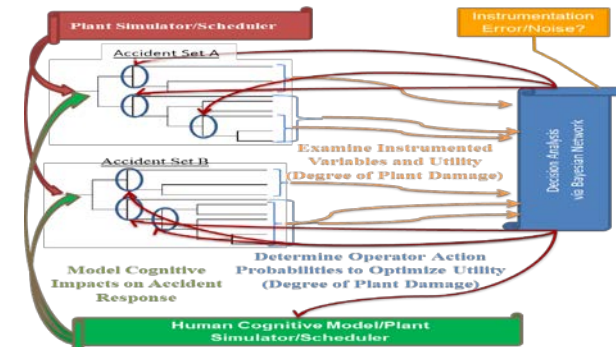
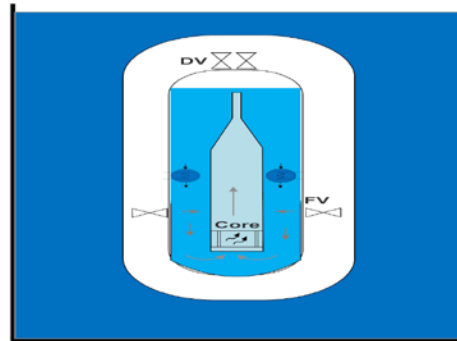
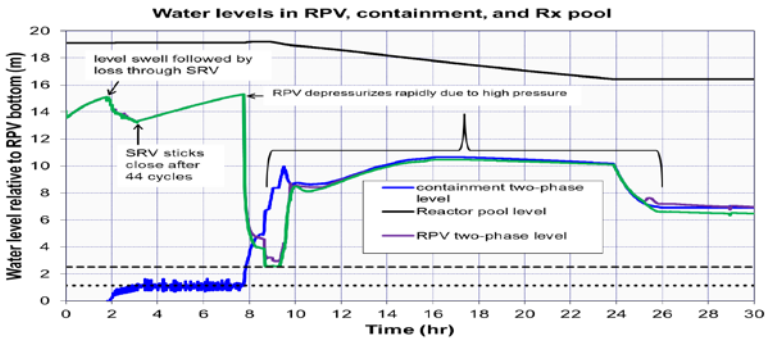


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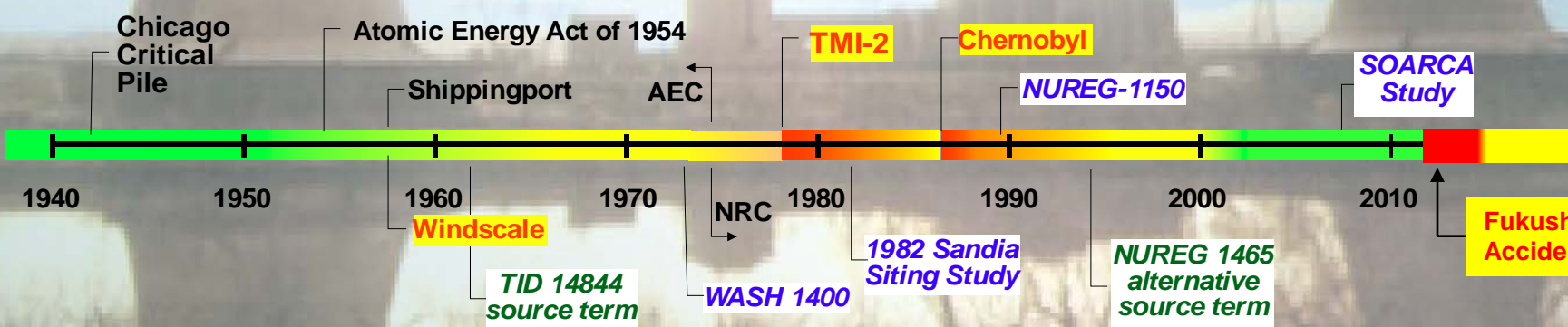
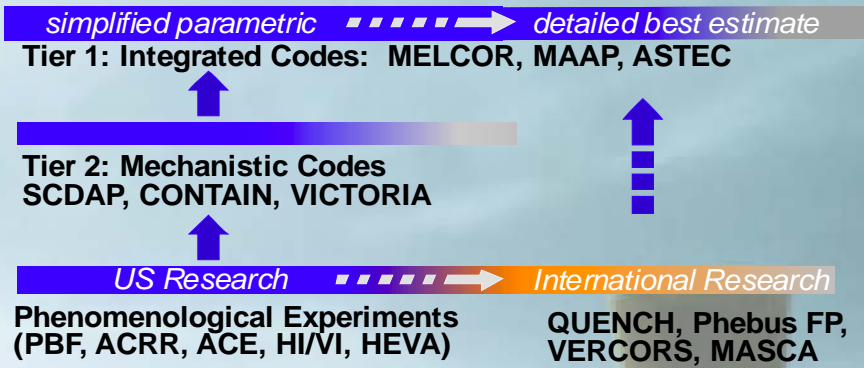
Nuclear Power Accident Analysis at Sandia National Labs

Matthew Denman
 Risk and Reliability Analysis
 Presentation to MIT, October 2015

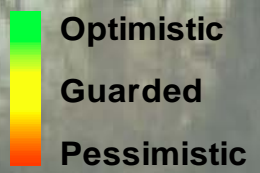
How Sandia Got Involved in the Safe Production of Nuclear Energy

