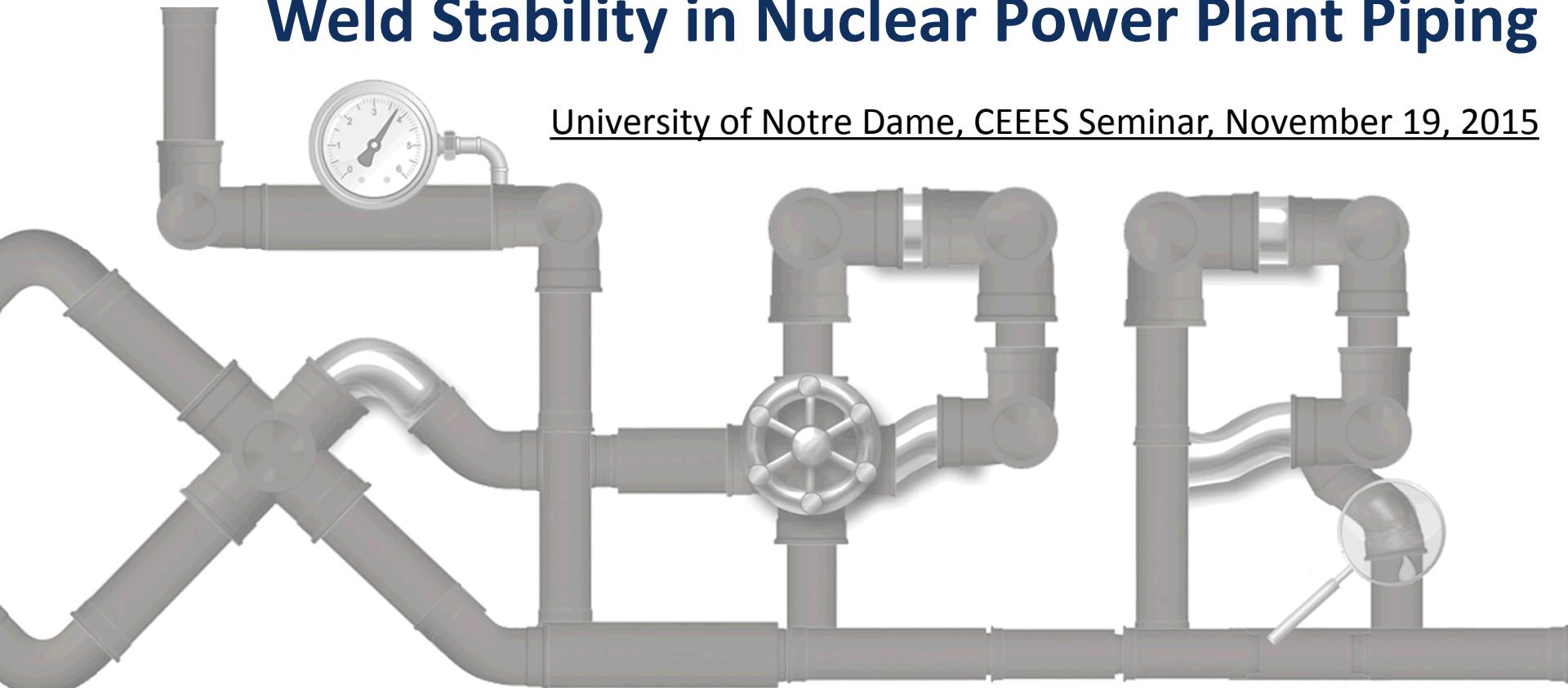


A Probabilistic Fracture Mechanics Approach to Weld Stability in Nuclear Power Plant Piping

SAND2015-10112PE

University of Notre Dame, CEEES Seminar, November 19, 2015



Scott E. Sanborn

sesanbo@sandia.gov

Department 6233 – Structural & Thermal Analysis

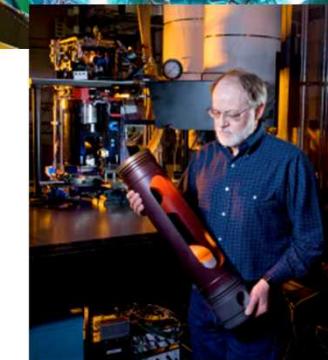
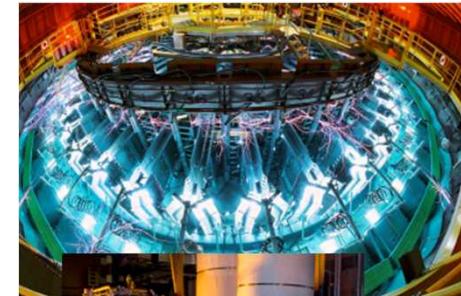
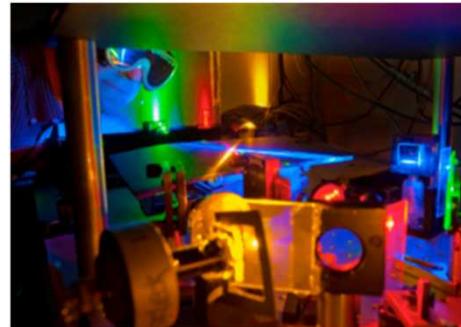
Sandia National Laboratories



Sandia National Laboratories is a multi-program laboratory managed and operated by Sandia Corporation, a wholly owned subsidiary of Lockheed Martin Corporation, for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-AC04-94AL85000.
SAND2015-XXXXX

“Exceptional service in the national interest”

Largest National Lab in the U.S. Department of Energy Laboratory Complex



Missions

- **Energy and climate**
- **Nuclear security engineering**
- **Defense systems**
- **Homeland security**

Sandia National Laboratories is a multi-program laboratory managed and operated by Sandia Corporation, a wholly owned subsidiary of Lockheed Martin Corporation, for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-AC04-94AL85000. SAND No. 2013-8635P

Energy & Climate



Energy Research

ARPAe, BES Chem Sciences, ASCR, CINT, Geo Bio Science, BES Material Science

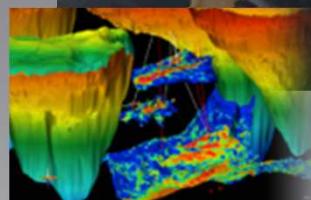
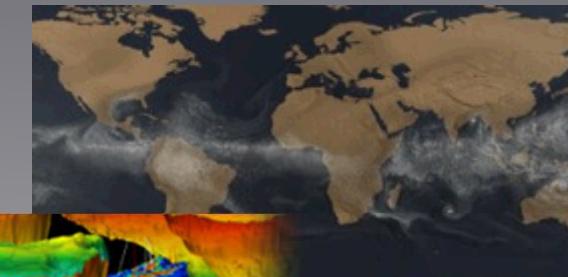
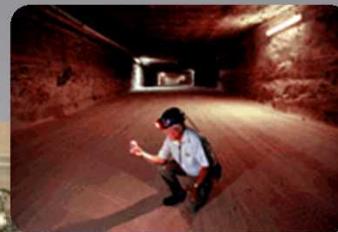
Renewable Systems & Energy Infrastructure

Renewable Energy, Energy Efficiency, Grid and Storage Systems



Nuclear Energy & Fuel Cycle

Commercial Nuclear Power & Fuel, Nuclear Energy Safety & Security, DOE Managed Nuclear Waste Disposal



Transportation Energy & Systems

Vehicle Technologies, Biomass, Fuel Cells & Hydrogen Technology



Outline

- Why nuclear power?
- The LBB problem
- Probabilistic Framework
- Deterministic Models
- Some results
- Wrap Up

Acknowledgements

Code Development Leads David Rudland – US NRC, Craig Harrington – EPRI



Computational Group

Remi Dingreville – Sandia National Laboratories

Mike McDevitt – EPRI

Cédric Sallaberry – EMC²

Aubrey Eckert-Gallup – SNL

Mariner, Paul – SNL

Patrick Mattie – SNL

Scott Sanborn – SNL

Dusty Brooks – SNL

Robert Kurth – EMC²

Dilip Dedhia – Structural Integrity Associates

David Harris – Structural Integrity Associates

Paul Williams – ORNL

Ken Geelhood – PNNL

Naveen Karri - PNNL

Ian Miller – GoldSim

Ryan Roper - GoldSim

Models Group

Marjorie Erickson – PEAI

Mike Benson – U.S. NRC

Mark Kirk – U.S. NRC

Kyle Schmitt – Dominion Engineering

John Broussard – Dominion Engineering

Glenn White – Dominion Engineering

Chris Casarez – Dominion Engineering

Do-Jun Shim – Emc2

Elizabeth Kurth – Emc2

Bud Brust – Emc2

Suresh Kalyanam – Emc2

Sean Yin – ORNL

Richard Bass – ORNL

Cliff Lange – Structural Integrity Associates

Steven Xu – Kinetics

Doug Scarth – Kinetics

Russ Cipolla – Aptech

Mike Hill – UC Davis

Steve Fyfitch – AREVA NP Inc.

Rick Olson – Battelle

Andrew Cox – Battelle

Lee Fredette – Battelle

Bruce Young – Battelle

Mark Dennis - EPRI

Carl Latiolais- EPRI

Thiago Seuaciuc-Osorio- EPRI

Inputs Group

Guy DeBoo – Exelon

Gary Stevens – U.S. NRC

Matt Homiack – U.S. NRC

Ashok Nana – AREVA NP Inc.

