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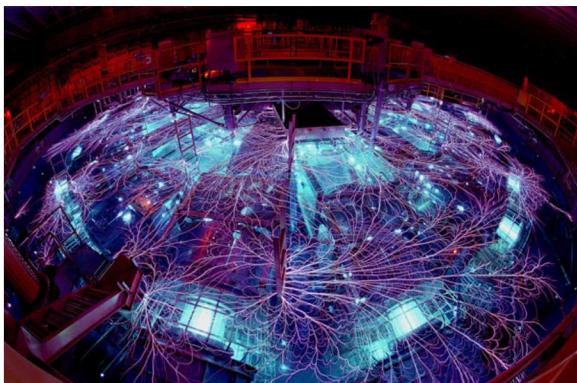


# Modeling kinetic processes in magnetized ICF plasmas

Paul F. Schmit

*Fundamental Science with Pulsed Power  
Breakout session*

July 23<sup>rd</sup>, 2014  
SAND-###-####



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# Thanks to many collaborators

S. A. Slutz,<sup>1</sup> C. A. Jennings,<sup>1</sup> R. W. Lemke,<sup>1</sup> M. R. Martin,<sup>1</sup> D. C. Rovang,<sup>1</sup>  
J. B. Greenly,<sup>2</sup> D. B. Sinars,<sup>1</sup> M. E. Cuneo,<sup>1</sup> M. C. Herrmann,<sup>1</sup>  
D. D. Ryutov,<sup>3</sup> B. E. Blue,<sup>4</sup> K. Tomlinson,<sup>4</sup> K. Killebrew,<sup>1,4</sup> D. Schroen,<sup>4</sup>  
R. M. Stamm,<sup>4</sup> G. E. Smith,<sup>5</sup> J. K. Moore,<sup>5</sup> J.-P. Davis,<sup>1</sup> D. H. Dolan,<sup>1</sup>  
D. G. Flicker,<sup>1</sup> M. R. Gomez,<sup>1</sup> R. A. Vesey,<sup>1</sup> K. J. Peterson,<sup>1</sup> A. B. Sefkow,<sup>1</sup>  
C. W. Nakhleh,<sup>6</sup> P. F. Knapp,<sup>1</sup> T. J. Awe,<sup>1</sup> E. C. Harding,<sup>1</sup> T. J. Rogers,<sup>1</sup>  
A. Laspe,<sup>1</sup> M. R. Lopez,<sup>1</sup> I. C. Smith,<sup>1</sup> B. W. Atherton,<sup>1</sup> M. Savage,<sup>1</sup>  
W. A. Stygar,<sup>1</sup> J. L. Porter,<sup>1</sup> M. K. Matzen,<sup>1</sup> ***and many more...***

<sup>1</sup>*Sandia National Laboratories, Albuquerque, NM*

<sup>2</sup>*Laboratory of Plasma Studies, Cornell University, Ithaca, NY*

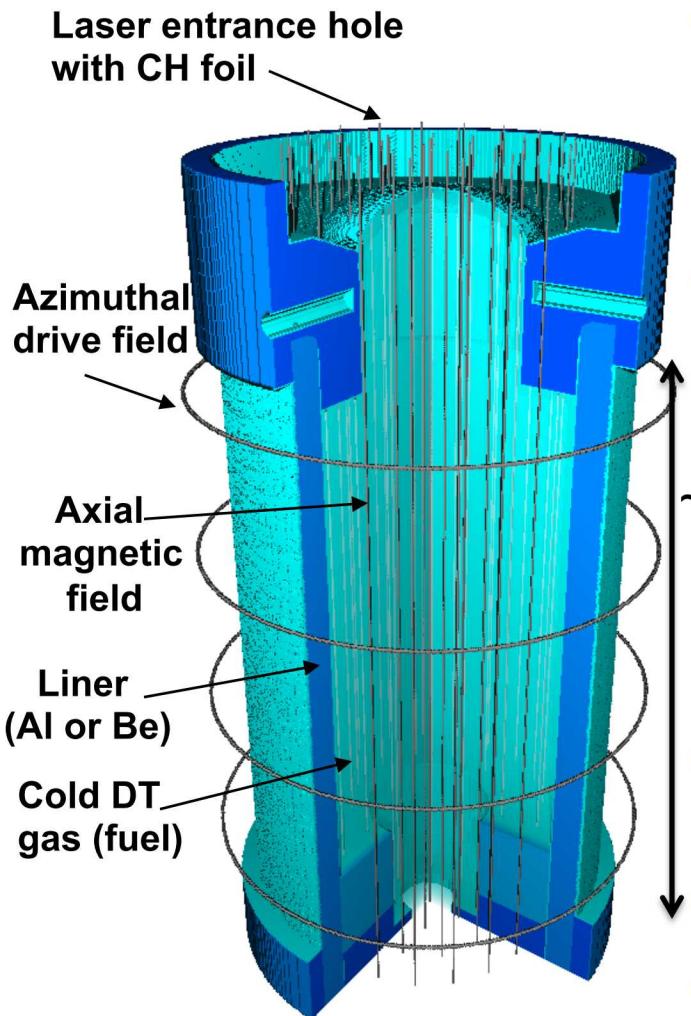
<sup>3</sup>*Lawrence Livermore National Laboratory, Livermore, CA*

<sup>4</sup>*General Atomics, San Diego, CA*

<sup>5</sup>*Raytheon Ktech, Albuquerque, NM*

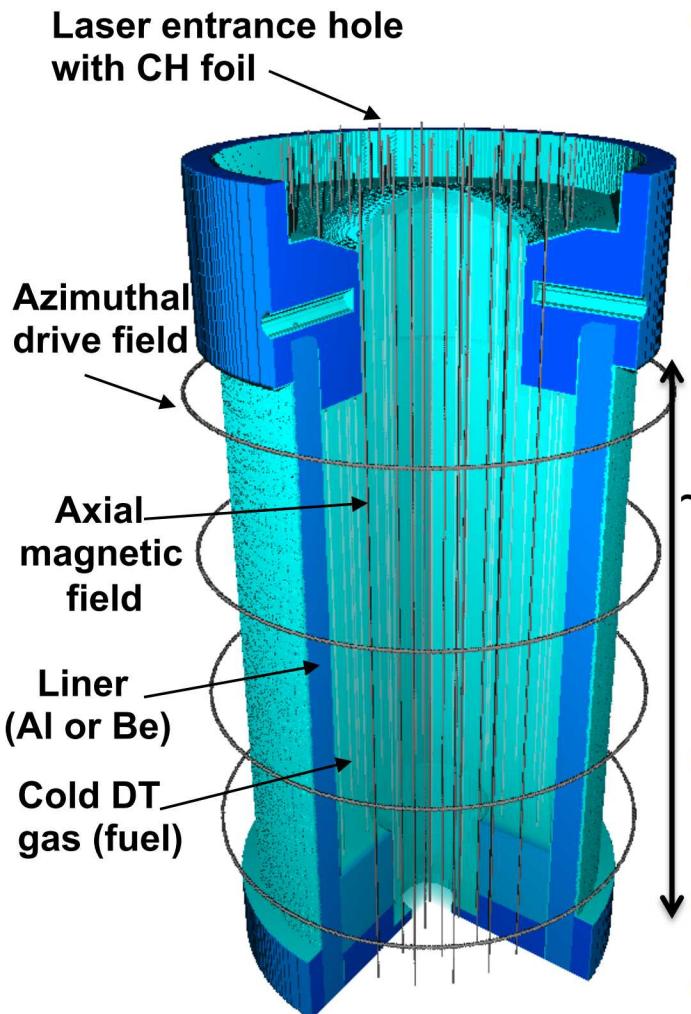
<sup>6</sup>*Los Alamos National Laboratory, Los Alamos, NM*

