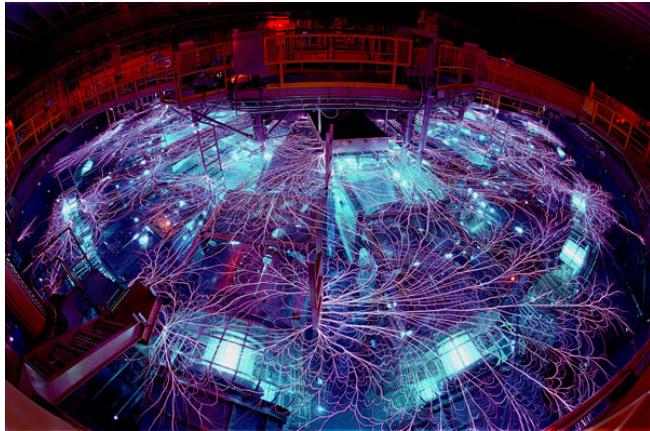
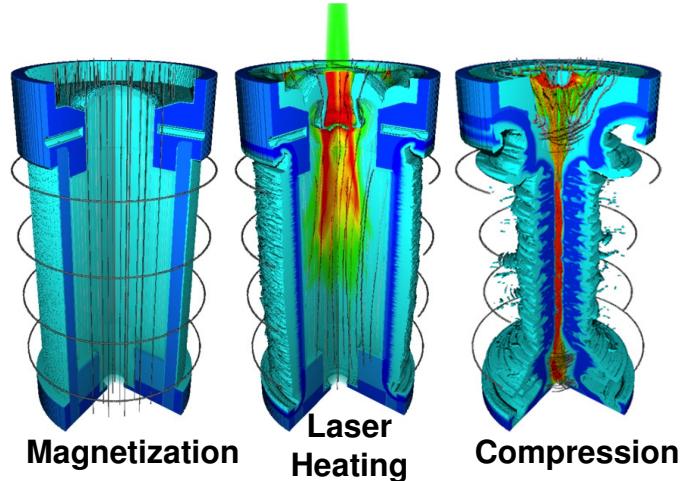


Exceptional service in the national interest



Progress in Magnetized Liner Inertial Fusion (MagLIF)



