

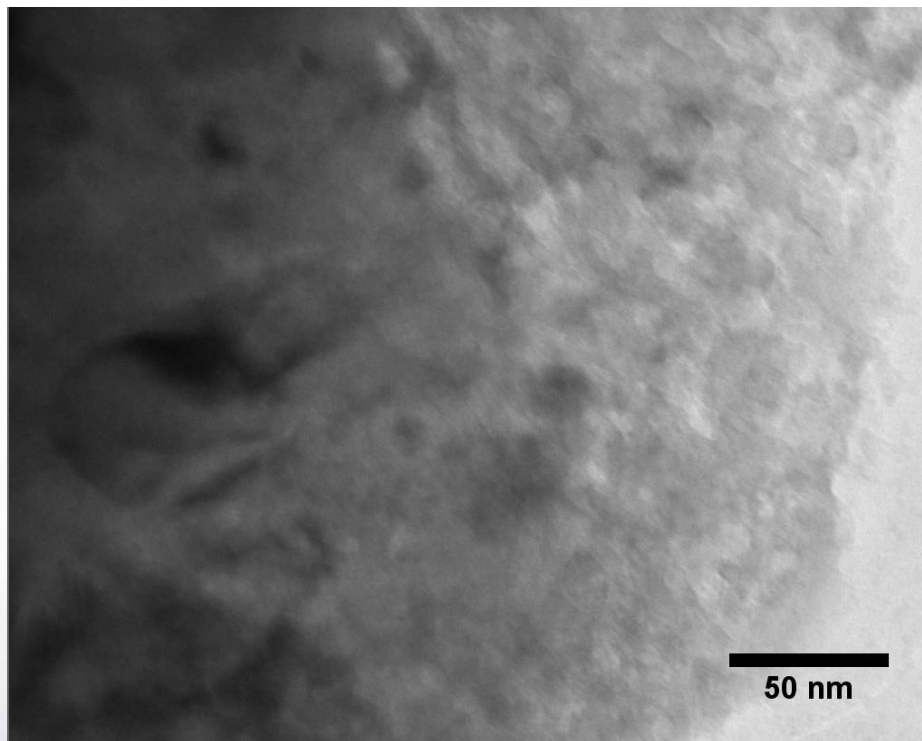
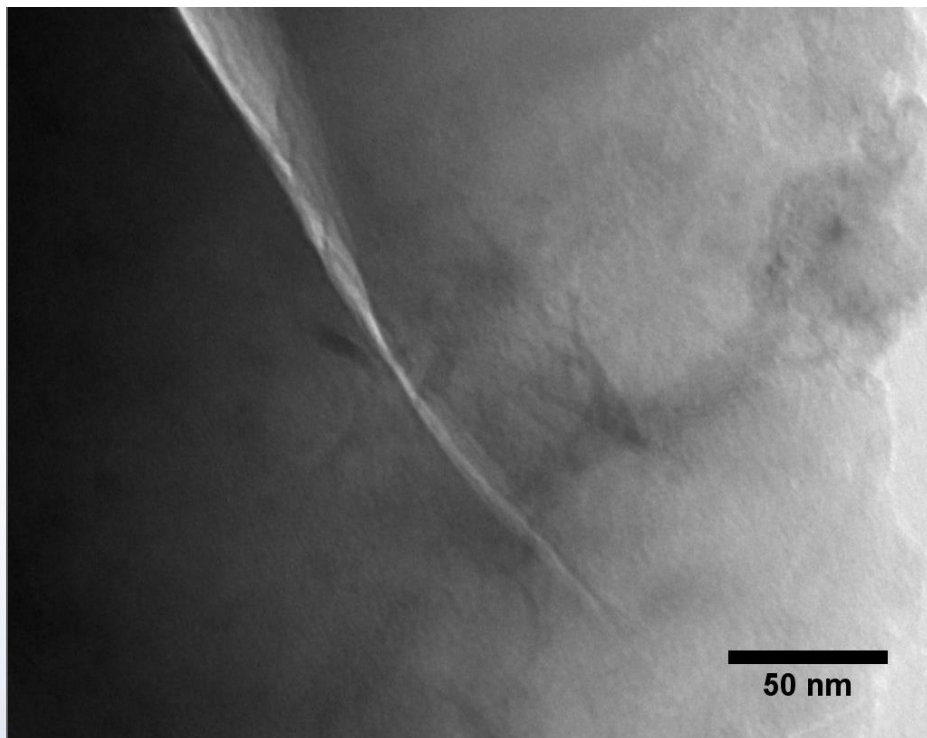
Studying Synergistic Radiation Effects in TPBAR Materials with In-situ Triple Beam Irradiation TEM

SAND2017-13384C

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Sandia National Laboratories & Pacific Northwest National Laboratory

October 9, 2017



Utilizing *In situ* TEM microscopy to deconvolute governing environments and elucidate the underlying mechanisms.



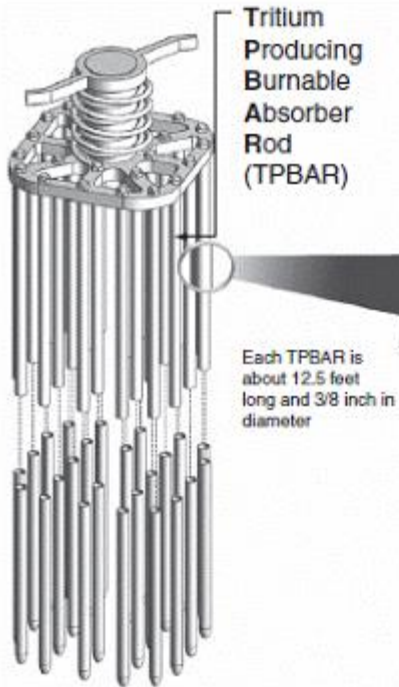
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TPBAR Design & Reactor Environment

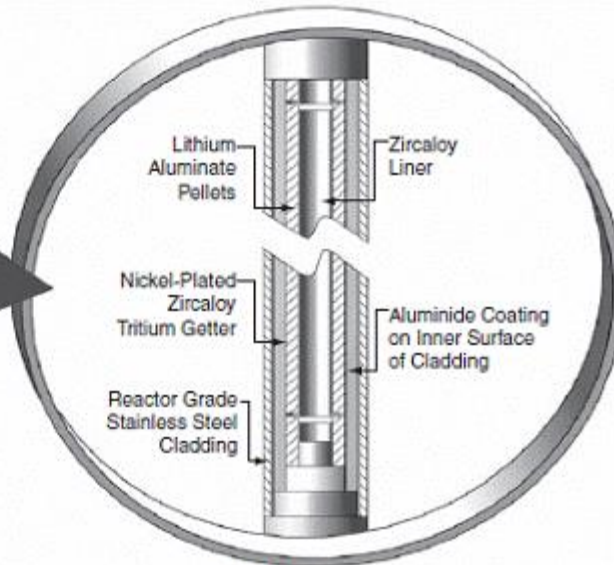
Tritium Producing Burnable Absorber Rod (TPBAR)



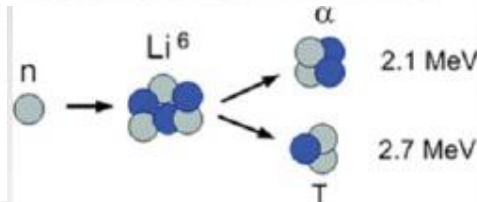
Each TPBAR is about 12.5 feet long and 3/8 inch in diameter

Assemblies consist of 12 to 24 TPBARs suspended from a base plate

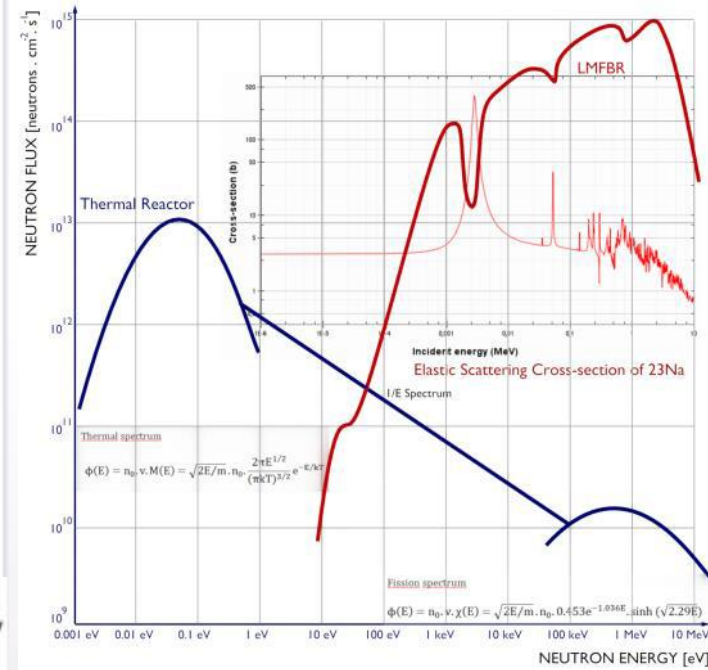
Source: INBSA



Not to scale



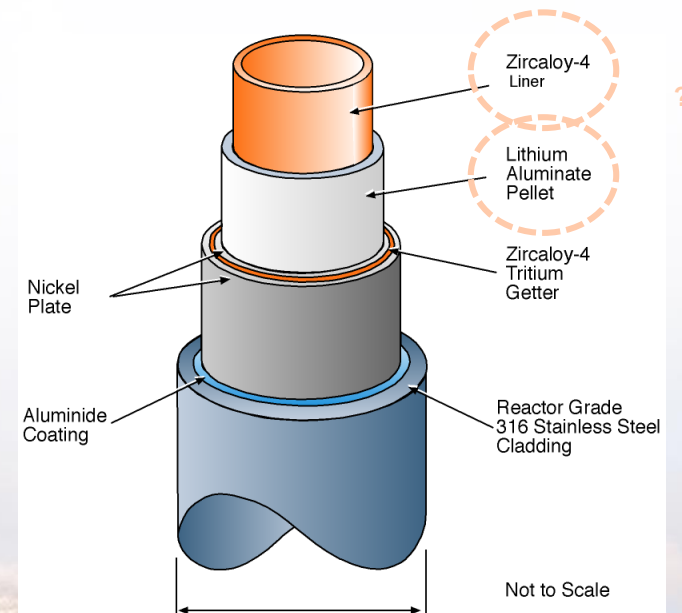
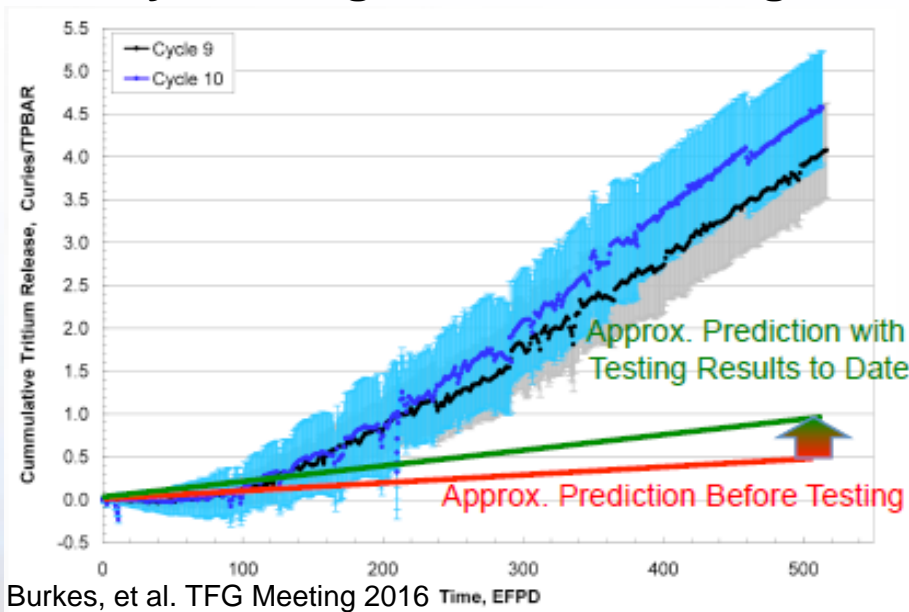
Tritium Producing Burnable Absorber Rod (TPBAR)



Extremely Complex Environment: Displacement Damage, Tritium Production, Helium Production, Mechanical Stress & Elevated Temperatures

Understanding Tritium Permeation in TPBAR

- TPBAR ^3H permeation is higher than predictive performance models
 - In 2004, during Cycle 6, the predicted levels were ~ 0.5 Ci/TPBAR/cycle and actual levels were ~ 4 Ci/TPBAR/cycle (0.04% of total ^3H produced)
- Mechanisms responsible for differences between predictions and observations are not well understood
- Currently building an understanding of fundamental ^3H -He-defect interactions



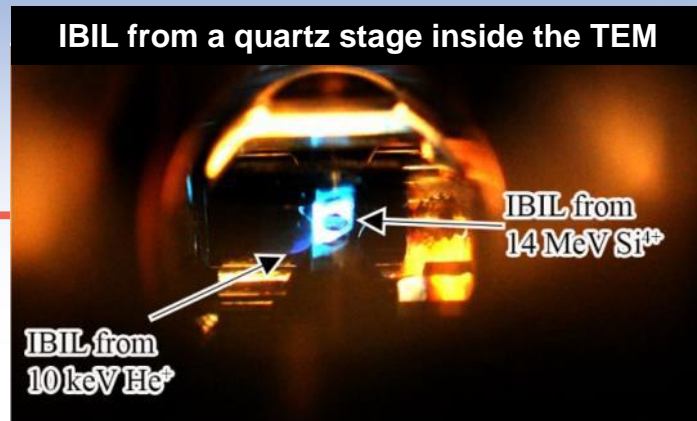
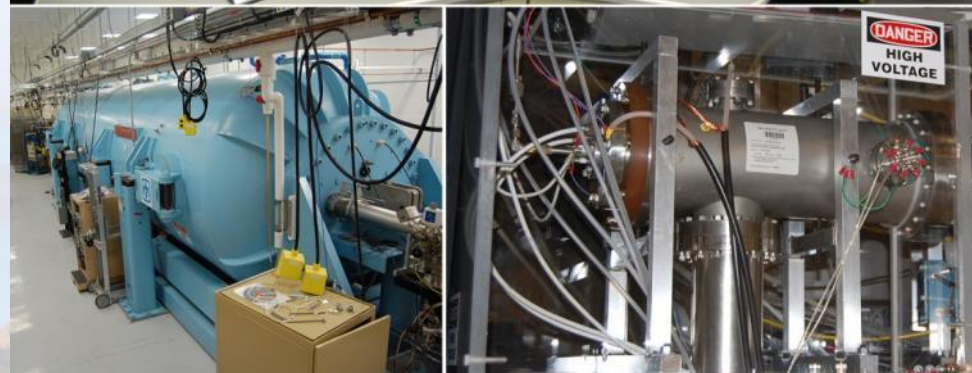
Simulating neutron irradiation in a reactor is complicated, and TPBAR adds the complication of ^3H production



Sandia's Concurrent *In situ* Ion Irradiation TEM Facility

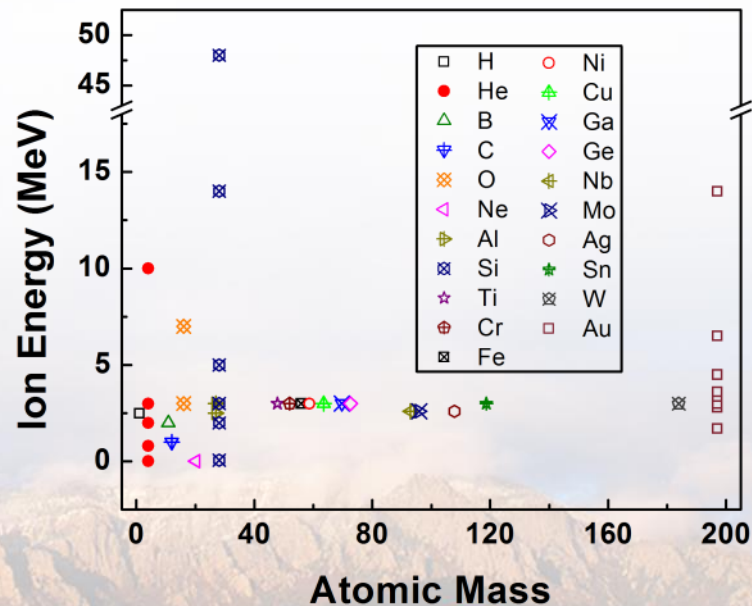
Collaborator: D.L. Buller

10 kV Colutron - 200 kV TEM - 6 MV Tandem

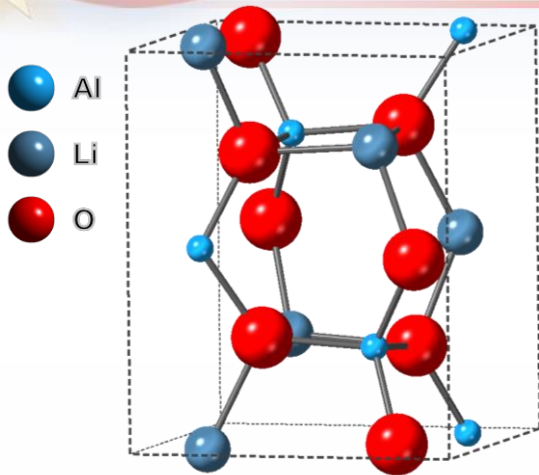


Direct real time observation of ion irradiation, ion implantation, or both with nanometer resolution

Ion species & energy introduced into the TEM



LiAlO₂ Background

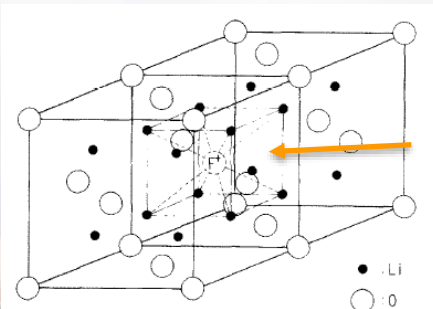


Previous Work

- Structural defects
 - Luo *et al* JNM 372 (2008) 53-58
- Volume swelling
 - Noda JNM 179-181 (1991) 37-41
- ³H detrapping
 - Oyaidzu *et al* JNM 375 (2008) 1-7
- Gas diffusion and release
 - Raffray *et al* JNM 210 (1994) 143-160

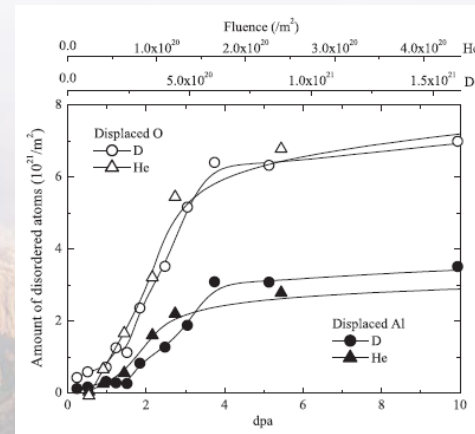
γ -LiAlO₂ is tetragonal (SG: P 41 21 2)

- H isotopes are thought to trap in oxygen vacancies
 - ²H release occurs at the same temperature as defect annealing in implanted LiAlO₂



Can be determined with luminescence

Noda JNM 179-181 (1991) 37-41

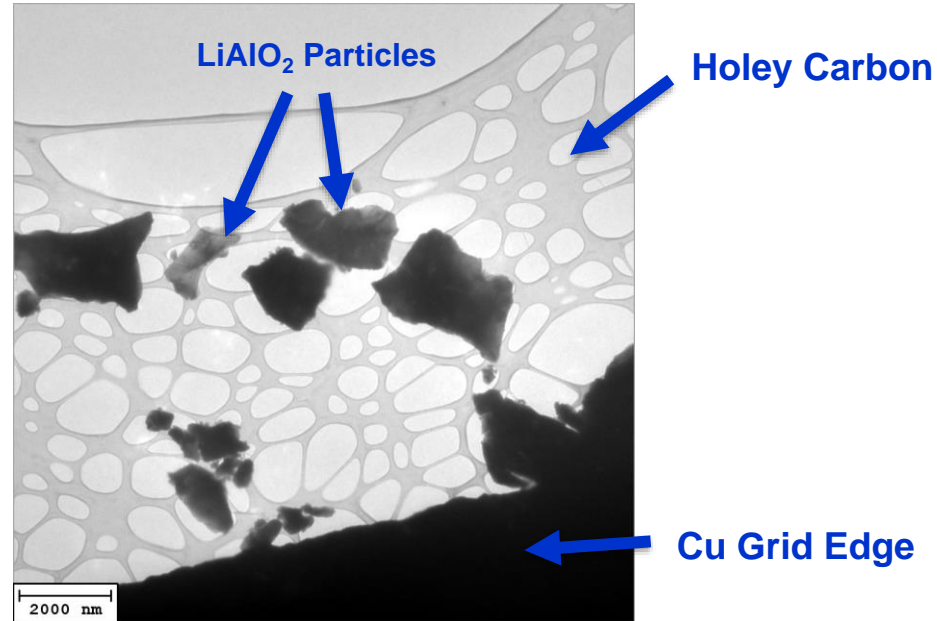


Katsui *et al.*
NIMB 268 (2010)
2735-2739



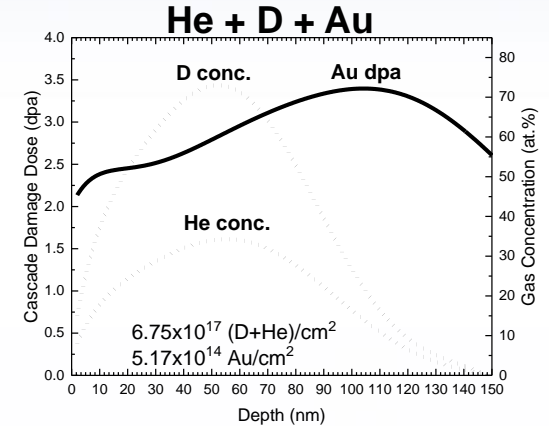
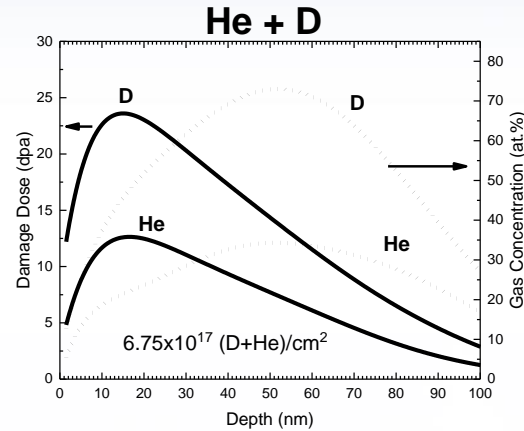
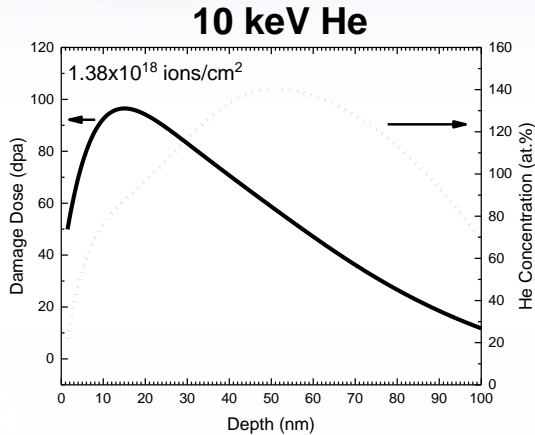
LiAlO₂ In-situ Ion Irradiation Preparation