# We are evaluating a **Magnetized Liner Inertial Fusion (MagLIF)\*** concept that may reduce fusion requirements



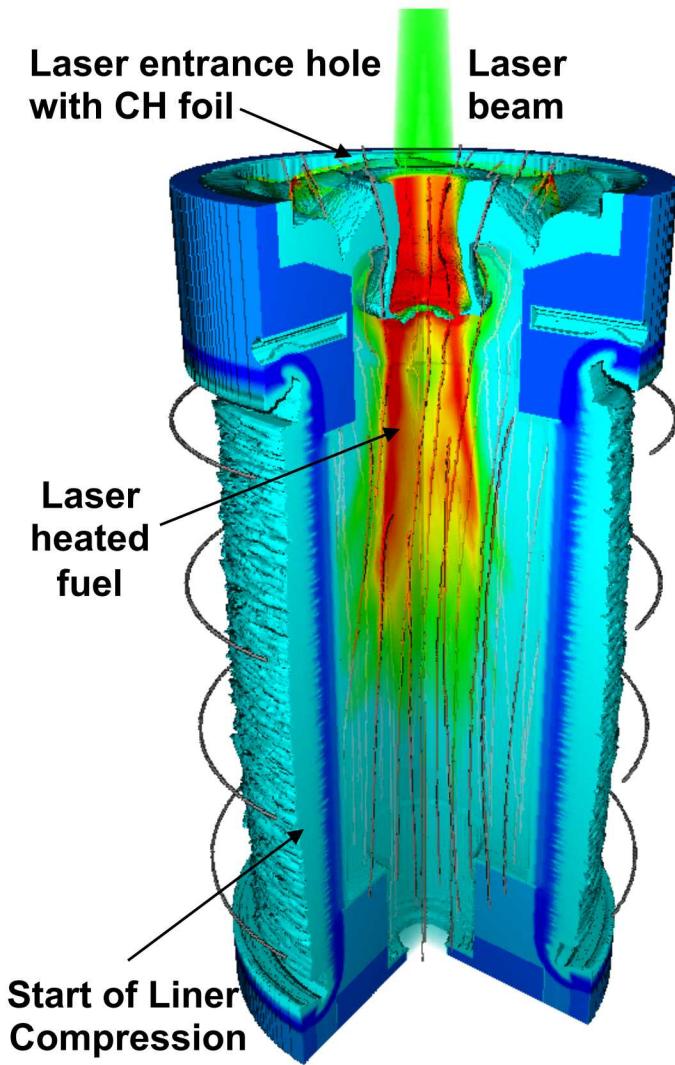
- An initial 30 T axial magnetic field is applied
  - Inhibits thermal conduction losses
  - May help stabilize implosion at late times
- During the ~100 ns implosion, the fuel is heated using the Z-Beamlet laser (about 6 kJ in designs)
  - Preheating to ~300 eV reduces the compression needed to obtain fusion temperatures to 23 on Z
  - Preheating reduces the implosion velocity needed to ~100 km/s, allowing us to use thick liners that are more robust against instabilities
- ~50-250 kJ energy in fuel; 0.2-1.4% of capacitor bank (Pulsed power is very energy efficient!)
- Stagnation pressure required is ~5 Gbar
- 100 kJ yield may be possible on Z using DT  
Early experiments would use DD fuel

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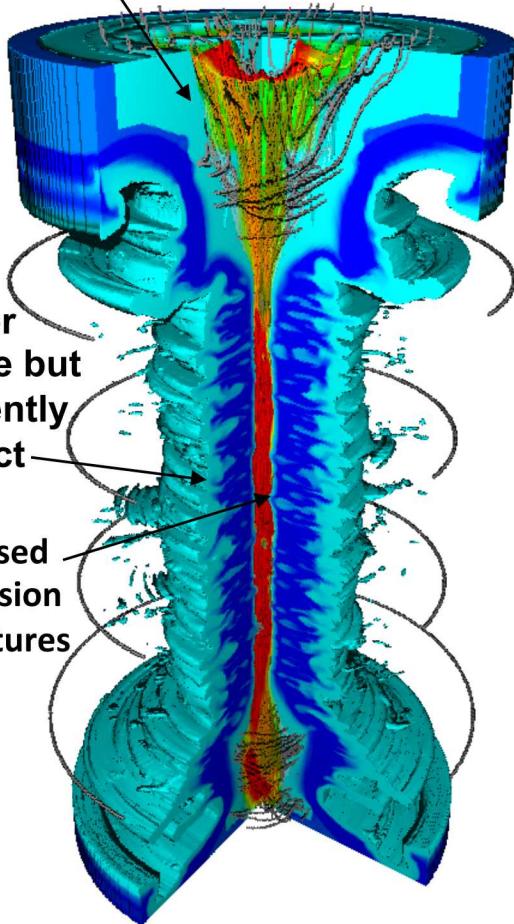
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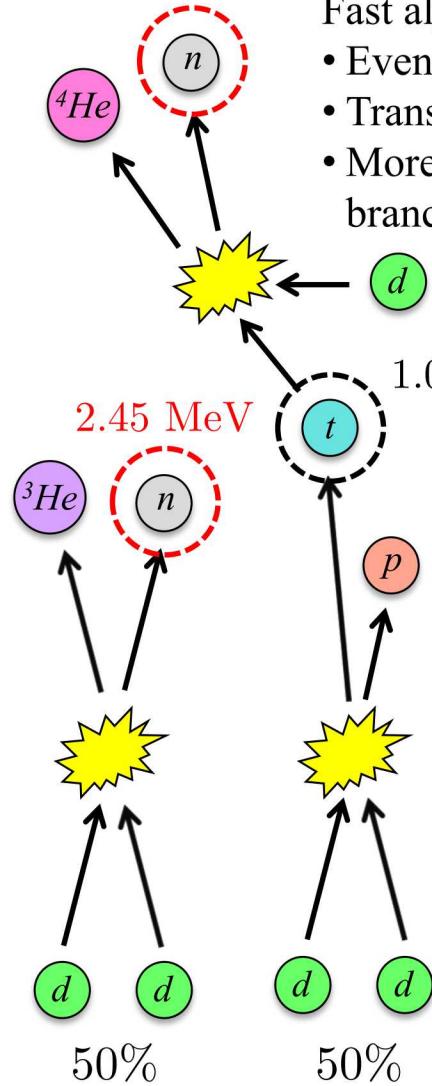
Axial field compressed by implosion



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# Kinetic processes in magnetized ICF plasma

14.1 MeV



Fast alphas:

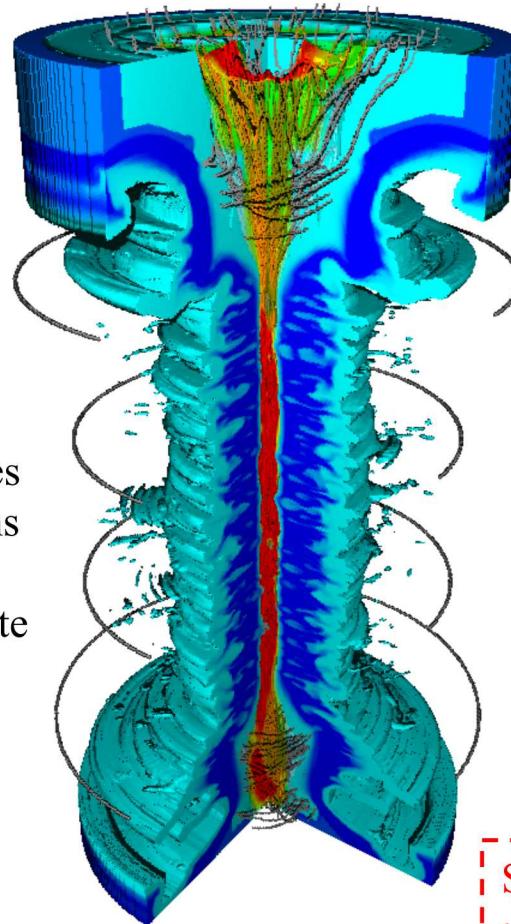
- Eventual DT fuel expts
- Transport/deposition
- More secondary reaction branches → diagnostics?

Fast tritons:

- Transport properties
- Secondary reactions → diagnostics
- 3.5 MeV  $\alpha$  surrogate

Fuel deuterons:

- Tail-depletion
- Transport properties
- Fuel-liner interface
- Mix



$$\rho_{\text{fuel}} \sim 0.5 \text{ g/cm}^3$$

$$\rho_{\text{liner}} \sim 10 \text{ g/cm}^3$$

$$(\rho R)_{\text{fuel}} \sim 2 - 10 \text{ mg/cm}^2$$

$$(\rho Z)_{\text{fuel}} \sim 0.2 - 1 \text{ g/cm}^2$$

$$(\rho R)_{\text{liner}} \gtrsim 1 \text{ g/cm}^2$$

$$T_{\text{fuel}} = 3 \text{ keV}$$

$$\Rightarrow \ln \Lambda_{ii} \sim \ln \Lambda_{ie} \sim 7.3$$

$$\Rightarrow \omega_{ci} \tau_{ii} \sim 1 \text{ at 100 MG}$$

$$T_{\text{fuel}} = 8 \text{ keV}$$

$$\Rightarrow \ln \Lambda_{ii} \sim \ln \Lambda_{ie} \sim 8.8$$

$$\Rightarrow \omega_{ci} \tau_{ii} \sim 3 \text{ at 100 MG}$$

$$\omega_{ce} \tau_{ee} = \mathcal{O}(100)$$

$$R/r_{L,\alpha} = \mathcal{O}(1)$$

Strongly magnetized electrons,  
marginally magnetized fuel ions,  
magnetized/confined fast ions,  
weak coupling → LFP

# Monte Carlo modeling of minority ion species

Test-ion kinetic equation, arbitrary number of background fuel species:

$$\frac{\partial F_a}{\partial t} = -\frac{\partial}{\partial \rho} \mathcal{F}_\rho F_a - \frac{\partial}{\partial \phi'} \mathcal{F}_\phi F_a - \frac{\partial}{\partial \mu} \mathcal{F}_\mu F_a - \frac{\partial}{\partial \varepsilon_k} \mathcal{F}_\varepsilon F_a + \frac{1}{2} \frac{\partial^2}{\partial \phi'^2} \mathcal{D}_{\phi\phi} F_a + \frac{1}{2} \frac{\partial^2}{\partial \mu^2} \mathcal{D}_{\mu\mu} F_a + \frac{1}{2} \frac{\partial^2}{\partial \varepsilon_k^2} \mathcal{D}_{\varepsilon\varepsilon} F_a$$

Drag terms

$\mathcal{F}_\rho = \sigma \varepsilon_k^{1/2} \cos \phi'$   
 $\mathcal{F}_\phi = \frac{Z_a}{2} \frac{\partial \Phi}{\partial \rho} \frac{\sin \phi'}{\sigma \varepsilon_k^{1/2}} - \sigma \varepsilon_k^{1/2} \frac{\sin \phi'}{\rho} + \chi_a$   
 $\mathcal{F}_\mu = \frac{Z_a}{2} \frac{\partial \Phi}{\partial \rho} \frac{\sigma \mu}{\varepsilon_k^{1/2}} \cos \phi' - \frac{\rho_m \Pi_a}{\varepsilon_k^{3/2}} \mu F(\varepsilon_k)$   
 $\mathcal{F}_\varepsilon = - \left( \frac{2 \rho_m \Pi_a}{\varepsilon_k^{1/2}} A_a \left\langle \frac{1}{A_b} \right\rangle [D(\varepsilon_k) - T_a D'(\varepsilon_k)] + Z_a \frac{\partial \Phi}{\partial \rho} \sigma \varepsilon_k^{1/2} \cos \phi' \right)$

${}^* \chi_a \equiv \frac{\omega_{ca}}{\nu_{\mu 0}}$  (magnetic field **only** shows up in gyrophase drag term)  
 ${}^* \Pi_a \equiv \frac{\langle Z_b^2 \ln \Lambda_{ab} \rangle}{\langle Z_b^2 \ln \Lambda_{ab} \rangle_0}$

Diffusion terms

$\mathcal{D}_{\phi\phi} = \frac{\rho_m \Pi_a}{\varepsilon_k^{3/2}} F(\varepsilon_k) \frac{1}{1 - \mu^2}$   
 $\mathcal{D}_{\mu\mu} = \frac{\rho_m \Pi_a}{\varepsilon_k^{3/2}} F(\varepsilon_k) (1 - \mu^2)$   
 $\mathcal{D}_{\varepsilon\varepsilon} = 4 \rho_m \Pi_a T_a A_a \left\langle \frac{1}{A_b} \right\rangle \frac{D(\varepsilon_k)}{\varepsilon_k^{1/2}}$

(Transition function,  $D$ , includes interactions with electrons and ions, assumed to be at same  $T$  locally)

$\frac{d\rho}{dt} = \mathcal{F}_\rho$   
 $\frac{d\phi'}{dt} = \mathcal{F}_\phi + \mathcal{D}_{\phi\phi}^{1/2} \Gamma_1(t)$   
 $\frac{d\mu}{dt} = \mathcal{F}_\mu + \mathcal{D}_{\mu\mu}^{1/2} \Gamma_2(t)$   
 $\frac{d\varepsilon_k}{dt} = \mathcal{F}_\varepsilon + \mathcal{D}_{\varepsilon\varepsilon}^{1/2} \Gamma_3(t)$

The formal solution to this equation can be found by solving an equivalent set of single-particle stochastic differential orbital equations for an ensemble of test particles