Nathan Palm – Westinghouse

QA Group

Nancy Kyle – Theseus

xLPR Team

Program Manager

Nate Leech – Demark

Program Integration Board

Denny Weakland - Ironwood Consulting

Bruce Bishop – Westinghouse

Rob Tregoning – U.S. NRC

Jay Collins – U.S. NRC

Nuclear Power - Importance

- **19% of electricity generated in the US^[1]**
- **63% of emissions-free electricity^[2]**
- **Licenses will be expiring; power lost if not renewed**
- **New plants are costly**

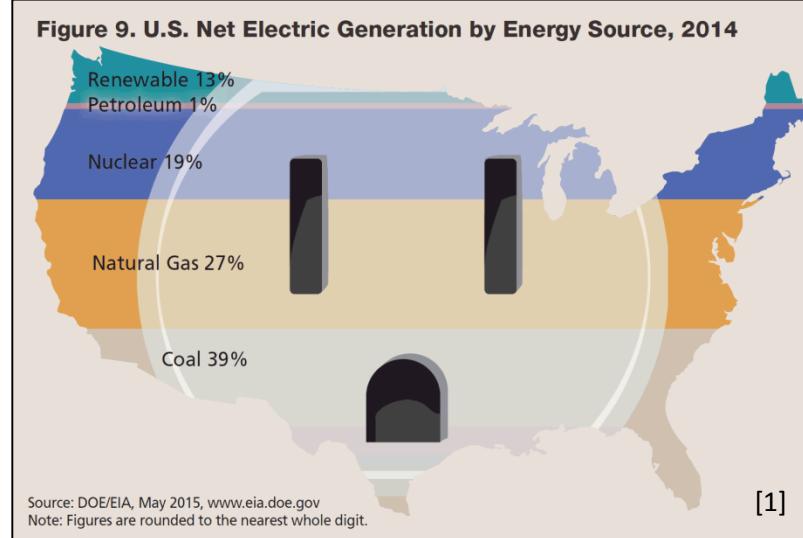
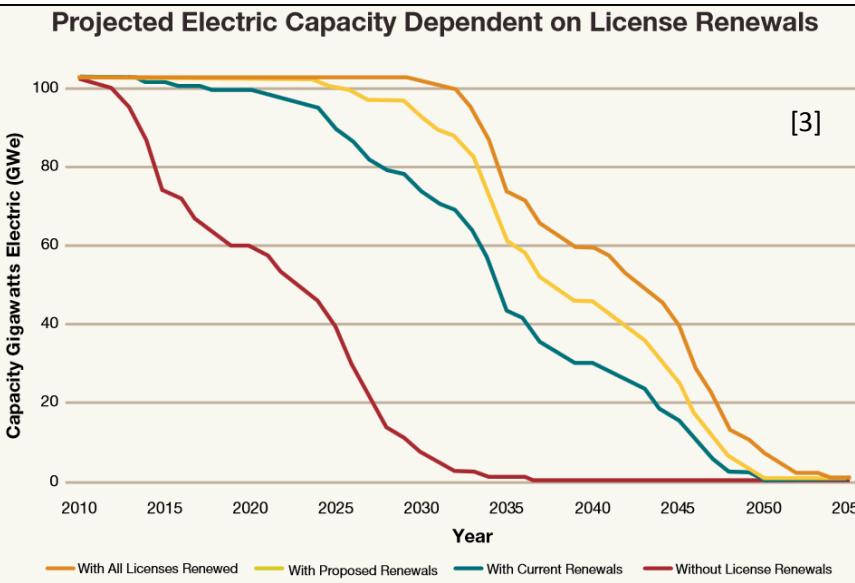
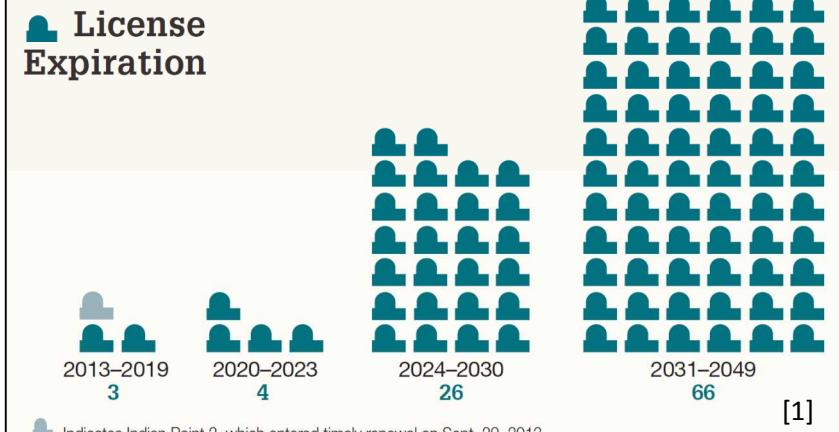


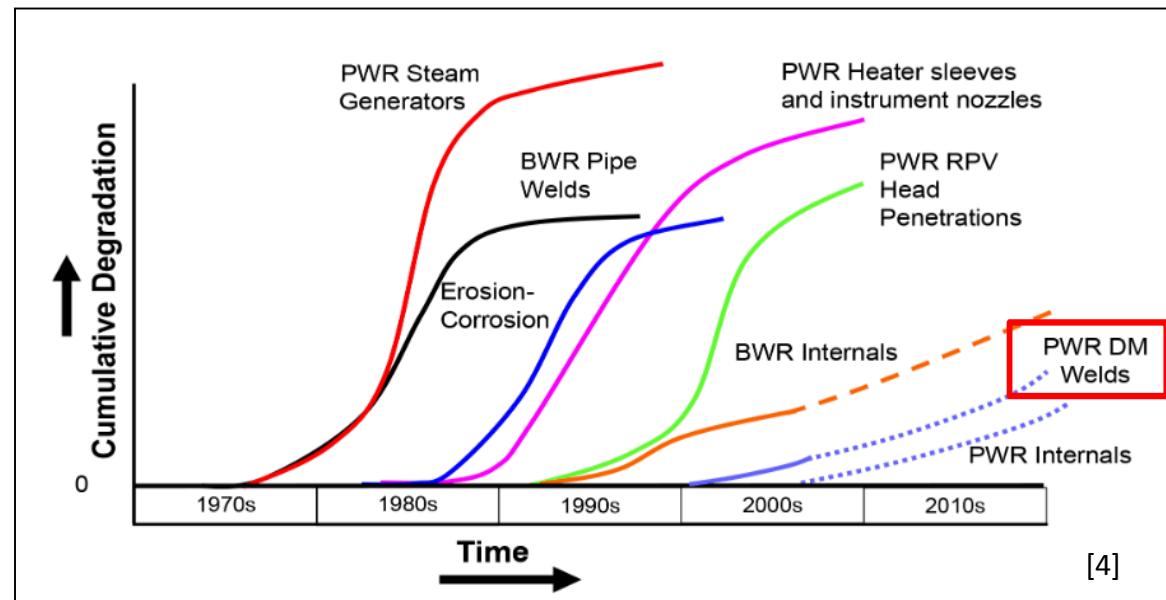
Figure 21. U.S. Commercial Nuclear Power Reactor Operating Licenses—Expiration by Year



Nuclear Power - Aging

- Many plants are old
- Materials aging and degradation have brought about some unique challenges

Figure 20. U.S. Commercial Nuclear Power Reactors—Years of Operation by the End of 2015 [1]

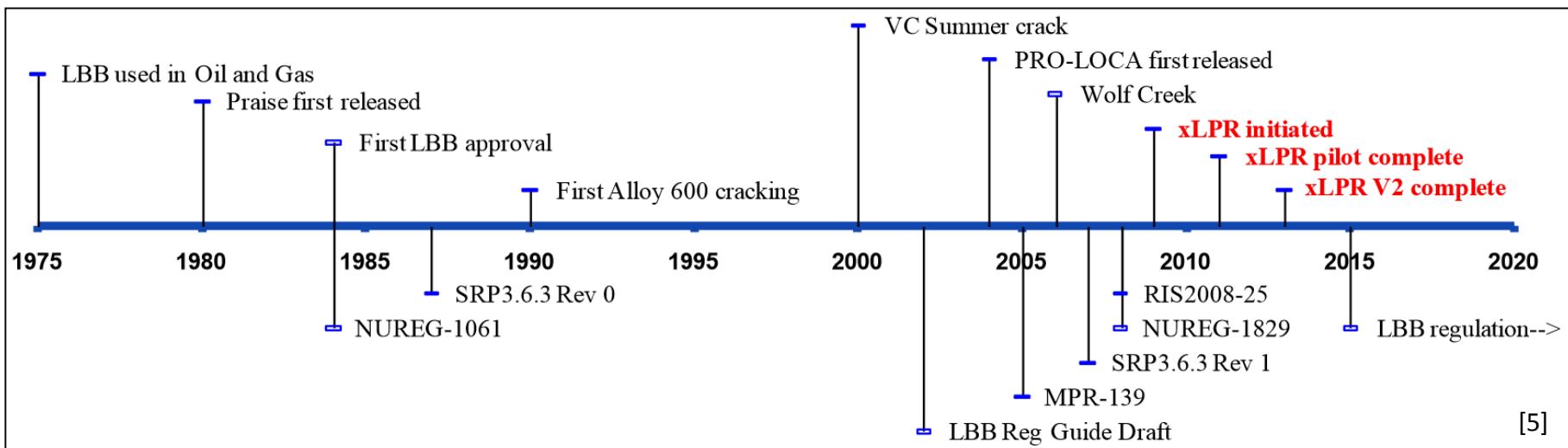


[4]