- Powders were drop-cast onto TEM grids

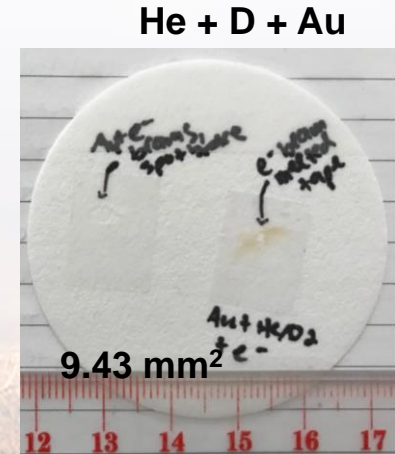
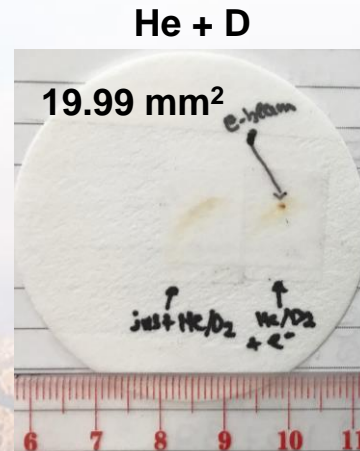
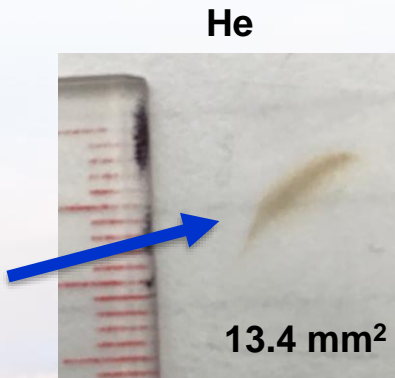


- Samples were heated to **310°C** using Hummingbird HT stage
- Four sets of irradiations:
 - 200 keV electrons → simulates Gamma (γ) irradiation
 - 200 keV e⁻ + 10 keV He → simulates γ and He accumulation from ⁶Li transmutation and ³H decay
 - 200 keV e⁻ + 10 keV He + 5 keV D → simulates γ , He, and ³H interaction
 - 200 keV e⁻ + 10 keV He + 5 keV D + **1.7 MeV Au** → simulates γ , gas build-up + cascades

LiAlO₂ In-situ Ion Irradiation Prediction (SRIM) and Beam Alignment



He & D beams are shaped like a comet due to TEM objective lens

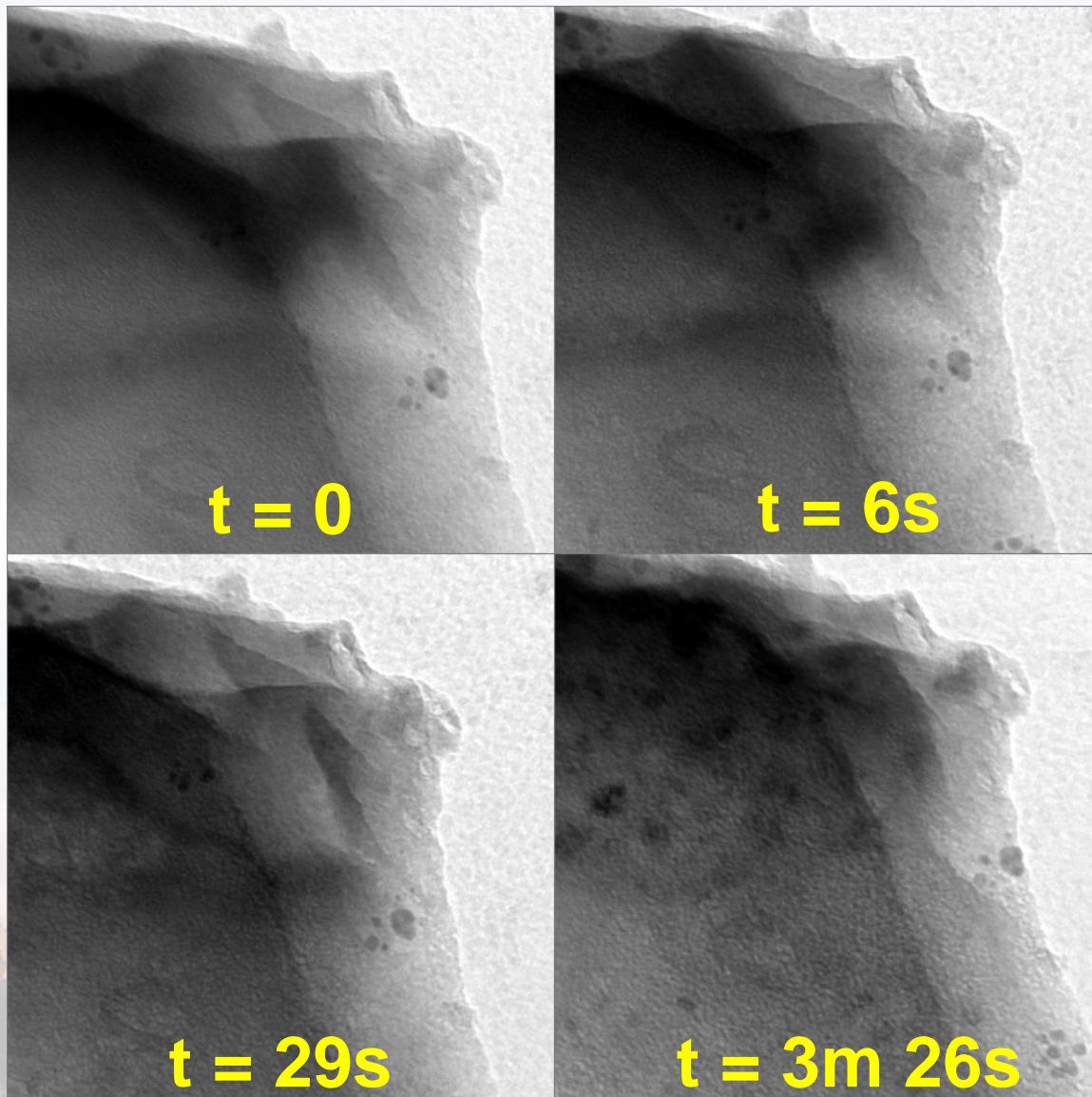
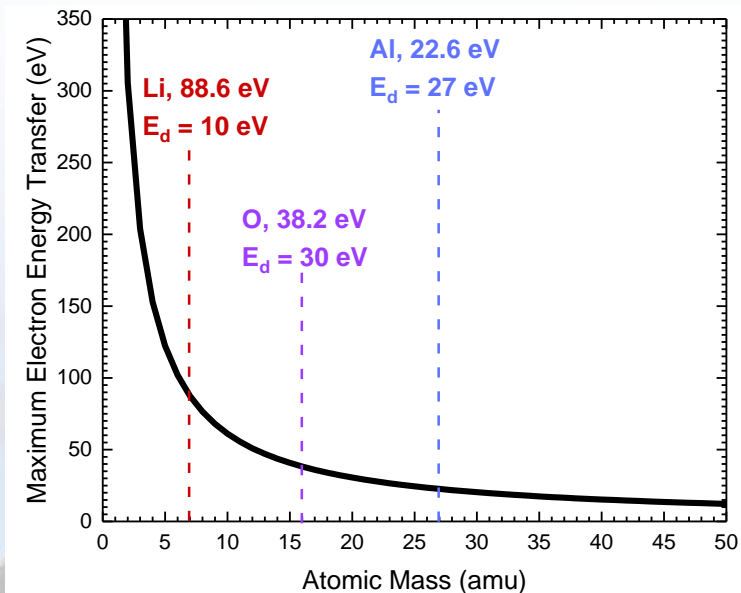


Most He/D diffuses out of thin film immediately



Electrons Alone Induced Void Growth

- Voids were observed to form under the electron beam in several particles
- Rate of void formation is not consistent between particles
- Possibly due to electron beam displacing Li and O atoms



In-situ TEM He implantation at 310°C

Before

-518 nm defocus

Focused

+518 nm defocus

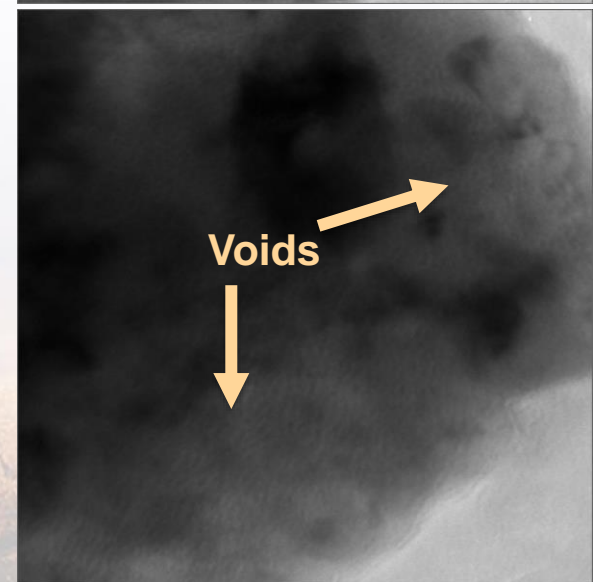
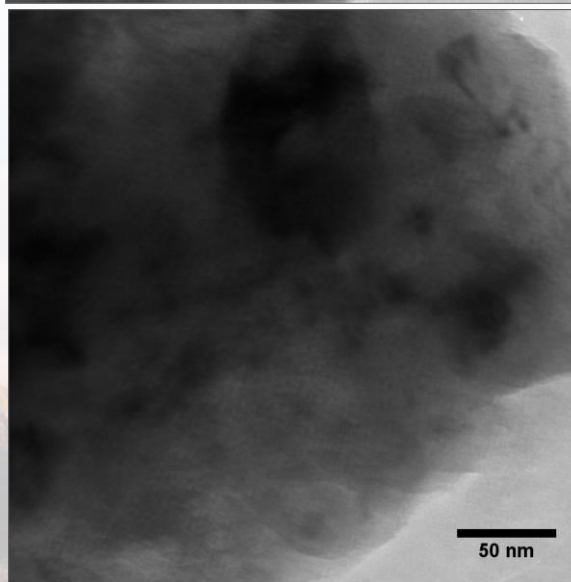
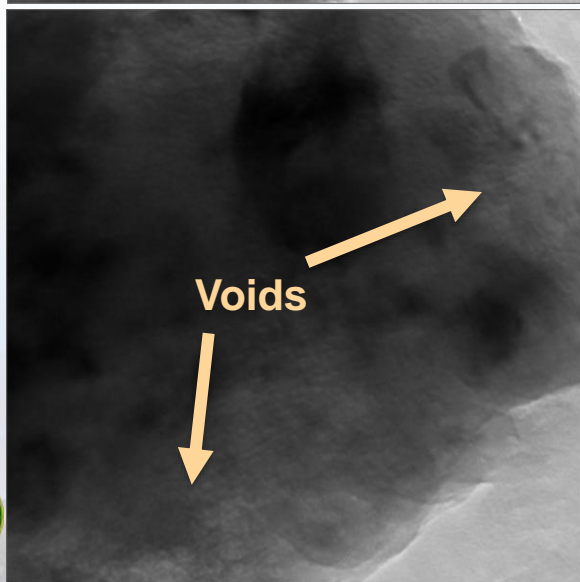
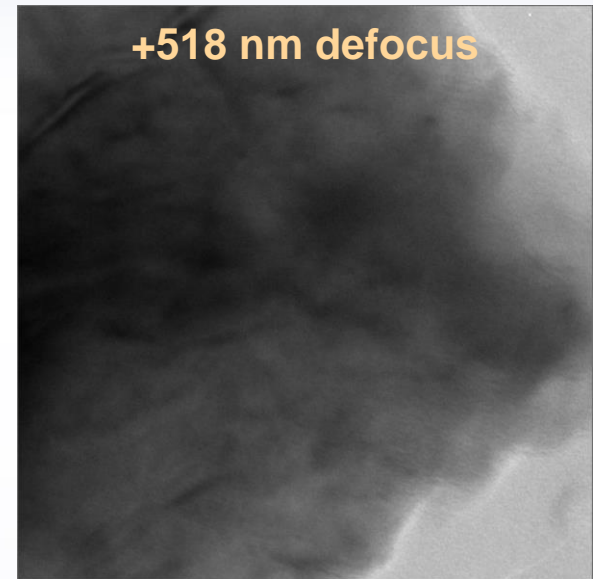
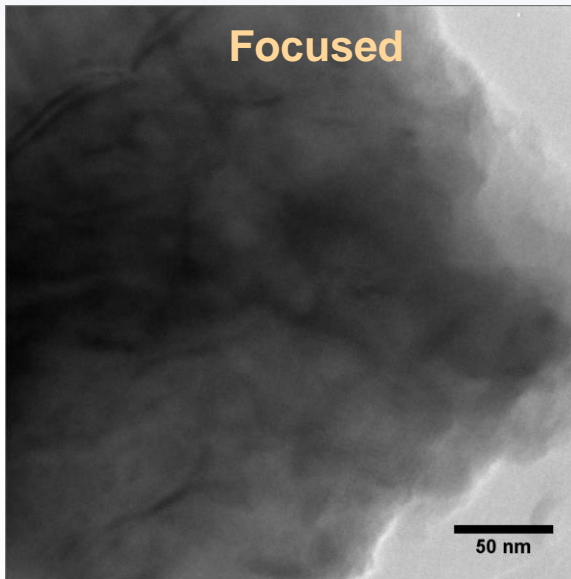
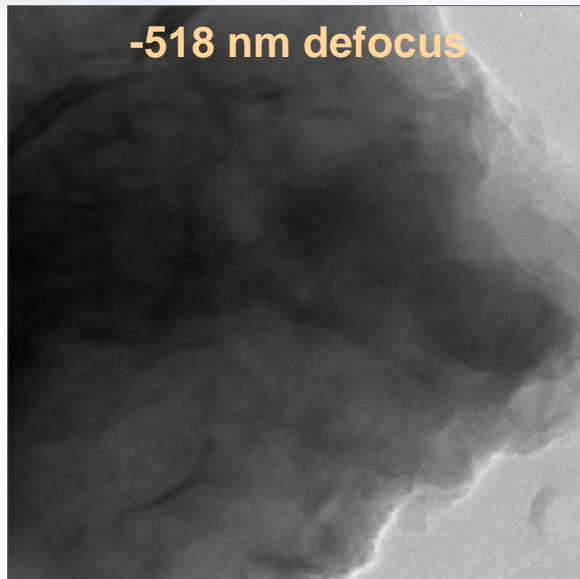
50 nm

After

Voids

Voids

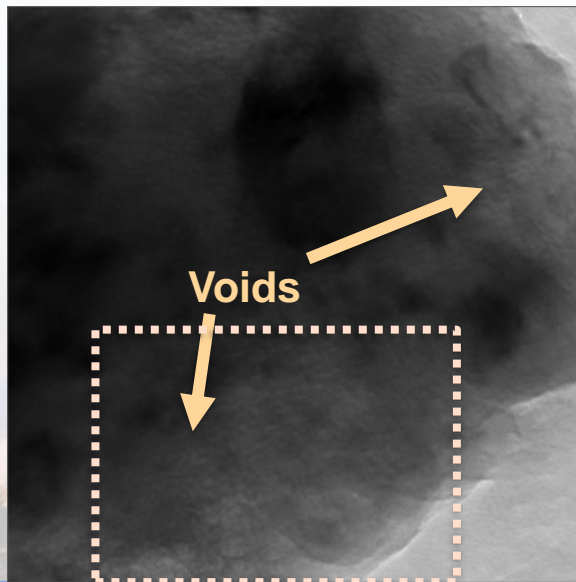
50 nm



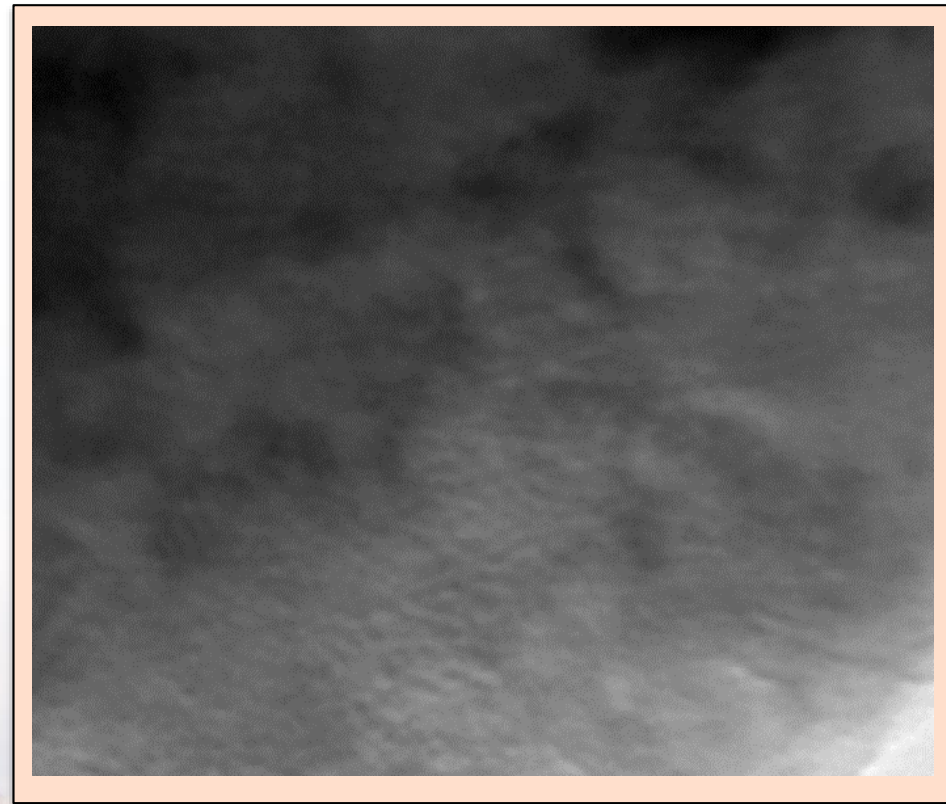
In-situ TEM He implantation at 310°C

- Each frame = 1 min of irradiation
- Because the voids are difficult to see in powders, I paused the video at a few points to show overfocus images
- Electron beam on for most of experiment
- Bubbles formed after ~13 min (1.5×10^{17} He/cm²)