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# Overview of Knudsen loss mechanism

- First work on tail-ion depletion & Knudsen layers in ICF by Petschek and Henderson:

VOLUME 33, NUMBER 19

PHYSICAL REVIEW LETTERS

4 NOVEMBER 1974

## Burn Characteristics of Marginal Deuterium-Tritium Microspheres

Dale B. Henderson

*Los Alamos Scientific Laboratory, Los Alamos, New Mexico 87544*

(Received 5 August 1974)

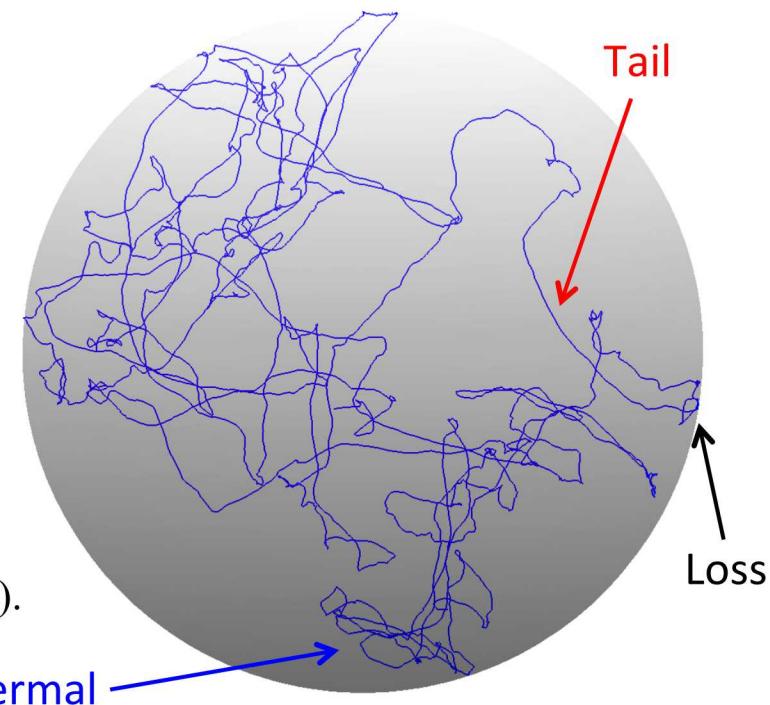
Long mean free paths for ions in the tail of the distribution may allow escape, quenching the burn of marginal ( $\rho R < 10^{-2}$  g/cm<sup>3</sup>) deuterium-tritium microspheres, possibly explaining the lack of success in experiments to date.

- Knudsen layer research in the ICF community has regained significant attention recently

- K. Molvig *et al.*, Phys. Rev. Lett. **109**, 095001 (2012).
- P. F. Schmit *et al.*, Phys. Plasmas **20**, 112705 (2013).
- B. J. Albright *et al.*, Phys. Plasmas **20**, 122705 (2013).
- X. Z. Tang *et al.*, Phys. Plasmas **21**, 032706 (2014).
- X. Z. Tang *et al.*, Phys. Plasmas **21**, 032707 (2014).
- C. J. McDevitt *et al.*, Phys. Plasmas **21**, 032708 (2014).
- M. J. Rosenberg *et al.*, Phys. Rev. Lett. **112**, 185001 (2014).

$$\text{Hydro : } \frac{\lambda_{\text{mfp}}}{L} \ll 1$$

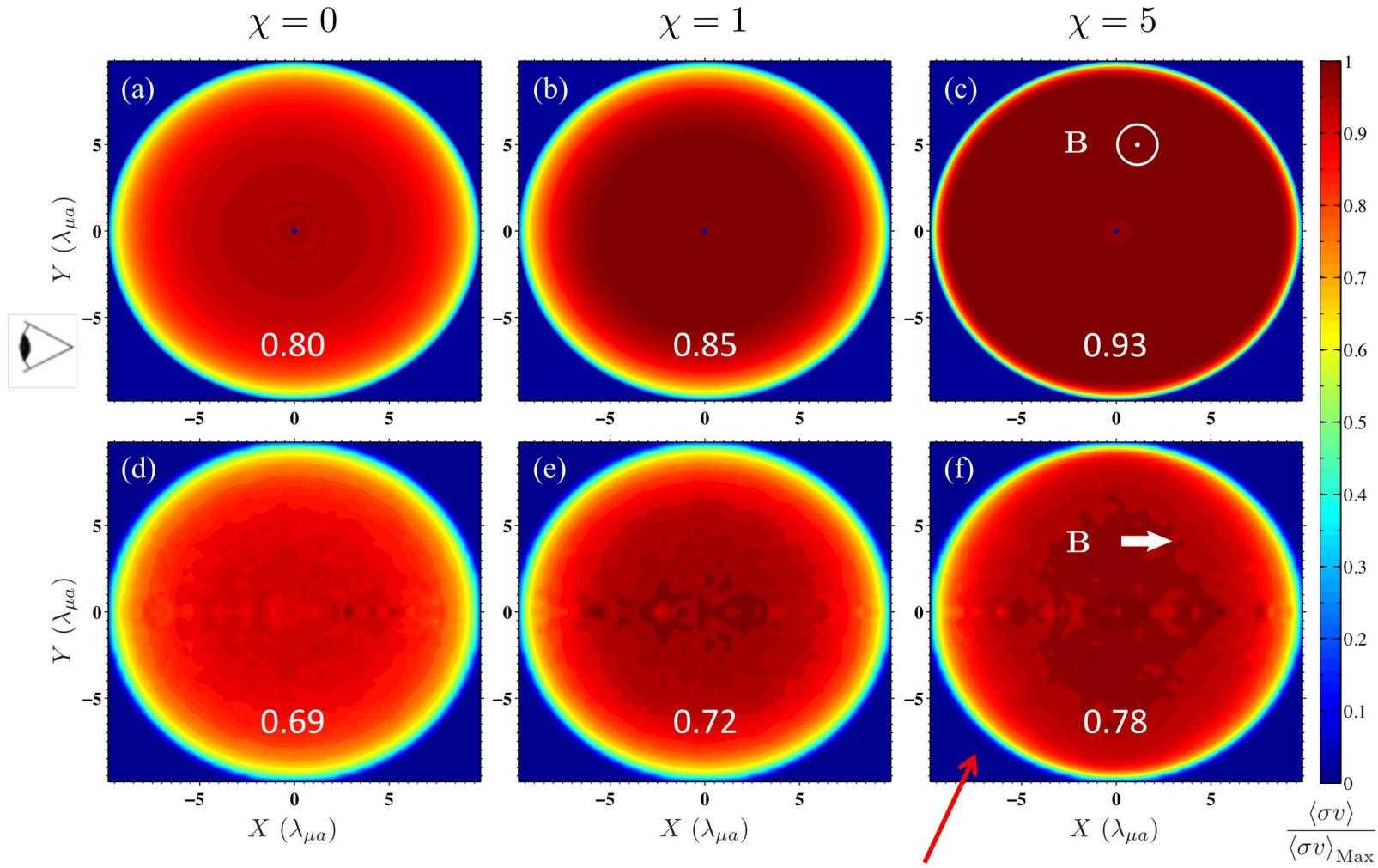
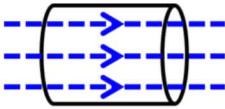
$$\text{Fuel ions : } \lambda_{\text{mfp}} \sim v^4$$



