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Linking External Radiation Transport Codes with GADRAS

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Motivation

- GADRAS has library of accurate detector response functions
 - Based on first principles + empirical adjustments based on characterization
- Most users of GADRAS use 1D Model, but not always applicable
 - Model is too complicated for 1D
 - Model is already built with another transport code
 - User is more familiar with other transport code
- Linkage from external transport codes to GADRAS is useful for the community
- Discussion will use MCNP as an example, but can be any transport code capable of giving similar results

Previous Work

- MCNP calculated 3D flux and gave GADRAS equivalent point source model [1]
 - Flux into 4π was collapsed and processed as if point source
 - Method worked well for distant sources
 - Scatter was computed by GADRAS
- Limitations:
 - Detector Response Function is based only on radiation entering through front of detector
 - Computed response was inaccurate for near-by, large sources

1) Published as Los Alamos National Laboratory report LA-UR-12-25637

Title: "Combined MCNP/GADRAS Simulation of HPGe Gamma Spectra"

Author(s): Mohini Rawool-Sullivan, John Mattingly, Dean Mitchell, and Jesson Hutchinson

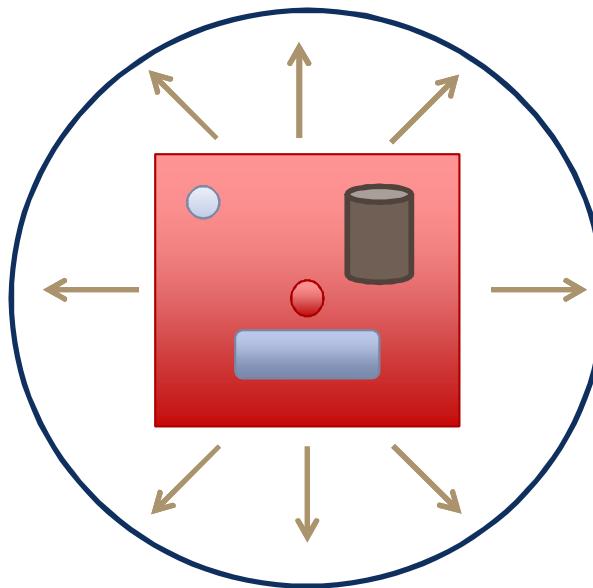
The Conference Record (CR) of 2012 Nuclear Science Symposium, Medical Imaging Conference

2012 Nuclear Science Symposium, Medical Imaging Conference & Workshop

on Room-Temperature Semiconductor X-Ray and Gamma-Ray Detectors,

2012-10-29/2012-11-03 (Anaheim, California, United States)

Previous Work



New Method

- Use current density (current per unit area, not flux!) on all sides (facets) of the detector as input to GADRAS
- GADRAS computes interactions within detector crystal and scattering that occurs within the detector housing
- External code should compute room scatter
- Environmental scatter computation from GADRAS should be suppressed

MCNP-GADRAS Coupling

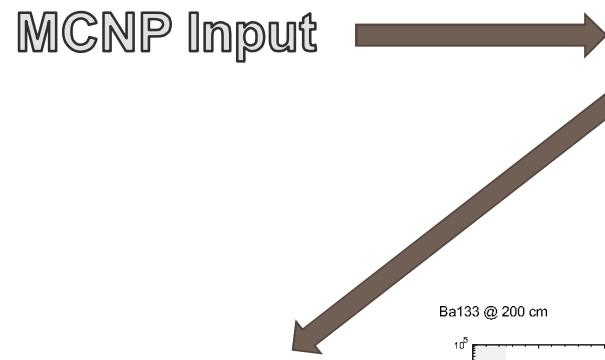
- Multi-faceted GAM files allow MCNP tallies to be input into GADRAS' detector response function

```
Simple Example Input Deck
1 0 -1 imp:p=0 $ detector volume
2 1 -0.0012 -2 1 imp:p=1 $ surrounding air
3 0 2 imp:p=0

1 rcc 100 0 0 10 0 0 10 $ detector surfaces
2 so 1000 $ universe sphere

mode p
sdef par=p pos=0 0 0 erg=0.6617
m1 7014 -75 $ air
8016 -25
f1:p 1.1 $ side of cylinder, side of detector
f11:p 1.2 $ bottom of cylinder, front face of detector
f21:p 1.3 $ top of cylinder, back face of detector
e0 {insert values from MCNPbins.dat}
```