After Irradiation

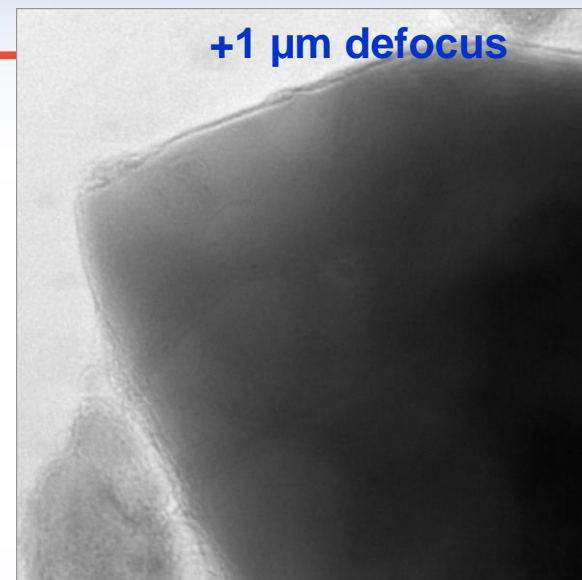
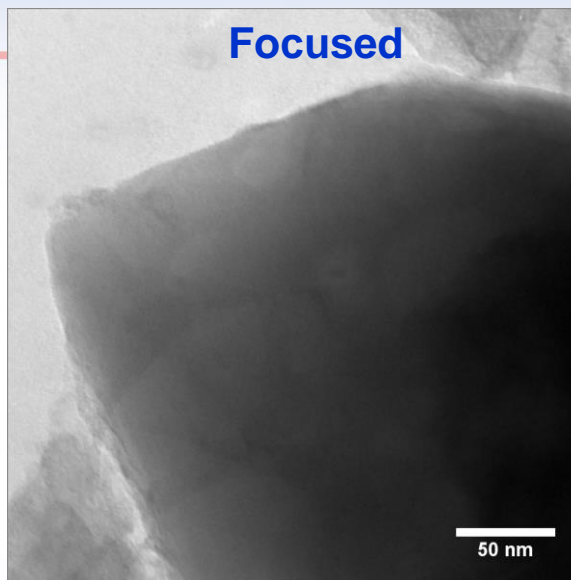
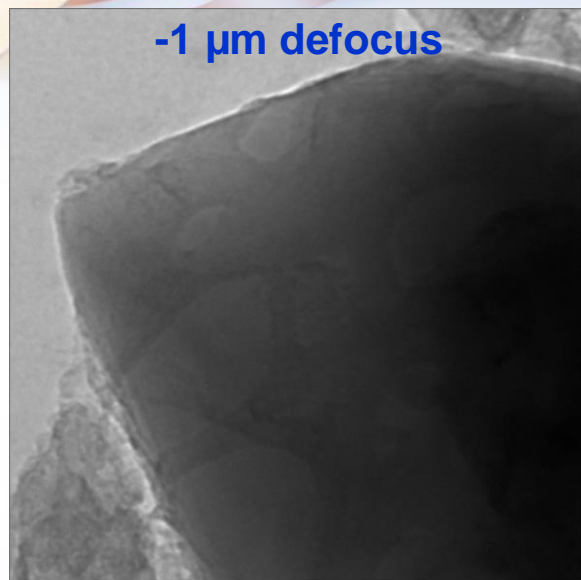


In-situ Video

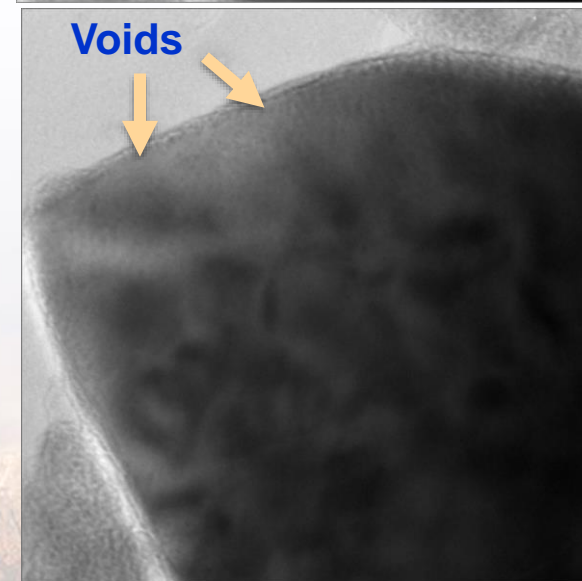
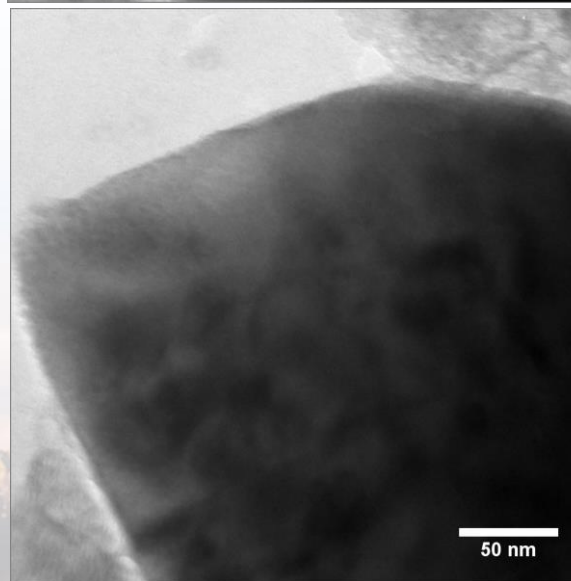
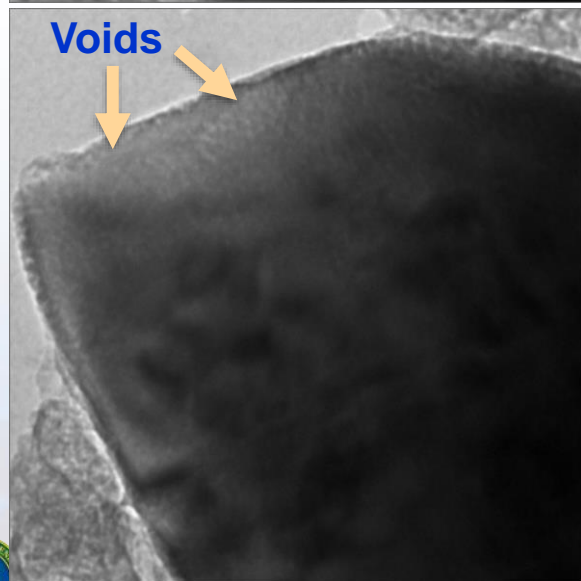


In-situ TEM He & D irradiation at 310°C

Before



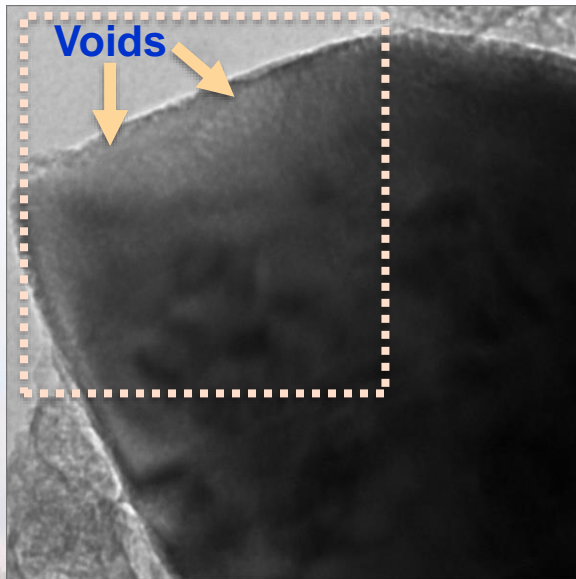
After



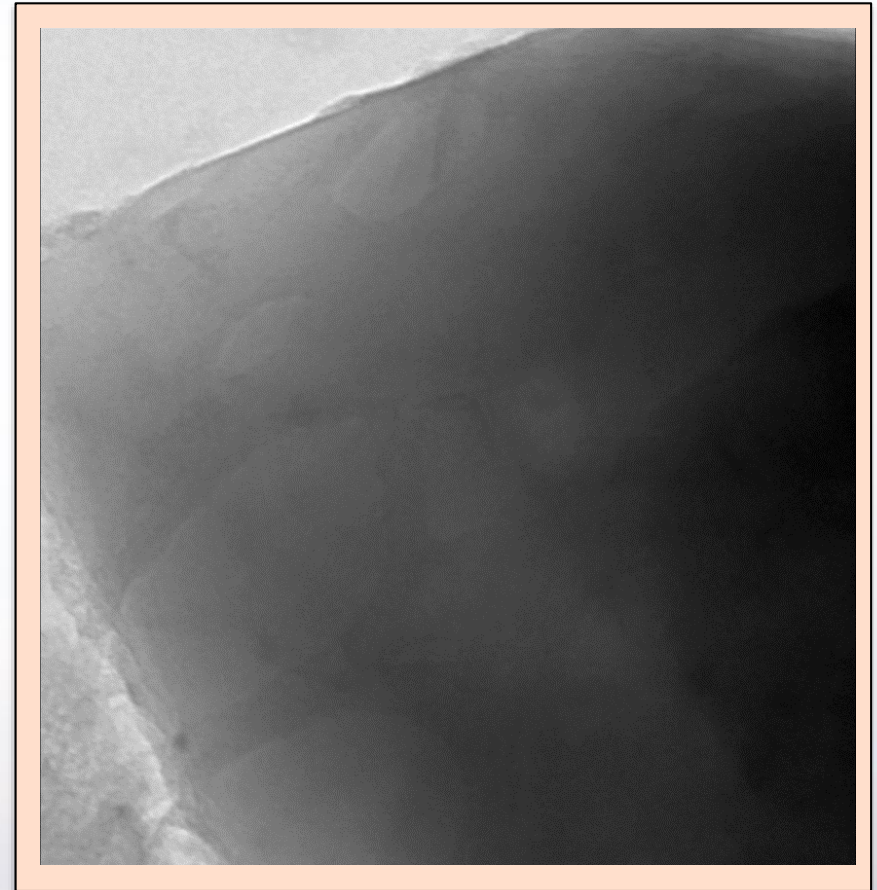
In-situ TEM He & D irradiation at 310°C

- Each frame = 5 min of irradiation
- All underfocus images
- Electron beam was off except for imaging
- Bubbles formed after ~60 min (1.7×10^{17} He/cm², 3.4×10^{17} D/cm²)

After Irradiation



In-situ Video



In-situ TEM He, D, & Au at 310°C

Before

-518 nm defocus

Focused

+518 nm defocus

After

50 nm

50 nm

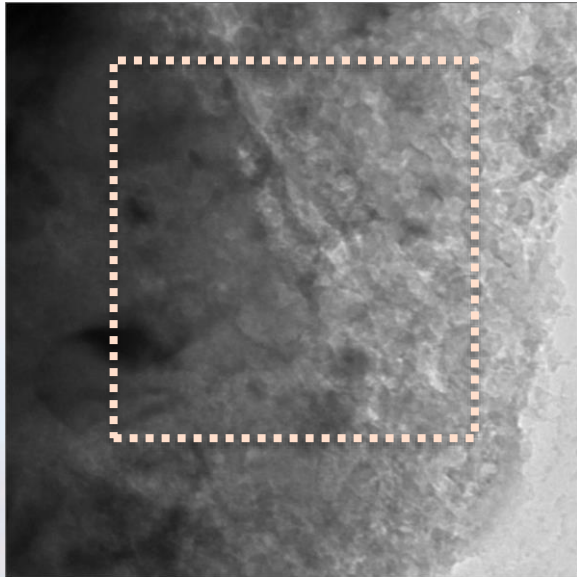
No single eucentric:
Drastic increase in
surface roughness!



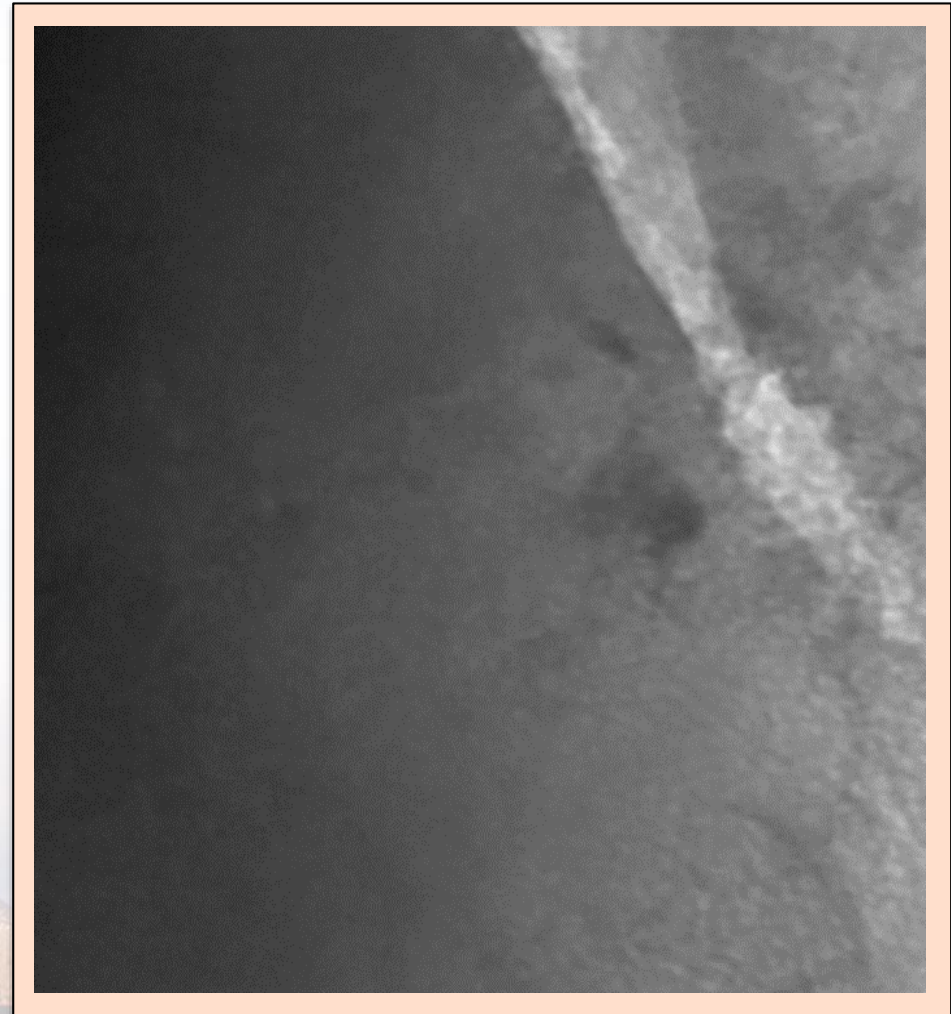
In-situ TEM He, D, & Au at 310°C

- Each frame = 5 min of irradiation
- Pre-existing voids could have an effect on this final microstructure
- Electron beam was on for most of the experiment

After Irradiation



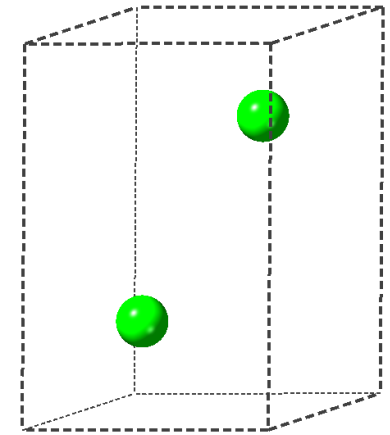
In-situ Video



Zircaloy Background

What is Zircaloy?

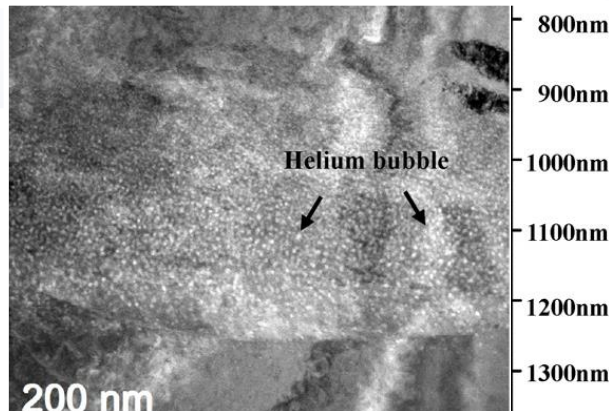
- Zircaloy-2: predominantly used as fuel cladding for BWRs
 - α -Zr, 1.5% Sn, 0.15% Fe, 0.1% Cr, 0.05% Ni
- Zircaloy-4: Removed the Ni and increased Fe content for less H uptake in certain reactor conditions
 - α -Zr, 1.5% Sn, 0.2% Fe, 0.1% Cr
- Zr-Nb alloys (e.g. Zirlo) are also common
- α -Zr has a **hexagonal close-packed (HCP)** crystal structure up to 810°C



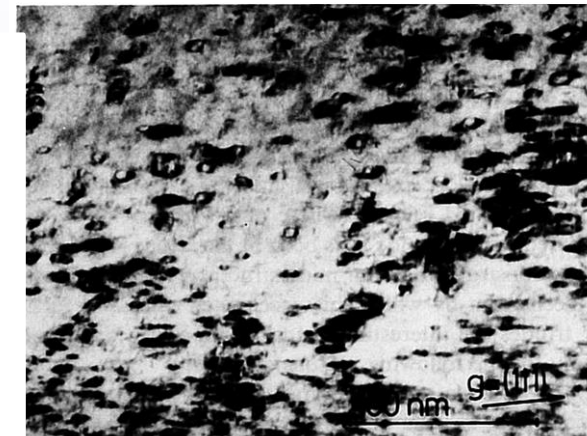
Crystal Structure of α -Zr (HCP)

Gas and defect behavior in Zr/Zr alloys

- ^3H , H, and He diffusion and release
- Bubble formation
- Irradiation induced metallic precipitate formation



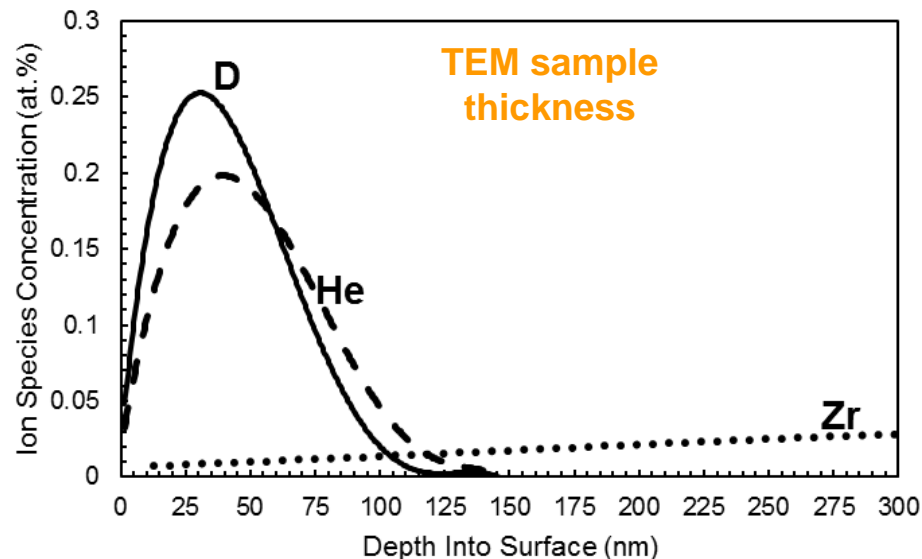
He bubbles in Zr-Nb alloy
Shen et al Mat Char 107 (2015) 309-316



TEM of Zr tritide after 325d
Schober et al JNM 141-143 (1986) 453-457

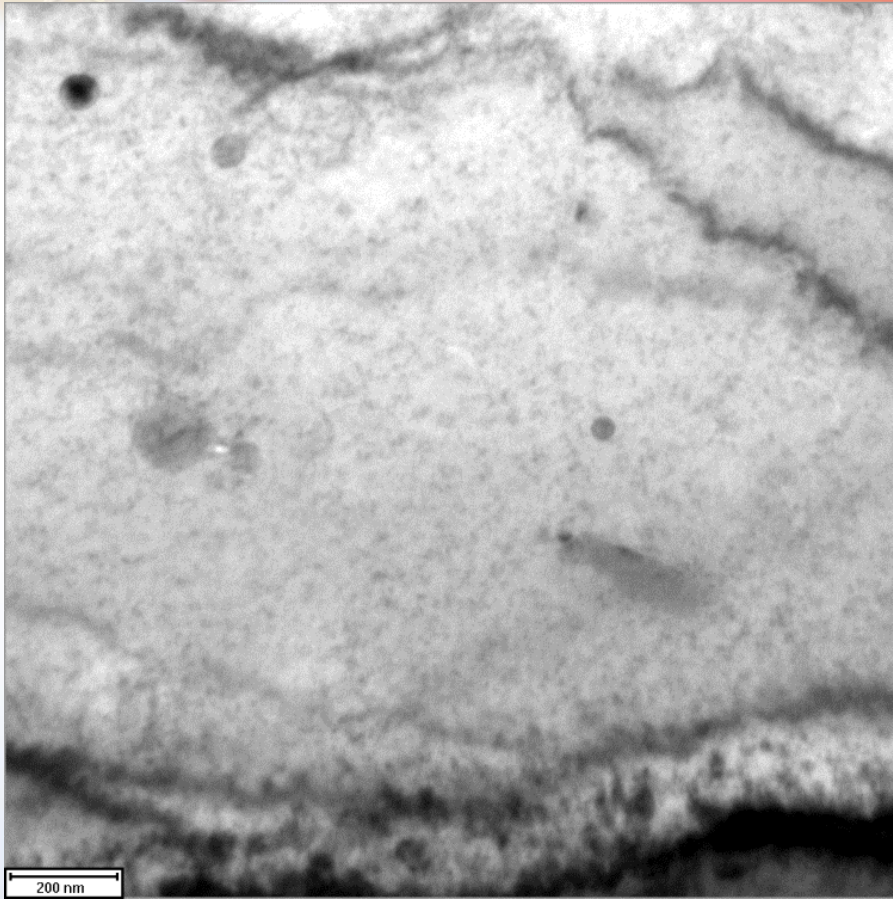
Zr alloy in-situ ion irradiation parameters

- Samples were prepared by electropolishing zirconium alloy samples (mostly ZIRLO)
- Several sets of irradiations done at 310°C, including:
 - 10 keV He → simulates He accumulation from ${}^6\text{Li}$ transmutation and ${}^3\text{H}$ decay
 - 10 keV He + 5 keV D + 3 MeV Zr → simulates gas build-up + displacement cascades
- SRIM, a Monte Carlo based program for simulating the number of displacements produced by an ion, was used to predict damage dose and concentration profiles.
- These preliminary experiments were run overnight and the exact gas concentrations/damage doses are not all known

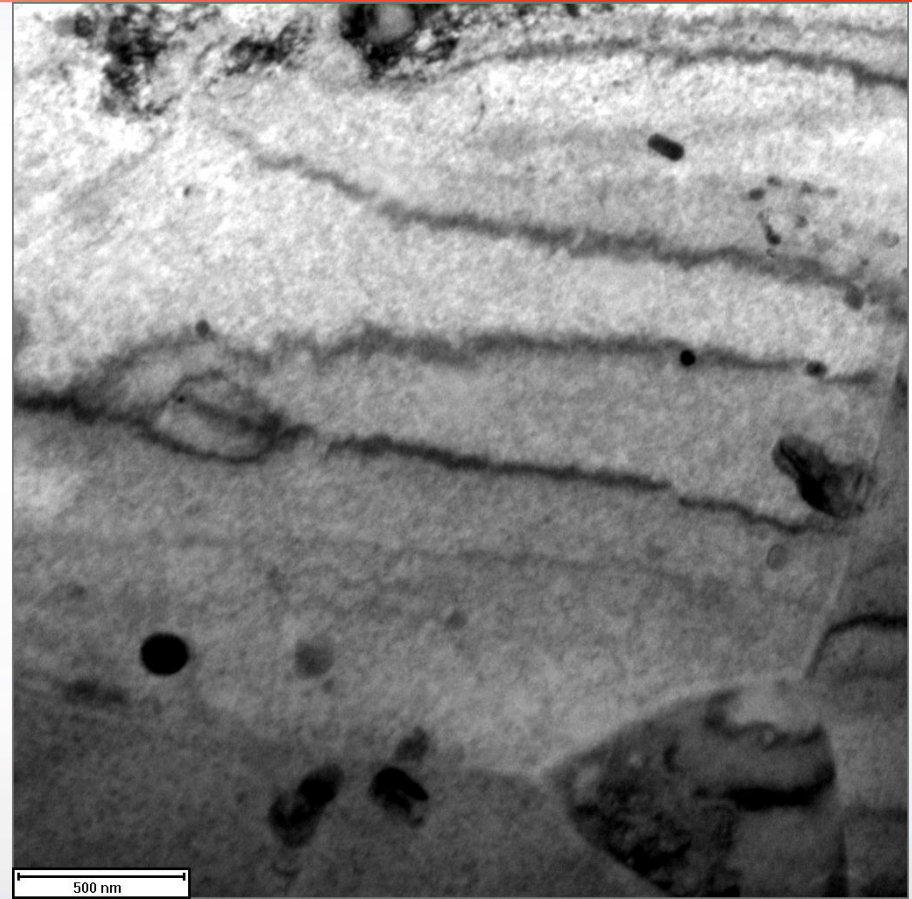


He and D profiles are implanted within the TEM sample, while most Zr passes through the sample, leaving only cascade damage.