# Tail-depletion and reactivity reduction with B-fields\*

Cylindrical and spherical systems: 5 keV, 1 g/cc, DD plasma,  $N_K = 0.1$

**Cylinder:**  
depletion  
suppressed  
*completely* by  
B-field



\*  $\chi$  defined alternatively as :  $\chi \equiv \frac{\omega_{ca}}{\nu_{\mu a}} \approx \frac{N_K}{N_B}$

Spherical symmetry-breaking of  
reactivity contours (3D  $\rightarrow$  1D)

# Exploring the dimensionless parameter landscape

Cylindrical system: 8 keV, 1 g/cc, DD plasma: volume-averaged reactivity reduction

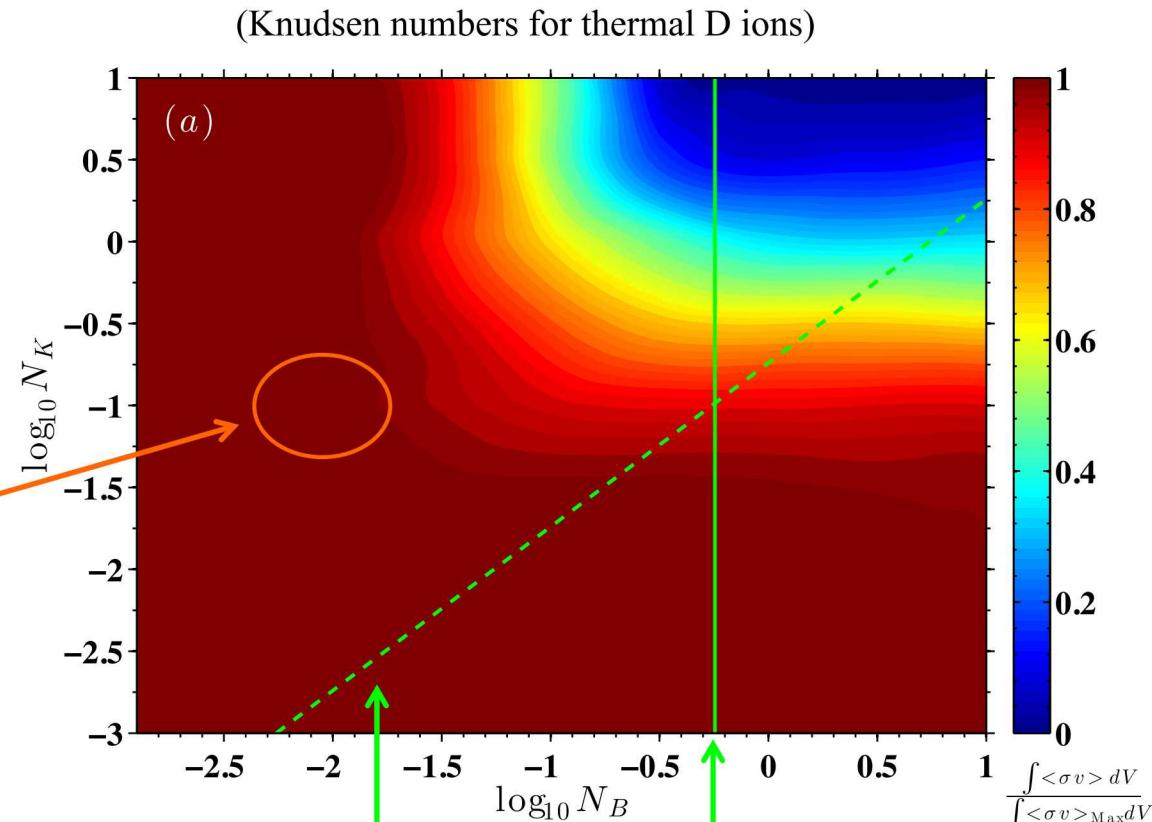
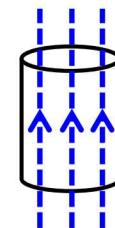
Relevant MagLIF timescales:

$$\tau_{dd}^\mu \sim \mathcal{O}(10 \text{ ps})$$

$$\tau_{\text{eq}} \sim \mathcal{O}(100 \text{ ps})$$

$$\tau_{\text{burn}} \sim \mathcal{O}(1 \text{ ns})$$

**MagLIF point design is well within the plateau regime for fully restored Maxwellian reactivities.**



$$N_K \sim \frac{\lambda_{\text{mfp}}}{L} \sim \frac{T^2}{nL}$$

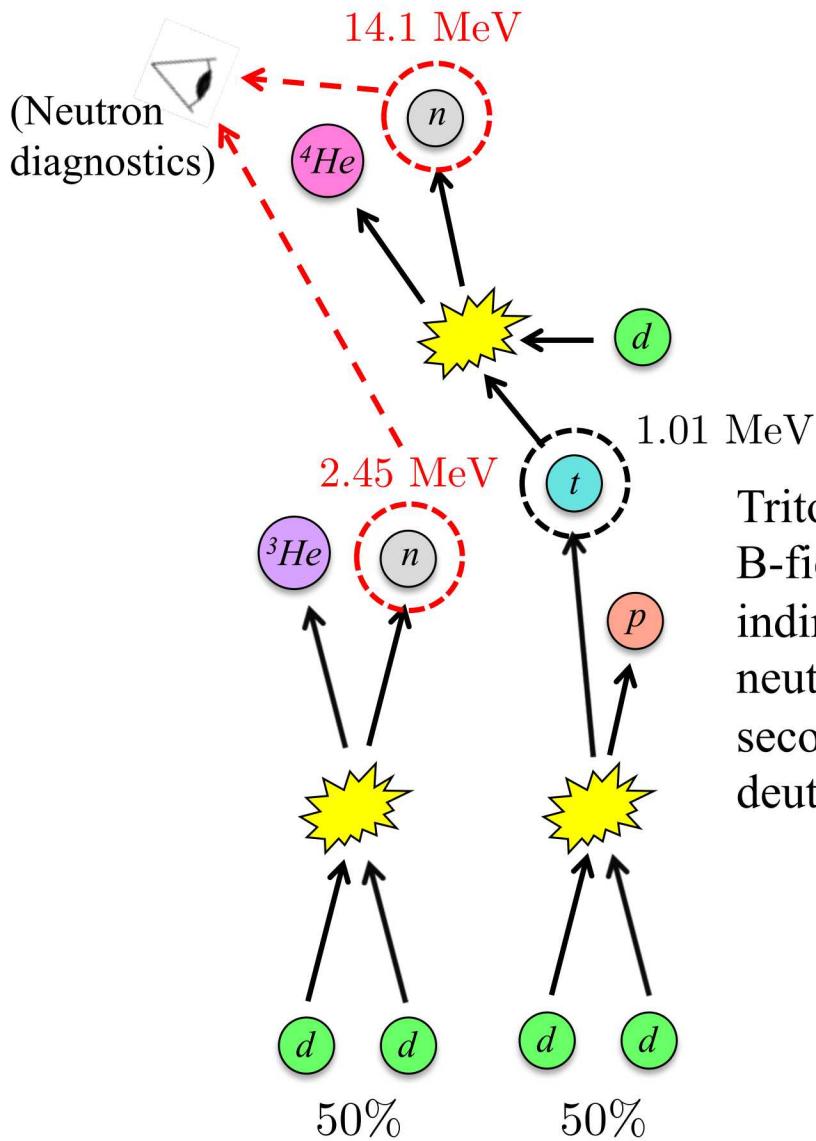
$$N_B \sim \frac{\rho_L}{L} \sim \frac{T^{1/2}}{BL}$$