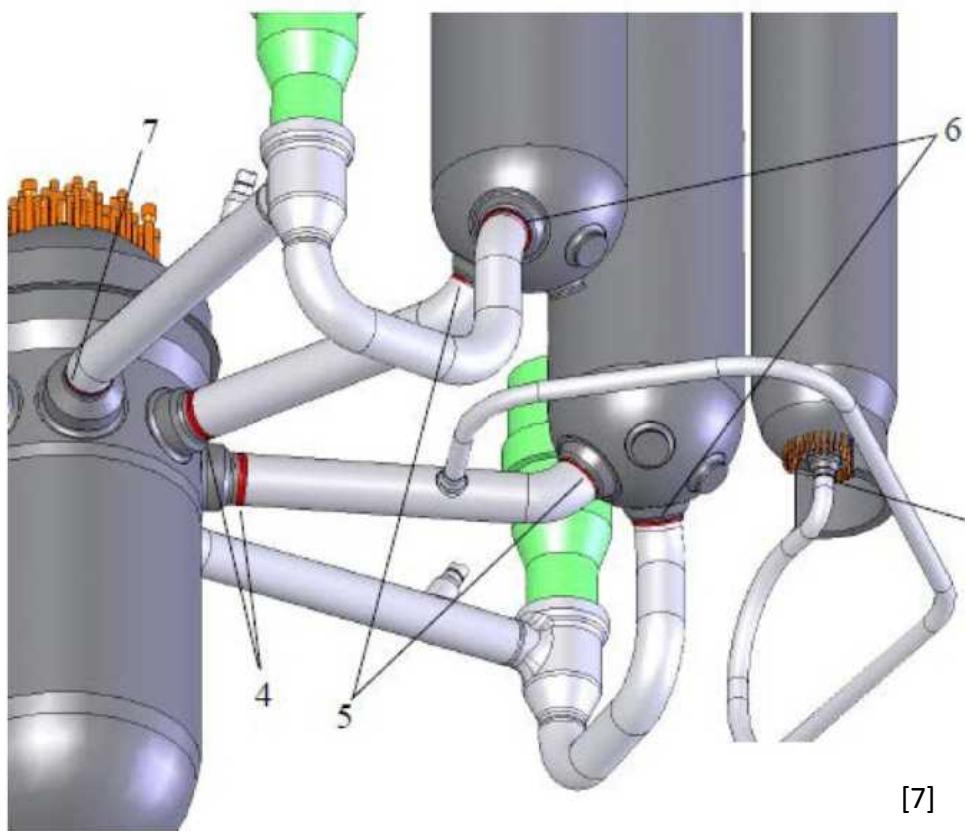
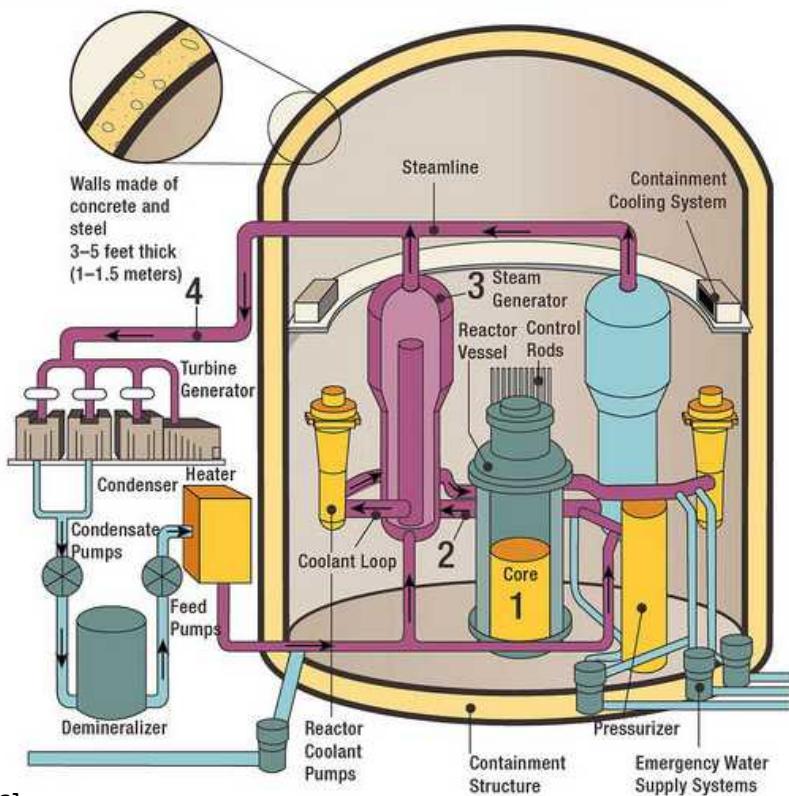
Leak Before Break (LBB) Problem

- 10CFR50 Appendix A General Design Criterion 4: local dynamic effects of piping rupture may be excluded from design basis
- LBB procedure used to justify approval of plant design excluding dynamic effects (no pipe whip restraints)
 - Assumes no active degradation mechanisms exist
 - However, after approval of LBB primary water stress corrosion cracking (PWSCC) was observed in many weld locations
- **eXtremely Low Probability of Rupture (xLPR)** ← Language in regulation

Probabilistic fracture mechanics tool that fully addresses and quantifies uncertainties and may be used to directly assess compliance with GDC-4

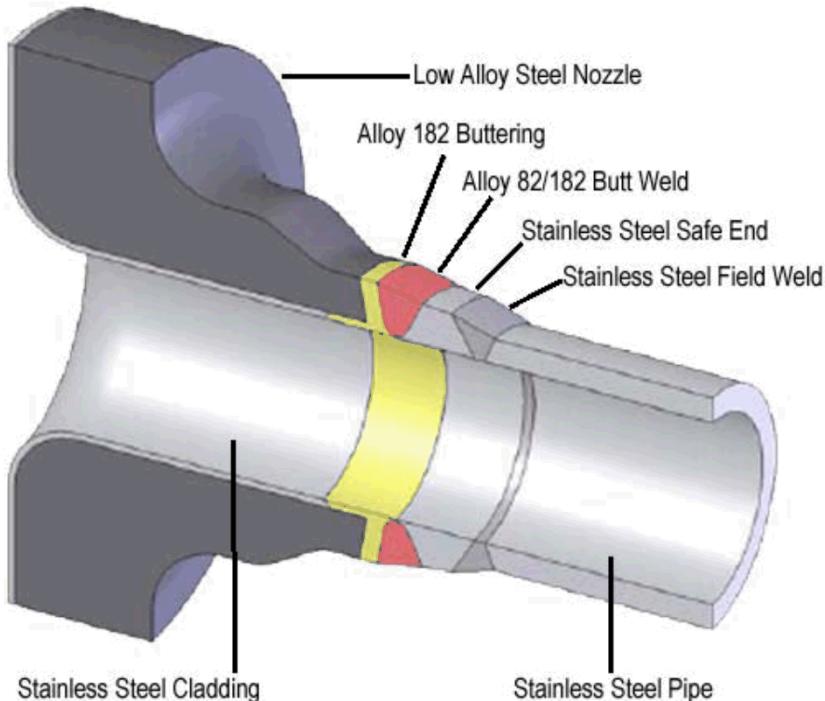


Primary Water Reactor Welds



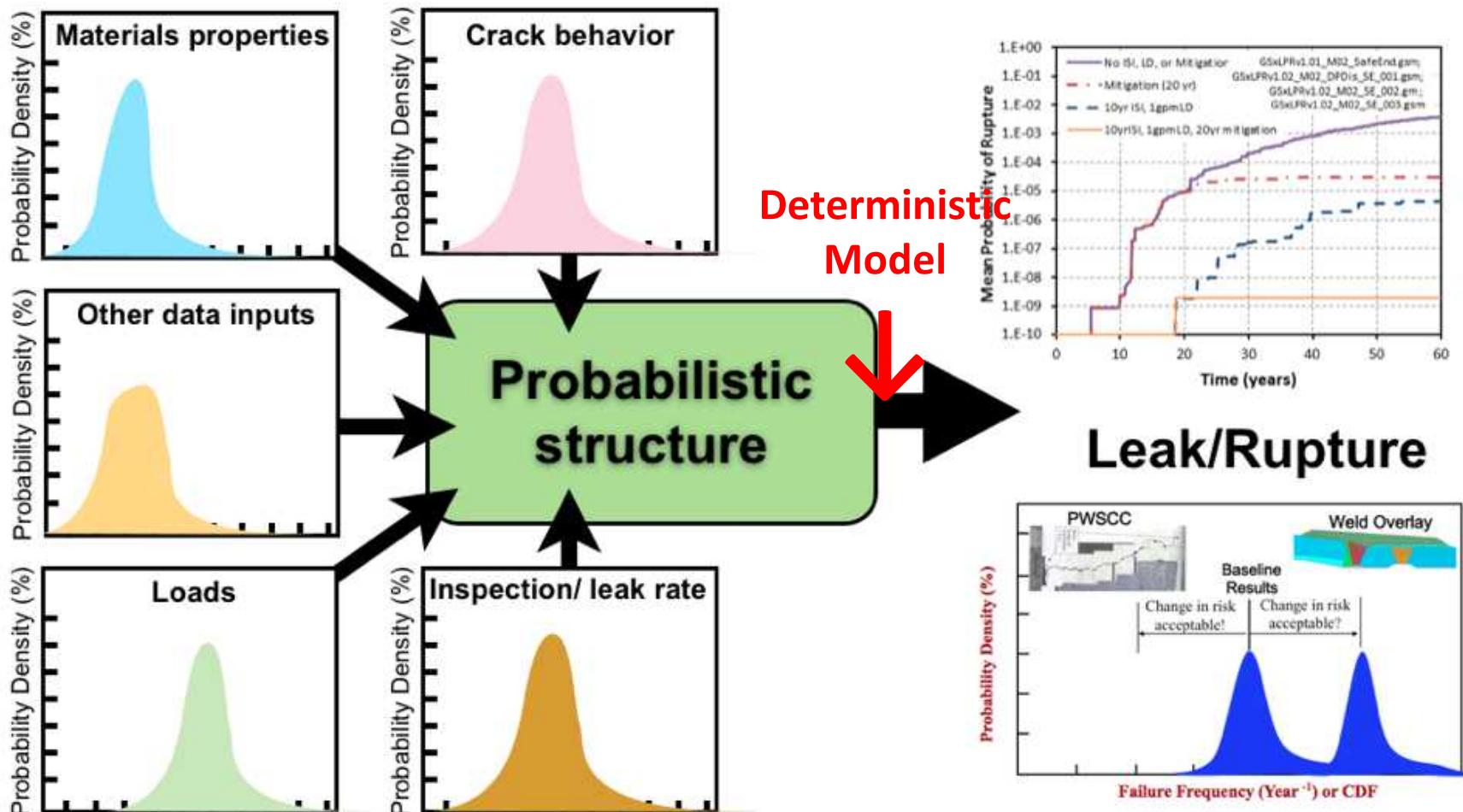
Example Dissimilar Metal Weld

Pressurizer Surge Nozzle



Probabilistic Fracture Mechanics

- To demonstrate an extremely low probability of rupture a probabilistic approach is necessary

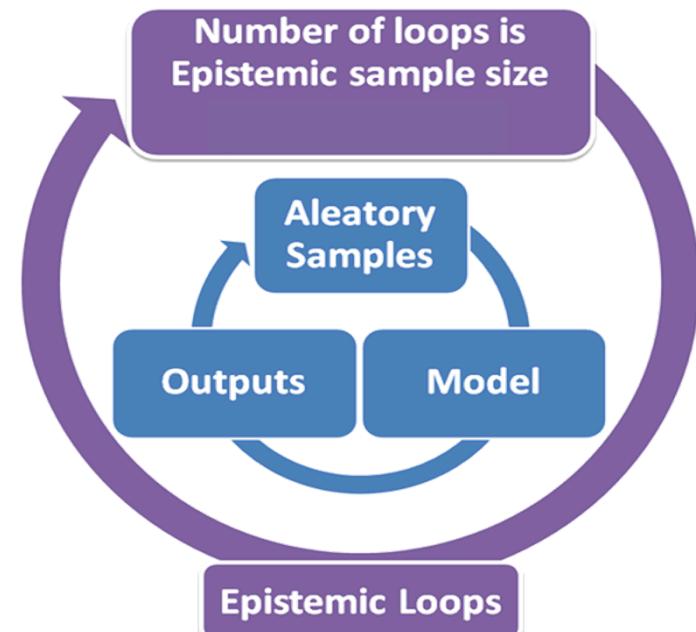


Uncertainty representation in xLPR

- **Aleatory uncertainty**
 - Inherent randomness. Represented with probability distributions.
- **Epistemic uncertainty**
 - Uncertainty originated from lack of knowledge on a fixed quantity. Represented with probability distributions.
- **Experts disagree on how to categorize certain inputs**
- **Inner aleatory loop**
- **Outer epistemic loop**
 - Allows evaluation of epistemic (model) uncertainty.