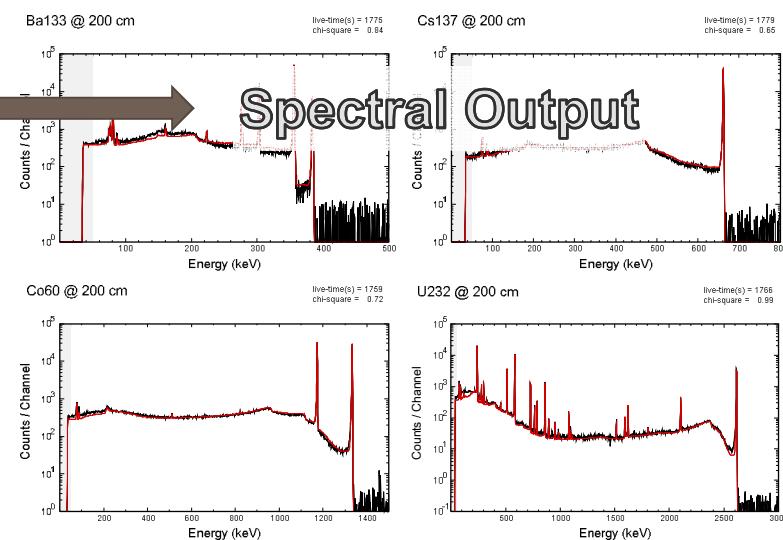
MCNP Input



GADRAS Input

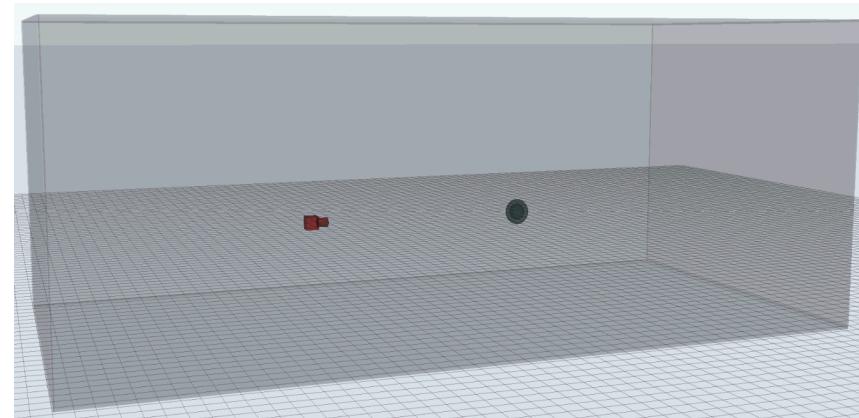
```
0 5 ! NewFormat,ModelGeometry
0 1475 78 ! PhotonLines, PhotonGroups, NeutronGroups
2.000E+02 3.952E+04 4.155E+01 1.063E+02 ! photon groups
2.935E+02 1.222E+01 2.138E+00 7.240E-01
2.957E+02 2.742E-01 1.290E-01 2.181E-01
...
1.179E+04 0.000E+00 0.000E+00 0.000E+00
1.200E+04 0.000E+00 0.000E+00 0.000E+00
1.965E+07 0.000E+00 7.831E-03 3.073E+00 ! neutron groups
1.733E+07 3.016E-01 4.618E+00 2.870E+01
...
5.000E-03 9.419E+00 2.706E-06 0.000E+00
2.000E-03 4.727E-02 9.549E-08 0.000E+00
5.000E-04 1.149E-03 0.000E+00 9.003E+03
1.000E-05 4.277E+04 0.000E+00 0.000E+00
```

Detector Current
Tally Output



Tallies and Conversions

- External transport code records separate tallies for front, side, and back of detector
 - Must be F1 tally
 - F8 tally uses MCNP's DRF
- Units differ between MCNP and GADRAS
 - Photon energy
 - MeV in MCNP
 - keV in GADRAS
 - Neutron energy
 - MeV in MCNP
 - eV in GADRAS
 - Tally/GAM file input
 - probability of particle entering cell in MCNP
 - particle/s/m² in GADRAS