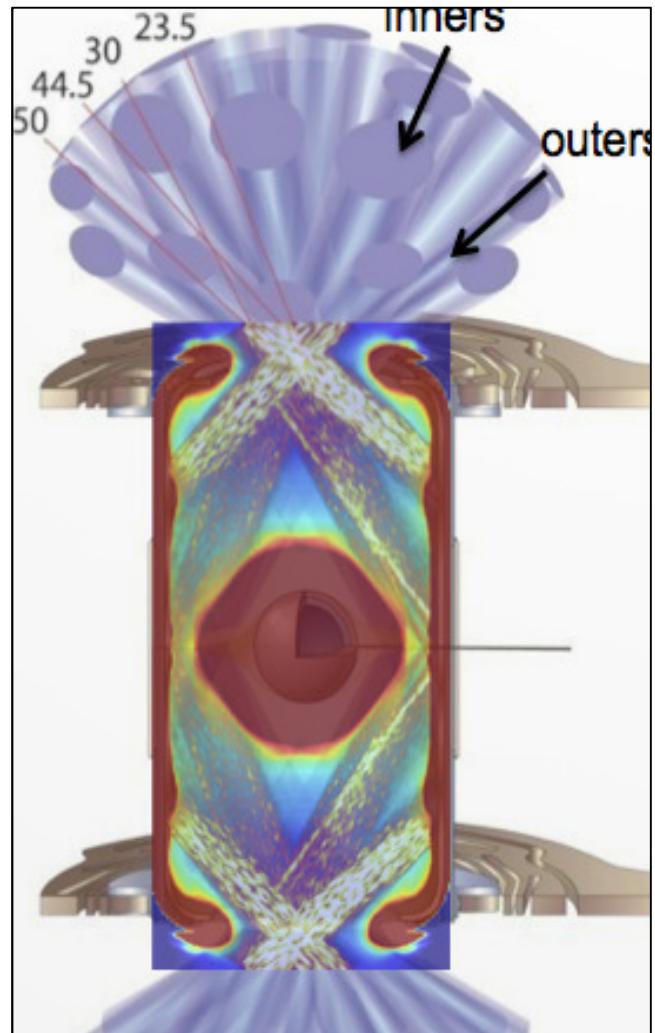
Stephanie Hansen

For the MagLIF team
Sandia National Laboratories

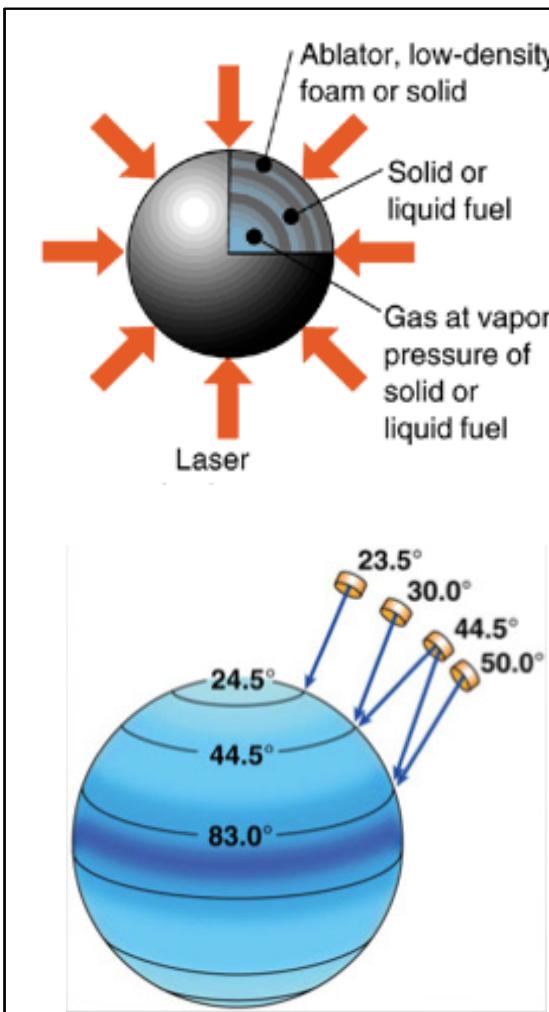
European Physical Society Conference on Plasma Physics
June 22-26, 2015