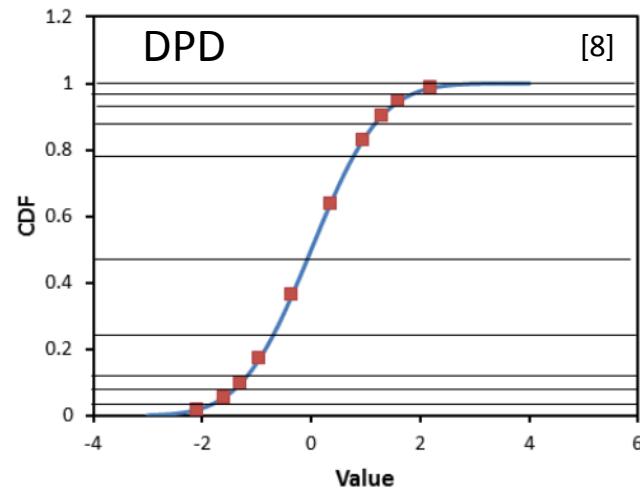
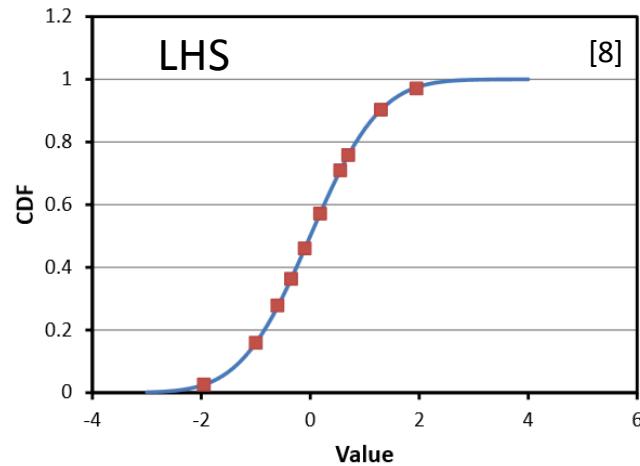
| Aleatory (Irreducible) |
|---|
| <ul style="list-style-type: none"> • Crack size • POD detection • Material properties • Crack growth parameters (Q/R,c,P) |

| Epistemic (Lack of knowledge) |
|---|
| <ul style="list-style-type: none"> • Loads • WRS • Crack growth (fweld) • Crack initiation parameters • POD parameters |



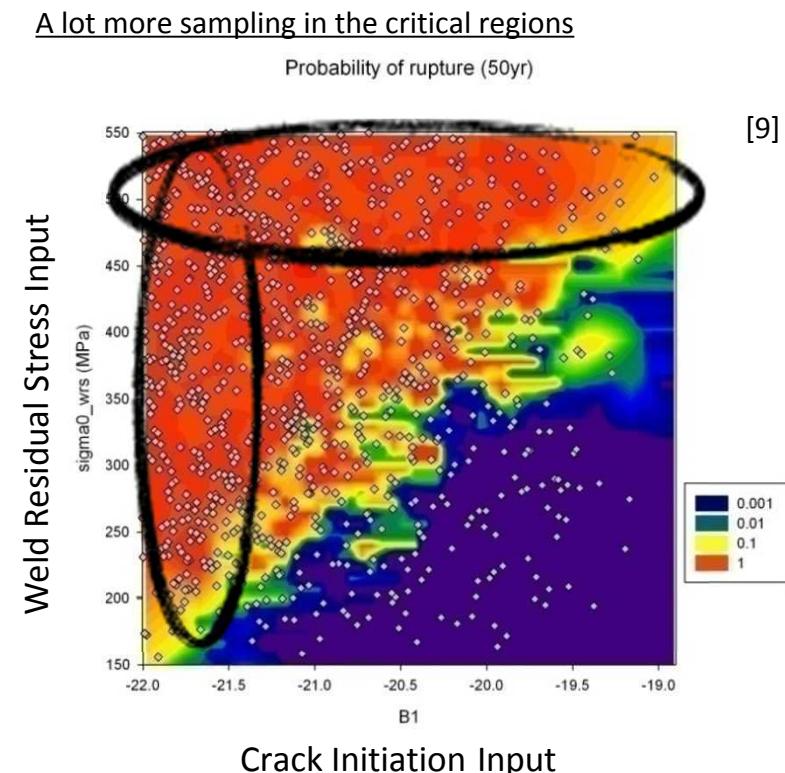
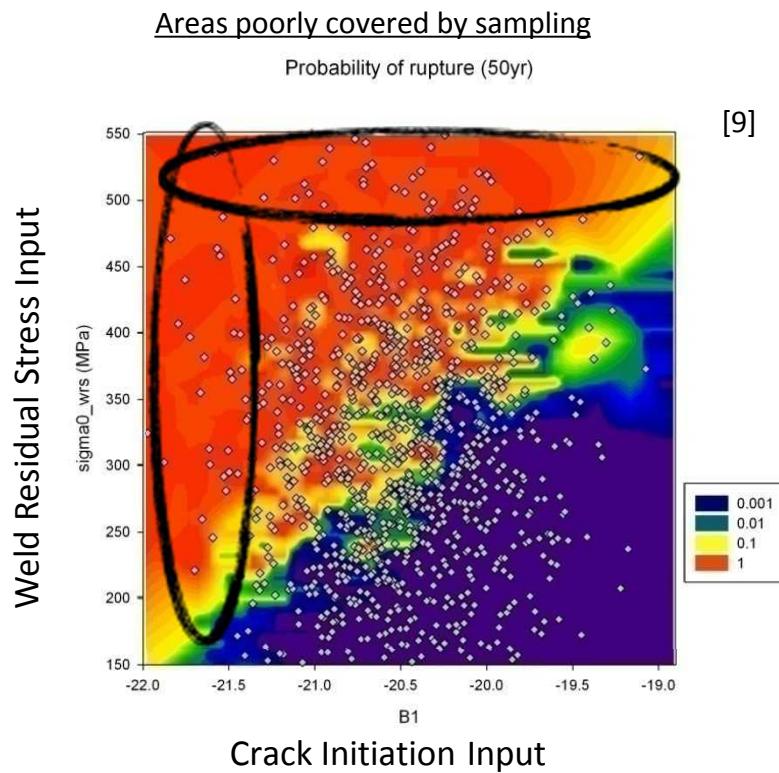
xLPR V2.0 – Sampling strategy

- For each loop, the xLPR allows:
 - Simple random sampling (sometimes called Monte Carlo)
 - Latin Hypercube Sampling (LHS)
 - dense stratification of each input into equal probably intervals, then **random sample** in interval
 - Discrete Probability Distribution (DPD)
 - dense stratification of each input into equal probably intervals, then **use conditional mean** in interval
 - better multidimensional coverage than LHS
 - better if variables are important *conjointly* and a reasonable range of values (not as dense as LHS) is required/sufficient
 - Importance sampling applied to selected values



xLPR V2.0 – Sampling strategy

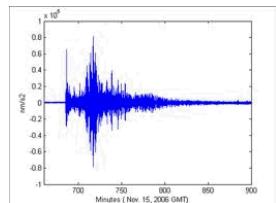
- For each loop, the xLPR allows:
 - Importance sampling applied to selected values
 - Define regions to focus sampling on but reduce the weights applied to those results



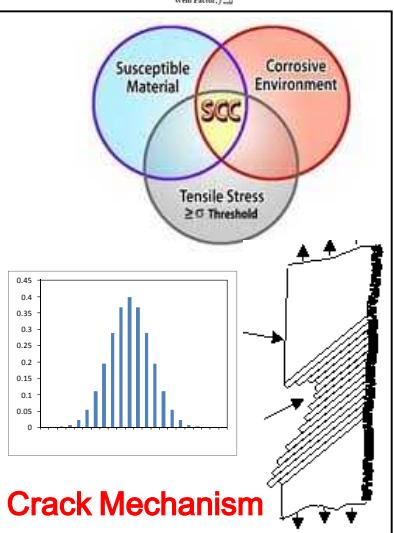
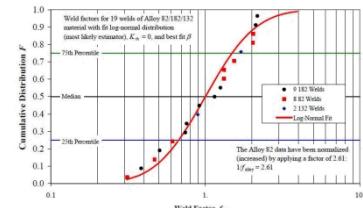
xLPR V2.0 – Conceptual flow