Tallies and Conversions

- Equations for converting between MCNP output and GADRAS input:
 - MCNP gives probability per source particle
 - Necessary info: source activity and facet surface area

$$\{\text{front current density}\} = \left\{ \frac{\text{front current}}{\text{source particle}} \right\} * \frac{(1 \times 10^{-6} \text{ Ci})(3.7 \times 10^{10} \frac{\text{Bq}}{\text{Ci}})(1 \frac{1/\text{s}}{\text{Bq}})}{\pi(10 \text{ cm})^2 \left(\frac{1 \text{ m}}{100 \text{ cm}} \right)^2}$$

$$\{\text{side current density}\} = \left\{ \frac{\text{side current}}{\text{source particle}} \right\} * \frac{(1 \times 10^{-6} \text{ Ci})(3.7 \times 10^{10} \frac{\text{Bq}}{\text{Ci}})(1 \frac{1/\text{s}}{\text{Bq}})}{2\pi(10 \text{ cm})(10 \text{ cm}) \left(\frac{1 \text{ m}}{100 \text{ cm}} \right)^2}$$

$$\{\text{back current density}\} = \left\{ \frac{\text{back current}}{\text{source particle}} \right\} * \frac{(1 \times 10^{-6} \text{ Ci})(3.7 \times 10^{10} \frac{\text{Bq}}{\text{Ci}})(1 \frac{1/\text{s}}{\text{Bq}})}{\pi(10 \text{ cm})^2 \left(\frac{1 \text{ m}}{100 \text{ cm}} \right)^2}$$

GAM File Structure

```
0 5 ! NewFormat,ModelGeometry
0 1476 78 ! PhotonLines, PhotonGroups, NeutronGroups
20.000  6.209E+01  1.416E+02  1.568E-02 ! photon groups
29.350  6.246E+00  9.240E-05  1.313E-03
29.570  2.240E-05  1.592E-11  1.369E-10
...
11790.000 0.000E+00  0.000E+00  0.000E+00
12000.000 0.000E+00  0.000E+00  0.000E+00
1.965E+07 3.501E-14  5.322E-13  5.340E-10 ! neutron groups
1.733E+07 0.000E+00  3.836E-12  5.535E-11
...
5.000E-03  3.114E+03  5.722E+02  3.873E+02
2.000E-03  1.053E+04  1.126E+03  8.363E+02
5.000E-04  7.401E+03  3.388E+02  2.243E+02
1.000E-05  0.000E+00  0.000E+00  0.000E+00
```



Energy



Front



Sides



Back

GAM File Structure

- Photon energy bins
 - Listed in ascending order (keV)
 - Bin defines lower bound of energy group (MCNP is opposite)
- Neutron energy bins
 - Listed in descending order (eV)
 - Bin defines upper bound of energy group
- Side current
 - GADRAS represents detectors as if they are symmetric normal to the detector axis
 - Current coming through top, bottom, left, and right sides are averaged together to give “side current density”