The United States ICF program has focused on three main approaches to ignition

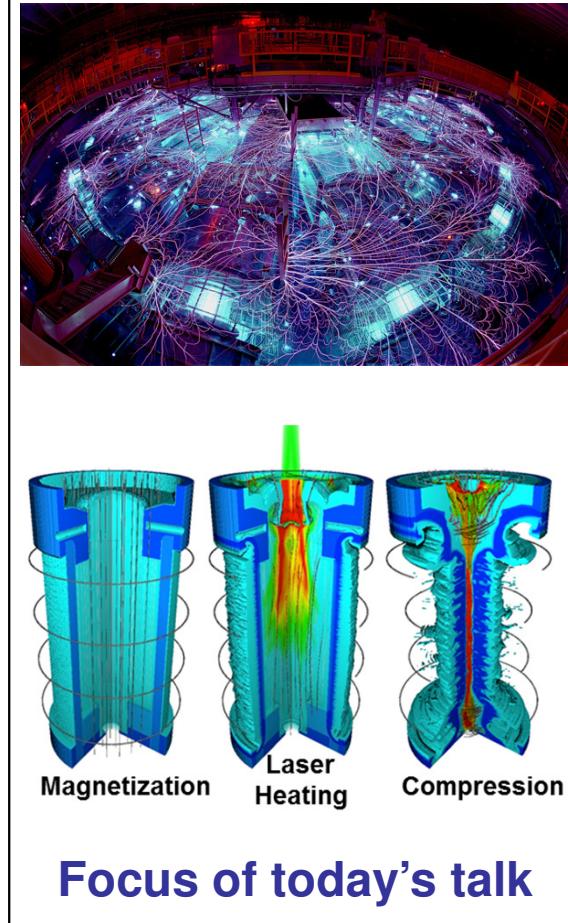
Indirect X-ray drive



Direct laser drive

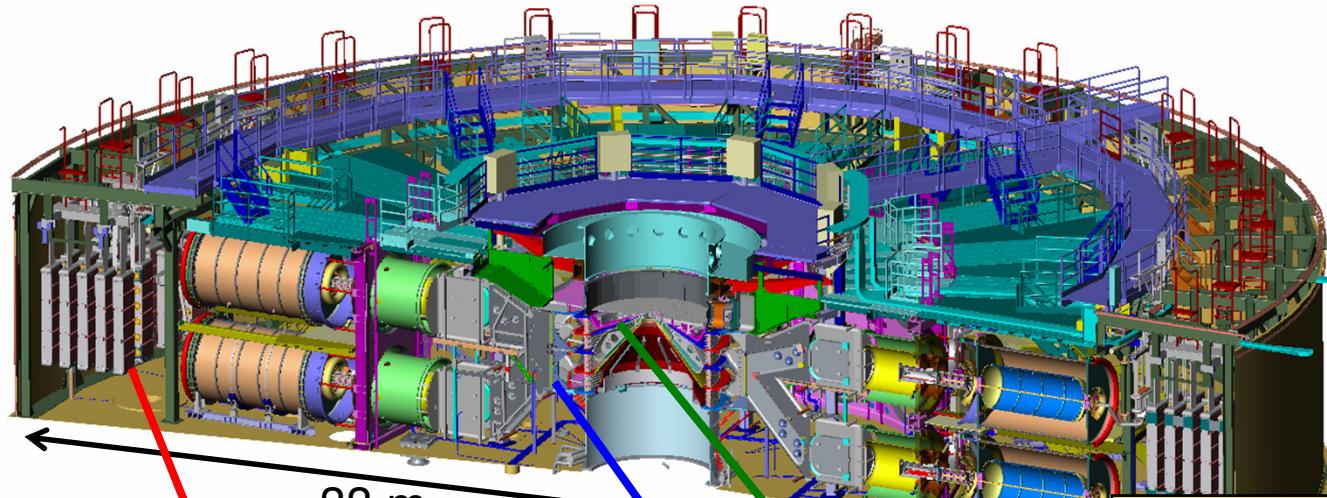


Direct magnetic drive

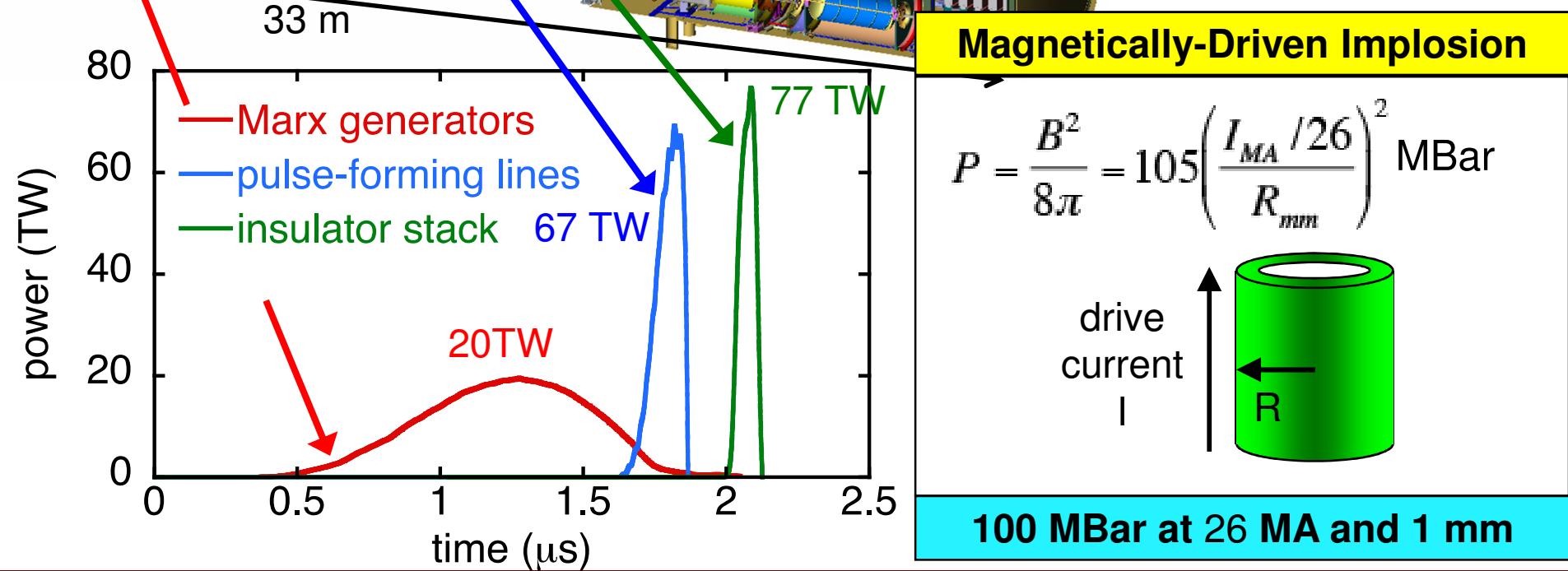


Focus of today's talk

Magnetic direct drive is based on the idea that we can efficiently use large currents to create high pressures



