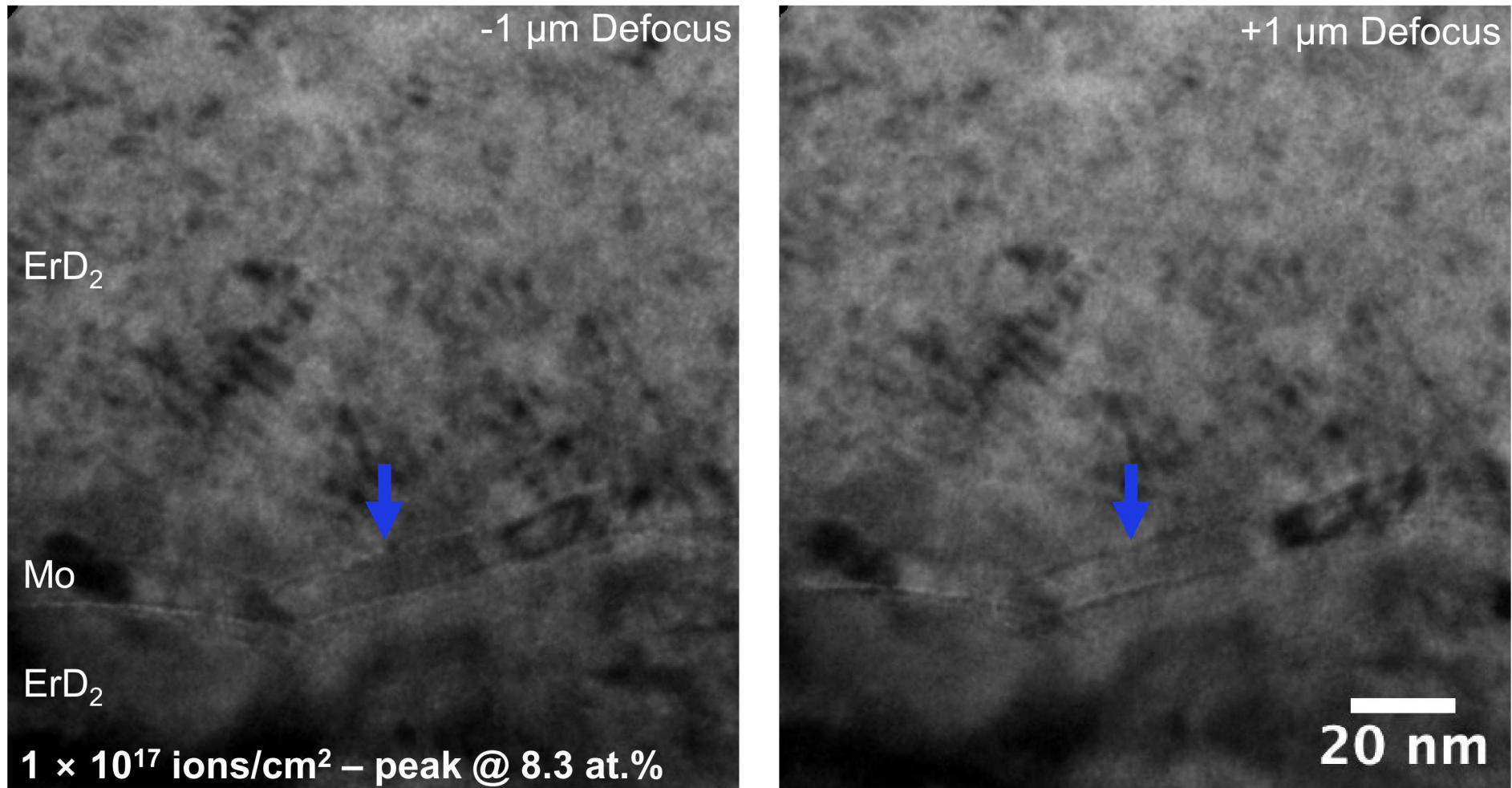
# As a First Attempt, Samples were Prepared with a Single Mo Layer



- Cr/Mo layers were sputter deposited at room temperature onto a Si substrate. 4 Er/Mo layers were electron beam evaporated.
- Samples were diced into 1 x 1 cm squares & loaded with deuterium.
- EDS maps of as-deposited structure showed Cr diffusion through the sputtered Mo layer and some  $\text{Er}_2\text{O}_3$  grains.



## He Clearly Traps at Mo Interfaces in $\text{ErD}_2/\text{Mo}$ Structure



- Bubbles formed along the Mo interface after “in-situ” implantation to  $8 \times 10^{16} \text{ ions/cm}^2$  (6.7 at.%)
- After implantation to  $1 \times 10^{17} \text{ ions/cm}^2$  (8.3 at.%) bubbles were clearly visible inside grains and along boundaries.

## Oxide Interfaces also Effectively Trapped He