Scan  $(N_K, N_B)$ -space at fixed  $T, n$  by varying  $B, L$ .

$$(\xi \approx 3.1)$$

\*Hot spot parameters: 8 keV, 0.5 g/cc, 100 MG B-field, 100 micron radius

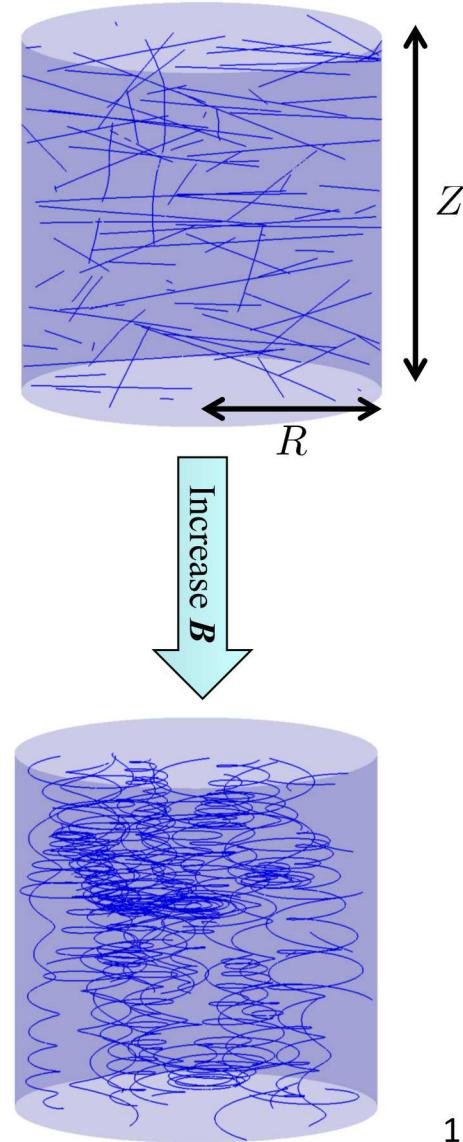
# Kinetic modeling of secondary reaction physics



We model ensembles of triton “test particles” and calculate their reactivities

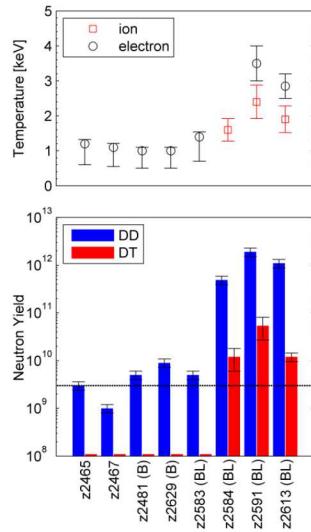
Tritons carry info about the B-field, which we extract indirectly via observation of neutrons produced by their secondary reactions with the deuterium fuel

A strong magnetic field causes triton path lengths to scale with  $Z$  instead of  $R$ , where  $Z \gg R$

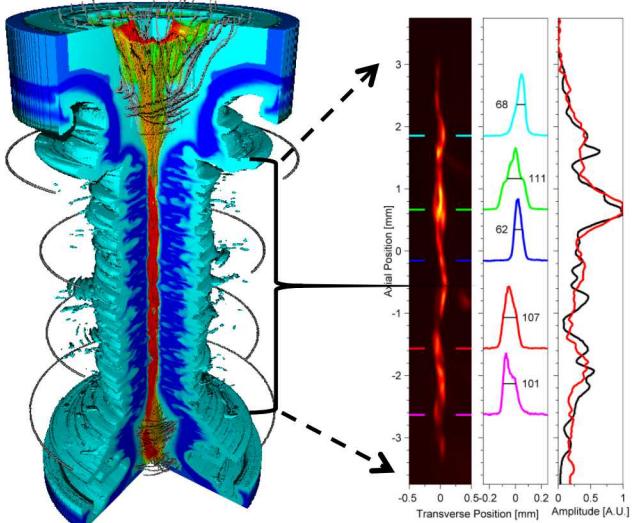


# Probing magnetization and mix with secondary nuclear reactions

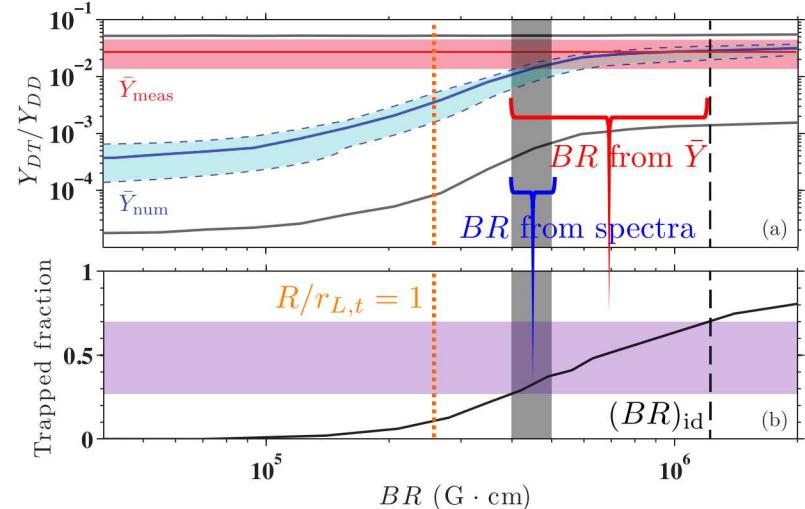
Roosevelt experimental temperature/yield data\*\*



Roosevelt experimental fuel self-emission data\*\*



Kinetic calculations of DT/DD vs.  $BR^*$



Experimental DT/DD  
BR from neutron spectra

Numerical DT/DD & confidence interval  
Estimated trapped triton fraction

First integrated tests of MagLIF:

- multi-keV  $T_e$  and  $T_i$
- $O(10^{12})$  primary DD neutron yields
- *Remarkable*,  $O(10^{10})$  secondary DT neutron yields

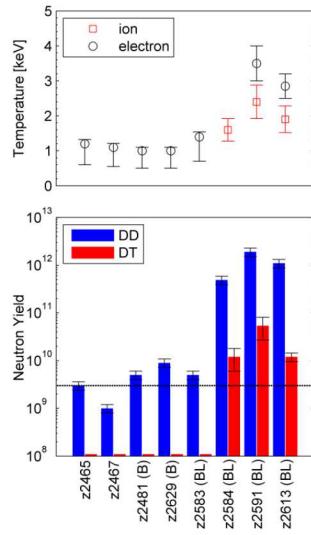
Self-emission data constrains stagnation radius:  $R \gtrsim 30 - 60 \mu\text{m}$