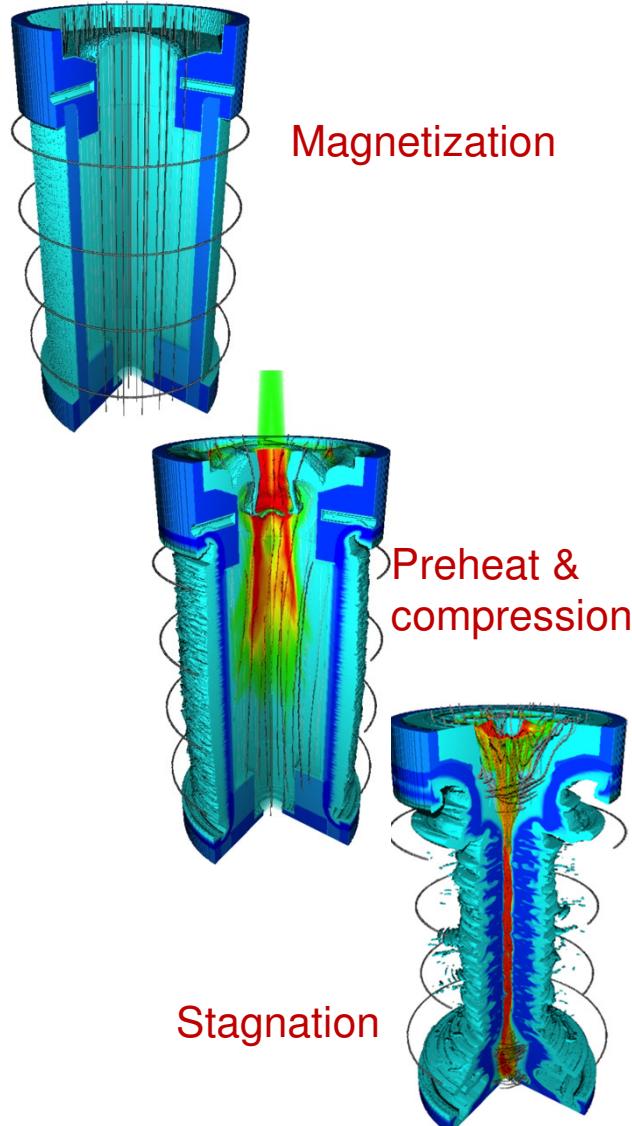
Z today couples 0.5 MJ out of 20 MJ stored to MagLIF targets, delivering 0.1 MJ to DD fuel.



Implosion time ~50 ns; stagnation ~0.1-1 ns

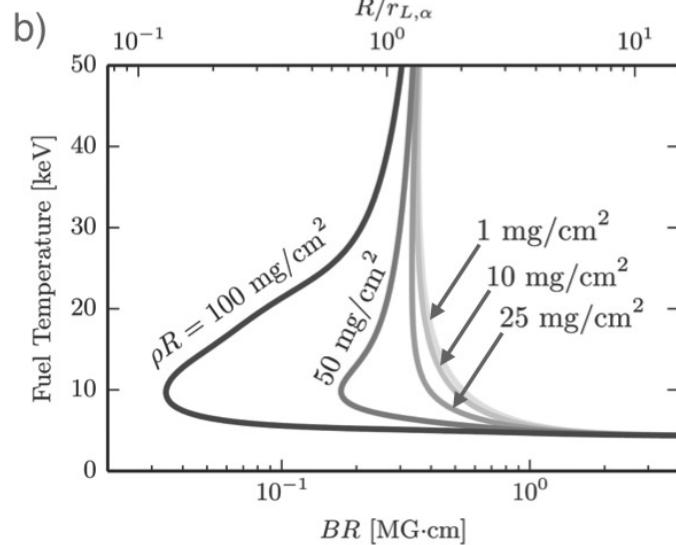
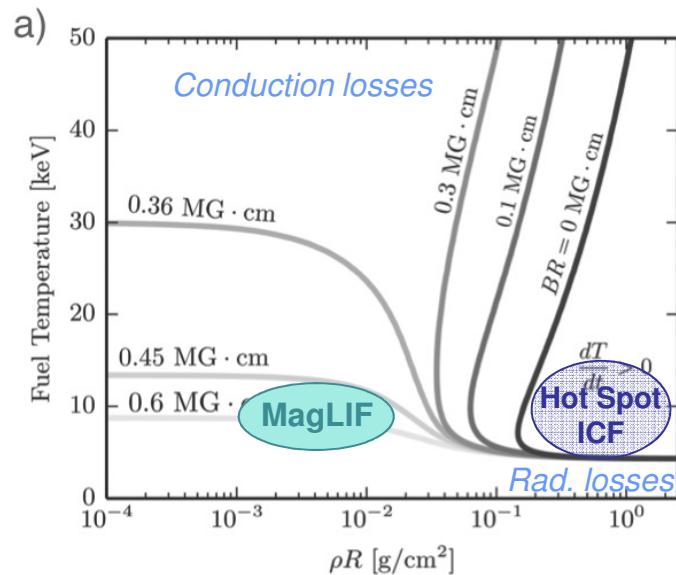
(1 atm = 1 bar = 10^5 Pascals) ³

