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# Breakout discussion: Science enabled by tritium on Z

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Z Fundamental Science Workshop  
Stagnation Breakout Session  
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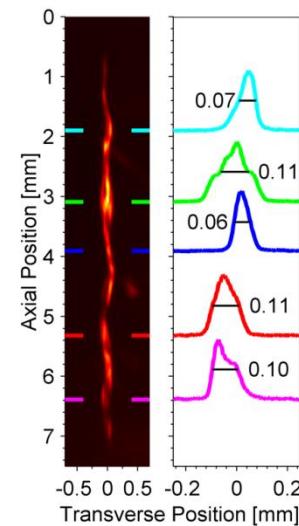
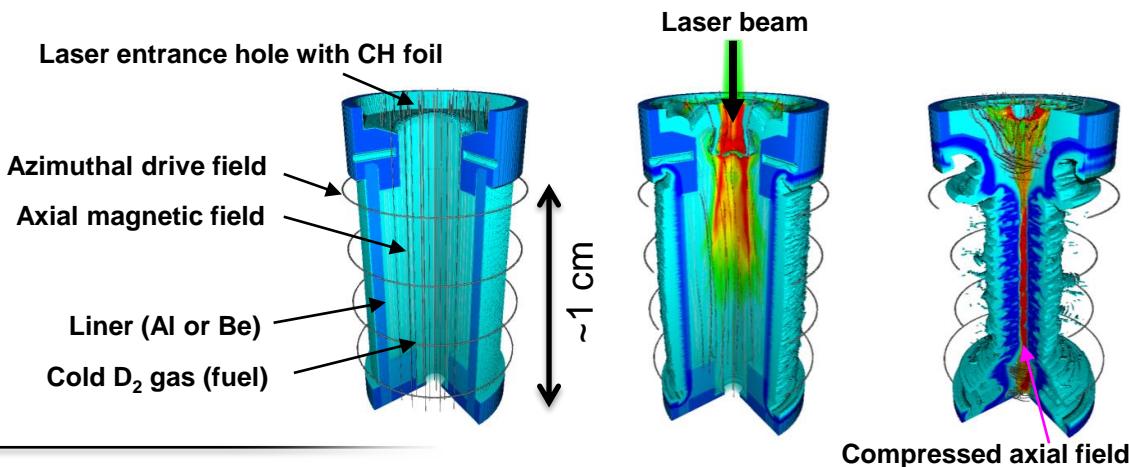
# Fielding tritium on Z will open the door to valuable collaborative MagLIF physics studies, but is nontrivial

- Tritium is not presently fielded on Z
  - Vacuum chamber is open every day, MITL grinding, tanks of oil and water
- Community needs to assess the cost-benefit of using tritium at Z
- Tritium would open the door to nuclear diagnostic techniques and target physics studies not presently possible
- These opportunities would encourage collaboration on Z with the broader HED community

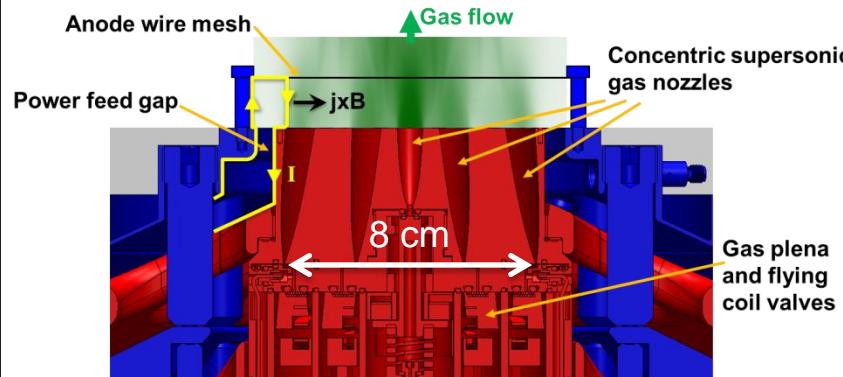


# ICF neutron sources at Z can have very different implosion dynamics and plasma conditions

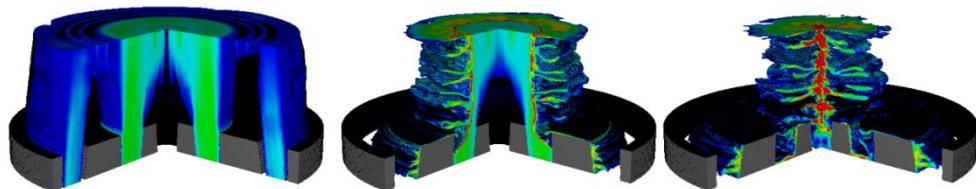
## MagLIF



## D2 gas puff



**MagLIF:** M. R. Gomez *et al.*, accepted to PRL (2014).  
**D<sub>2</sub> gas puff:** C. A. Coverdale *et al.*, PoP 14, 022706 (2007).