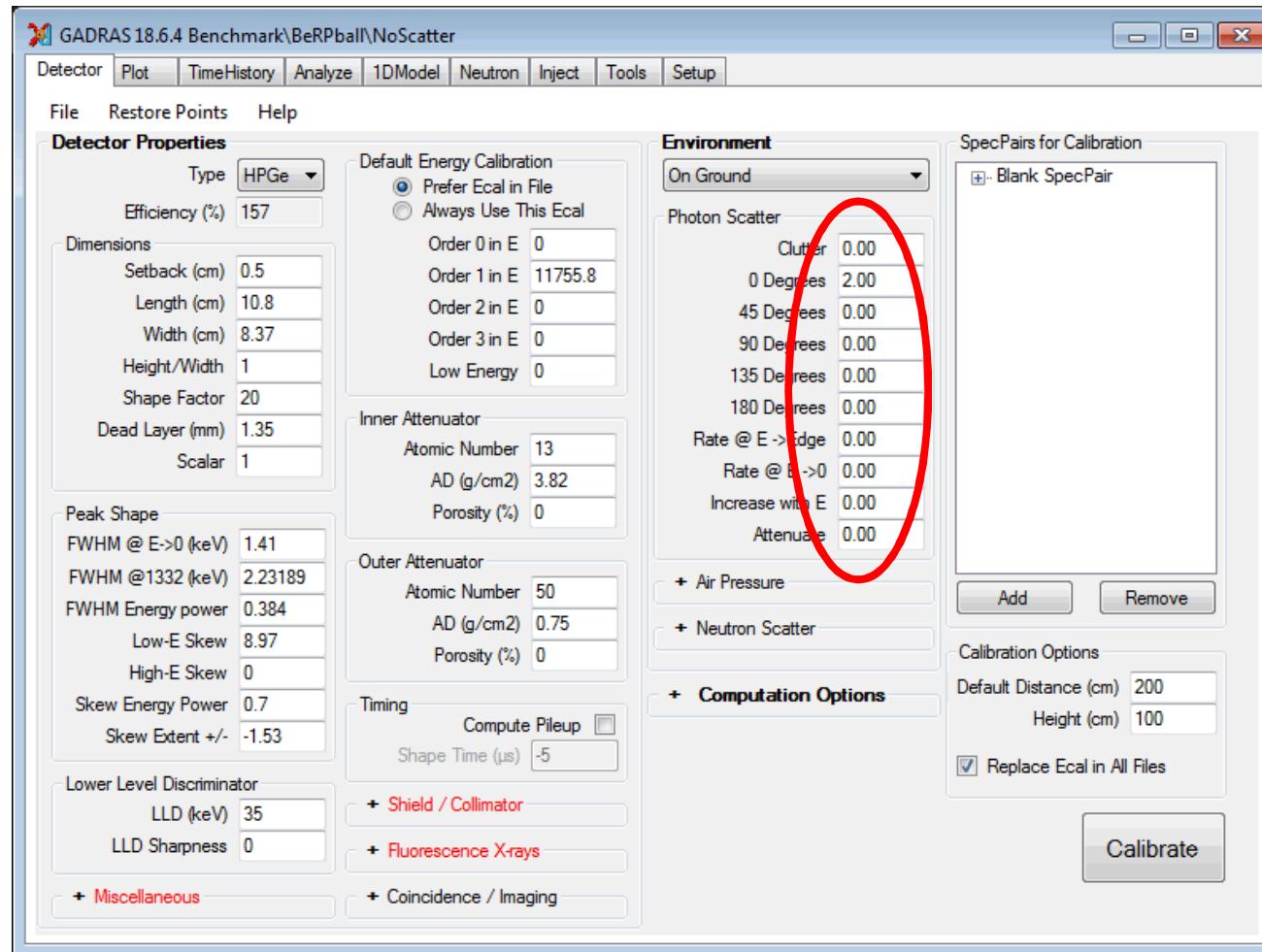
Capabilities

- Current densities can be used for multiple detectors
 - The same MCNP results can be used for different detectors
 - If source-detector distance changes significantly, new MCNP models should be used
 - Detector size in MCNP model doesn't matter unless source-detector distance is small
- Energy bin structure is not mandated, but results are rebinned according to default structure
 - Recommended photon structure in MCNPbins.dat

GADRAS Scattering

- Scattering traditionally estimated with calibration spectra
- Parameters correspond to radiation scatter angles
- Two types of scattering:
 - Environmental scattering within room
 - Internal scattering in the detector
- Collimators are not yet implemented

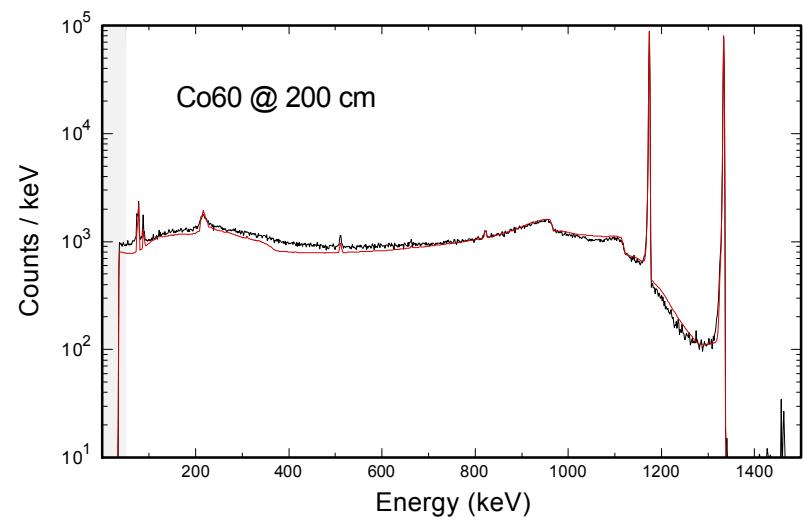
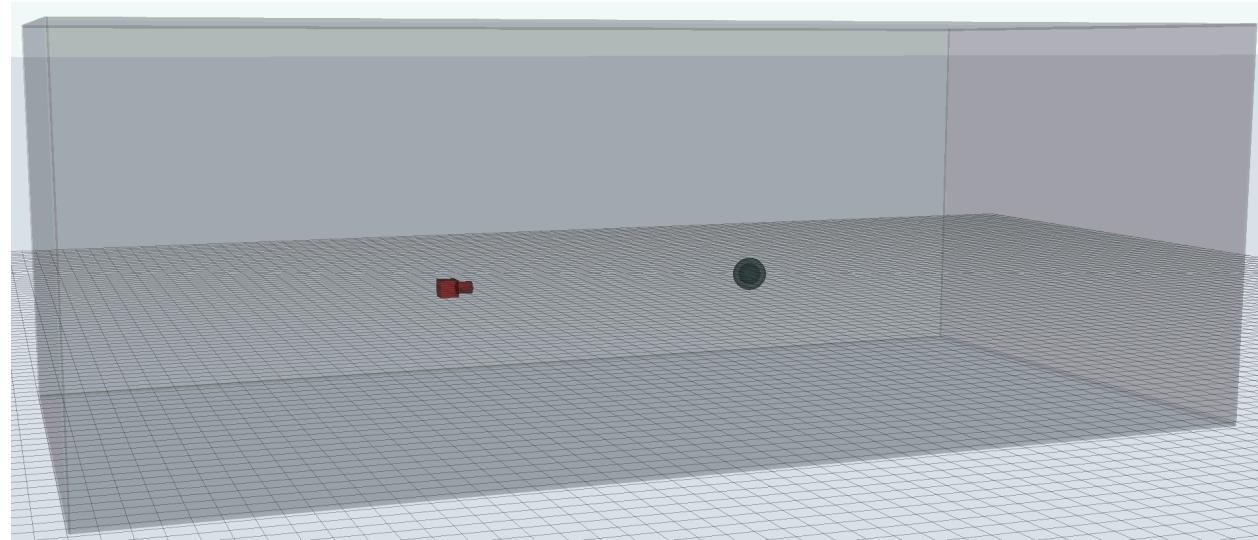
Detector Settings