- **Atomic Energy Commission divided 1974**
 - Department of Energy to develop commercial nuclear energy
 - US Nuclear Regulatory Commission to assure nuclear energy development provides adequate protection of public health and safety
- **Sandia made a National, multi-mission laboratory**
 - Expertise in nuclear science and engineering
 - No past involvement with commercial nuclear power

Timeline of Nuclear Safety Technology Evolution



Nuclear Power Outlook

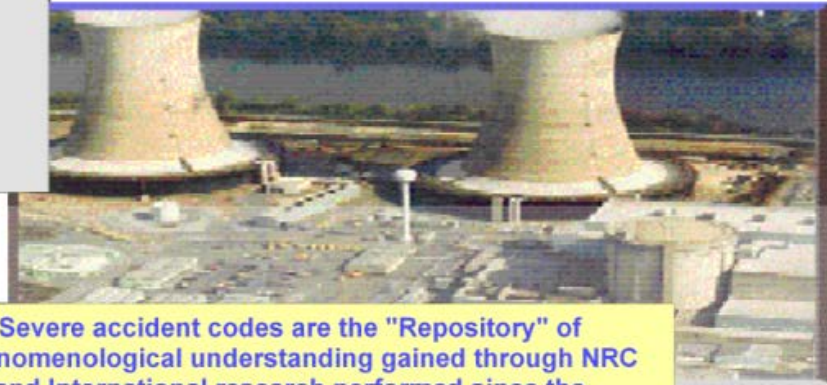


Emerging Issues.....

- Risk Informing Regulation**
 - Modernization, NUREG-1465
- License Amendments and Extension**
 - MOX, High Burnup
 - Plant aging
- Emergency Response Planning**
- Advanced Reactors**
 - AP1000, ESBWR, US-EPR
- NGNP - HTGR, VHTGR, H2 Economy**
- GNEP - Fast Burner Reactor, Reprocessing**

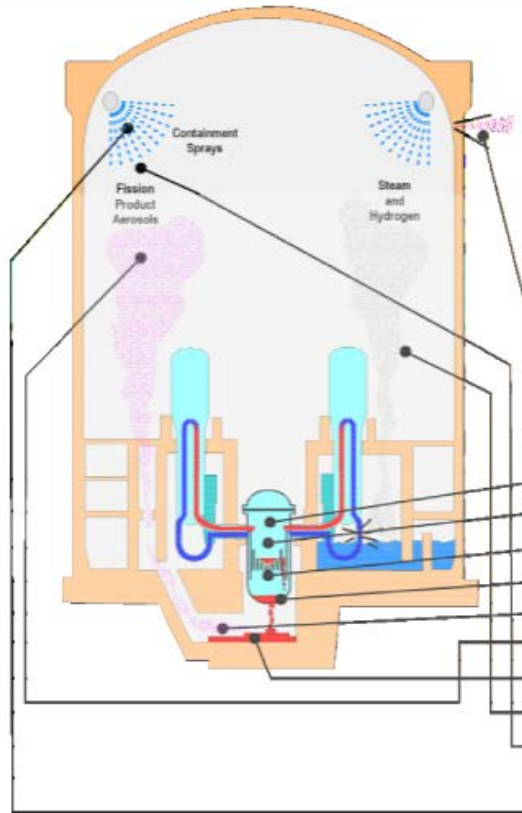
MELCOR Severe Accident Phenomena

Modeling and Analysis of Severe Accidents in Nuclear Power Plants



Severe accident codes are the "Repository" of phenomenological understanding gained through NRC and International research performed since the TMI-2 accident in 1979

Integrated models required for self consistent analysis



Important Severe Accident Phenomena

	MELCOR	CONTAIN	VICTORIA	SCDAP	RELAP-5
Accident initiation	█				
Reactor coolant thermal hydraulics	█				
Loss of core coolant	█				
Core meltdown and fission product release	█		█		
Reactor vessel failure	█				
Transport of fission products in RCS and Containment	█	█	█		
Fission product aerosol dynamics	█	█	█		
Molten core/basemat interactions	█				
Containment thermal hydraulics	█				
Fission product removal processes	█	█	█		
Release of fission products to environment	█	█	█		
Engineered safety systems - sprays, fan coolers, etc	█	█			
Iodine chemistry, and more	█				

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Source: Tokyo Electric Power Company



Modeling and Analysis of Severe Accidents in Nuclear Power Plants

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Integrated models required for self consistent analysis

Important Severe Accident Phenomena

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The Fukushima Accident Reconstruction



Sandia National Laboratories is a multi-program laboratory managed and operated by Sandia Corporation, a wholly owned subsidiary of Lockheed Martin Corporation, for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-AC04-94AL85000.

Timeline of Major Fukushima Damage Events

Earthquake at 14:46: LOSP

Tsunami at 15:41: SBO

level loss

▲ SC Saturated
 ■ Fuel Damage

▨ fresh water ▨ sea water

▲ Containment vent
 ▲ H2 Explosion

Unit 1

- Loss of isolation condenser cooling following tsunami and station blackout produced "hands off" damage progression

low pressure emergency injection

- Low pressure water injection was well aligned with operator depressurization
- Water injection believed to minimize core damage initially
- Loss of injection later believed to lead to significant core damage

▲ SC Saturated

▨ RCIC operating ▨ HPCI operating ▨ Level loss

Unit 3

▲ RPV Depressurization

▨ low pressure emergency injection
 ■ Fuel damage

more damage possible?