	$Y_n$ (DD)	$Y_n$ (DT)	$T_e$ (keV)	$T_i$ (keV)	$n_i$ ( $\text{cm}^{-3}$ )	$\Delta t$ (ns)	Diameter
<b>MagLIF</b>	$2 \times 10^{12}$	$5 \times 10^{10}$	$\sim 3$	2.5	$\sim 10^{23}$	< 2	$\sim 50 \mu\text{m}$
<b>D<sub>2</sub> gas puff</b>	$4 \times 10^{13}$	$< 4 \times 10^9$	2.2	$\sim 10$	$2 \times 10^{20}$	$\sim 30$	6 mm

# Several key physics issues could be addressed with DT experiments

Physics	Measurement	Tritium fuel content		
		<0.1%	0.1%	1%
Behavior of tritium in the Z pulsed power environment	Sampling of tritium contamination, migration			
Scaling of yield to DT—thermonuclear?	DT yield			
Ion temperature and non-thermal population	Precision nTOF and DT/DD yield ratio			
Liner/fuel mix	DT yield with tritiated gas fill and deuterated liner			
Fuel morphology	Neutron imaging			
Thermonuclear reaction history	Gamma Ray History/GCD, Thompson parabola			
Liner/fuel density, non-thermal effects (peak shifts)	Compact/Magnetic Recoil Spectrometer (CRS/MRS), precision nTOF			

# Diagnostic Capabilities enabled by tritium use will open new physics understanding for MagLIF

- Better SNR, higher dynamic range n-spectral measurements
  - More precise ion temperature
  - High precision Be down scatter measurements for liner pR
  - MRS or CRS measurements both axially and radially
- Neutron imaging enabled by higher yields
  - Is the neutron producing volume the same as the x-ray producing volume?
  - Down-scatter image for liner pR uniformity measurements
- $\gamma$  reaction history enabled by higher yields and preferable  $\gamma$ -branching ratio
  - Is the x-ray history the same as the  $\gamma$ -history?
  - Does the reaction history have structure indicating multiple isolated burn regions?
- Novel mix studies are enabled by separated reactant experiments using tritium or tritiated hydrogen gas
  - Deuterated window to study window mix
  - Deuterated coating on liner interior to study liner mix
  - Deuterated top/bottom caps to study mix from laser interactions
  - Combine w/ neutron imaging to study transport of mix material

# Gradual increase in MagLIF tritium fuel content will provide increasing scientific opportunities

Proposed Z Timeline

FY15	FY16	FY17	FY18	FY19	
					
Tritium Surrogates $D_2$ , $^3He$	Trace Tritium ES&H $<0.1\%$	Trace Tritium 10x DT Yield $\sim0.1\%$	Minority Tritium $>10^{13}$ DT Yield $\sim1\%$		Tritium Operations 10-50%
			DT yield scaling, ion temperature and non-thermal population Nuclear tracers for liner/fuel mix Neutron imaging, high sensitivity for DD MagLIF, mixed DD/DT imaging (CR-39?)		
	Brems background measurements for GCD, shielding studies Wedge range filter, CRS design		GRH/GCD, Thompson parab., CVD dia. MRS neutron spectroscopy		

# Our ability to minimize the impact on the facility depends on the ability to purge the tritium from the Z target chamber