10 keV He⁺ Implantation at 310°C



**After Implantation.
Damage, No Cavities.**



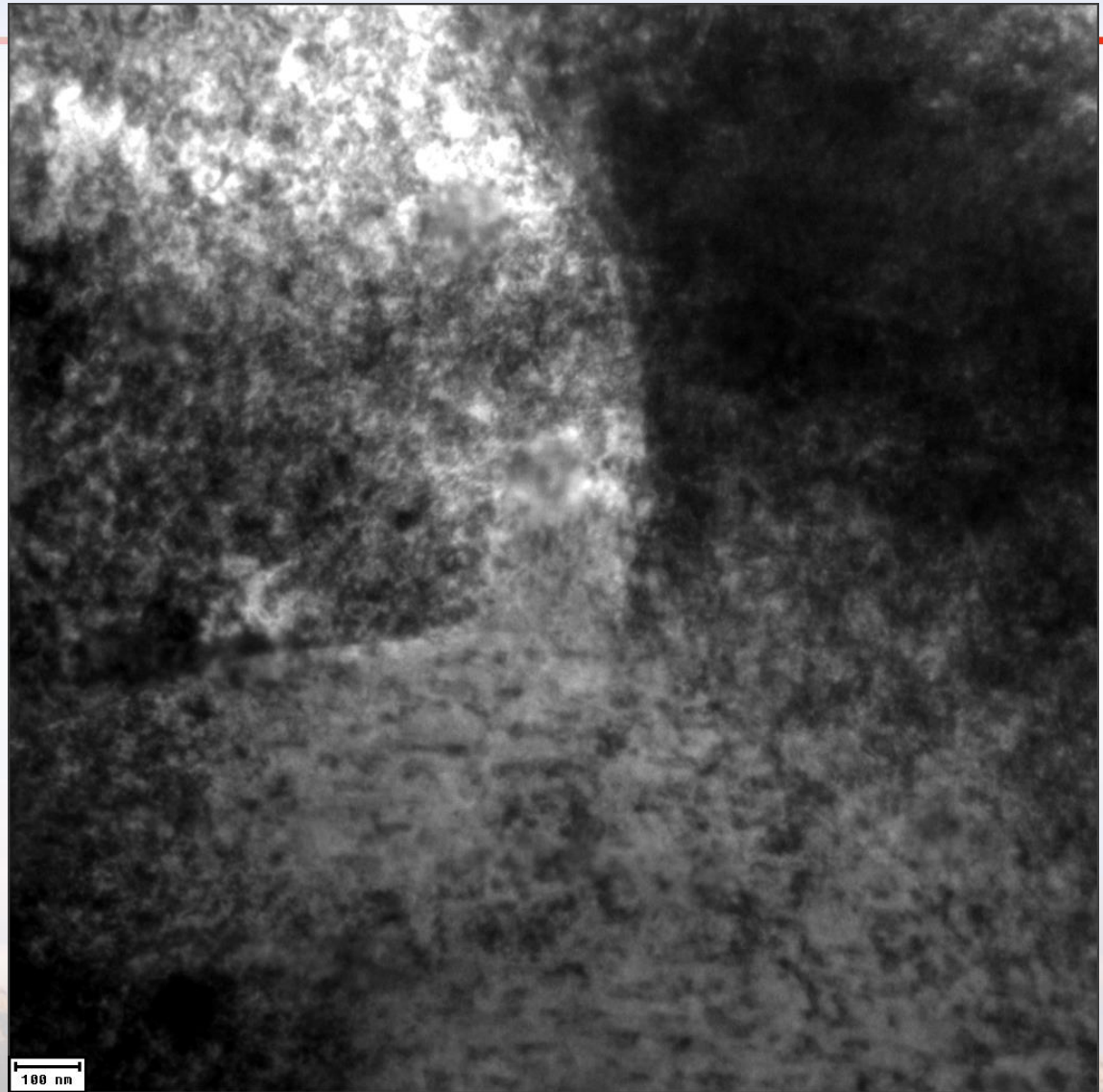
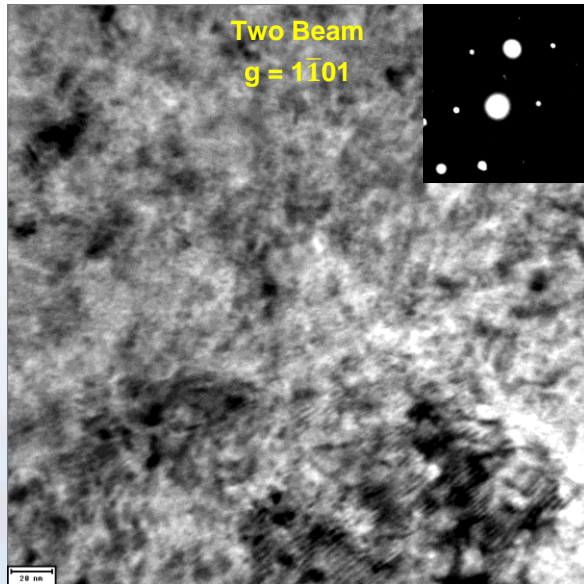
**Still no cavities after subsequent
irradiation with 3 MeV Zr.**



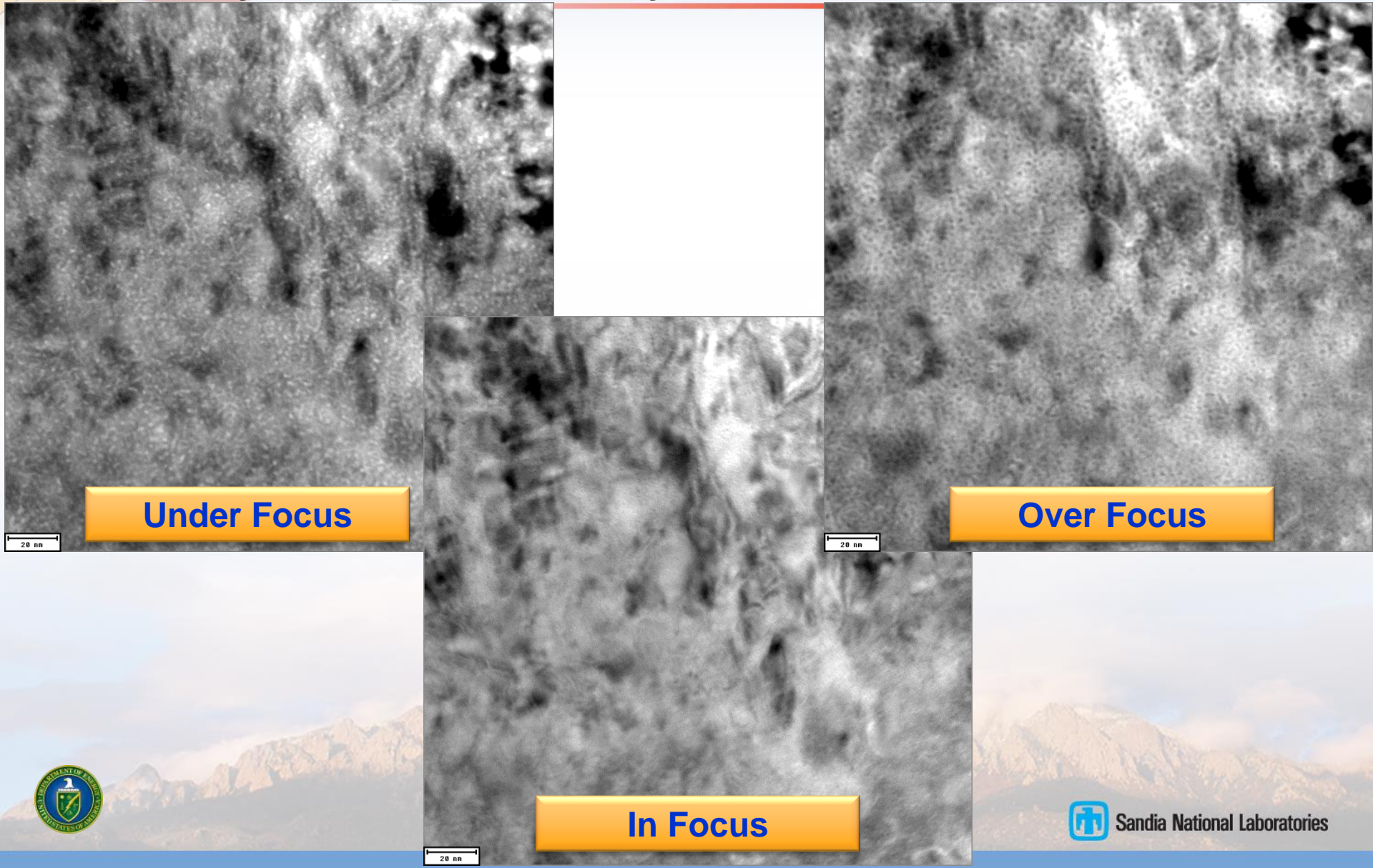
Concurrent D & He Implantation & Zr Irradiation

After triple beam irradiation

- Very dense, complex defect structure
- No visible cavities
- Fuzzy defects difficult to characterize



Cavities were observed in He implanted Zr alloy samples 30 days after irradiation



Under Focus

Over Focus

In Focus





Summary

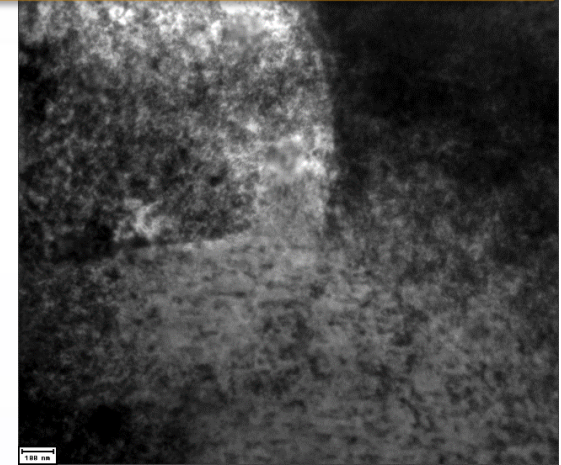
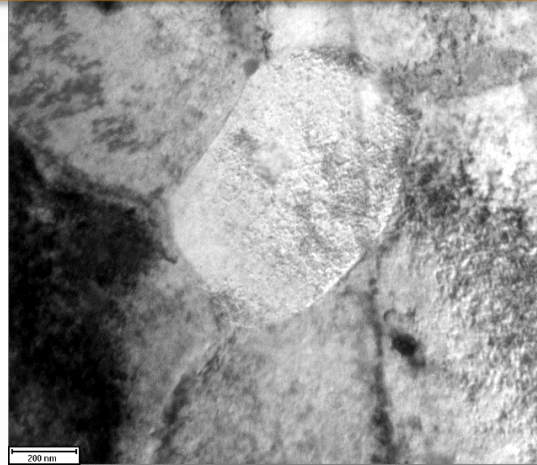
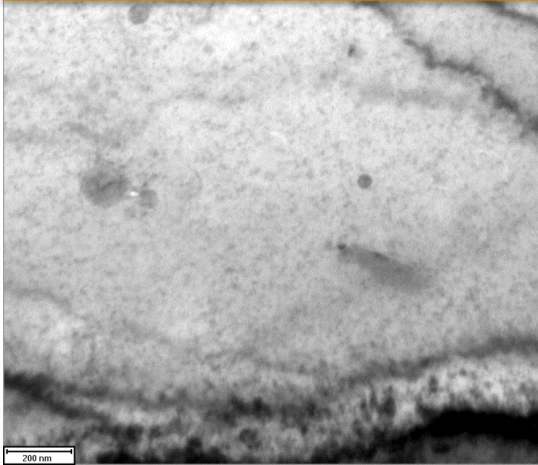


Zr Alloys

He accumulation →

Damage + He →

Damage + He + ^3H

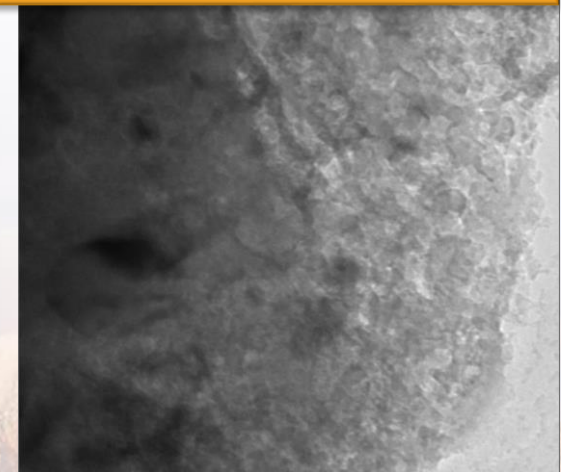
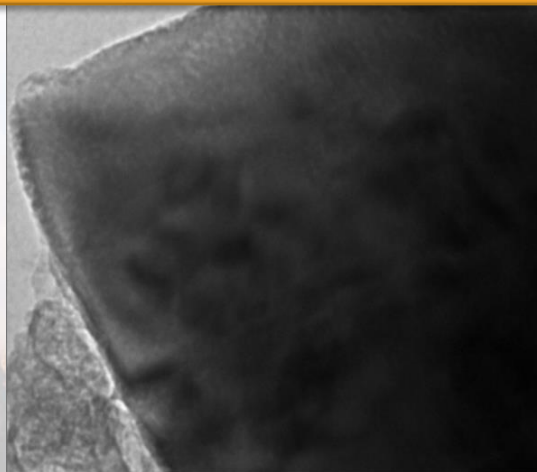
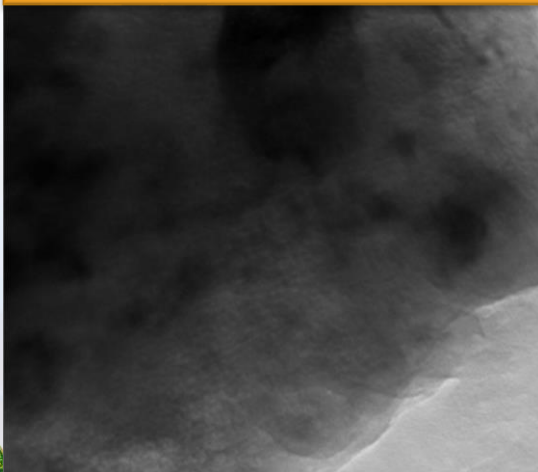


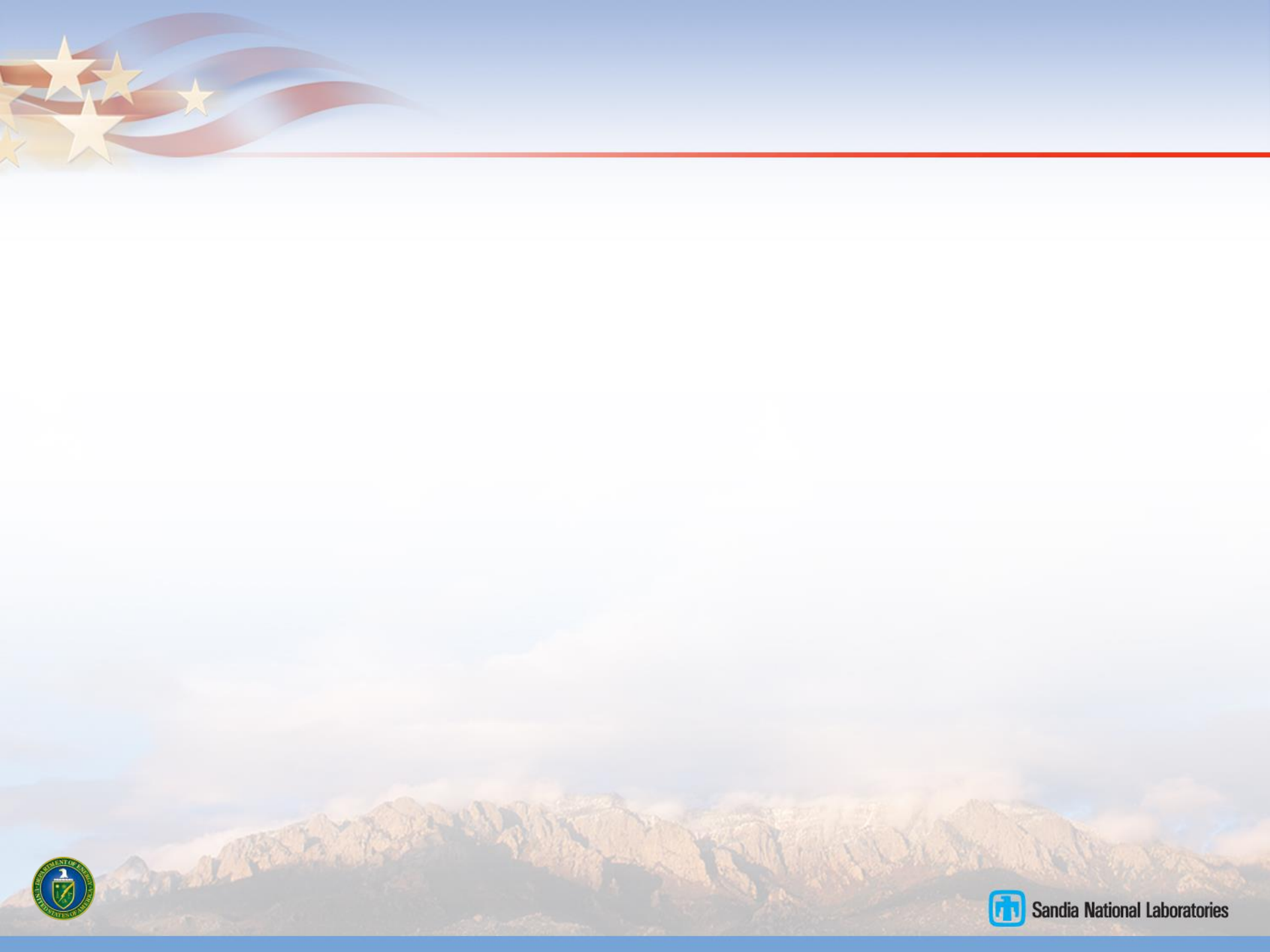
LiAlO₂

He accumulation →

^3H + He →

Damage + He + ^3H

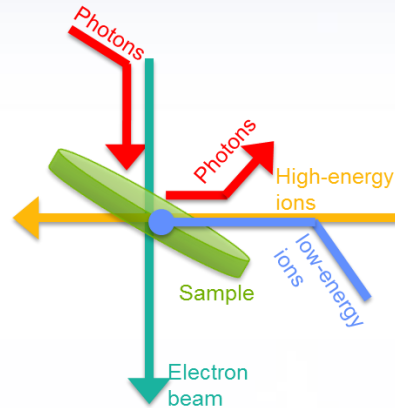




Summary

Sandia's I³TEM capabilities:

- *In situ* high energy ion irradiation from H to Au
- *In situ* gas implantation
- Heating up to 1,000 °C
- Quantitative and bulk straining
- Two-port microfluidic cell
- Gas flow/heating stage
- Electron tomography
- Precession Electron Diffraction



Currently applying the current I³TEM capabilities to various material systems in combined and harsh environmental conditions

Sandia's I³TEM future capabilities being developed:

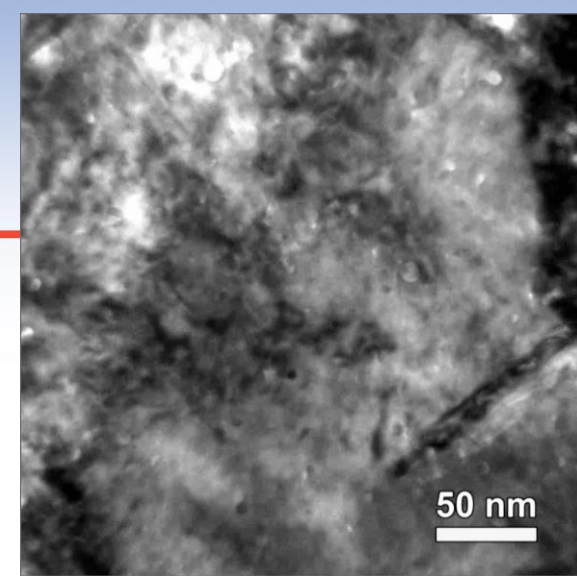
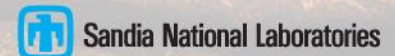
- *In situ* ion irradiation TEM in liquid or gas (currently capable)
- DTEM: Nanosecond resolution (laser optics being developed)
- Beamline: Add 1 MV NEC Tandem & convert 90° magnet to bend beams 45°

Collaborators:

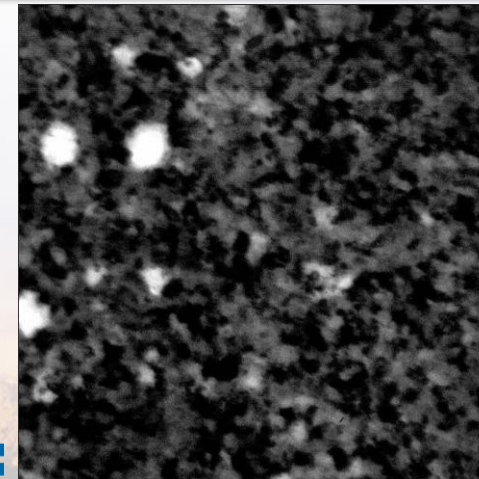
- IBL: D.C. Bufford, D. Buller, C. Chisholm, B.G. Clark, J. Villone, B.L. Doyle, S. H. Pratt, M. Steckbeck & M.T. Marshall
- Sandia: B. Boyce, T.J. Boyle, P.J. Cappillino, J.A. Scott, B.W. Jacobs, M.A. Hekmaty, D.B. Robinson, J.A. Sharon, W.M. Mook, F. Abdeljawad, & S.M. Foiles
- External: A. Minor, L.R. Parent, I. Arslan, H. Bei, E.P. George, P. Hosemann, D. Gross, J. Kacher, & I.M. Robertson

This work was partially supported by the US Department of Energy, Office of Basic Energy Sciences.

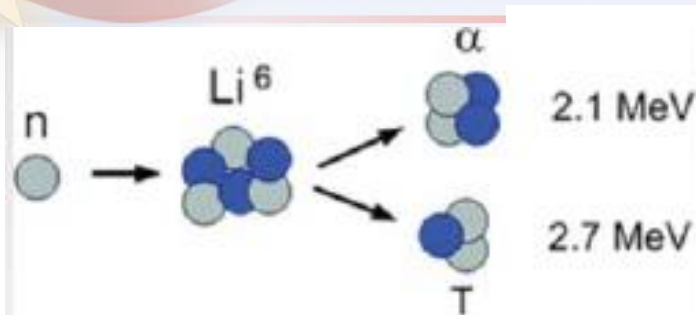
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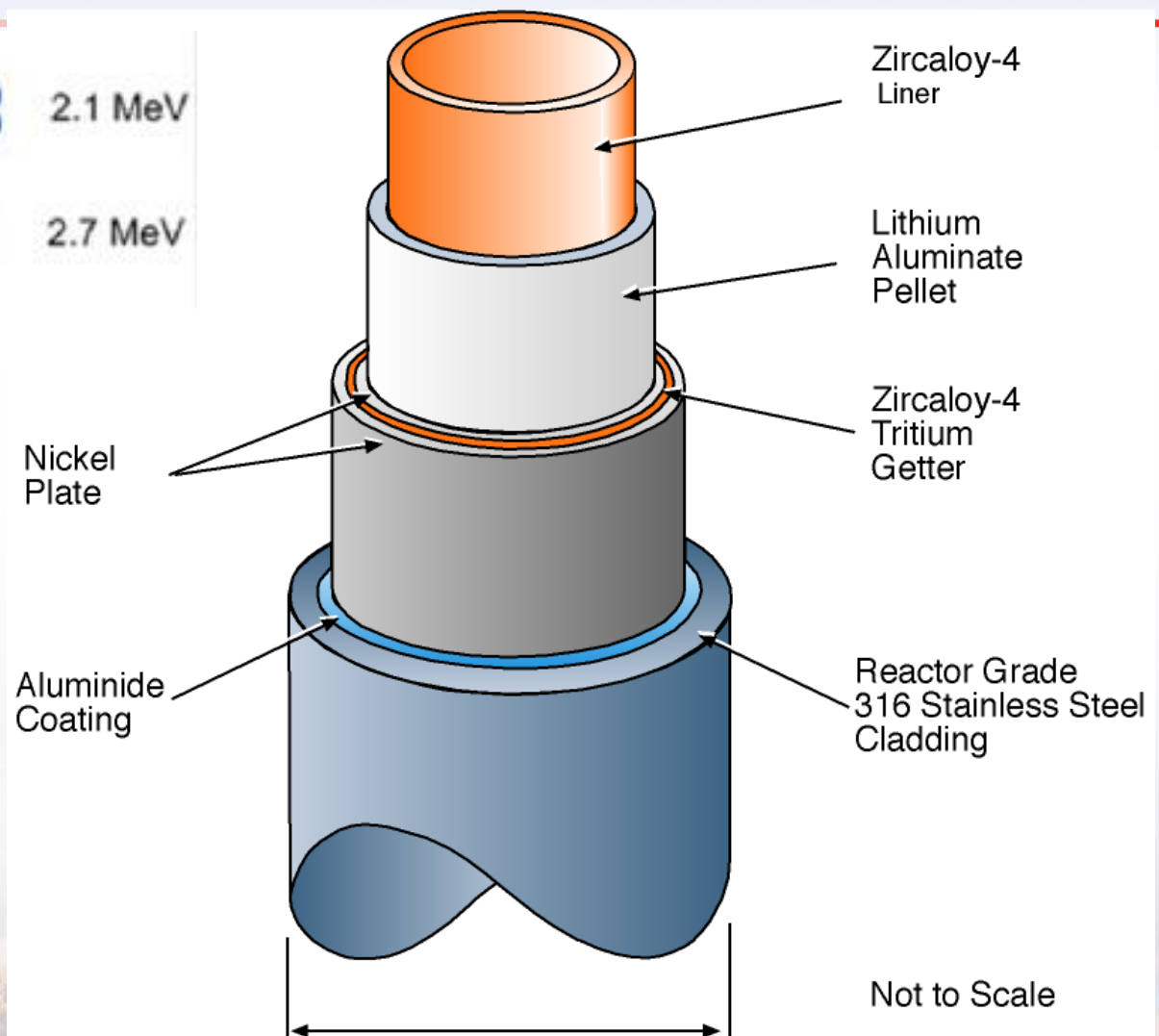
Sandia's I³TEM although still under development is providing a wealth of interesting initial observations and harsh environments



Tritium Producing Burnable Absorber Rod



- Displacement Damage
- Helium Implantation
- Tritium Implantation
- Elevated Temperatures



Investigating the **nm** Scale to Understand the **km** Scale



1 nm

1 μ m



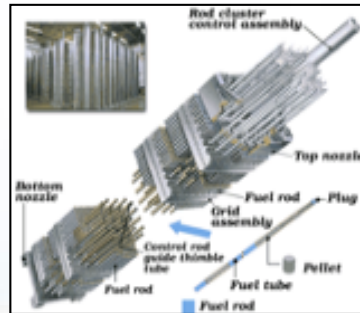
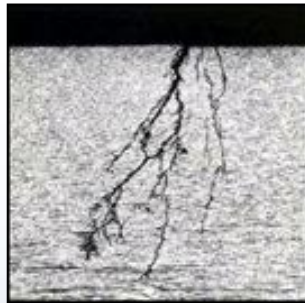
1 mm

1 m



1 km

10⁷ m



In situ Ion Irradiation TEM (I³TEM)



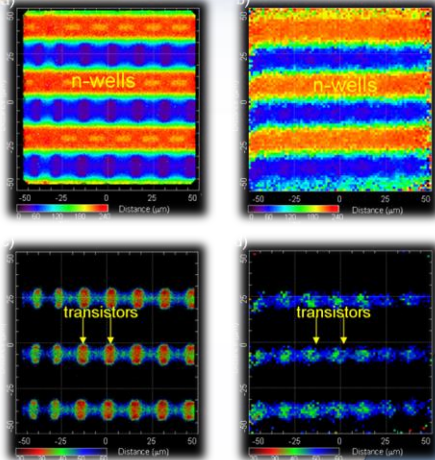
Ion Beam Lab (IBL)



Sandia National Laboratories



Sandia's Ion Beam Laboratory



Ion Beam Analysis (IBA)

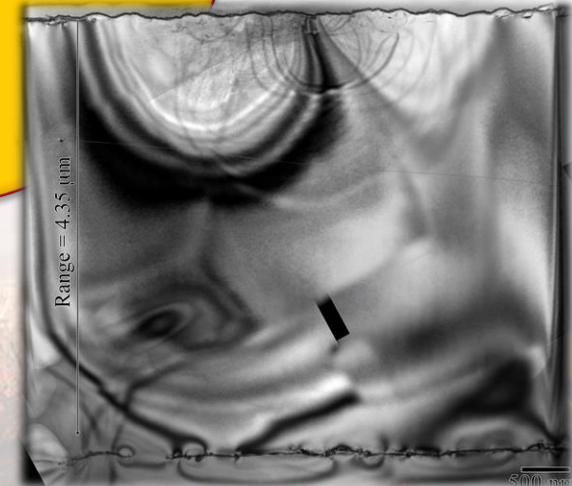
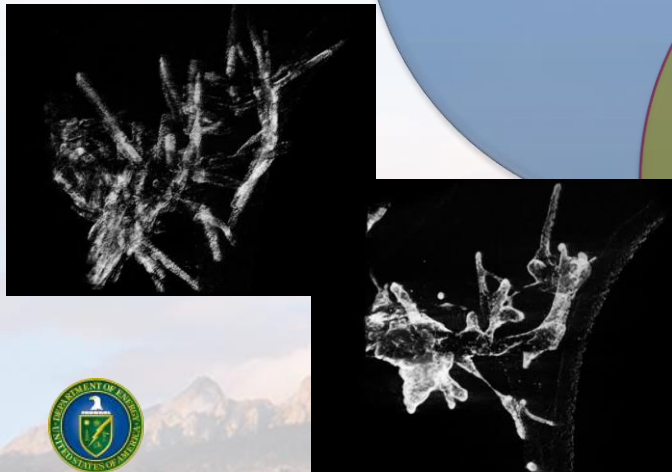
TECHNIQUE	SCHEMATIC	EXAMPLE ANALYSIS SPECTRUM
		Target (Ino., Det.) Residual
RBS		
ERD		
HIBS		
NRA		
PIXE		
SEU/IBICC		

Radiation Effects Microscopy (REM)



Ion Beam Modification (IBM)

In situ Ion Irradiation Transmission Electron Microscopy (I³TEM)

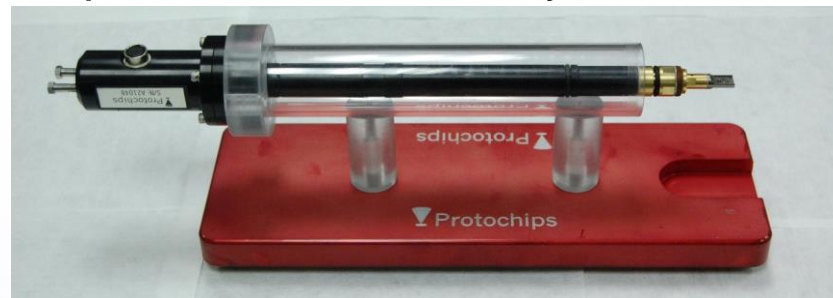


In situ TEM Hydrogen Exposure

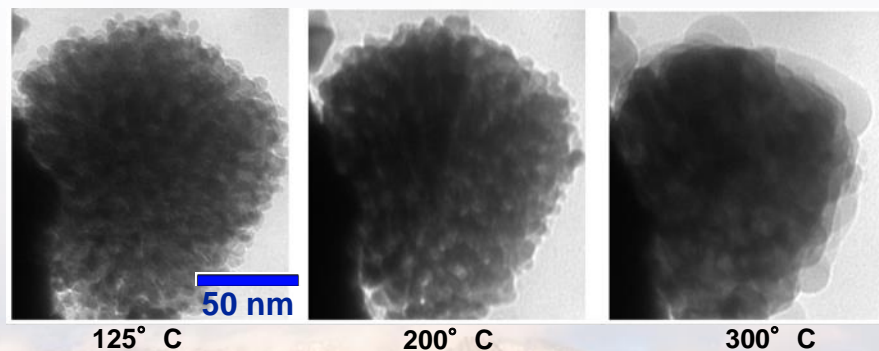
Contributors: B.G. Clark, P.J. Cappillino, B.W. Jacobs, M.A. Hekmaty, D.B. Robinson, L.R. Parent, I. Arslan. & Protochips, Inc.

Vapor-Phase Heating TEM Stage

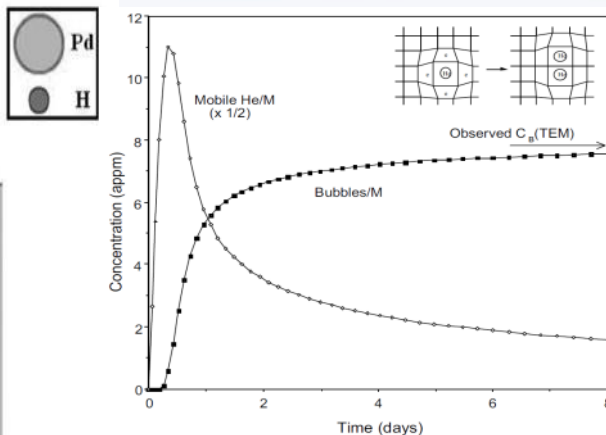
- Compatible with a range of gases
- *In situ* resistive heating
- Continuous observation of the reaction channel
- Chamber dimensions are controllable
- Compatible with MS and other analytical tools



- 1 atm H₂ after several pulses to specified temp.



New *in situ* atmospheric heating experiments provide great insight into nanoporous Pd stability



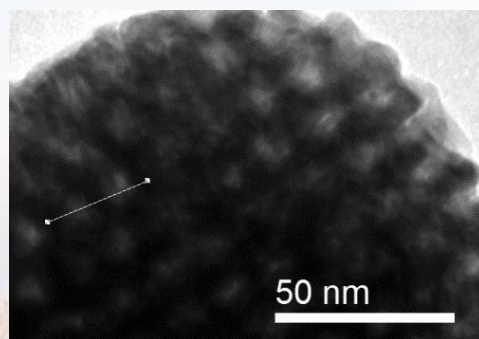
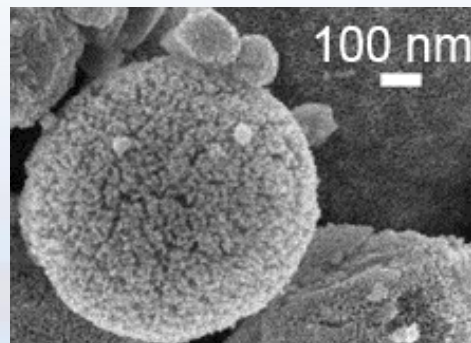
Cowgill, D., *Fusion Sci. & Tech.*, 28 (2005) p. 539

Trinkaas, H. *et al.*, *JNM* (2003) p. 229

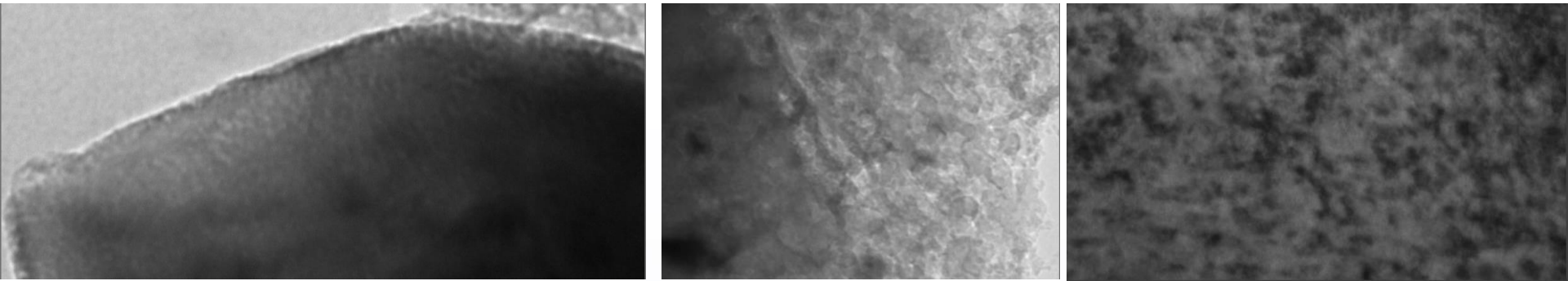
Thiebaut, S. *et al.* *JNM* (2000) p. 217

R. Delmelle, J., *Phys. Chem. Chem. Phys.* (2011) p.11412

Harmful effects may be mitigated in nanoporous Pd



Simulations of Damage and Gas Accumulation in TPBAR Materials with In-situ Triple Ion Beam Irradiation TEM



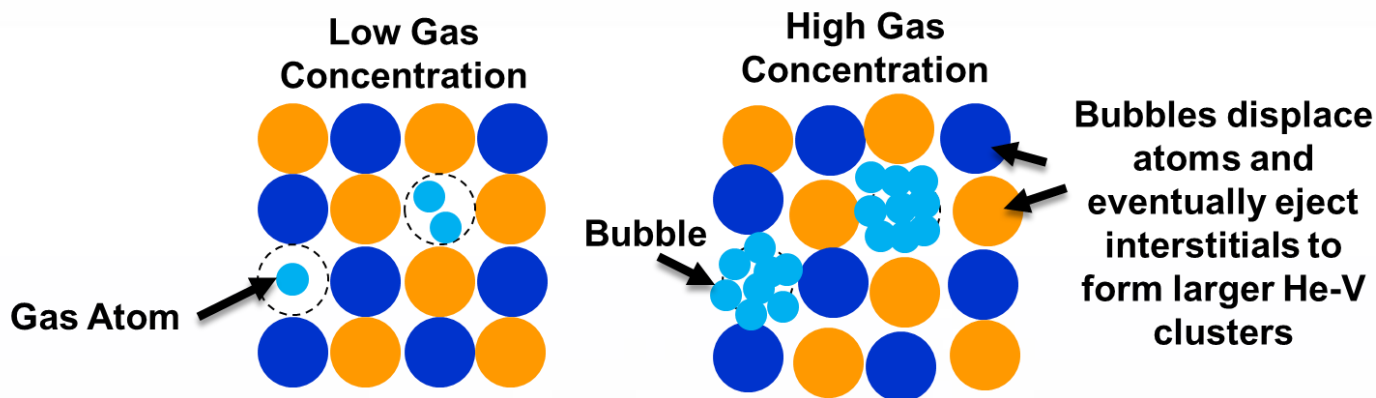
**Caitlin A. Taylor, Brittany Muntifering, David Senior, Clark Snow, and
Khalid Hattar**

September 6th, 2017



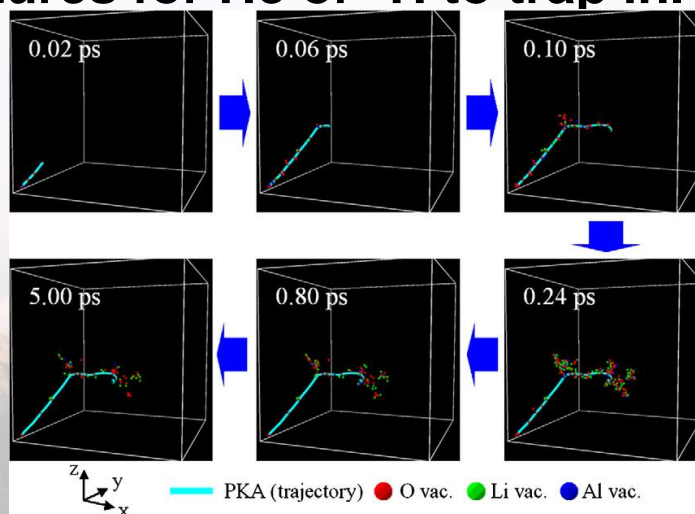
Bubbles May Affect ^3H Release

- Bubbles form due to He trapping in lattice defects



- Neutron irradiation produces displacement cascades, providing complex defect structures for He or ^3H to trap in.