▲▲ Containment vents
 ▲ H2 Explosion

low pressure emergency injection

Fuel damage

Unit 2

▲ SC Saturated

▨ RCIC - CST ▨ RCIC from suppression pool ▨ Level loss

▲ RPV Depressurization ▨ low pressure emergency injection

■ Possible Fuel damage
 ▲ Containment vent
 ▲ Noise heard ?

- Unit 2 operated RCIC/HPCI pumps well beyond expected duration

Unit 4 (SFP)

▲ Explosion in Unit 4

Friday 11

Saturday 12

night

Sunday 13

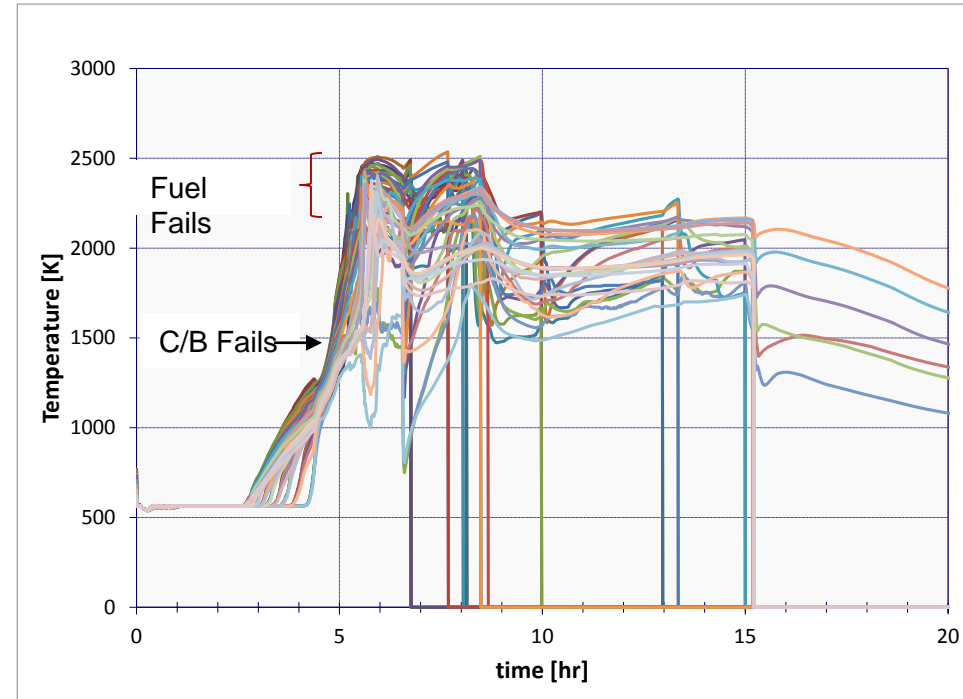
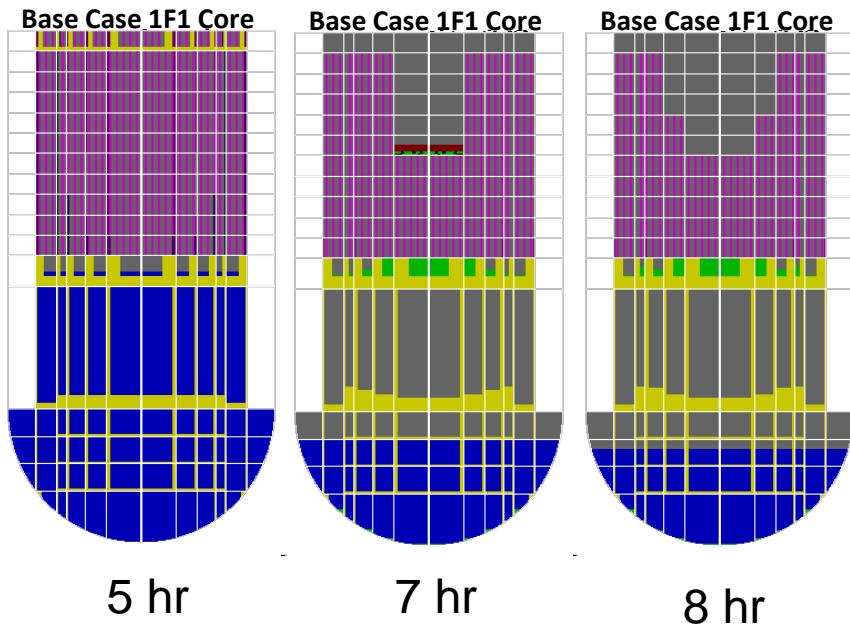
Monday 14