$h = 10 \text{ mm}$

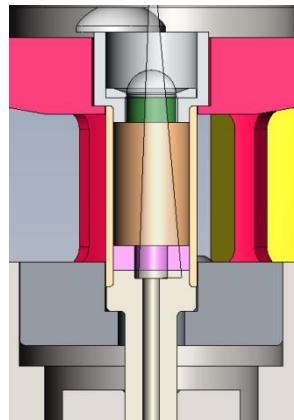
$r_{\text{fuel}} = 2.75 \text{ mm}$

$V = 238 \text{ mm}^3$

$\rho = 1.5 \text{ mg / cc}$

$1 \% \text{ T} = 41.1 \text{ mCi}$

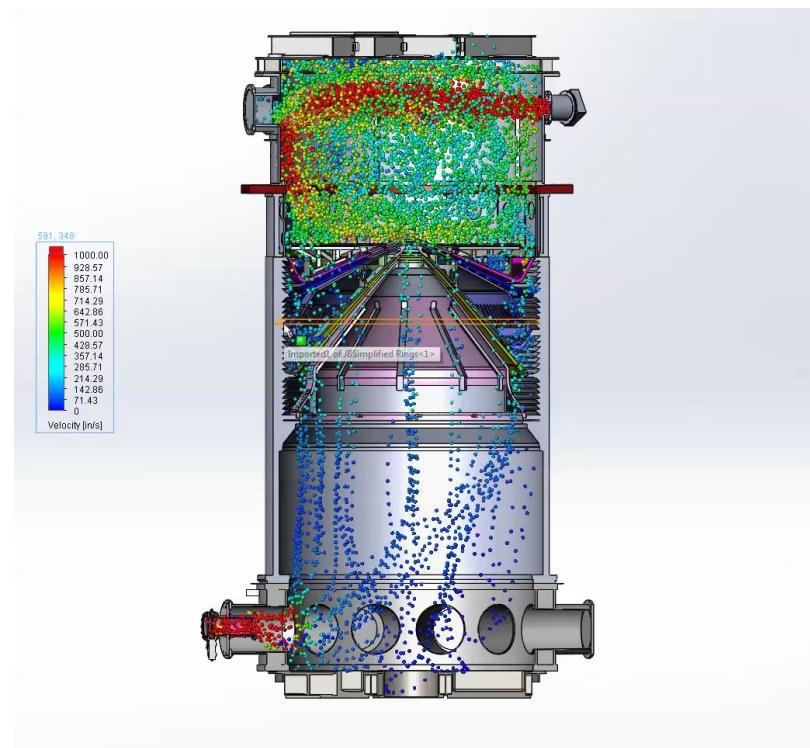
MagLIF target



Flow analysis of the Post Shot Air Exchange System for Z center section

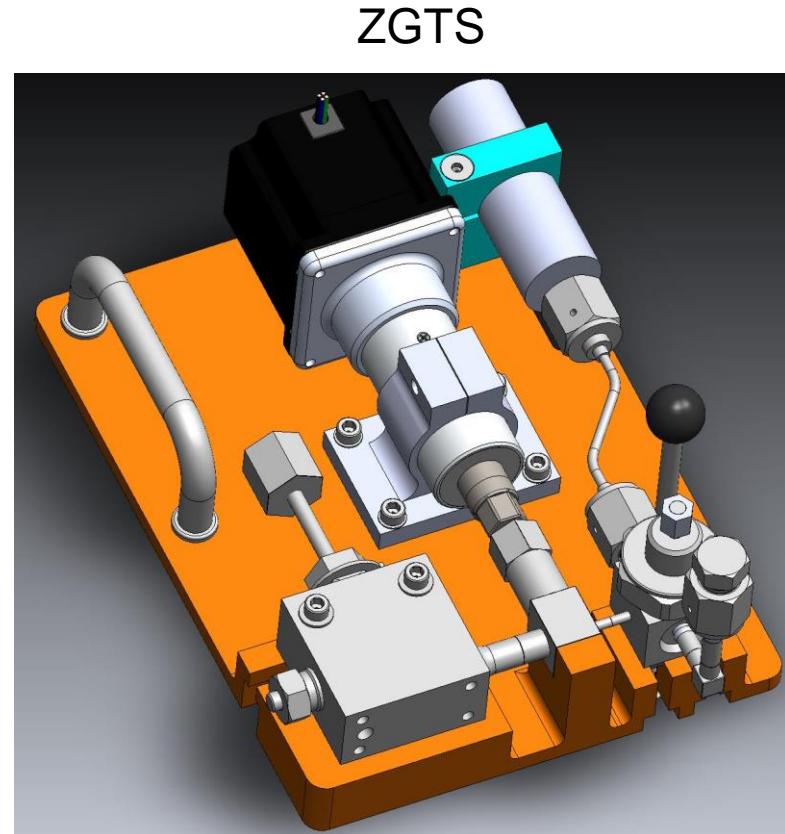
Volume =  $66 \text{ m}^3$

Total surface area =  $464 \text{ m}^2$



- Purge efficiencies required to keep Z below control limits for tritium
  - Assuming entire surface area
  - 99.5 % for  $10,000 \text{ dpm} / 100 \text{ cm}^2$   
(Contaminated area)
  - $\sim 50 \%$  for  $1 \text{ e}6 \text{ dpm} / 100 \text{ cm}^2$   
(Highly contaminated area)

# We recently completed development of the Z Gas Transfer System (ZGTS) capable of filling MagLIF targets in-situ on Z



- Robust tritium capable gas transfer system
  - Uses metal diaphragm puncture valve
  - Minimizes tritium inventory
  - Controls when and where tritium is used
  - Fills target in-situ just prior to shot