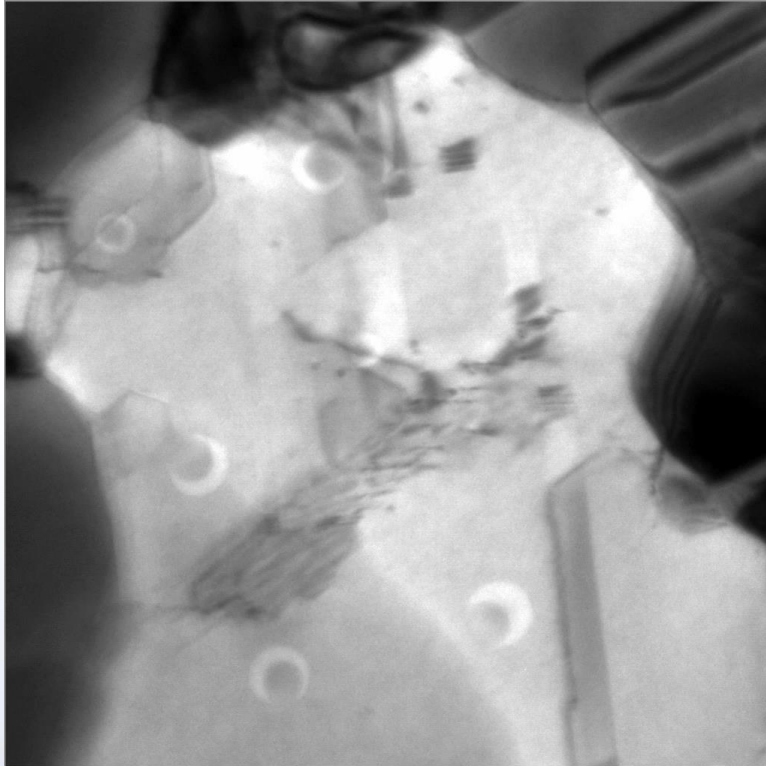
MD simulation of displacement cascade in LiAlO_2 (PKA = 5 keV)



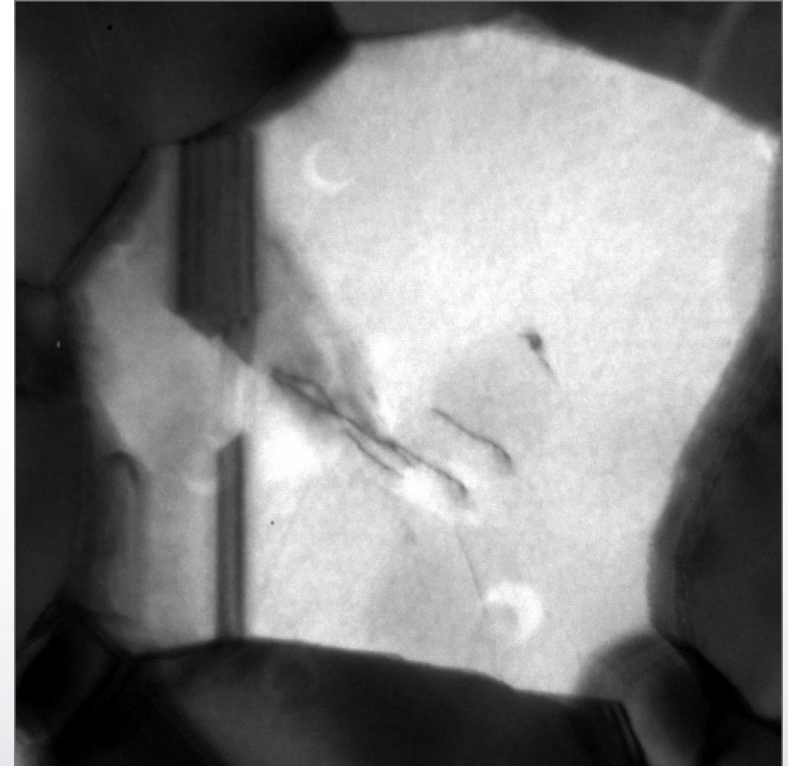
Dose Rate Effects

Collaborators: C. Chisholm , P. Hosemann, & A. Minor

7.9×10^9 ions/cm²/s



6.7×10^7 ions/cm²/s



VS



Improved vibrational and ion beam stability permits us to work at 120kx or higher permitting imaging of single cascade events



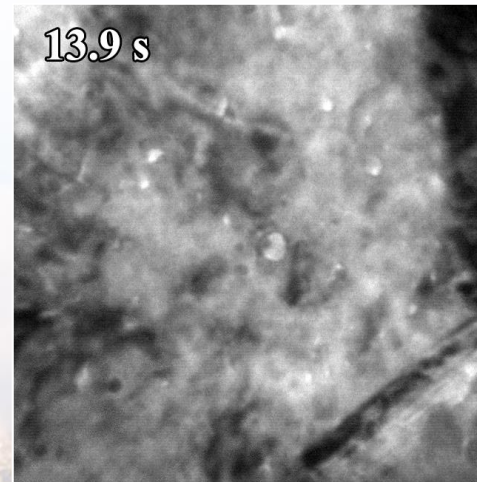
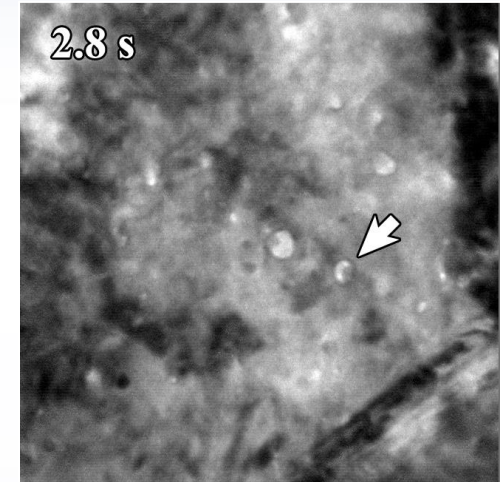
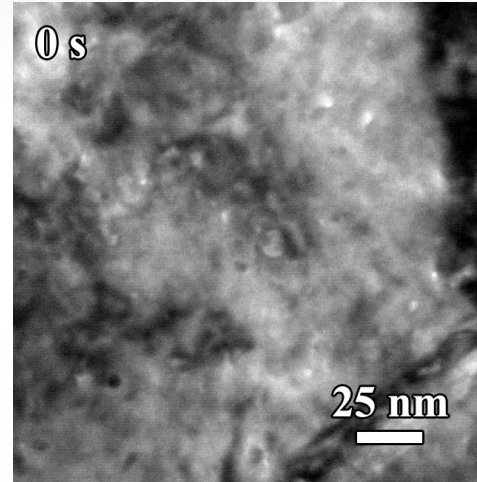
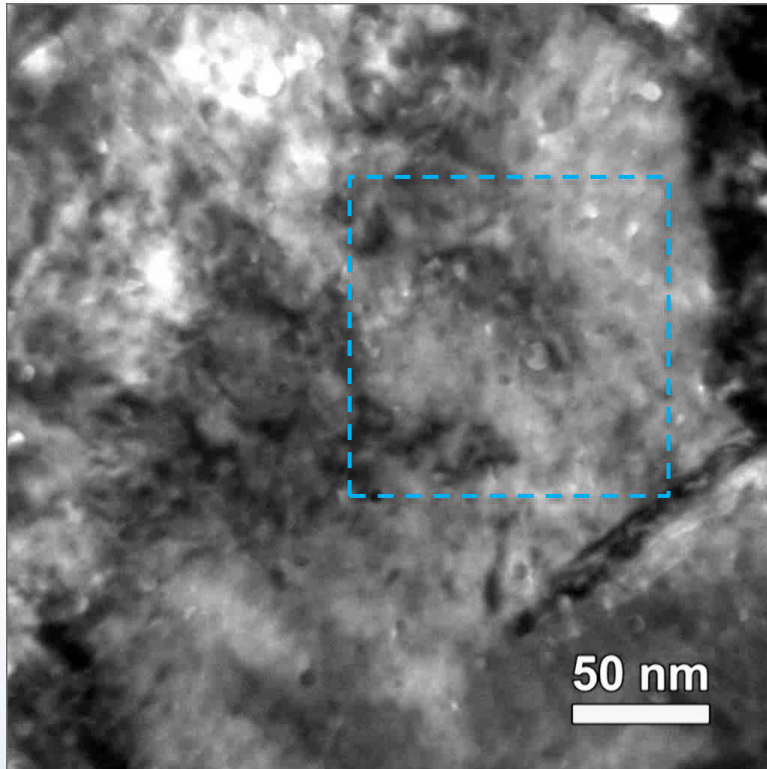
Sandia National Laboratories

Simultaneous *In situ* TEM Triple Beam:

2.8 MeV Au⁴⁺ + 10 keV He⁺ / D₂⁺

Collaborator: D.C. Bufford

Video playback speed x1.5.



In-situ triple beam He, D₂, and Au beam irradiation has been demonstrated on Sandia's I³TEM!

Intensive work is still needed to understand the defect structure evolution that has been observed.

■ Approximate fluence:

- Au 1.2×10^{13} ions/cm²
- He 1.3×10^{15} ions/cm²
- D 2.2×10^{15} ions/cm²

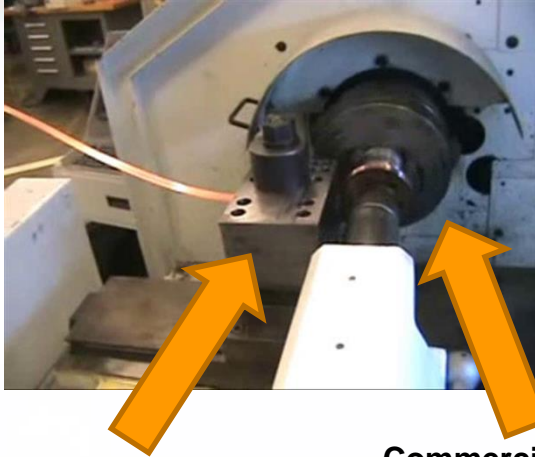
■ Cavity nucleation and disappearance



Sandia National Laboratories

What Insight into Structural Stability is Gained from I³TEM Experiments?

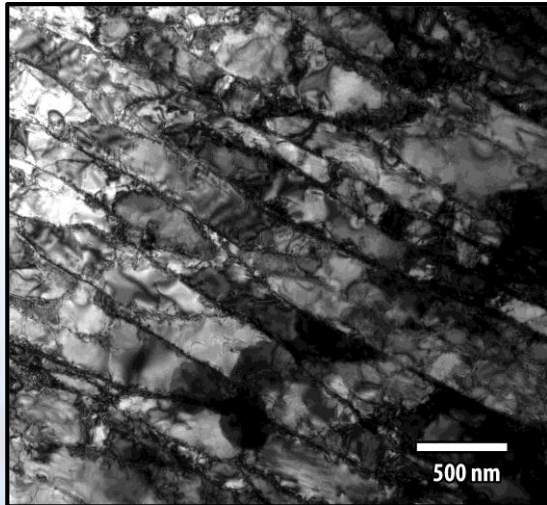
Collaborators: O. El-Atwani, J. P. Allain, D. Buller, & J.A. Scott



Extrusion
Machining tooling

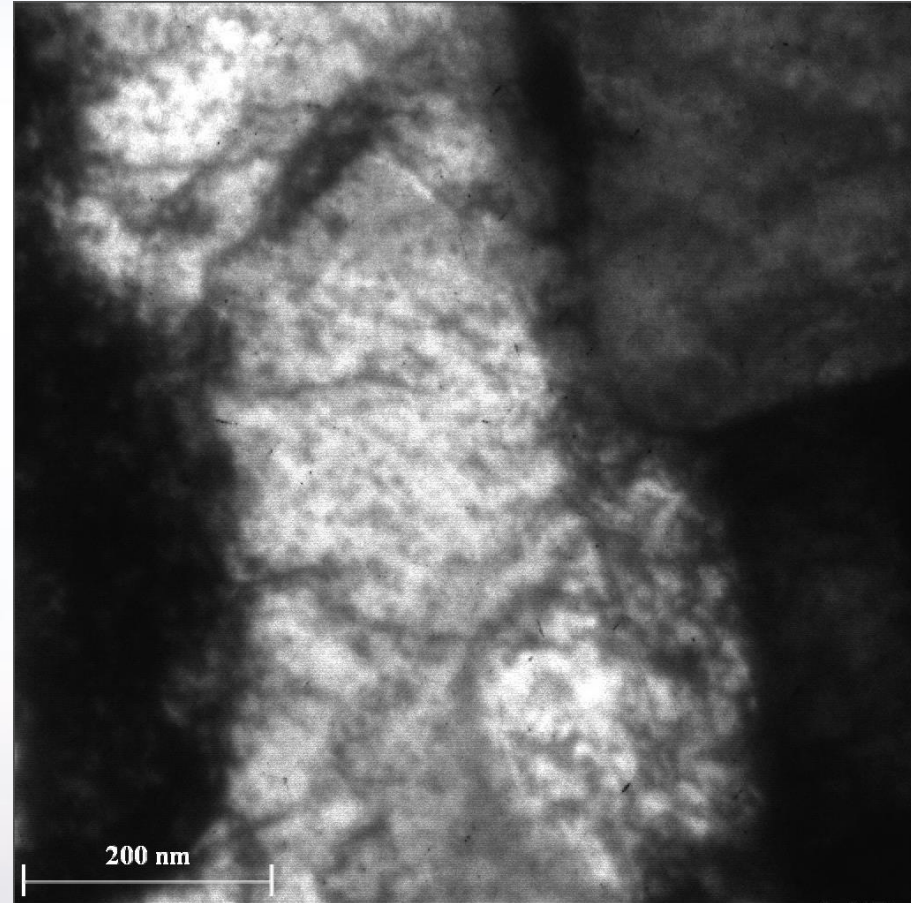
Commercially
available lathe

From NW
components through
proposed NE
cladding to waste
storage:
Understanding
Radiation Damage is
Essential



UFG Tungsten

- I³TEM W irradiation and He implantation of SPD-W developed for ITER applications



I³TEM is providing insight into:

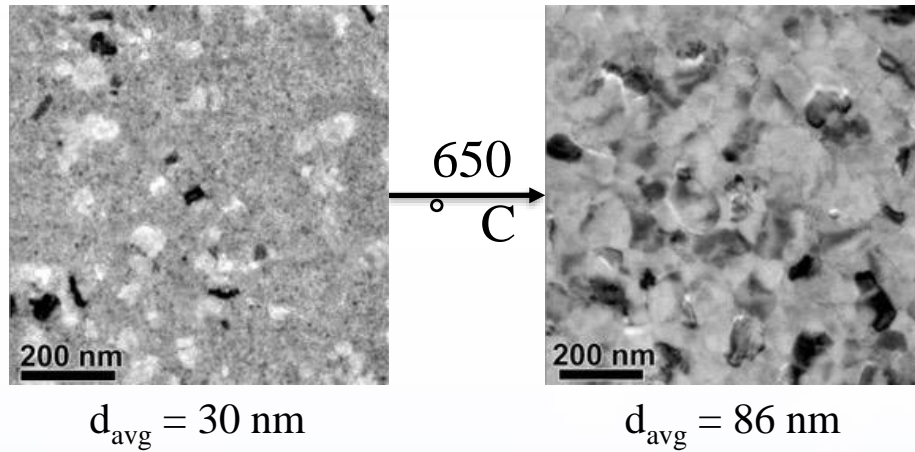
- 1) Loop formation
- 2) Loop stability & migration
- 3) Rad & structural defect interactions



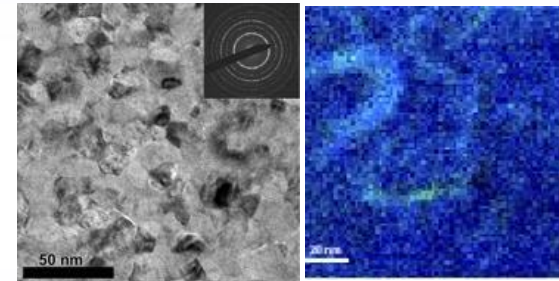
Scaling down to Nanocrystalline Tungsten Alloys

Collaborators: O.K. Donaldson, T. Kaub, G. Thompson, and J. Trelewicz

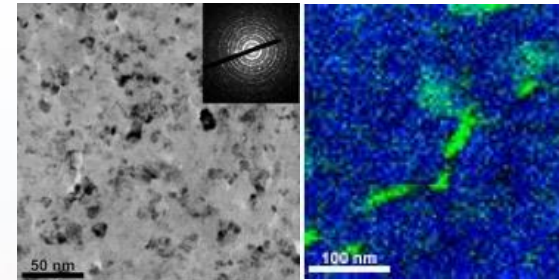
Nanocrystalline W



Nanocrystalline W-20at.%Ti

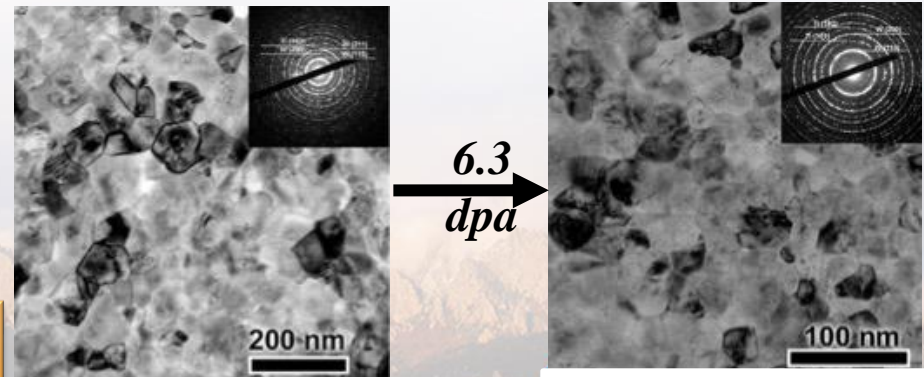
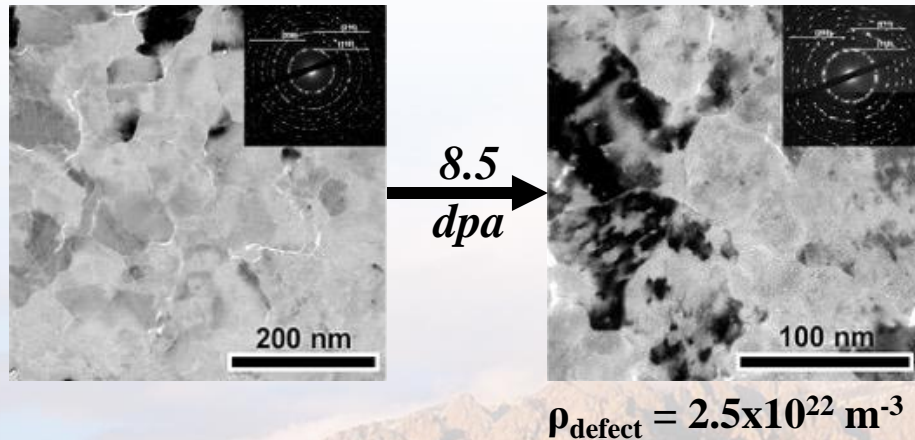