day

Tuesday 15

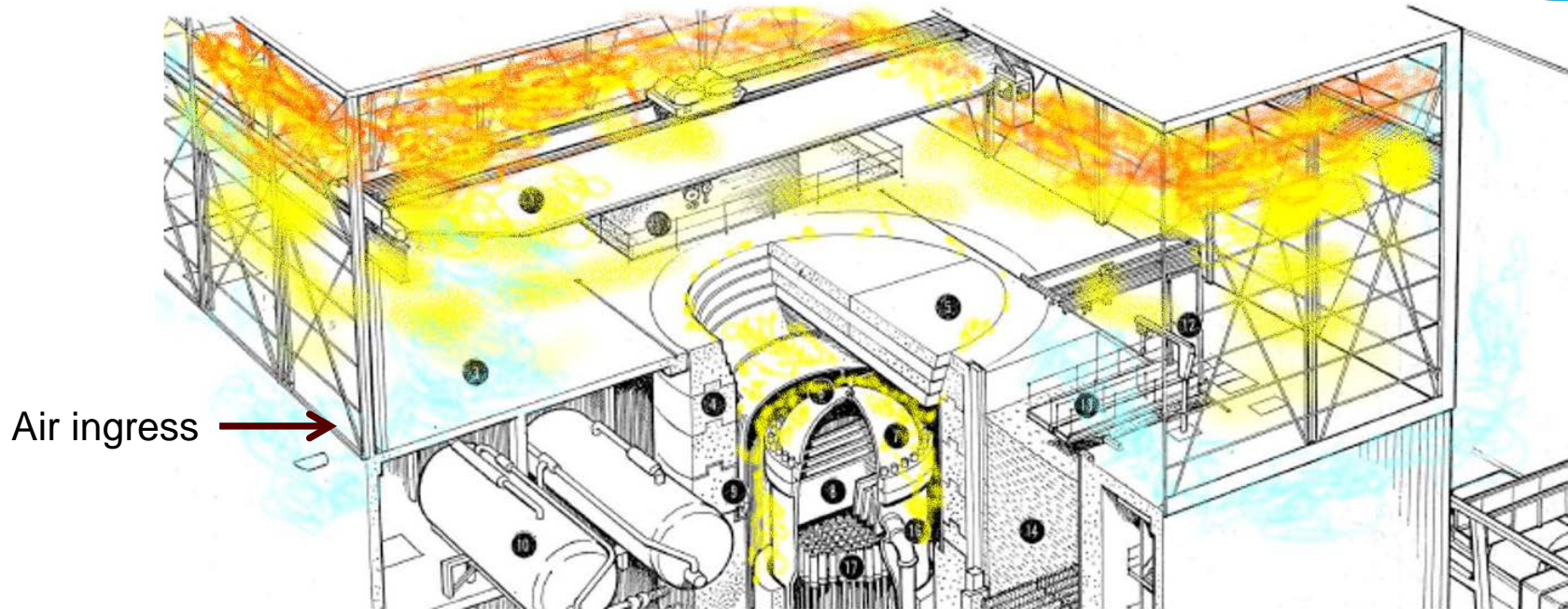
Wednesday 16

Sequential Degradation of Control Blades and Fuel Rod Geometry



- Core damage starts at ~ 4 hours – Control Blade fails first
- Progressive fuel damage after 6 hours
- Core exit gas temperatures very high

Combustible Conditions Follow PCV Venting in 1F1



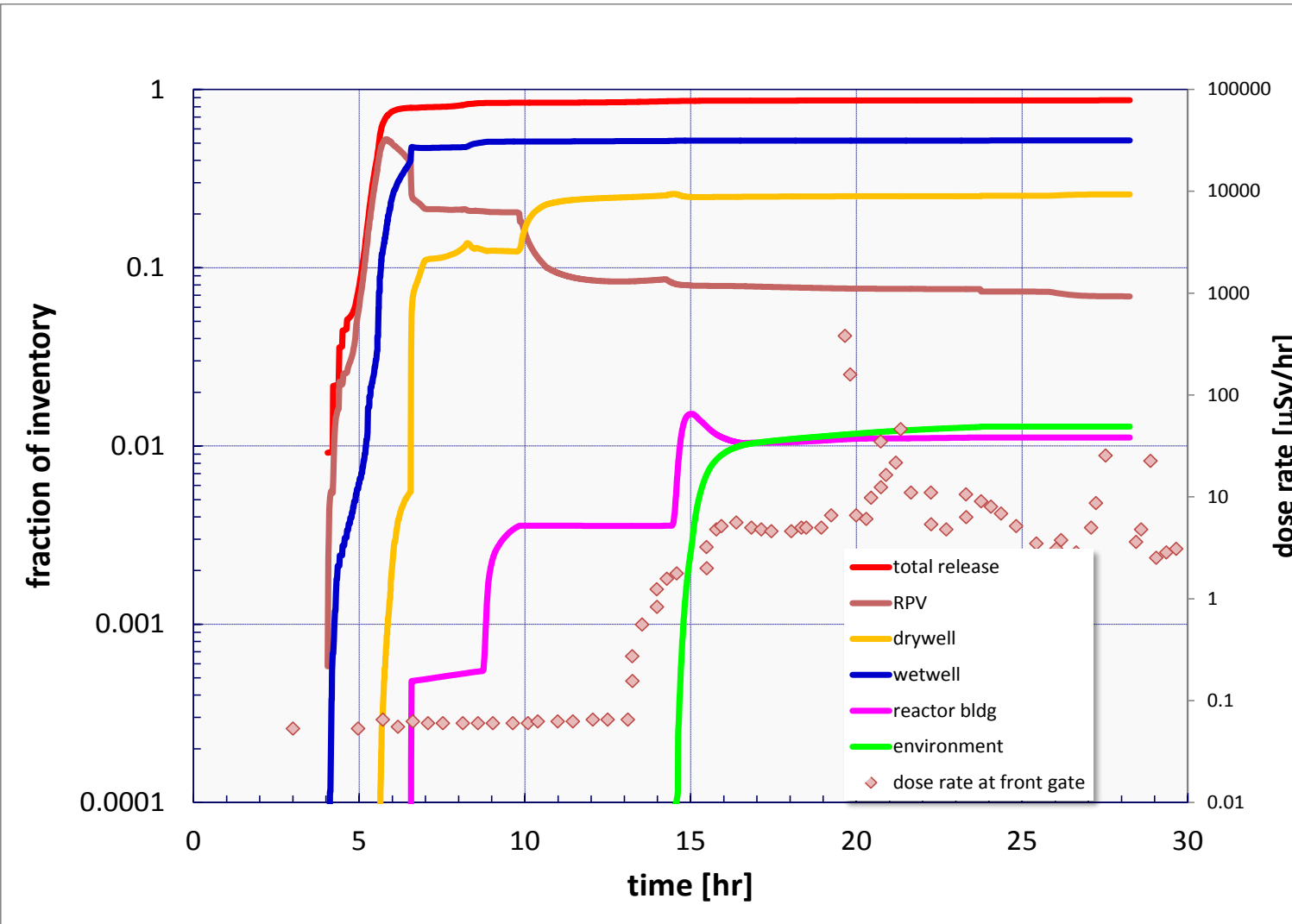
- At around ~23 hours, steam and hydrogen leakage from PCV greatly reduced
 - Water injection was stopped
 - PCV was depressurized by operator venting action
- Continuing condensation without steam source....
 - Reduces steam molar fraction to below 50% in refueling bay, and
 - Produces partial vacuum that draws in outside air
- Air ingress and steam condensation leads to conditions favoring combustion
- Hydrogen stratification produces flammable or detonable concentrations of H_2/O_2