Deterministic Model Within Probabilistic Framework

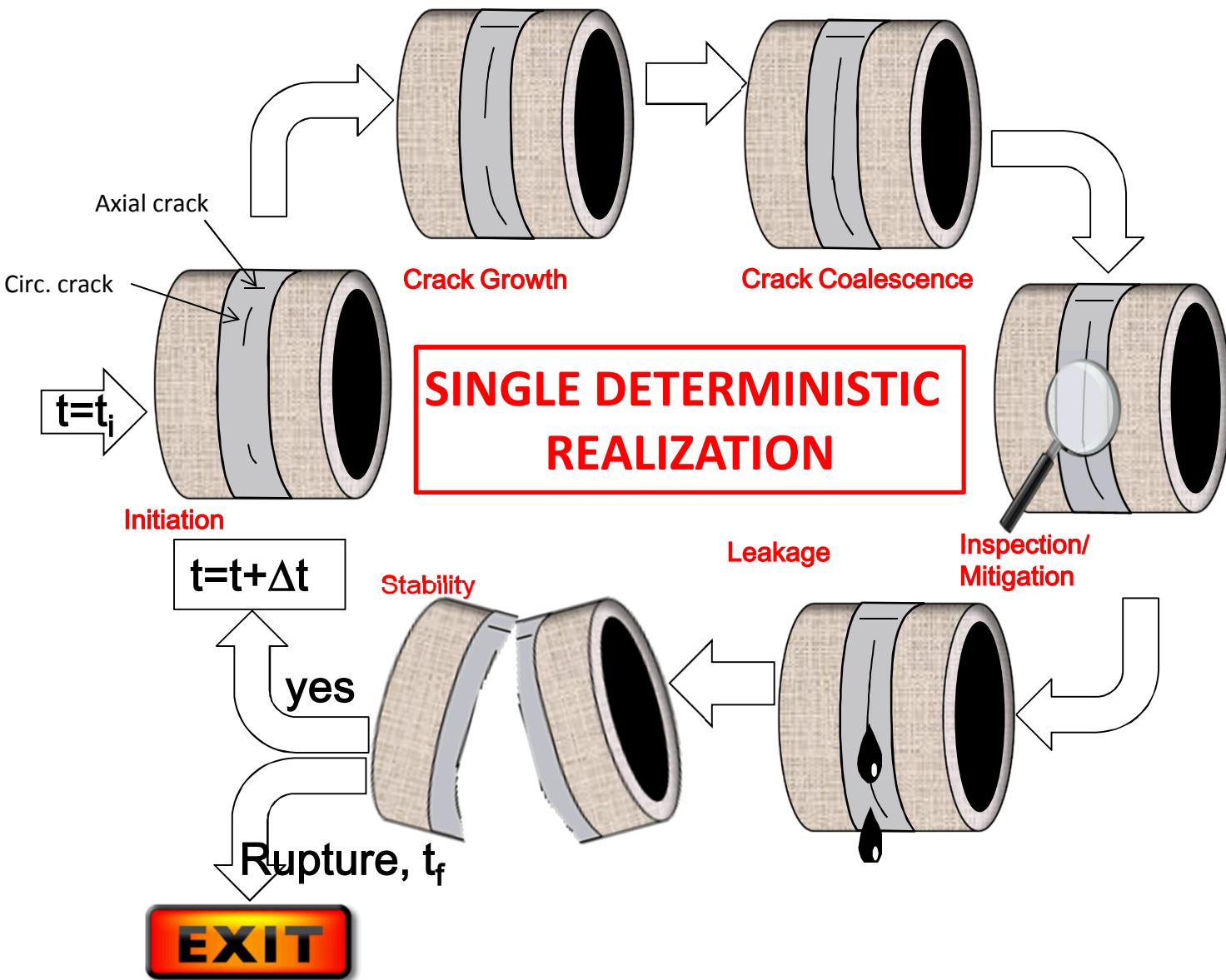
Loads



Material Properties

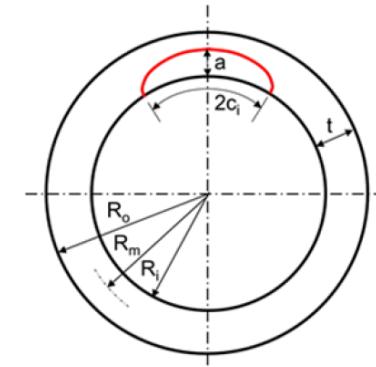


Crack Mechanism

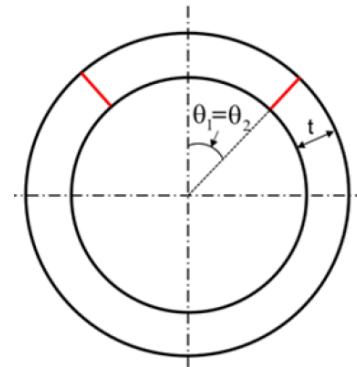


Deterministic Model

- Must be realistic representation of physical phenomena *but* also fast enough to run $\sim 10^6$ realizations
- Key assumptions
 - Cracks can be either circumferential (circ.) or axial oriented cracks
 - Idealized crack shapes
 - No interaction between circ. and axial cracks
- Each deterministic model individually validated against available data (field data, lab data, other models, etc.)
- Probabilistic framework ties deterministic models together and evolves cracks through time



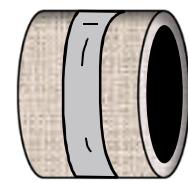
Ideal surface crack
(SC)



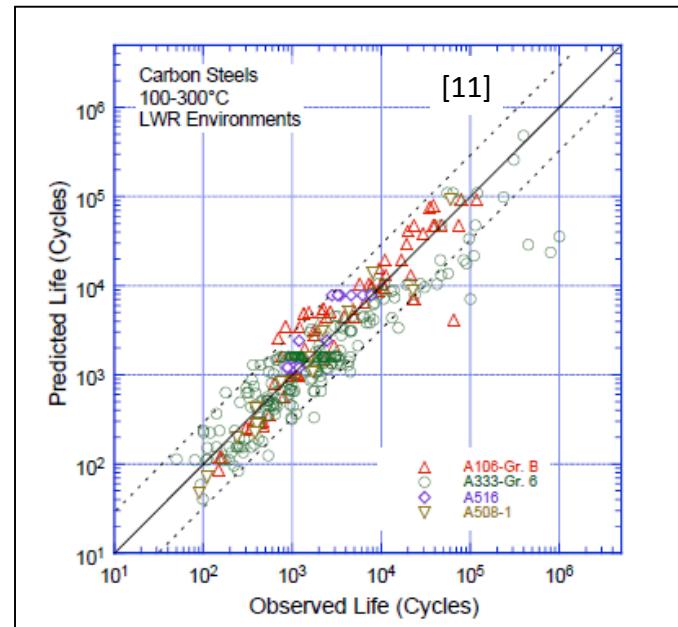
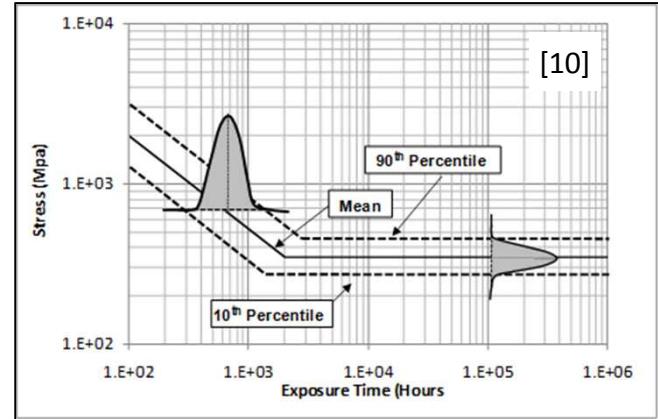
Ideal through wall
crack (TWC)

Deterministic Model

Crack Initiation

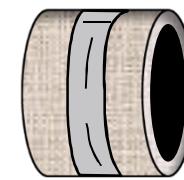


- PWSCC
 - 3 models available
 - Calibrated to field data
- Fatigue
- Initial Flaw (pre-existing)
- Flaw is initiated at engineering size



Deterministic Model

Crack Growth



- Loads – WRS, bending, axial, transients, etc.
- Stress intensity factors – “K”
 - Many solution methods available for ideal crack geometries
- K drives Growth
 - General PWSCC form

$$\dot{a}_{PWSCC} = \begin{cases} \alpha f_{comp} f_{flaw} e^{-\frac{\varrho_g}{R_{gas}} \left(\frac{1}{T} - \frac{1}{T_{ref}} \right)} (K_I - K_{th})^\beta & K_I > K_{th} \\ 0 & K_I \leq K_{th} \end{cases}$$

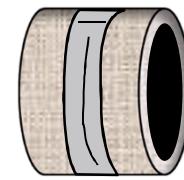
- General fatigue form

$$\dot{a}_{fatigue,i} = \frac{1}{\Delta t} \left[\frac{da_i}{dN} \cdot N_{cyc,i} \right]$$

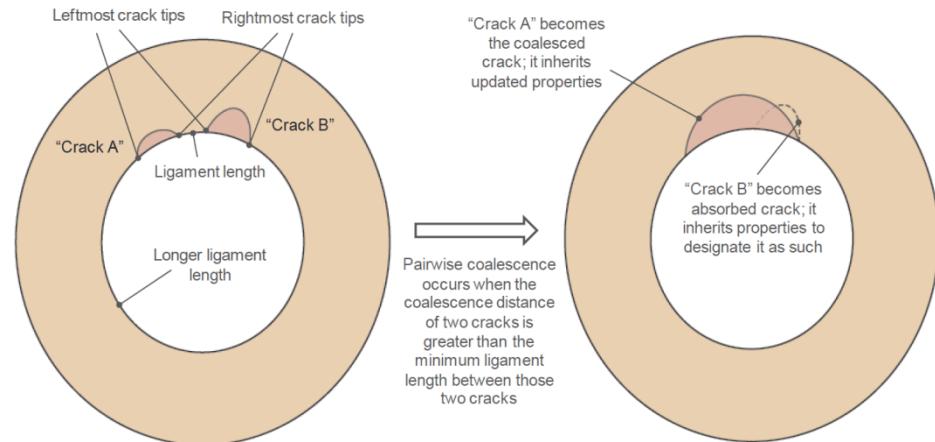
$$\frac{da_i}{dN} = C_{cstm} \tau_{r,i}^{p_{cstm}} \left(\frac{\Delta K_i}{(1 - a_{cstm} R_i)^{b_{cstm}}} \right)^{m_{cstm}}$$

Deterministic Model

Crack Coalescence



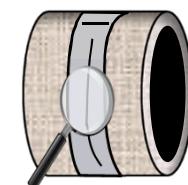
- Only applicable to circumferential cracks
- Cracks assumed to be in the same plane
- Rule based to determine if two cracks are close enough to interact



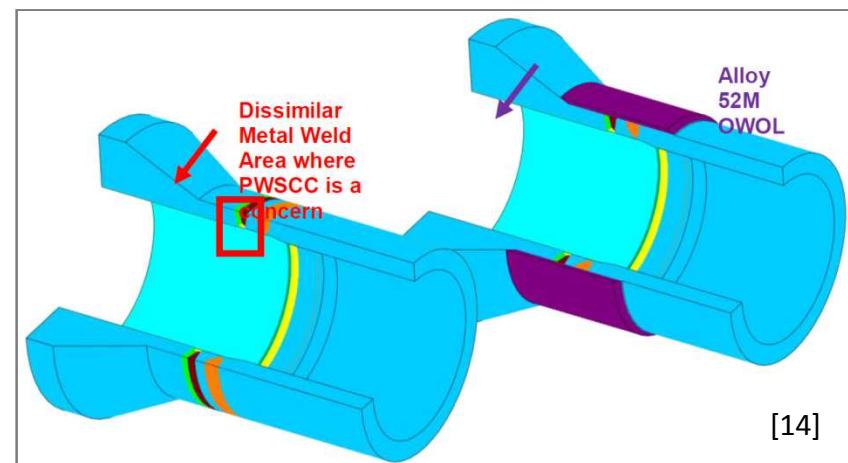
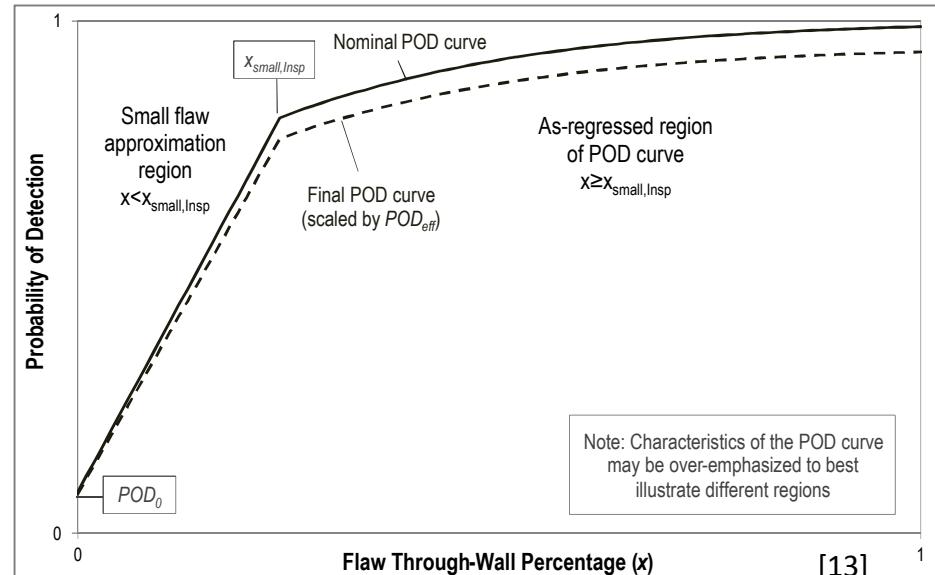
[12]