1000°C Anneal



Grain growth is hampered by the addition of Ti

Ti solute is heterogeneously distributed after annealing

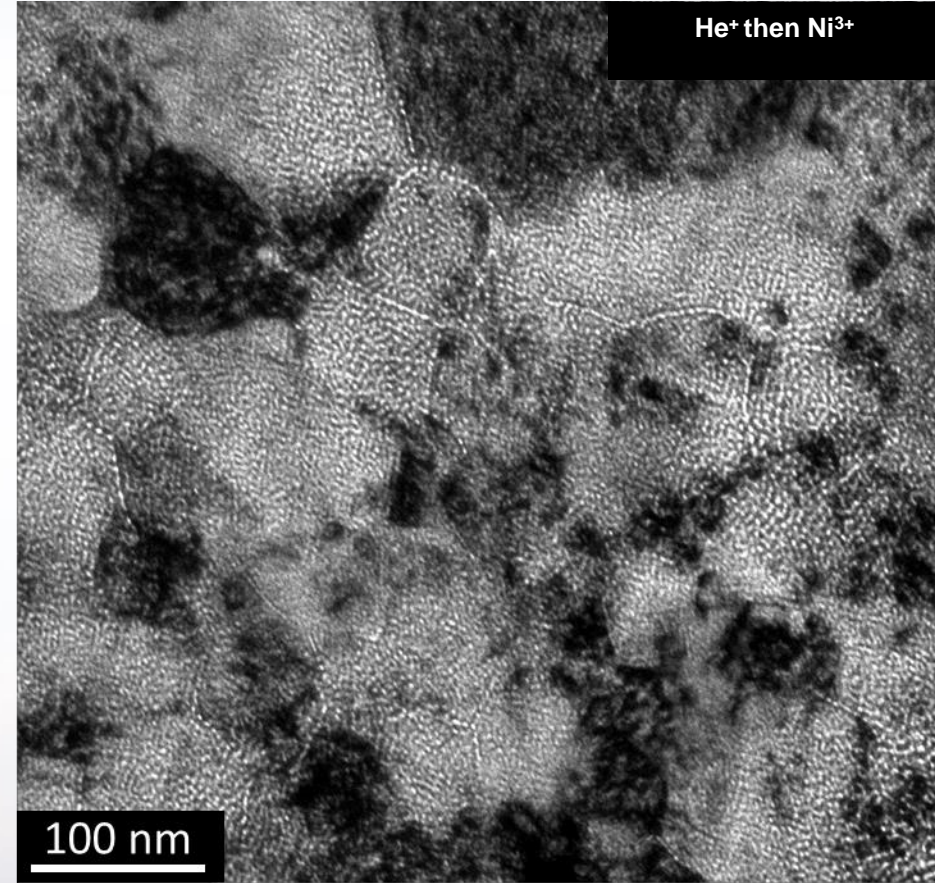
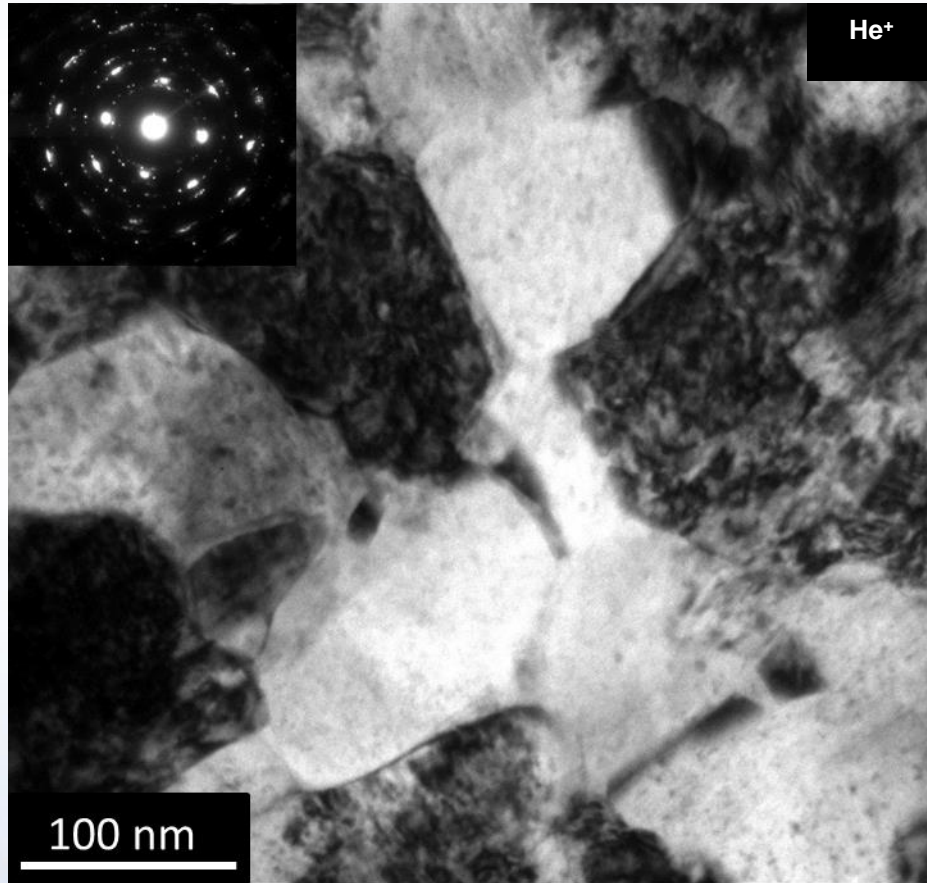


$\rho_{\text{defect}} = 2.8 \times 10^{22} \text{ m}^{-3}$

Alloying does not negatively effect radiation tolerance, while improving thermal and mechanical properties

10 keV He⁺ Implantation followed by 3 MeV Ni³⁺ Irradiation

Collaborator: B. Muntifering & J. Qu



10¹⁷ He⁺/cm²

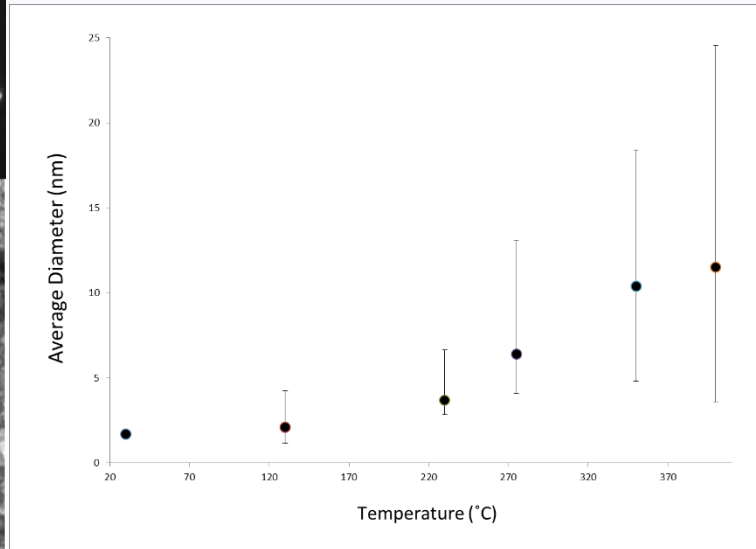
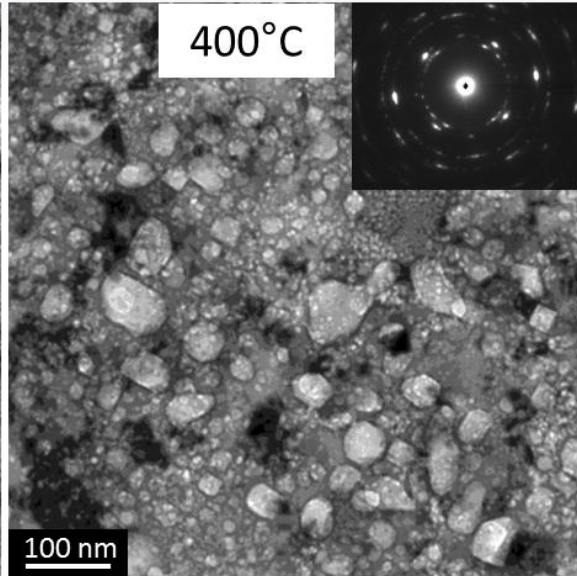
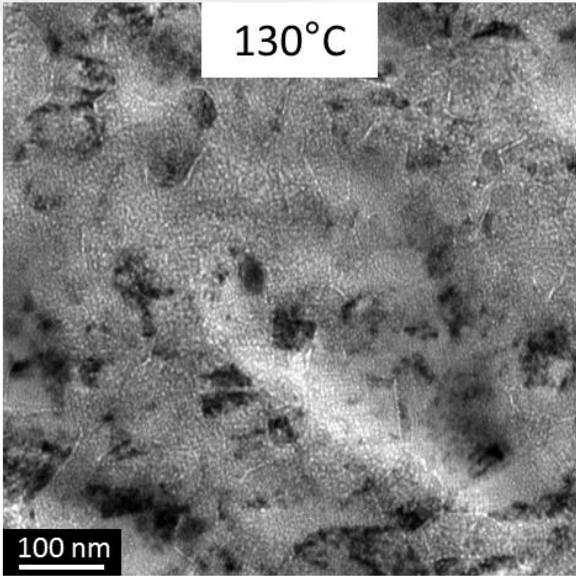
Visible damage to the sample

0.7 dpa Ni³⁺ irradiation

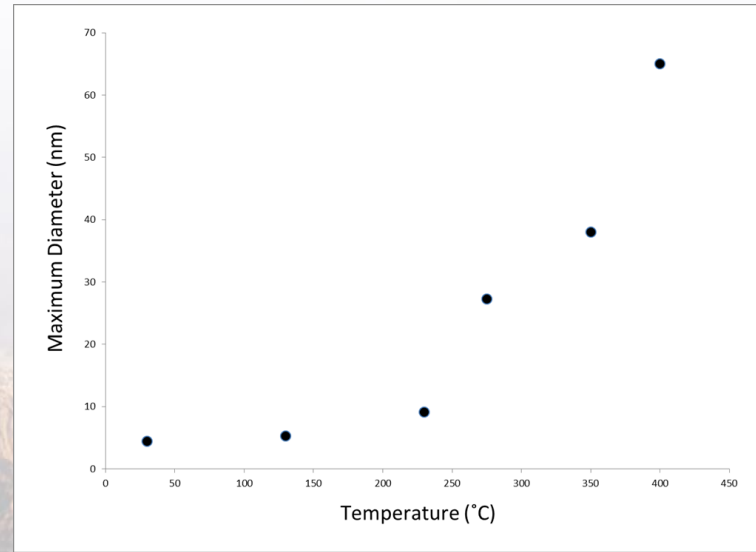
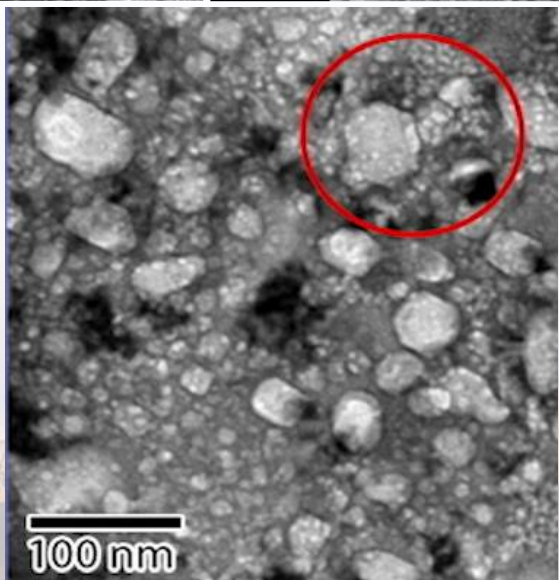
High concentration of cavities along
grain boundaries



Cavity Growth during In-situ Annealing of 10 keV He⁺ Implanted and then 3 MeV Irradiated Ni³⁺

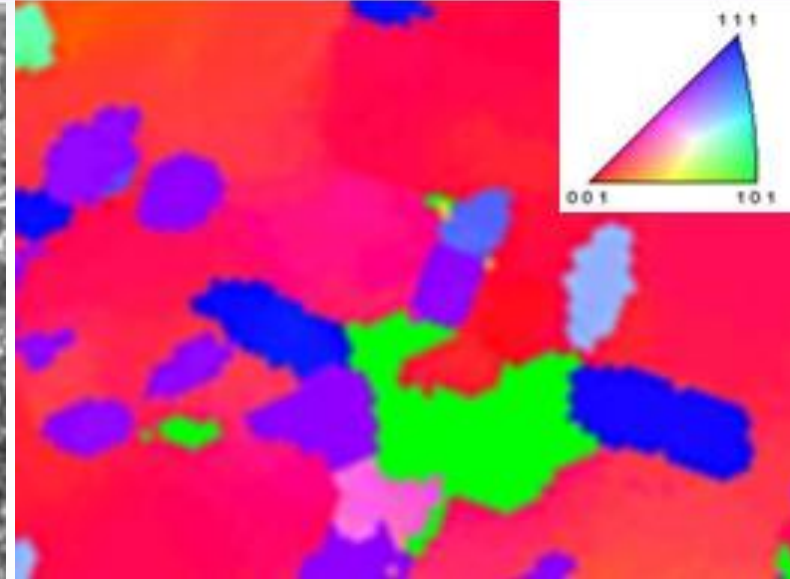
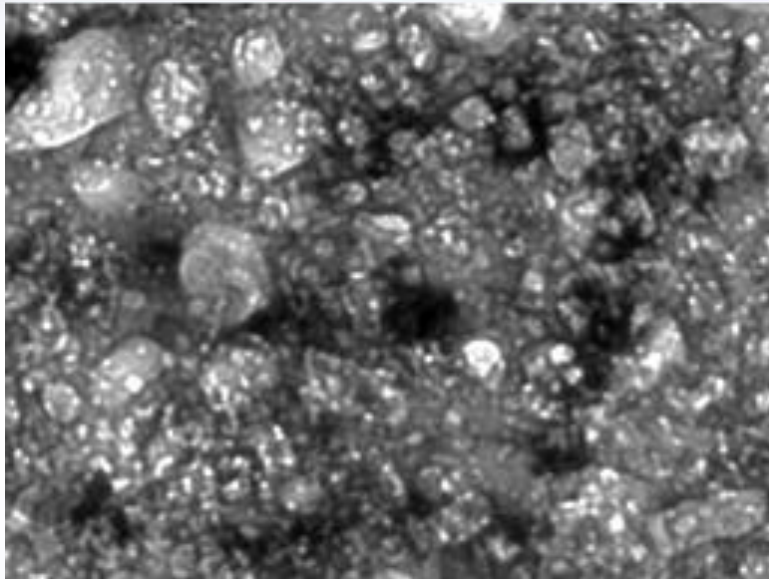


Bubble to cavity transition and cavity evolution can be directly studied

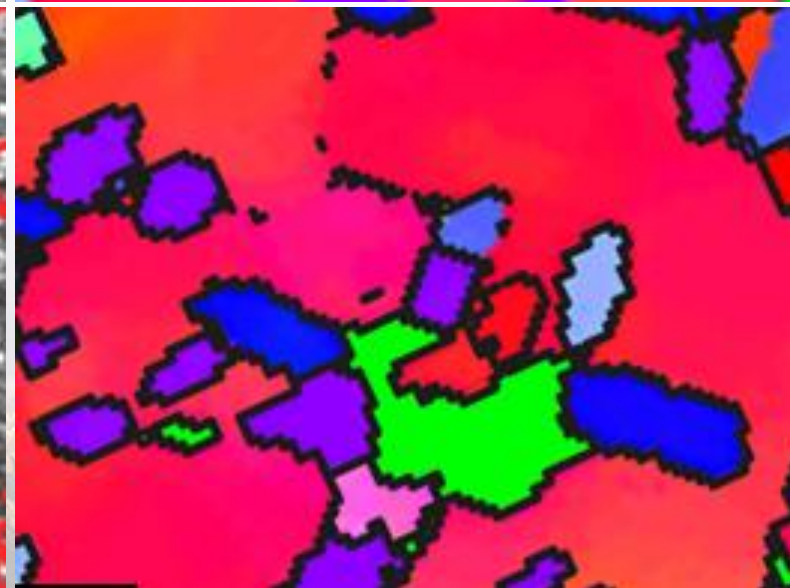
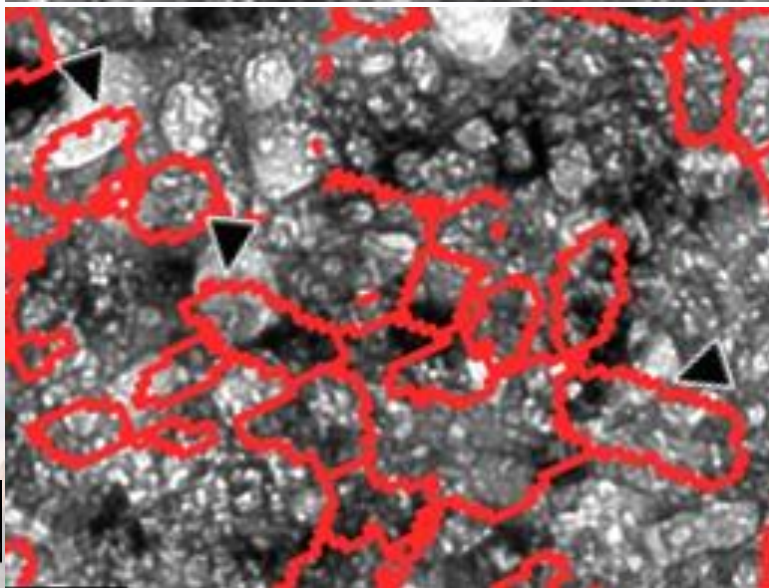


Precession Electron Diffraction Reveals Hidden Grain Structure

Cavities in helium implanted, self-ion irradiated, nc nickel film annealed to 400 °C



Cavities span multiple grains at identified grain boundaries

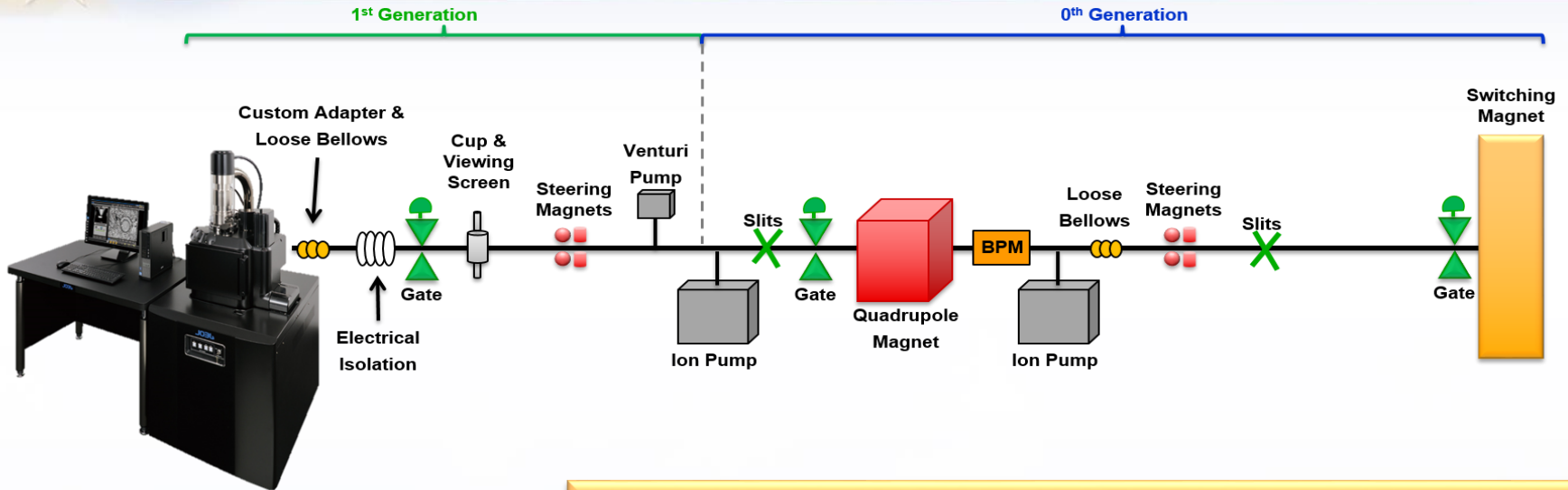


100 nm

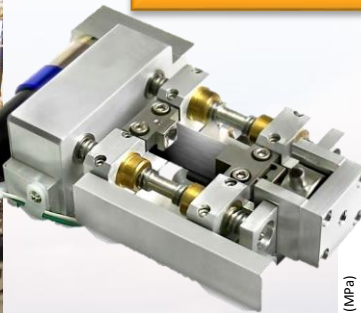
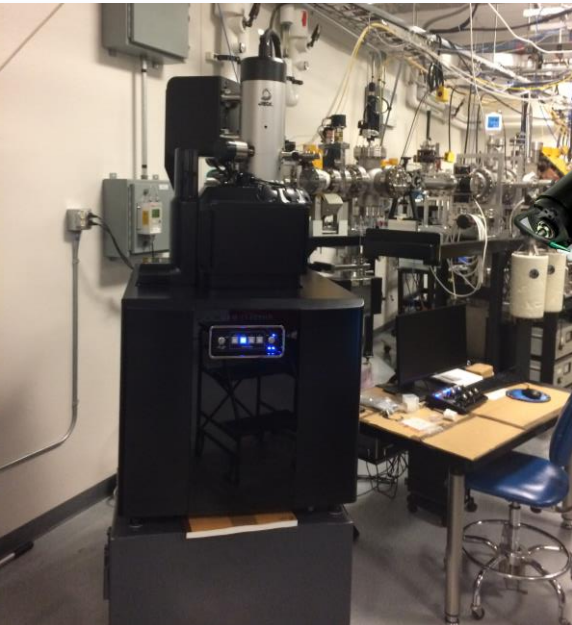
Schematic of the *In situ* SEM Beamline

Collaborators: D.L. Buller & S. Briggs

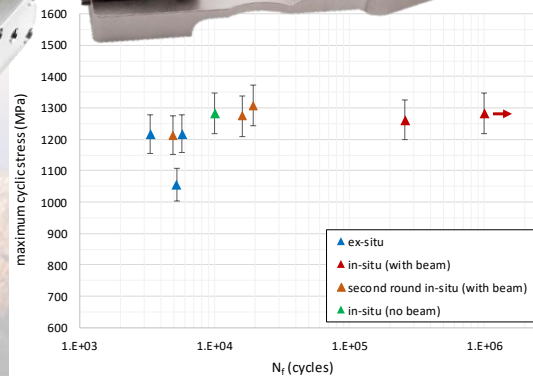
8/24/2017



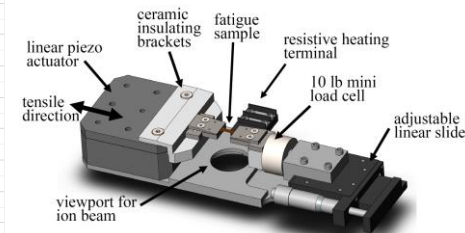
Beam Line planned for the *in situ* SEM will be developed in phases. Ultimate plan is for multiple accelerators being attached for dual or triple beam experiments.