Damage from Explosions



Used by permission from TEPCO
Kenji Tetawa

Cs Release 1F1



Radiation monitors at front gate jump at ~13 hr

Cs Env release about 400,000 Ci

Extended legacy in Probabilistic Risk Assessment

Foundations

Establish basis for assessing and managing risks

Evolutionary PRA

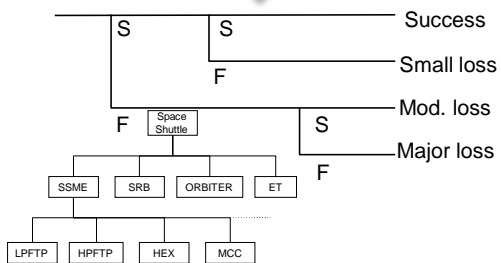
Address gaps in foundational methods

Revolutionary PRA

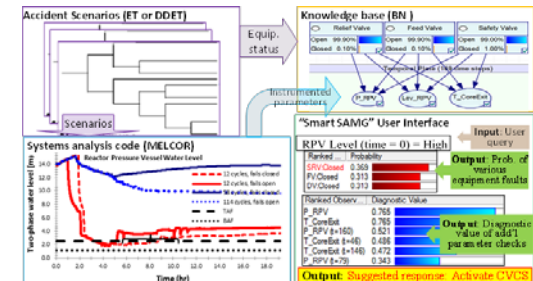
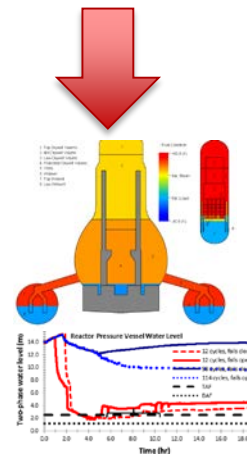
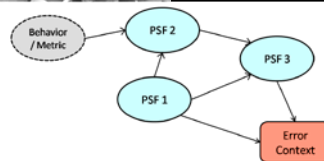
Introduce dynamic elements

New frontiers

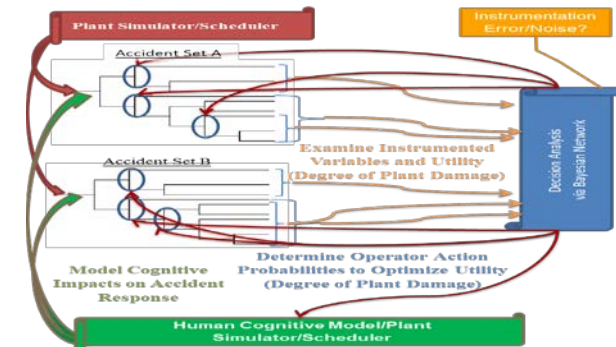
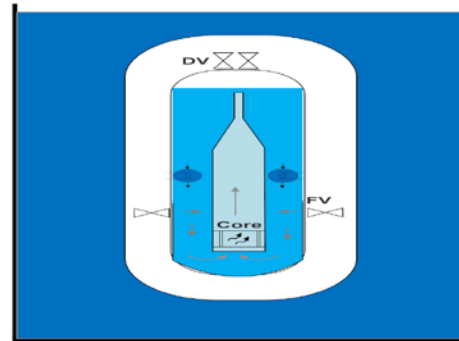
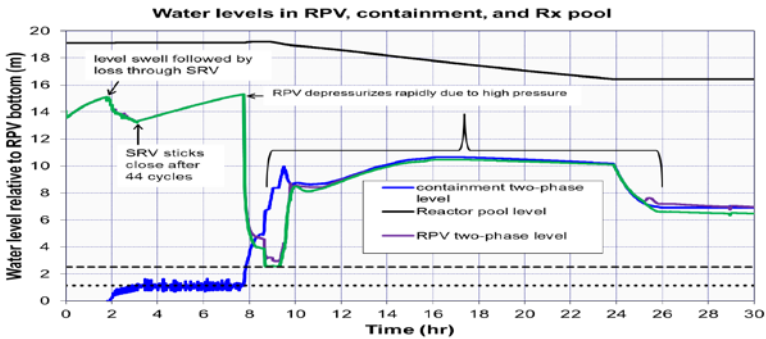
Use foundation to solve problems beyond PRA