Deterministic Model

Inspection and Mitigation

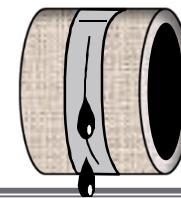


- **Ultrasonic test inspection**
- **Given a flaw of certain depth:**
 - What is the probability of detection?
 - What is the “depth” evaluated from the inspection?
- **Repair weld if evaluated depth is > some threshold**
- **Mitigation**
 - **Stress/structural: Inlay, Onlay, MSIP**
 - **Chemistry: H₂, Zinc**

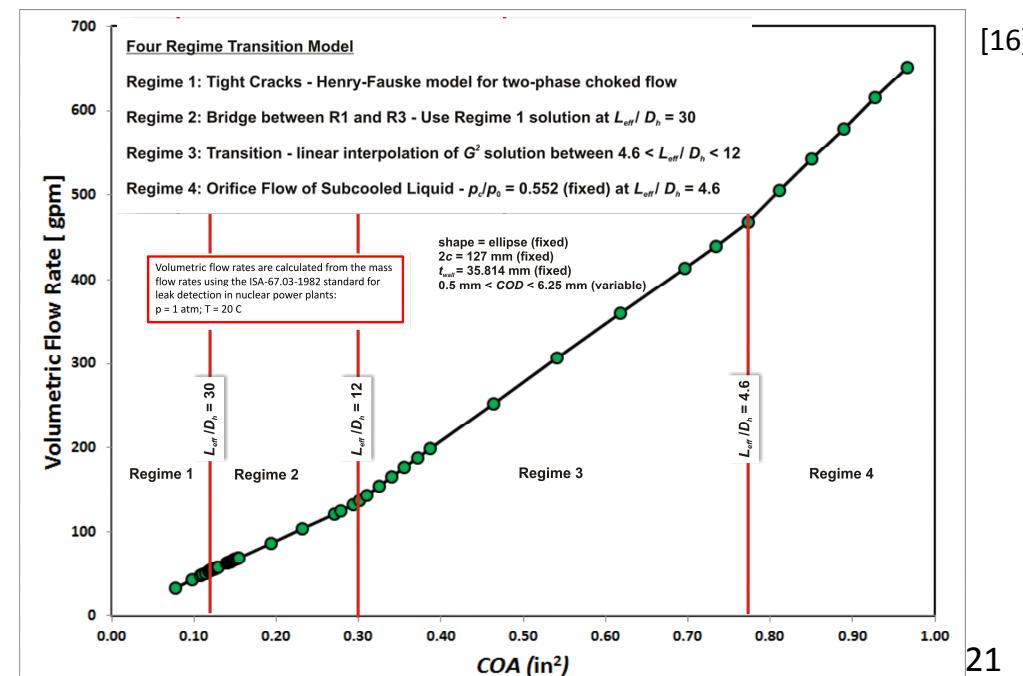
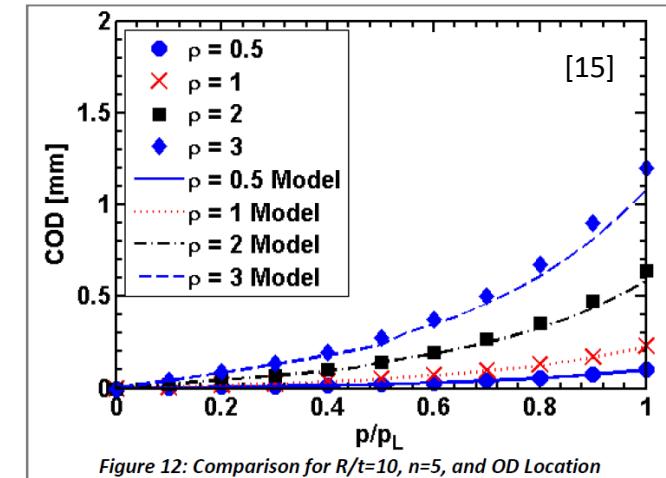


Deterministic Model

Leak Detection

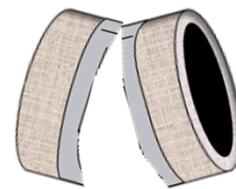


- For ideal shaped TWCS under given loads:
 - crack opening displacement (COD) obtained by analytical solutions that have been benchmarked against finite element analyses
 - COD → COA
- LEAPOR (leak rate preprocessor) generates look-up tables prior to running simulation



Deterministic Model

Stability

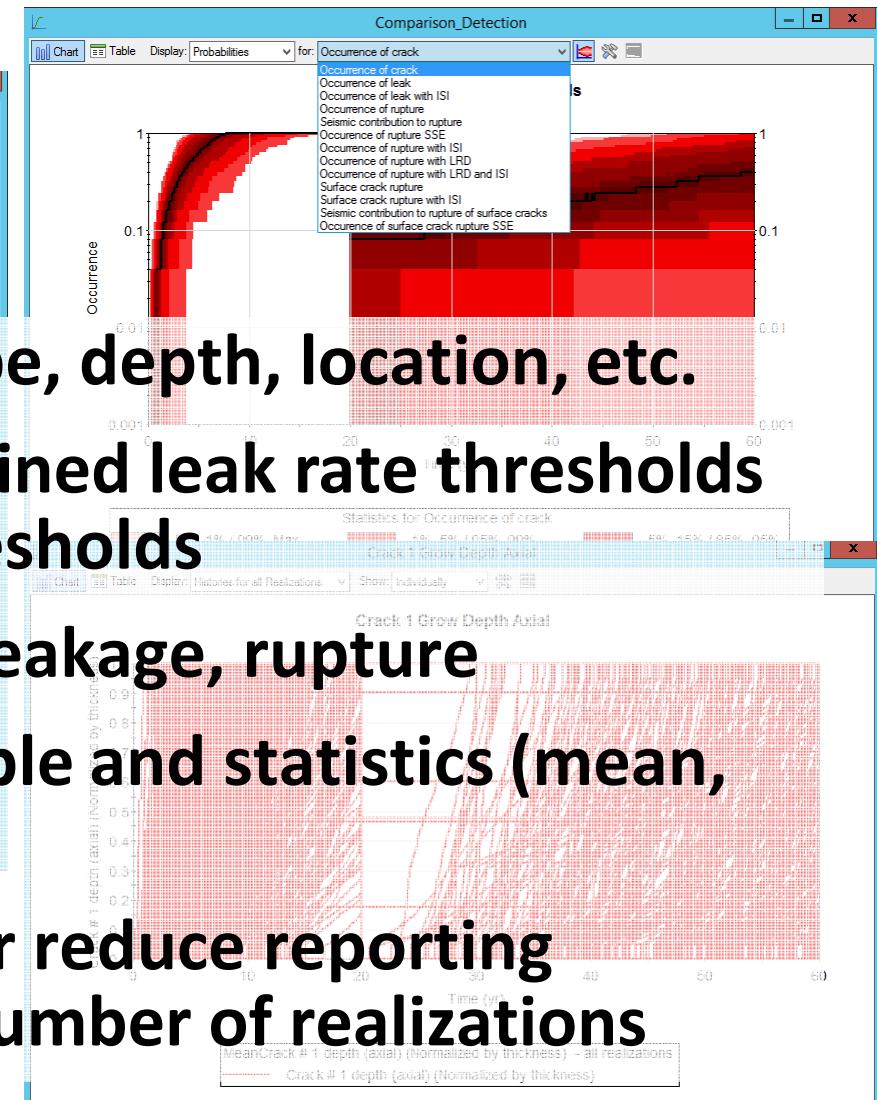
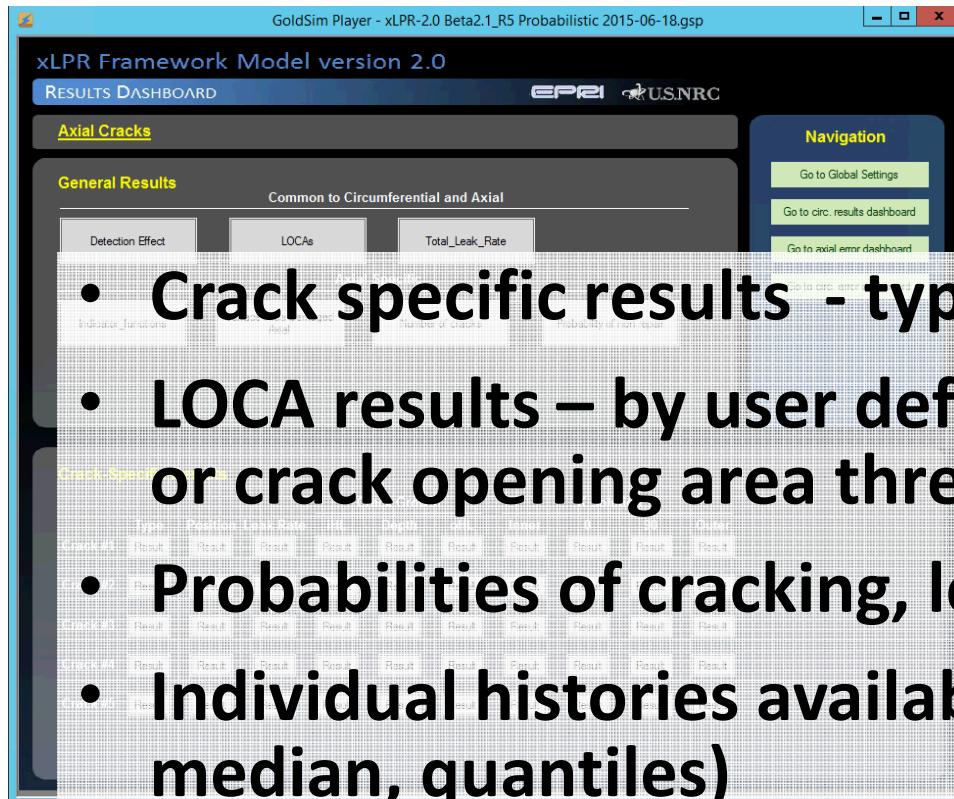