Kinetic ions + Monte Carlo nuclear reactions:

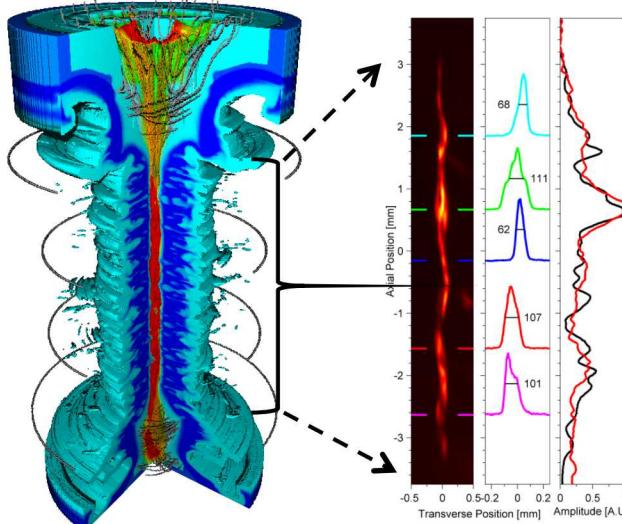
- Inputs determined by expt data & uncertainty
- DT/DD yield scales strongly with  $BR$
- Triton gyroradius:  $R/r_{L,t} \propto BR$
- For  $R \approx 50 \mu\text{m}$ , indicates  $B \gtrsim 80 \text{ MG}$
- Models assume homogeneous fuel conditions, effects of inhomogeneities being studied

# Probing magnetization and mix with secondary nuclear reactions

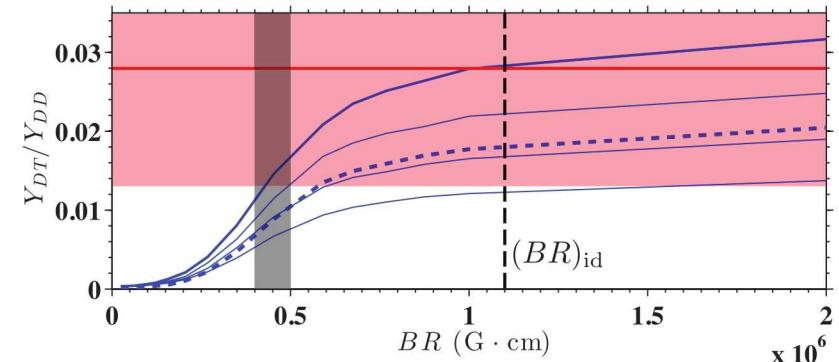
Roosevelt experimental temperature/yield data\*\*



Roosevelt experimental fuel self-emission data\*\*



DT/DD vs. BR vs. beryllium mix\*



dashed :  $\rho_d R = 1 \text{ mg/cm}^2$ ,  $n_{\text{Be}}/n_{\text{tot}} \equiv c_{\text{Be}} : 0.1$

solid :  $\rho_d R = 2 \text{ mg/cm}^2$ ,  $c_{\text{Be}}$  (ascending order) : 0.3, 0.2, 0.1, 0

Experimental DT/DD      Numerical DT/DD  
BR from neutron spectra

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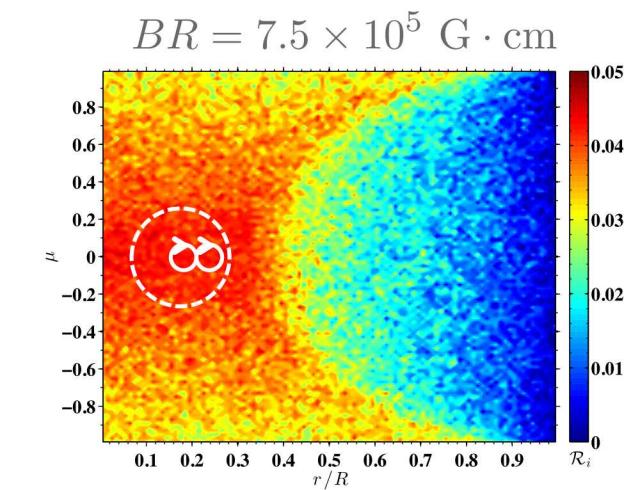
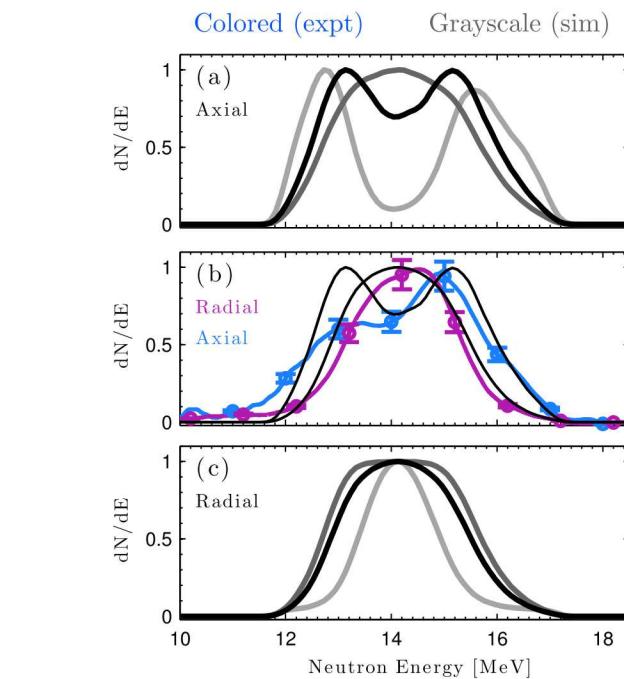
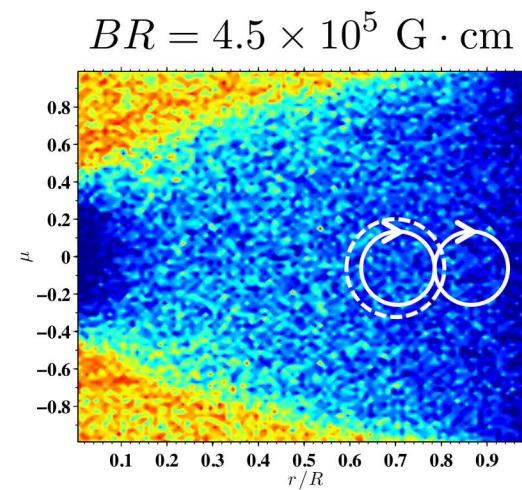
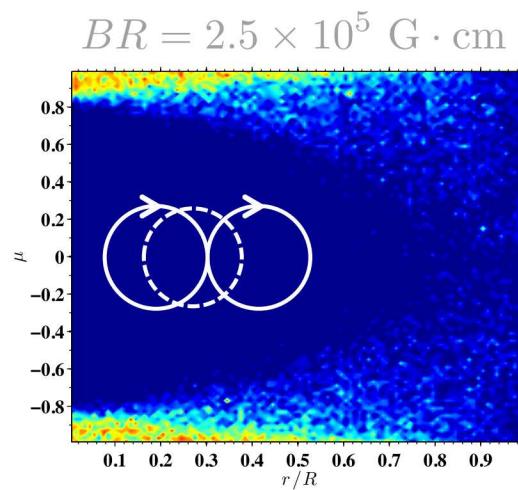
Self-emission data constrains stagnation radius:  $R \gtrsim 30 - 60 \mu\text{m}$