$$\text{Risk} \sim \sum_n \sum_j (f_{nj} \cdot c_{nj})$$



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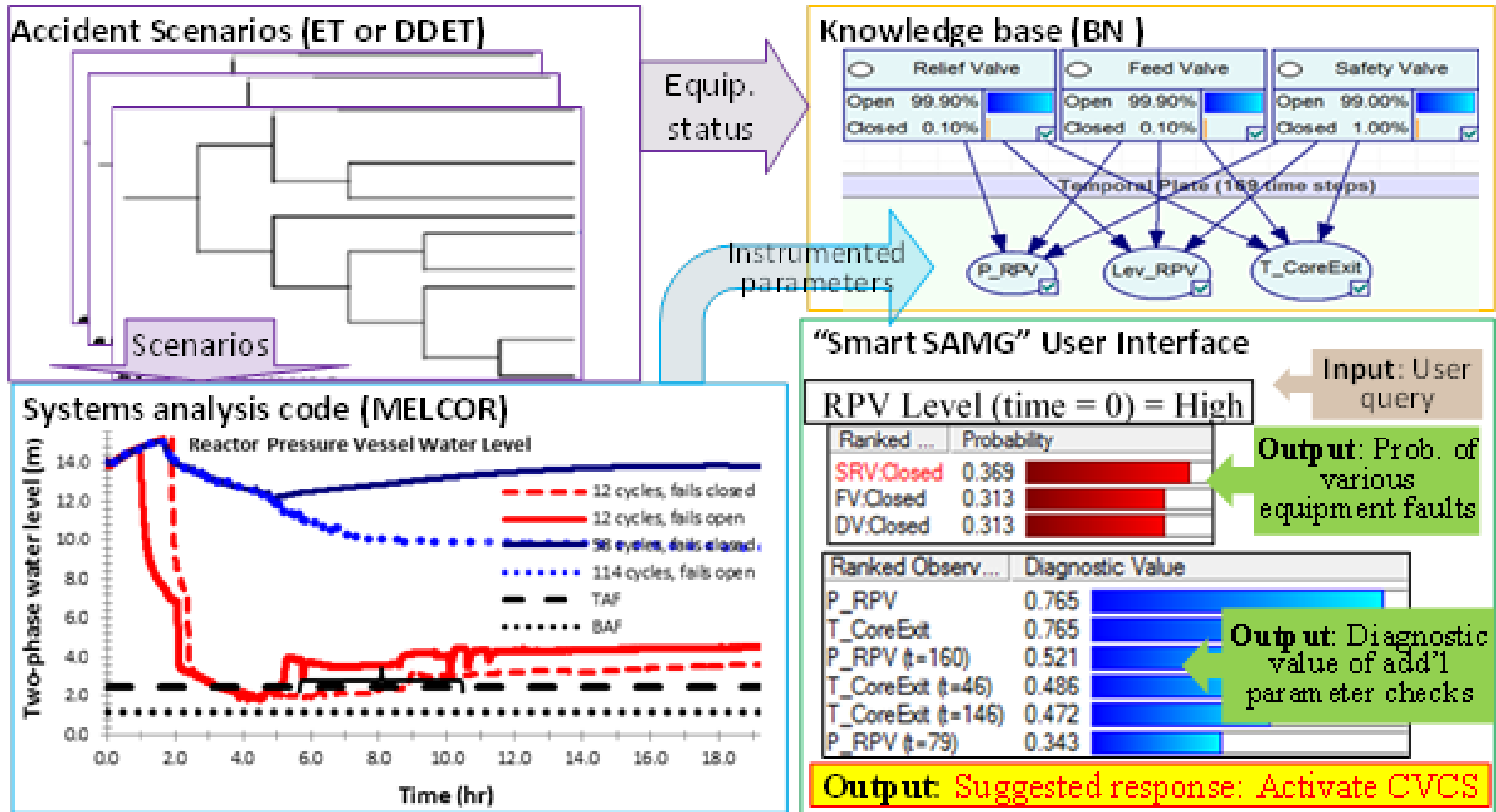


New Frontiers

“Smart Procedures”