- Loads on cracked pipe are greater than the remaining section can maintain
 - With and without Safe Shutdown Earthquake loads
- Circumferential cracks
 - Surface Crack (SC) – Net Section Collapse
 - Through Wall Crack (TWC) – Net Section Collapse and/or elastic-plastic tearing instability
- Axial cracks
 - SC – limit load analysis in Ductile Fracture Handbook
 - TWC – limit load and elastic-plastic tearing instability

Deterministic Model – Crack Stability

- **Circumferential cracks**
 - Surface Crack (SC) – Net Section Collapse
 - Through Wall Crack (TWC) – Net Section Collapse and/or elastic-plastic tearing instability
- **Axial cracks**
 - SC – limit load analysis in Ductile Fracture Handbook [21]
 - TWC – limit load and elastic-plastic tearing instability
- **For Dissimilar Metal (DM) welds material properties used in stability modules are a combination of the base metal material properties**
- **Stability checked with/without SSE loads**

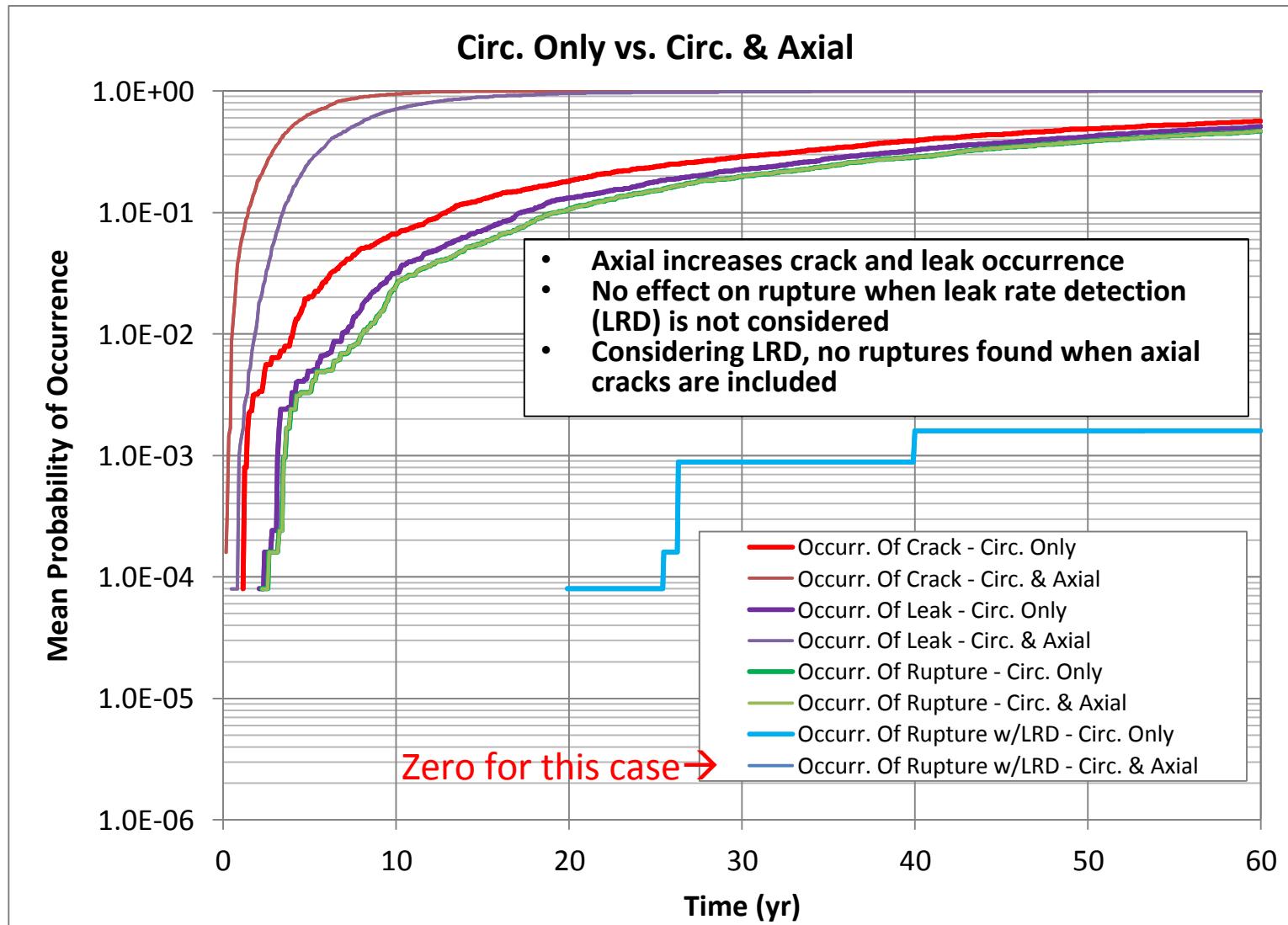
Results Available from xLPR Runs



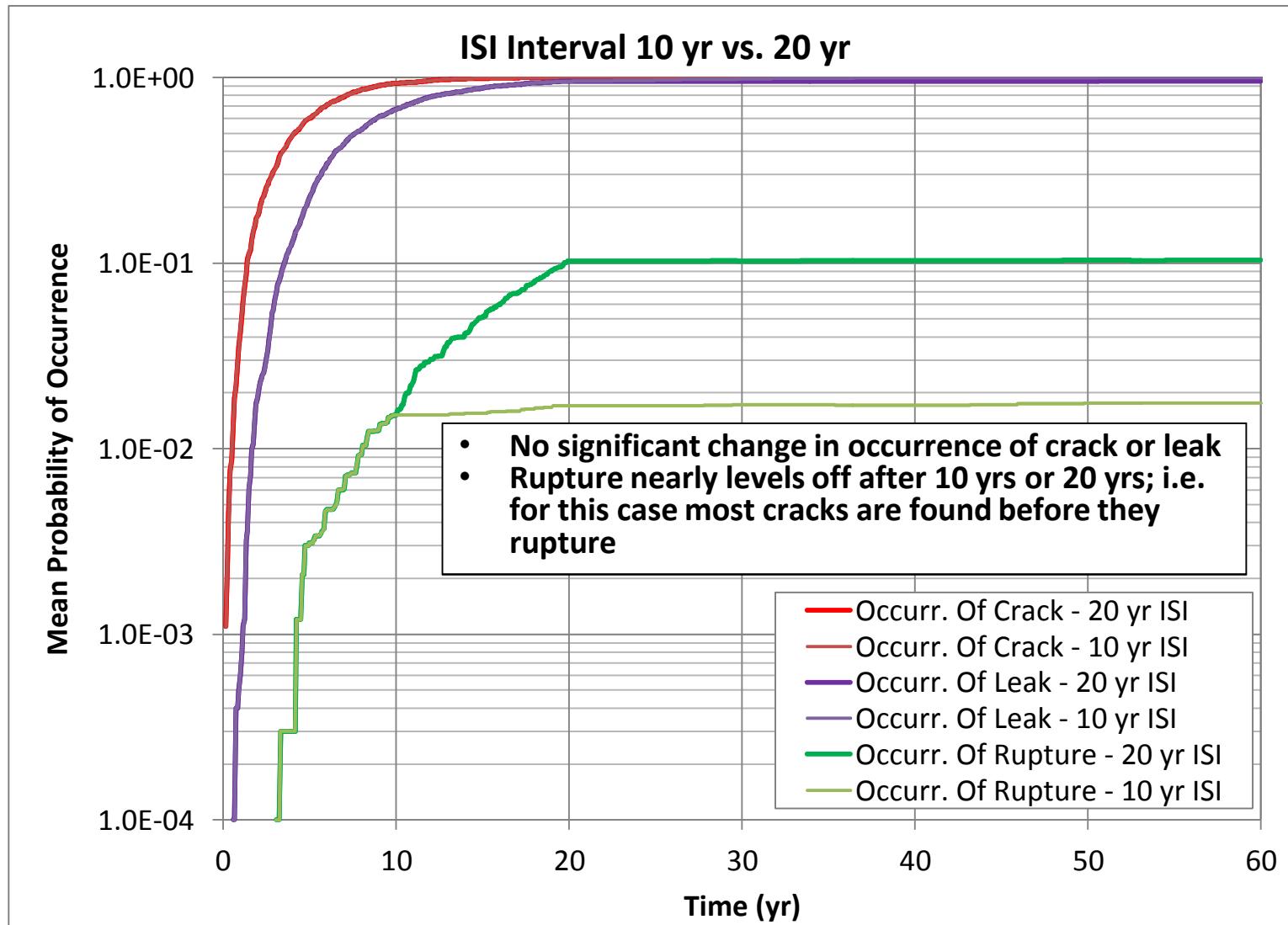
Some Example Results

- The following results are meant to demonstrate different options available in the code and their effects on the important results.
 - Beta version of the xLPR V2.0 code – bugs may still exist
 - Not all inputs may be realistic – input databases still being created
- The results presented are based on a dissimilar metal weld for a reactor pressure vessel nozzle
 - Base metals - SA-508 and SA-182
 - Weld material – Inconel 182
- Same geometry, materials, loads (except transients), WRS, and sample size used for all the following demonstrations