The Magnetized Liner Inertial Fusion (MagLIF)* concept is well suited to pulsed power drivers and may reduce inertial fusion requirements



- **Axial magnetization of fuel/liner ($B_{z0} = 10-30$ T)**
 - Inhibits thermal conduction losses, may help stabilize liner compression, traps charged fusion products ($\beta: 5 \sim 80$; $\omega\tau > 200$)
- **Laser preheat (2-10 kJ)**
 - Reduces amount of radial fuel compression needed to reach fusion temperatures ($R_0/R_f = 23-35$)
- **Liner compression of fuel (70-100 km/s, ~100 ns)**
 - “Slow”, quasi-adiabatic compression of fuel
 - Low velocity requirements allow use of thick liners ($R/\Delta R \sim 6$) that are robust to instabilities and provide sufficient ρR at stagnation to inertially confine fuel
- Combination allows fusion at ~100x lower fuel density than traditional ICF (~5 Gbar vs. 500 Gbar)
- DD equivalent of 100 kJ DT yield may be possible on Z with upgrades from our initial setup
e.g., 10 T \rightarrow 30 T; 2 kJ \rightarrow >6 kJ; 19 MA \rightarrow >24 MA

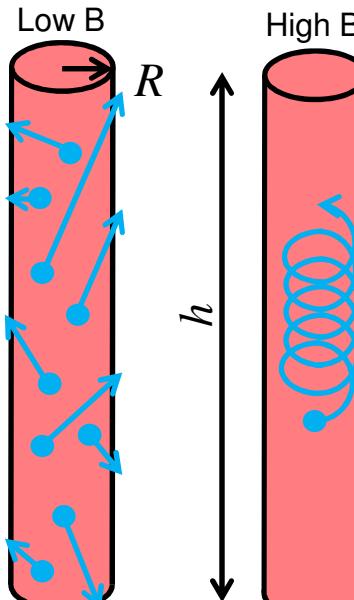
Magnetization can be used to reduce thermal conduction losses and ρR /pressure requirements



- Axial magnetization of fuel/liner ($B_{z0} = 10-30$ T)

- **Inhibits thermal conduction losses, may help stabilize liner compression, traps fusion products ($\beta: 5 \sim 80$; $\omega\tau > 200$)**

$$\frac{R}{r_\alpha} = \frac{BR [T \cdot \text{cm}]}{26.5} = \frac{BR [G \cdot \text{cm}]}{2.65e5} \approx 4BR [MG \cdot \text{cm}]$$



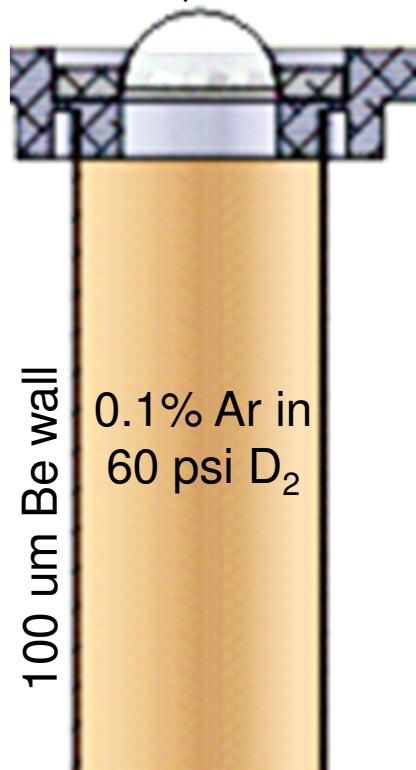
Fraction of trapped α 's (tritons)
is a function of **BR** only