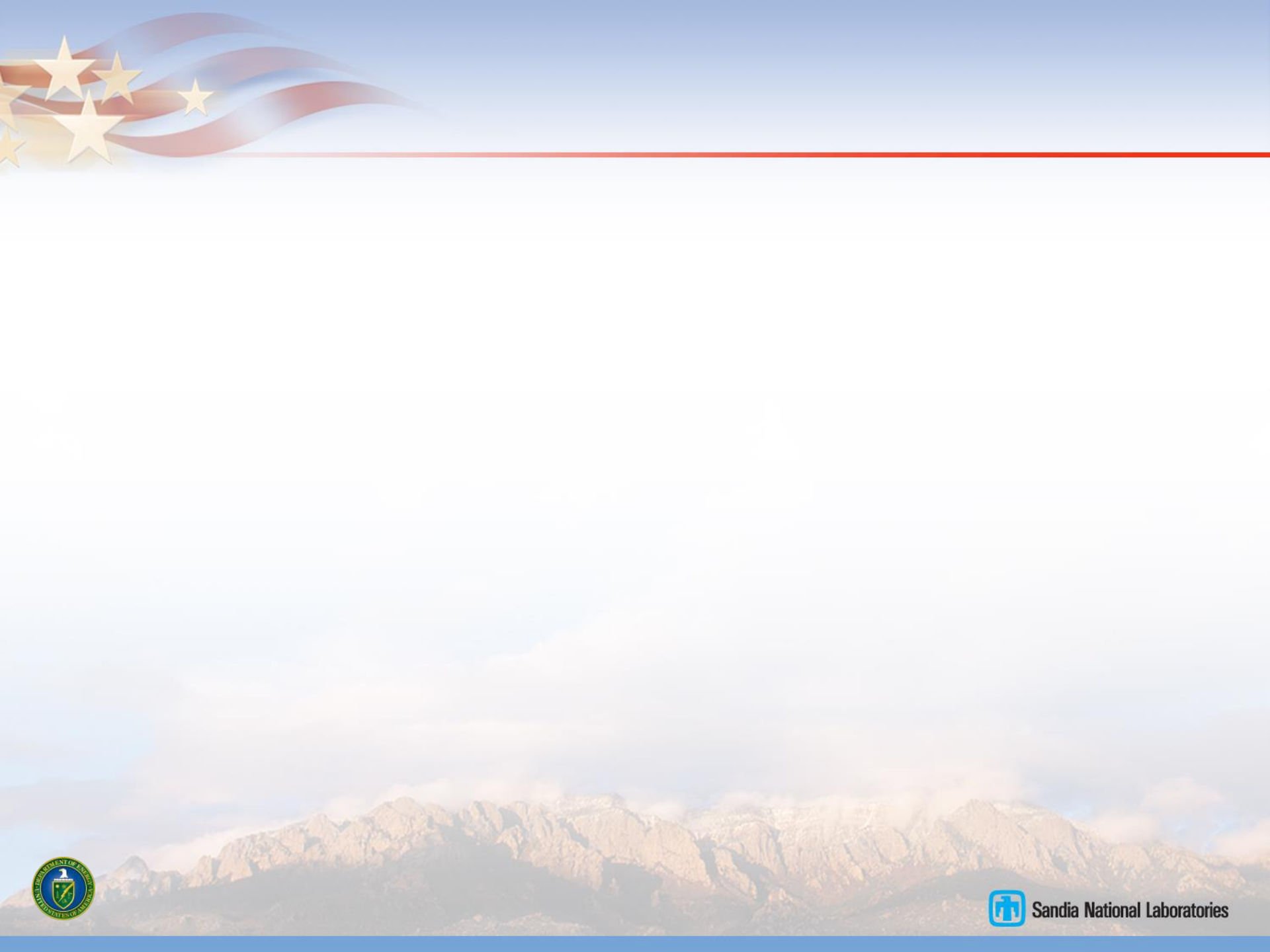


Hysitron PI85 Nanoindenter



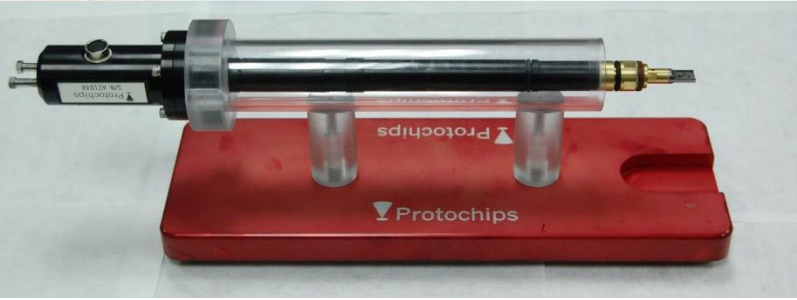
Custom-built Piezo Fatigue tester





Feasibility of Studying Zircaloy 2 at Nominally 1 atm

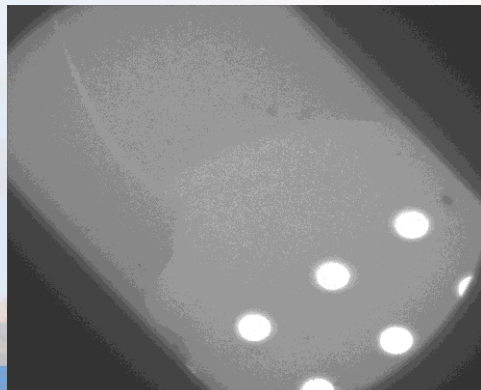
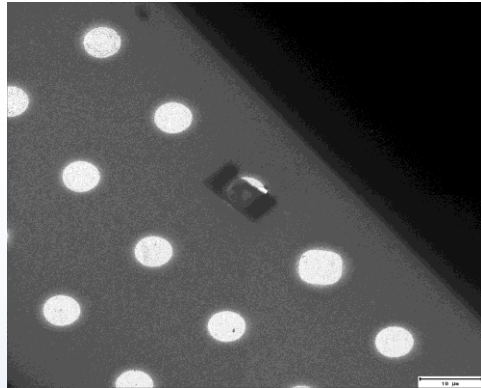
Collaborators: S. Rajasekhara and B.G. Clark



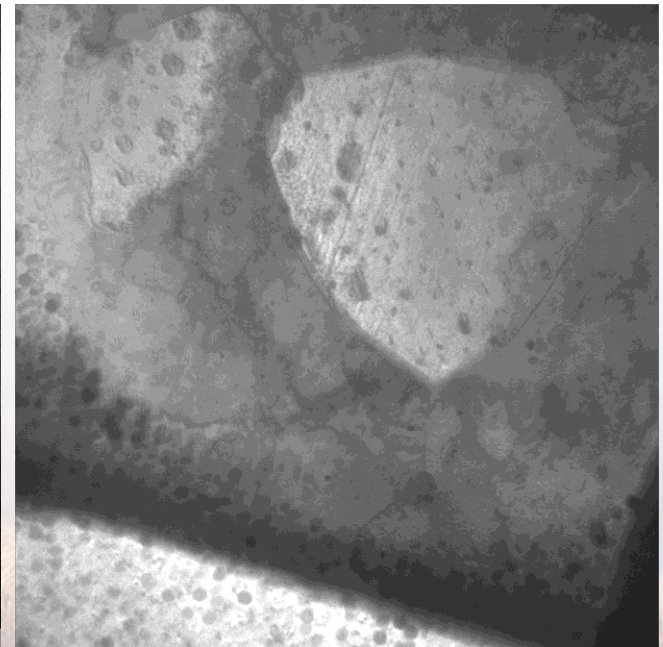
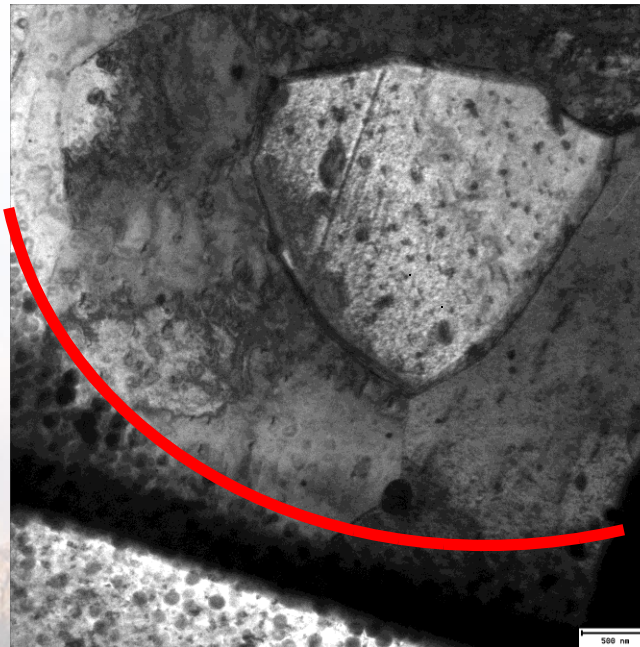
Vapor-Phase Heating TEM Stage

- Compatible with a range of gases
- *In situ* resistive heating
- Continuous observation of the reaction channel
- Chamber dimensions are controllable
- Compatible with MS and other analytical tools

Vacuum & Single Window



Nominally 1 atm H₂ & Two Windows

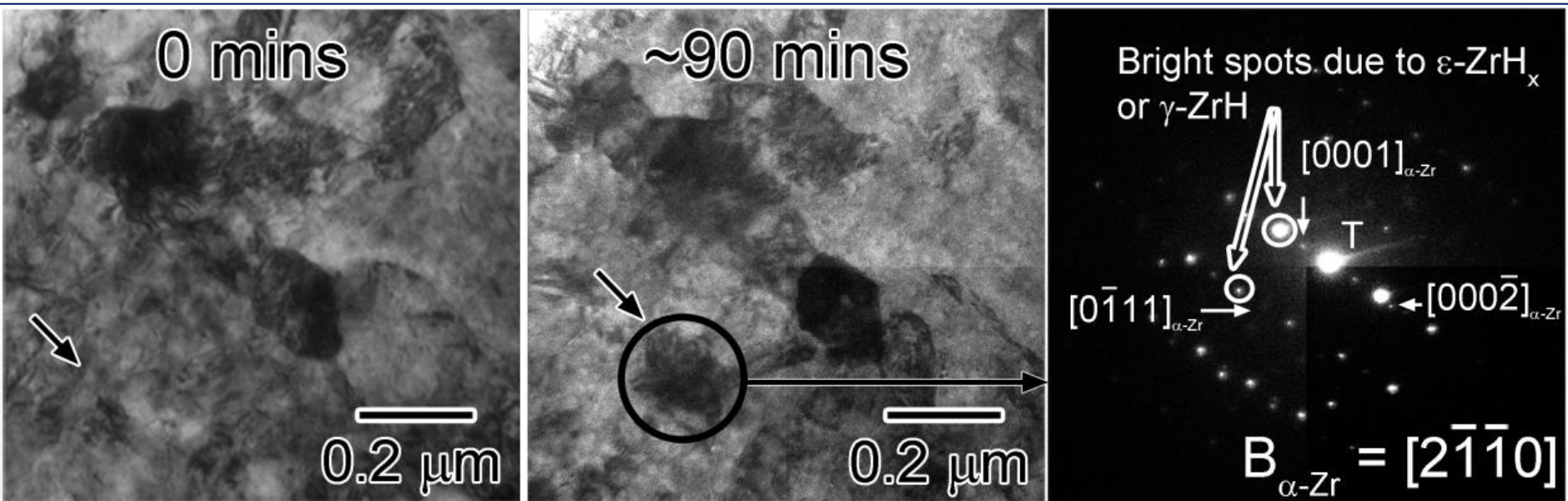


Most features are observed in both despite the decreased resolution resulting from the additional SiN window and 5 μm of air

In situ Observation of Hydride Formation in Zirlo

Collaborators: S. Rajasekhara and B.G. Clark

Absolute hydrogen pressure: 327 torr (~ 0.5 atm),
Ramp rate: 1 °C/s, Final temperature: ~ 400 C, Dwell time: ~ 90 mins



Hydride formation shown, for the first time by use of a novel TEM gas-cell stage, at elevated temperature and hydrogen pressure



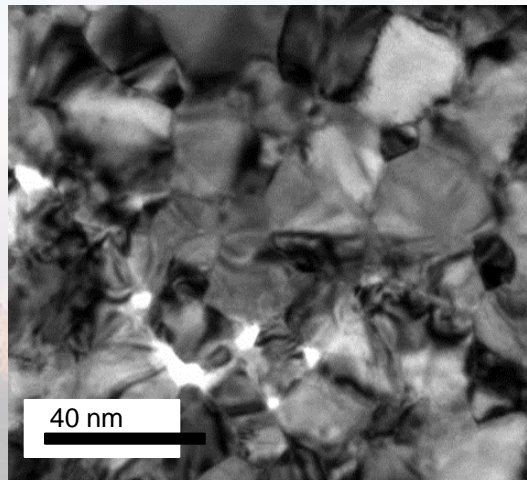
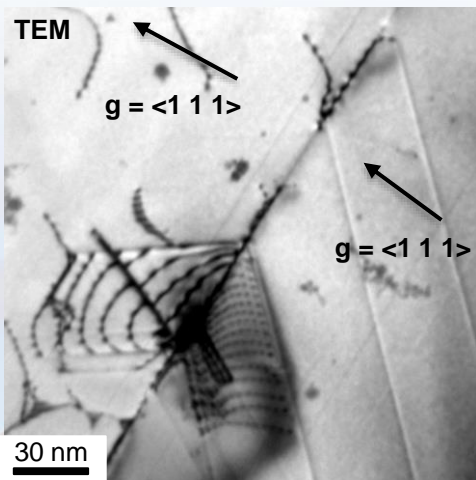
In situ Mechanical Testing

Qualitative “Bulk” Mechanical Testing

Minimal control over displacement and no “out-of-box” force information

- Successful in studies in observing dislocation-GB interactions/mechanisms
- Ideally both grains have kinematic BF 2-beam conditions: challenging in ST holder

Traditional Gatan Heating and Straining Holder

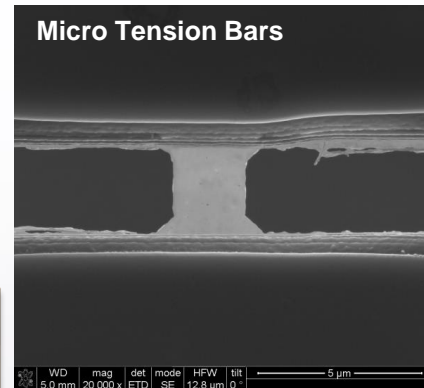


Quantitative Mechanical Testing

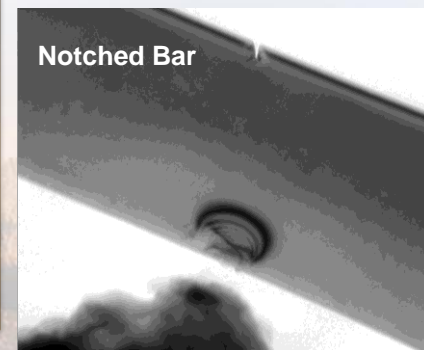
Minimal control over displacement and no “out-of-box” force information

- Sub nanometer displacement resolution
- Quantitative force information with μN resolution

Hysitron PI-95 Holder



- 1) Indentation
- 2) Tension
- 3) Fatigue
- 4) Creep
- 5) Compression
- 6) Bend

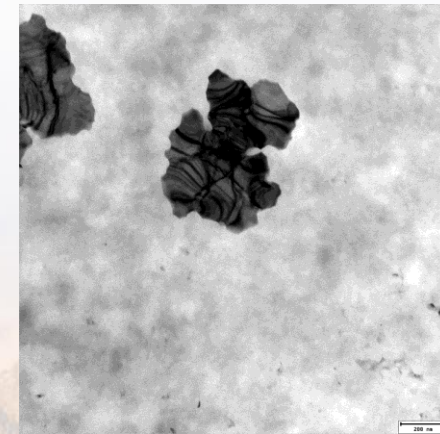
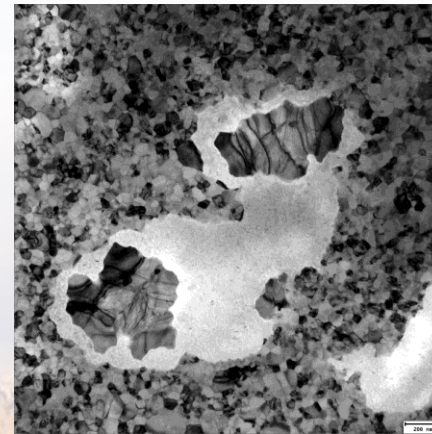
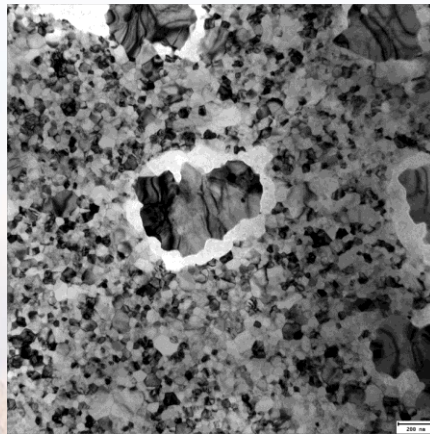
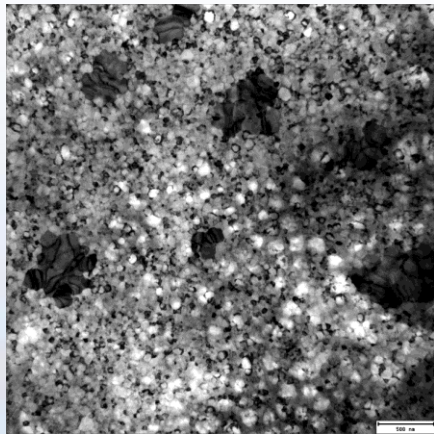
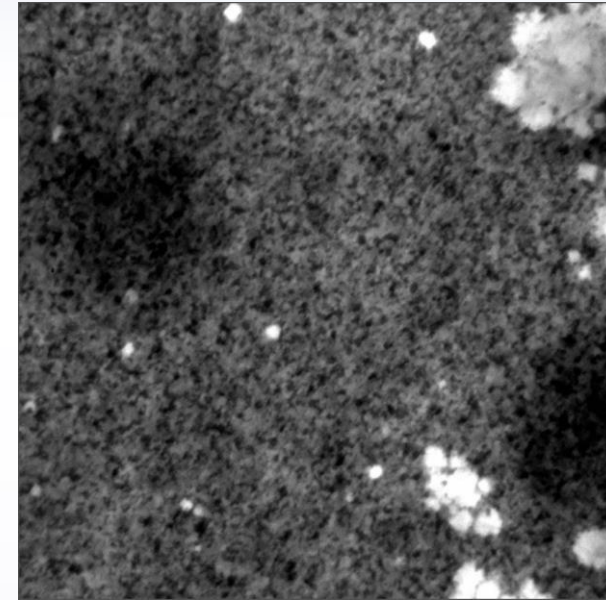
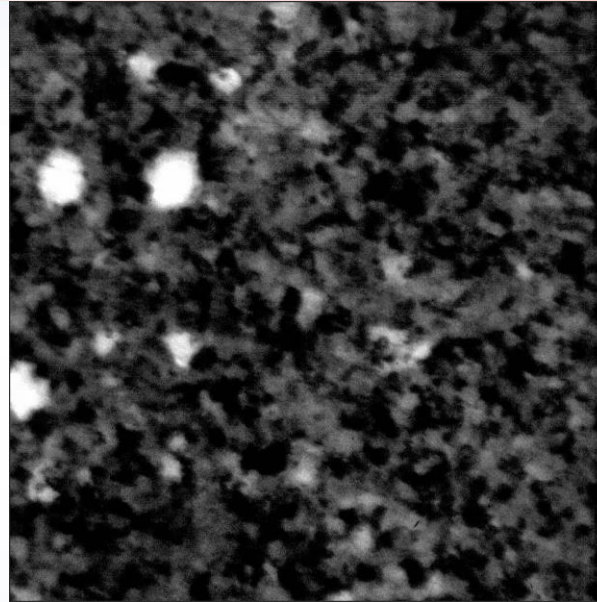
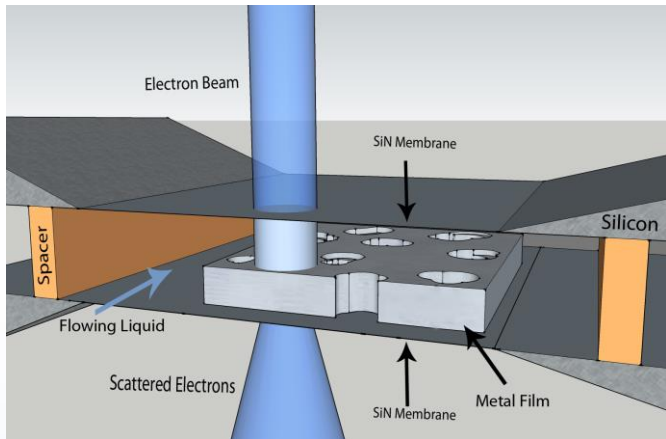


Can We Gain Insight into the Corrosion Process through *In situ* TEM?

Contributors: D. Gross, J. Kacher, I.M. Robertson & Protochips, Inc.

Microfluidic Stage

- Mixing of two or more channels
- Continuous observation of the reaction channel



**Pitting mechanisms during dilute flow of acetic acid over 99.95% nc-PLD Fe involves many grains.
Large grains resulting from annealing appear more corrosion tolerant**