DT/DD also constrains fuel-pusher mix:

- Assumed uniform mix (worst-case scenario)
- Mix lowers DT/DD in magnetized limit (enhanced stopping  $\rightarrow$  shorter triton ranges)
- $(BR)_i$  assumes no flux losses and  $R = 50\mu\text{m}$
- Mix likely  $< 10\%$ , emission analysis agrees\*\*
- Reducing measurement uncertainty important

# Probing magnetization and mix with secondary nuclear reactions

- Measured DT neutron energy spectra anisotropic
- Calculated spectra match data well, suggesting narrower range:  $BR \approx 4 - 5 \times 10^5 \text{ G} \cdot \text{cm}$
- Axial view shows double-peak, due to Doppler shift from most reactive tritons
- Radial view shows single peak
- Spectra features highly sensitive to  $BR$ !*
- Figures below show triton reaction probability ( $\mathcal{R}_i$ ) based on initial radial position and pitch-angle relative to magnetic field ( $\mu = \cos \theta$ )



# HYDRA (fluid) and LSP (particle) simulations are used to generate synthetic neutron spectra\*

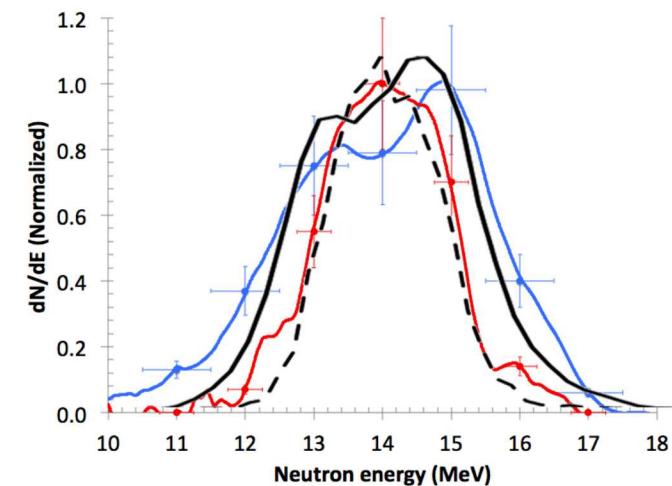
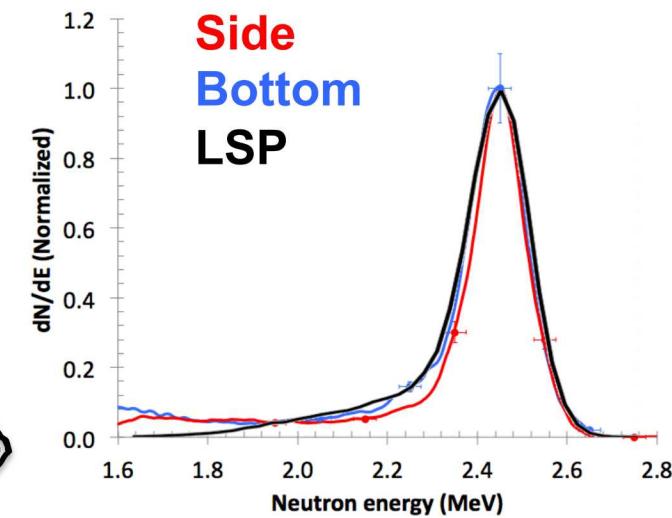
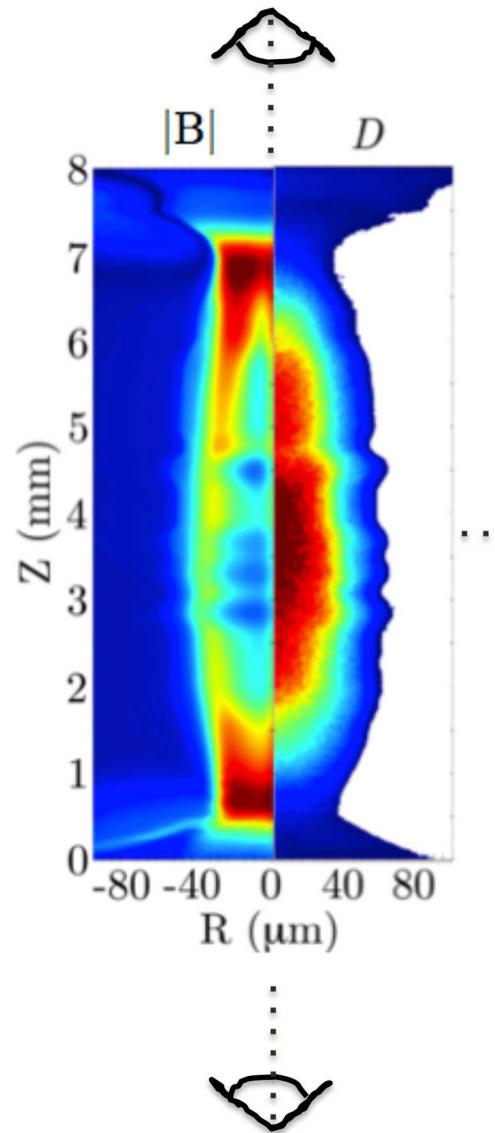
PIC simulations are initialized with HYDRA output ( $n$ ,  $T$ ,  $B$ ) just before stagnation, and then run through burn.

All ions are evolved kinetically.

Binary scattering between all species is treated.

Binary fusion events are treated.

Synthetic neutron detectors are located to the side, top, and bottom of the stagnation column.



# Other interesting kinetic topics in ICF with possible consequences/departures in MagLIF

- Kinetic transport effects at the plasma-liner (plasma-metal) interface
  - Recent developments in theory and modeling of *unmagnetized* plasma-metal interfaces:
    - “Non-linear structure of the diffusing gas-metal interface in a thermonuclear plasma,” K. Molvig *et al.*, 2014 Anomalous Absorption Conference. **Transport theory in lo-Z/hi-Z plasma.**
    - “Kinetic effects at material interfaces in ICF implosions,” S. C. Wilks *et al.*, 2014 Anomalous Absorption Conference. **Lsp modeling of lo-Z/hi-Z plasma.**
    - “A new theory of mix in Omega capsule implosions,” D. A. Knoll *et al.*, 2014 Anomalous Absorption Conference. **Transient double-layer effects.**
  - Differences in MagLIF:
    - Axial magnetic field marginally to moderately magnetizes fuel ions and strongly magnetizes electrons near stagnation, but liner plasma far more collisional. Different local transport modes?
    - Does  $\mathbf{B}$  suppress double-layers/ambipolar fields? If not completely, and radial E-field exists near density gradient, possible azimuthal  $\mathbf{E} \times \mathbf{B}$  rotation
    - Extra D.o.F to account for. Velocity shear transport barrier?