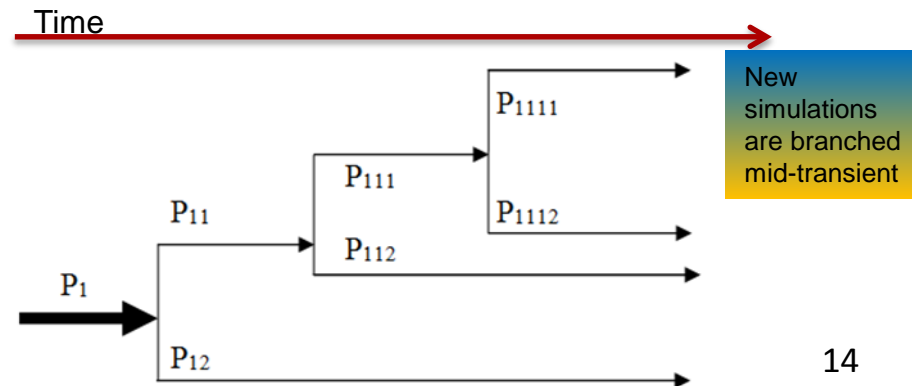
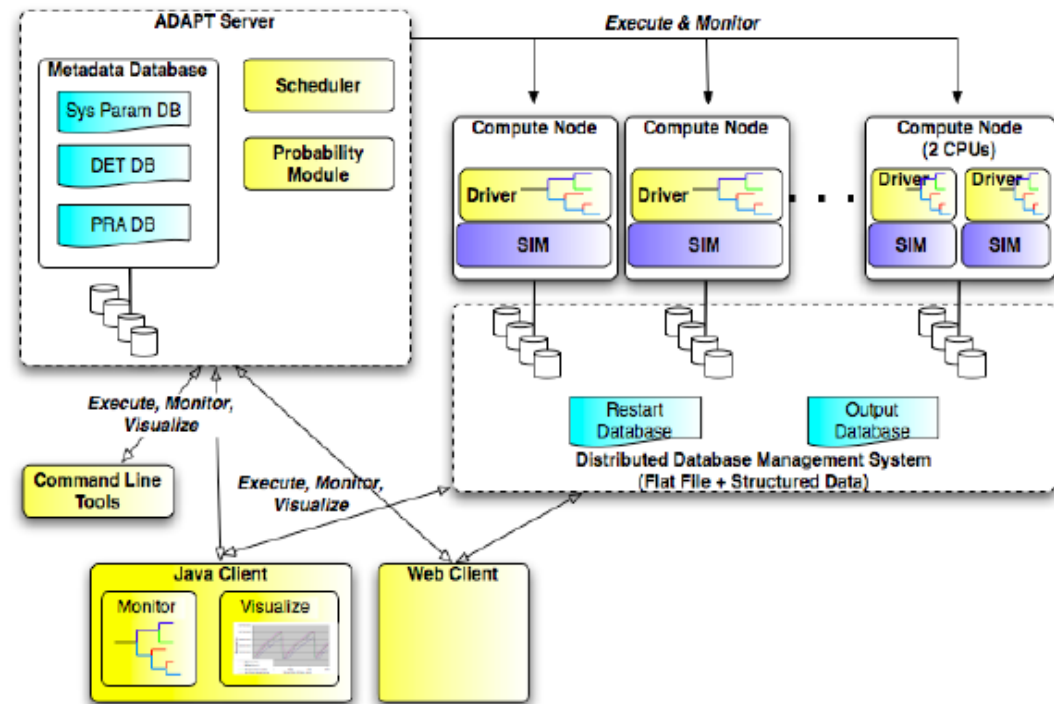


Approach: "Smart Procedures" Model development methodology



Discrete-Dynamic-Event-Trees (DDET) [Uncertainty Exploration]

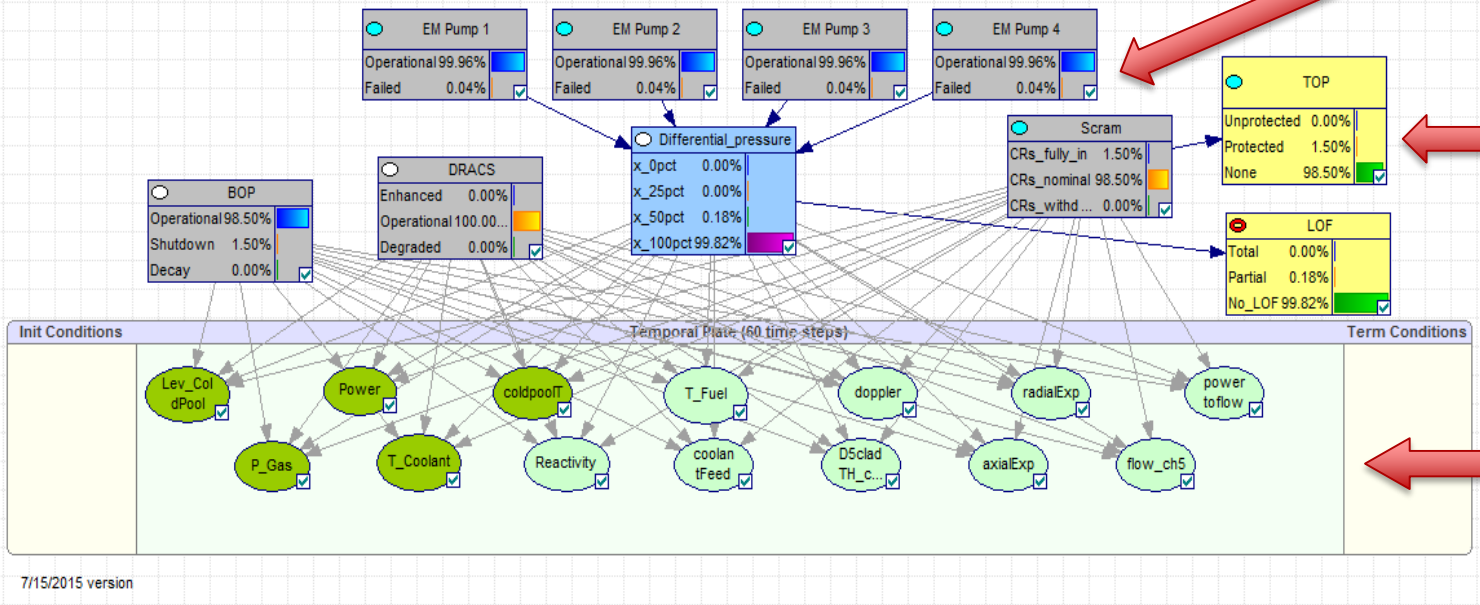
- DDET is a methodology for exploring large spectrum of possible accident scenarios via **Dynamic Programming**.
 - Simulates multiple accident sequences by branching based on physics calculations.
 - Scheduler (ADAPT) was created by a Sandia LDRD completed in 2008.