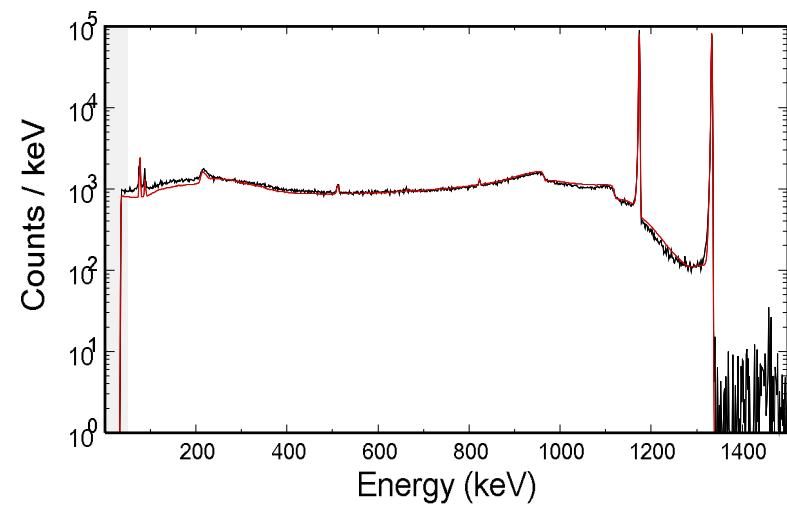
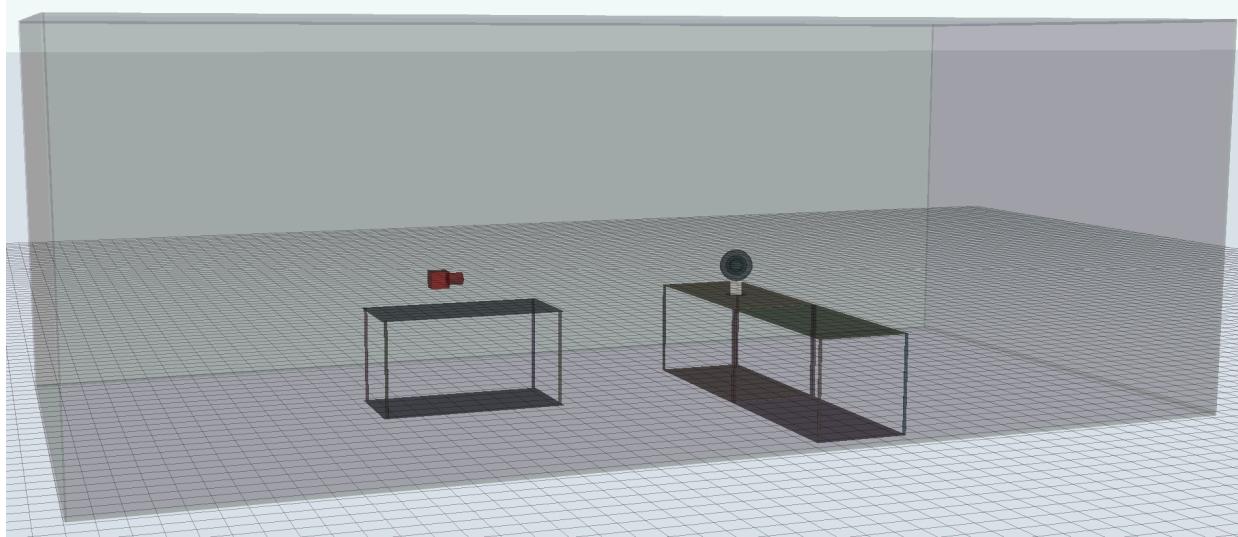
Experimental Verification