Measurements to date suggest
we are reaching 0.4 MG-cm:
enough to return 40% of fusion
product energy to the fuel.
Effects saturate at $BR > 0.5$ MG-cm

Magnetization and preheat: dedicated preheat experiments indicate that the external magnetic field effectively insulates the preheated plasma

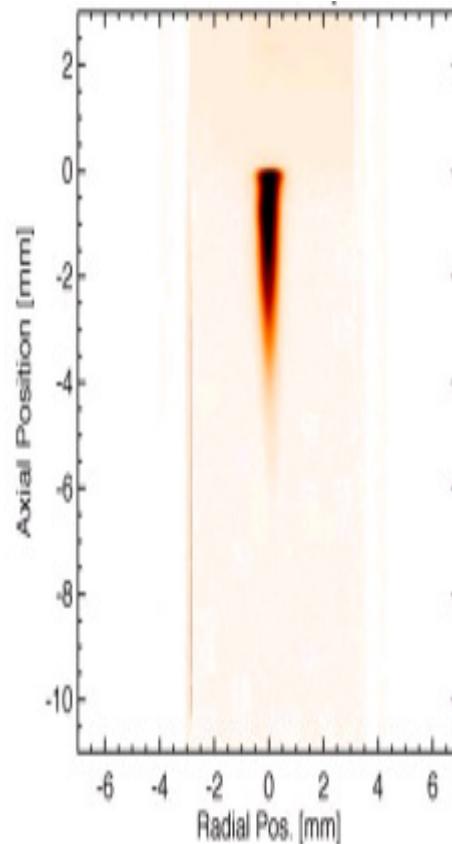
2.5 kJ laser onto

1.5 μ m foil:

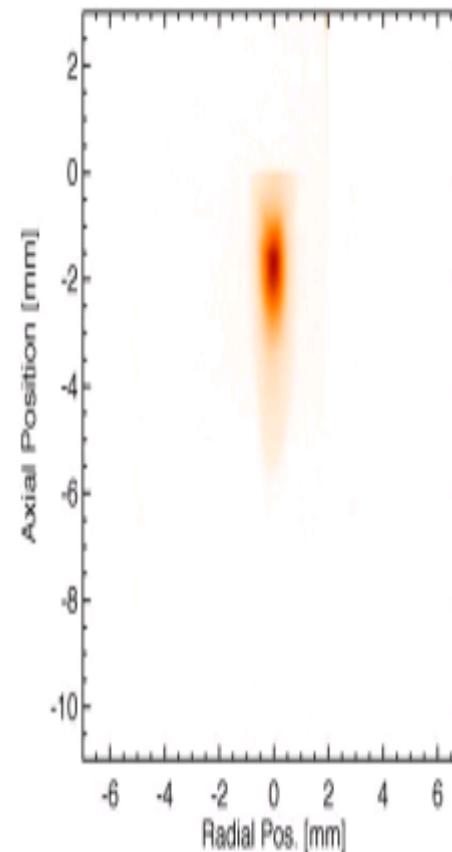


3.12 keV argon self-emission images

$B_z = 8.5$ T



$B_z = 0$ T

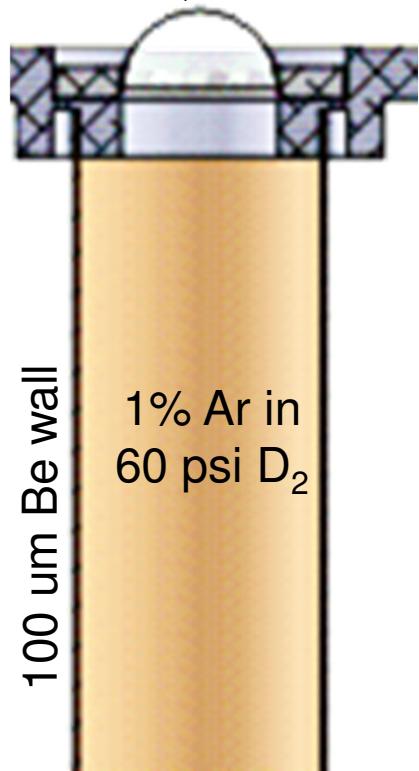


These data may help constrain models of thermal conductivities in high magnetic fields