Evolution of accident sequences is determined by physics and engineering calculations, not a priori analyst decisions.

Accomplishment: Proof-of-concept model for LOF and TOP diagnosis

- Yellow nodes denote accident types.
- Grey nodes denote systems and components.
- Green nodes denote reactor parameters (light green are not instrumented in the control room, lime green are instrumented)
- Blue nodes denote physical states/conditions (unmonitored)



Equipment status

Accident states

Plant Parameters (monitored or not); 2hrs

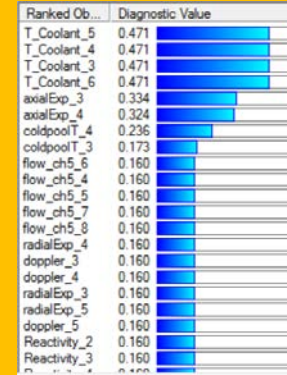
7/15/2015 version

BN-based tool can be used to provide insight into instruments are most essential for diagnosis of specific accidents. This information can provide insight into, reactor design e.g., which instruments need to be hardened for severe accidents.

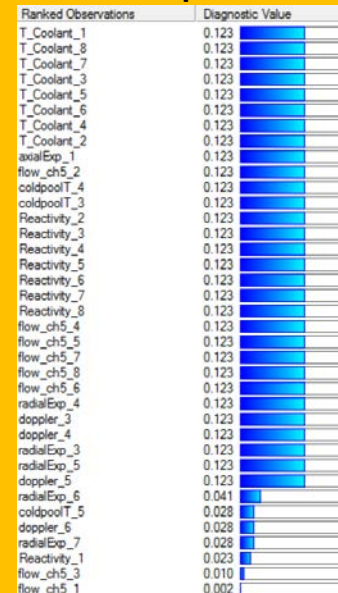
“SMART Procedures” Next steps

- Experiment with model to get insights on value of specific indicators (in progress).
 - Example of insights:
 - Reactivity is a better diagnostic indicator than Power
 - Hot pool temperature (T_Coolant) has high diagnostic value for both LOF and TOP
 - Cold Pool Temperature has better diagnostic value for LOF than TOP
 - Power and Reactivity are redundant
- Expand model scope to cover additional accident sequences
- Conceptualize addressing “failed” instrumentation states
- Test and Validation activities on existing model

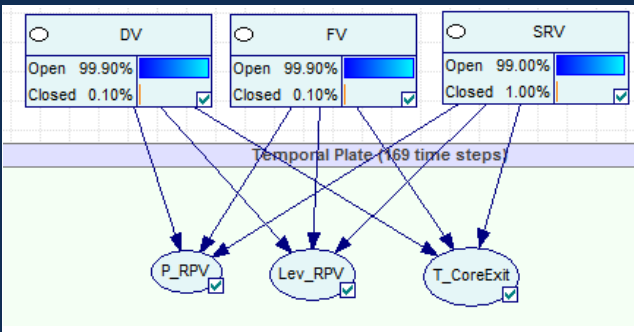
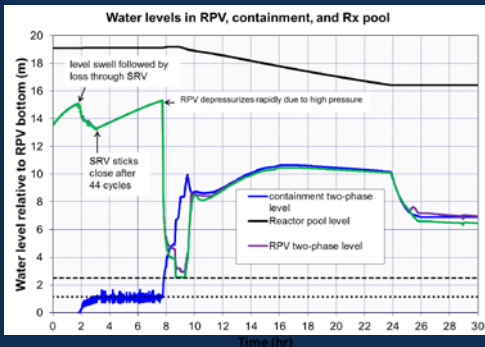
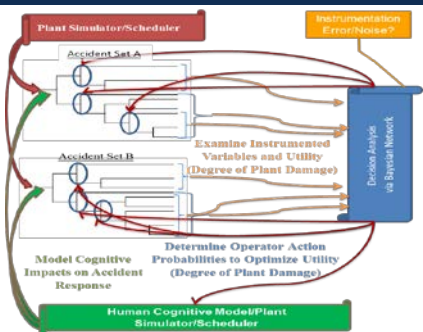
Loss Of Flow Diagnosis



Transient Overpower Diagnosis



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Ranked Observ...	Diagnostic Value
Lev_RPV	0.896
P_RPV	0.856
T_CoreExit	0.856
T_CoreExit (t=46)	0.651
P_RPV (t=160)	0.650
T_CoreExit (t=146)	0.615
P_RPV (t=79)	0.485
P_RPV (t=25)	0.464
T_CoreExit (t=72)	0.443
T_CoreExit (t=63)	0.439
Lev_RPV (t=160)	0.433
Lev_RPV (t=61)	0.421
T_CoreExit (t=44)	0.414
P_RPV (t=58)	0.406
Lev_RPV (t=156)	0.406
T_CoreExit (t=123)	0.382
T_CoreExit (t=108)	0.372
Lev_RPV (t=161)	0.361
T_CoreExit (t=98)	0.361
P_RPV (t=128)	0.359
T_CoreExit (t=70)	0.358

Thank you!

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