- Data taken at Nevada National Security Site (NNSS)
 - HPGe – 150%
- Basic MCNP model incorporated walls, floor, and ceiling
- Advanced model incorporated steel carts, legs, other detectors, and electronics modules
- 200 cm source-distance, 100 cm above ground
- ^{137}Cs , ^{133}Ba , ^{60}Co , ^{232}U , ^{252}Cf moderated and unmoderated, BeRPBall moderated and unmoderated

MCNP Model

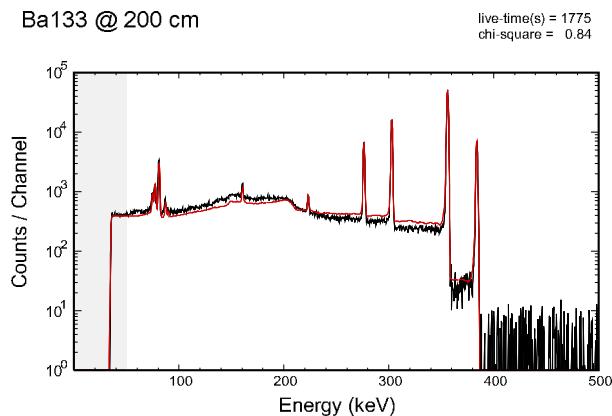


MCNP Model