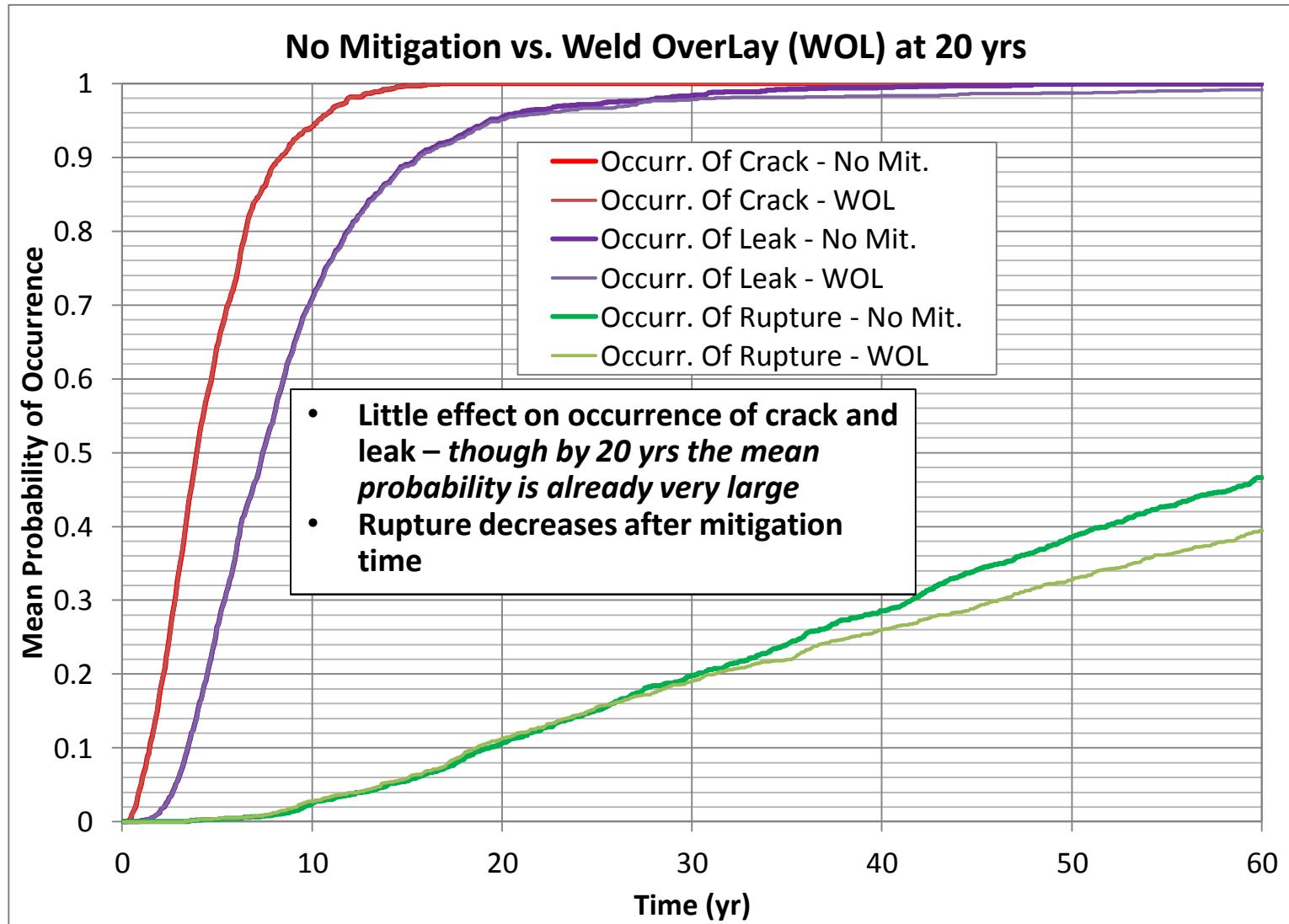
Effect of Including Axial Cracks



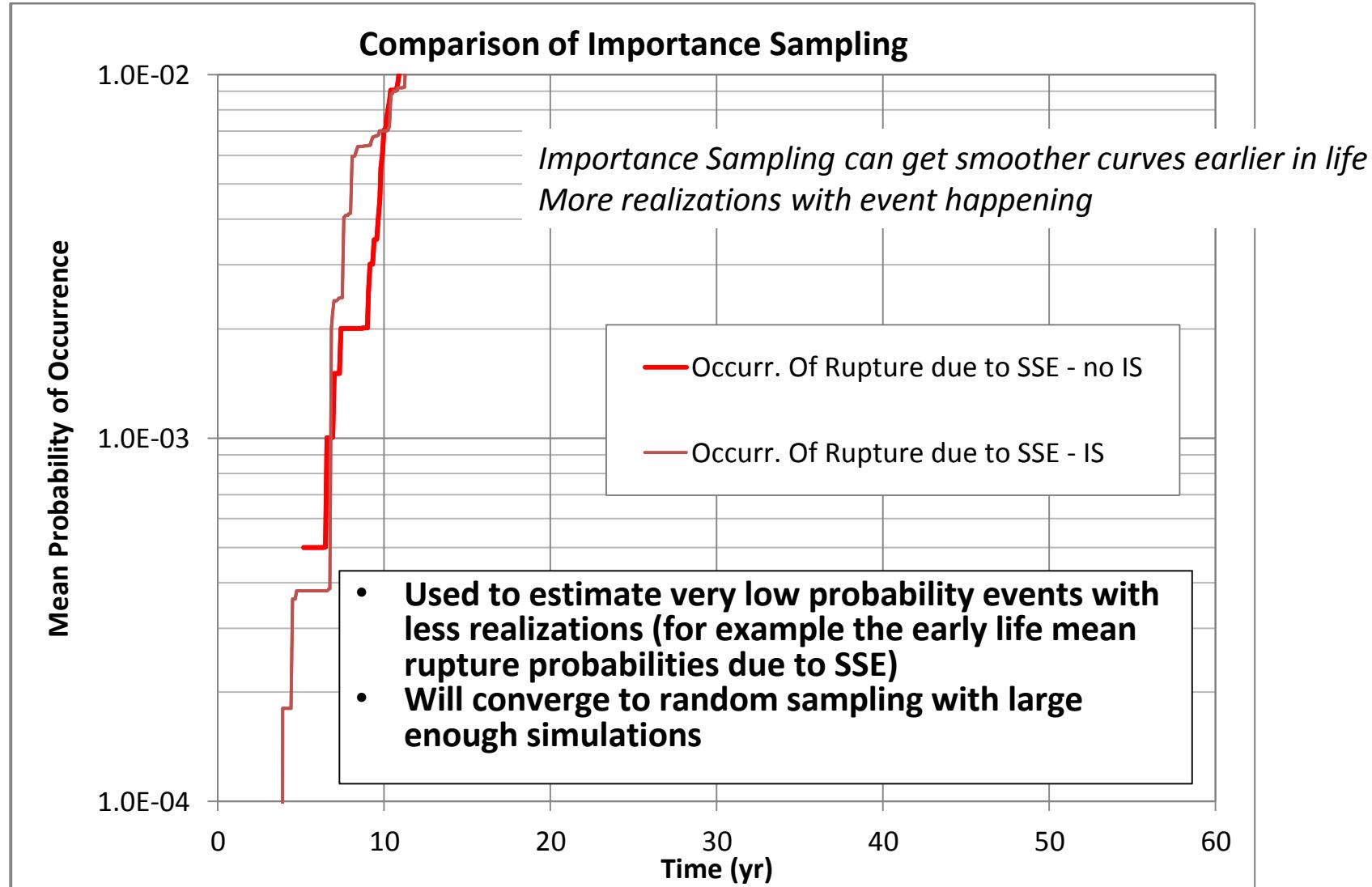
Effect of ISI Interval



Effect of Mechanical Mitigation



Use of Importance Sampling



Wrap Up

- **LBB Problem**
- **Probabilistic Approach**
- **Deterministic Models**
- **Some results**
- **Questions**

References



1. NUREG-1350, "Information Digest, Vol. 27, Section 3: Nuclear Reactors", U.S. Nuclear Regulatory Commission, Washington, DC
<http://pbadupws.nrc.gov/docs/ML1525/ML15254A456.pdf>
2. M Korsnick, "The Role of Nuclear in the Power System of the Future", Presented at *EPRI 2014 Summer Seminar, The Integrated Grid*.
http://www.epri.com/About-Us/Documents/Summer_Seminar_2014/Session%204.1_Korsnick_FINAL.pdf
3. NUREG-1350, "Information Digest, Vol. 27, Section 7: Appendices", U.S. Nuclear Regulatory Commission, Washington, DC
<http://pbadupws.nrc.gov/docs/ML1525/ML15254A460.pdf>
4. "Renewing Licenses for the Nation's Nuclear Power Plants – A Report by the APS Panel on Public Affairs" 2013, American Physical Society, College Park, MD. <http://www.aps.org/policy/reports/popa-reports/upload/nuclear-power.pdf>
5. M Kirk, Status of Recent Practical Applications of PFM to the Assessment of Structural Systems in Nuclear Plants in the United States" Presented at *International Symposium on Improvement of Nuclear Safety Using Probabilistic Fracture Mechanics Organized by the PFM Subcommittee, Atomic Energy Research Committee of the Japan Welding Engineering Society*, October 2014, Tokyo, Japan.
<http://pbadupws.nrc.gov/docs/ML1429/ML14290A069.pdf>
6. <http://www.nrc.gov/reactors/pwrs.html>
7. EJ Sullivan, MT Anderson, "Assessment of the Mechanical Stress Improvement Process for Mitigating Primary Water Stress Corrosion Cracking in Nickel Alloy Butt Welds in Piping Systems Approved for Leak-Before-Break". PNNL-22070, January 2013, Pacific Northwest National Laboratory, Richland, WA.
8. DL Rudland, C Harrington, "xLPR Version 1.0 Report: Technical Basis and Pilot Study Problem Results". 2011. U.S. Nuclear Regulatory Commission, Washington, DC & Electric Power Research Institute, Palo Alto, CA.
<http://pbadupws.nrc.gov/docs/ML1106/ML110660292.pdf>
9. PD Mattie, CJ Sallaberry, JC Helton, and DA Kalinich, "Development, Analysis, and Evaluation of a Commercial Software Framework for the Study of Extremely Low Probability of Rupture (xLPR) Events at Nuclear Power Plants" SAND2010-8480, December 2010, Sandia National Laboratories, Albuquerque, NM. <http://pbadupws.nrc.gov/docs/ML1107/ML110700019.pdf>
10. xLPR-SRD-CI-SCC, *xLPR Software Requirements Specification for Crack Initiation PWSCC*, Version 4.0, 2014.
11. O.K. Chopra, W.J. Shack, *Effect of LWR Coolant Environments on the Fatigue Life of Reactor Materials*, NUREG/CR-6909, U.S. Nuclear Regulatory Commission, Washington, DC, 2007 February.
12. xLPR-SRD-Coalescence, *xLPR Software Requirements Description for the Crack Coalescence Module*, Version 2.3, 2015.
13. xLPR-SRD-ISI, *xLPR Software Requirements Description for ISI Module*, Version 2.2, 2014.
14. Mitigation Team, xLPR Models Group, *White Paper on Incorporation of PWSCC Mitigation Strategy into xLPR Version 2.0*, 2012.
15. xLPR-MVR-AxCOD, *xLPR Module Validation Report for Axial Through-Wall Crack-Crack Opening Displacement Module*, Version 1.0, 2015.
16. xLPR-SDD-LRM, *xLPR Software Design Description for Leakage Rate Module*, Version 2.0, 2015.