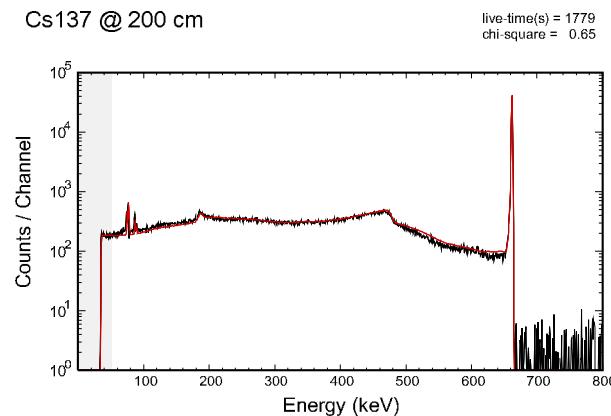


Gamma Models

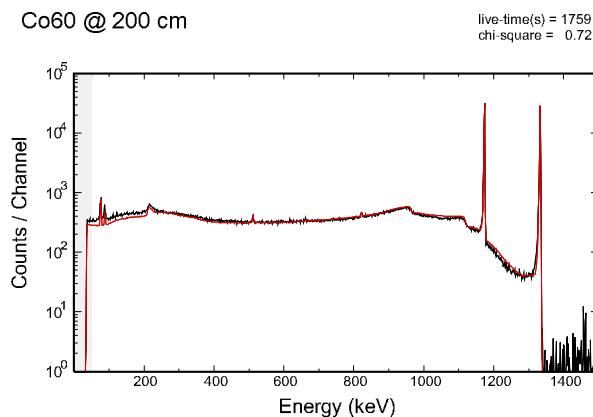
Ba133 @ 200 cm



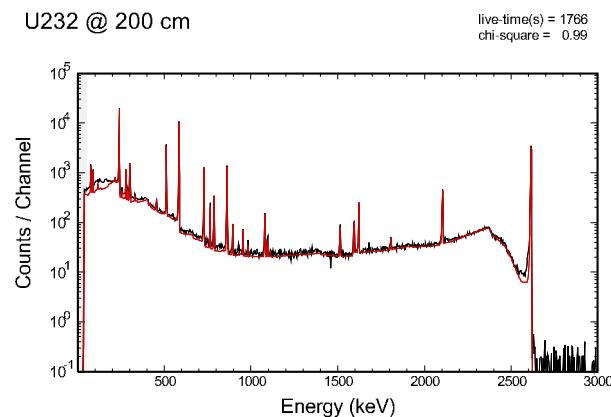
Cs137 @ 200 cm



Co60 @ 200 cm

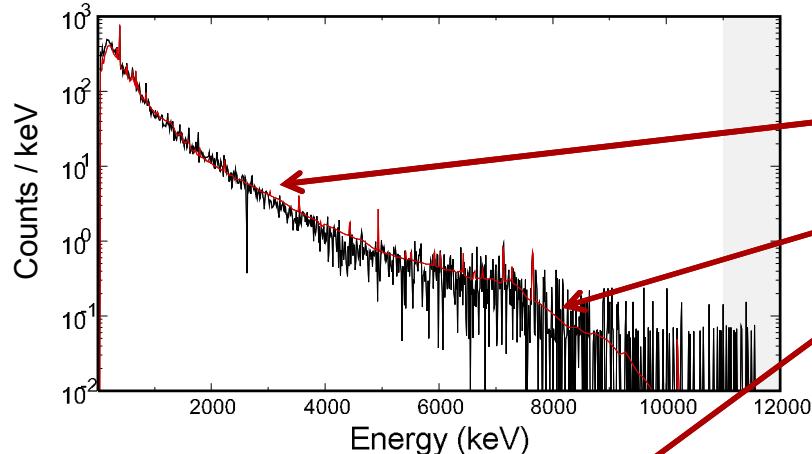


U232 @ 200 cm

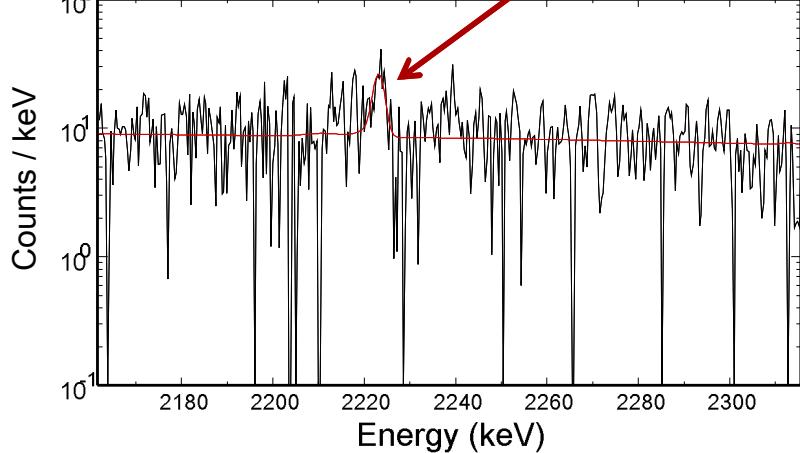


Cf-252

Cf252 @ 200 cm H=100cm

live-time(s) = 1787
chi-square = 0.66

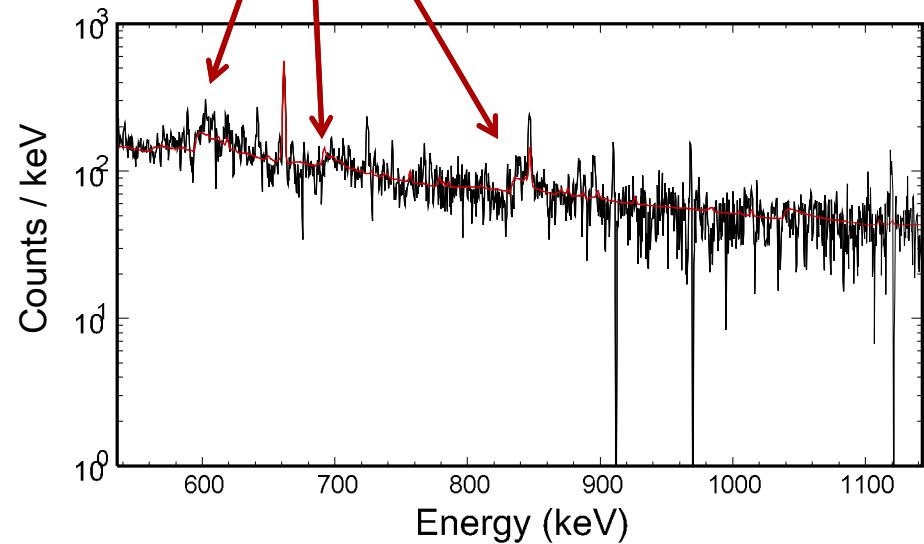
Cf252 @ 200 cm H=100cm

live-time(s) = 1787
chi-square = 0.66

Features replicated:

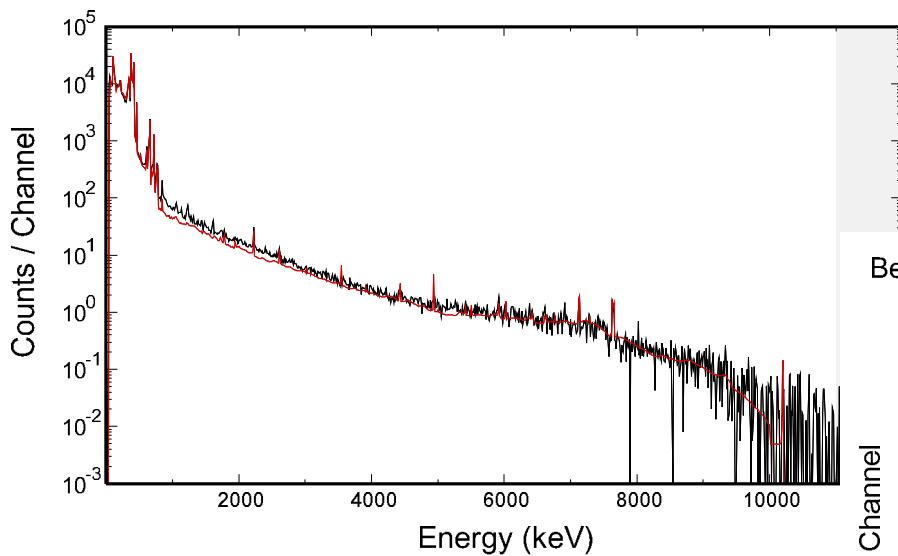
- Continuum from
- Fission gammas
- Neutron capture in HPGe
- Neutron capture reactions
- Inelastic scatter

Cf252 @ 200 cm H=100cm

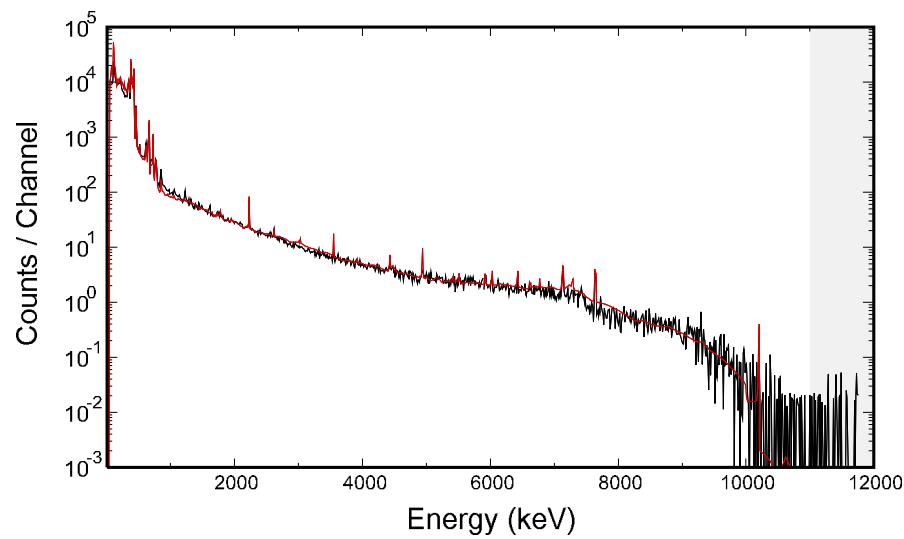
live-time(s) = 1787
chi-square = 0.66

BeRPBall

BeRPBall Bare @ 200 cm

live-time(s) = 3299
chi-square = 1.50

BeRPBall 1.0" reflector @ 200 cm

live-time(s) = 3324
chi-square = 4.03

Common Pitfalls

- Not turning off scattering in GADRAS' DRF
- Gamma group structure does not capture discrete lines
 - Lines were left out because thought not important
 - Line groups are too wide
 - Line group energy bin doesn't match up with actual source energy
- Source term incorrect
 - Some users use GADRAS' 1DModel to generate MCNP input