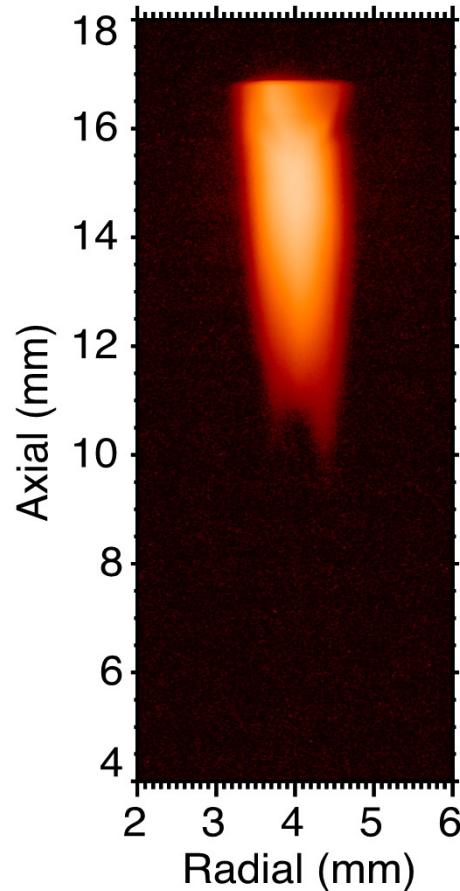
Preheat: Dedicated preheat experiments indicate poor laser coupling to the fuel (100-300 J from 2-4 kJ)



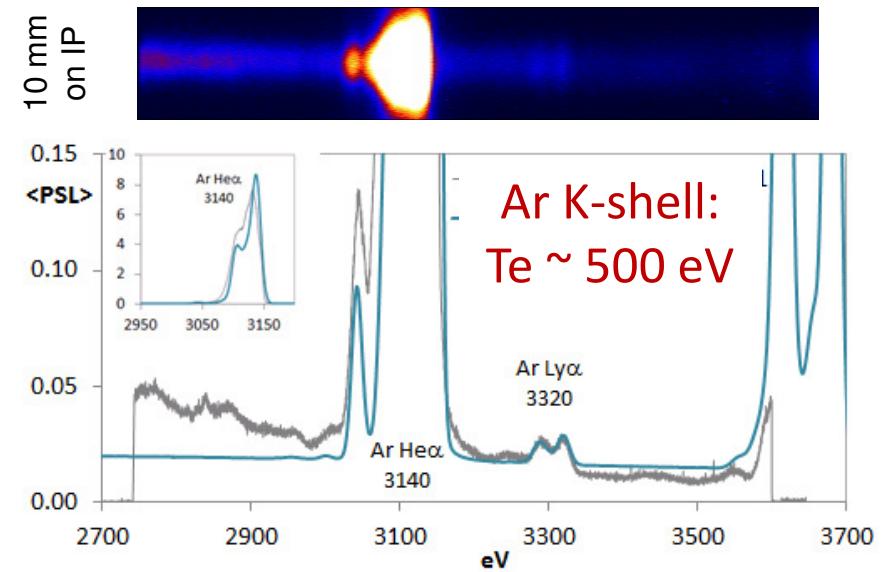
2.5 kJ laser onto
1.5 μm foil:



3.12 keV
self-emission
image



Radially resolved Ar
emission spectrum

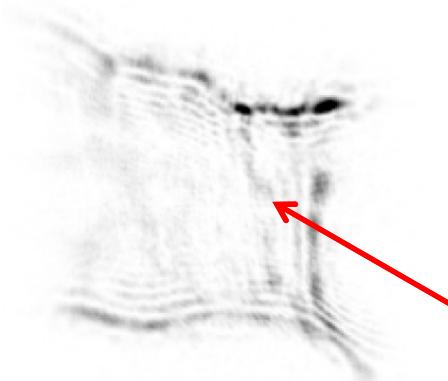


Spectra and imager data:
~500 eV temperature in ~mm³ volumes →
only ~10% of laser energy is coupled to
the fuel – consistent with ~10¹² DD yields
in integrated experiments

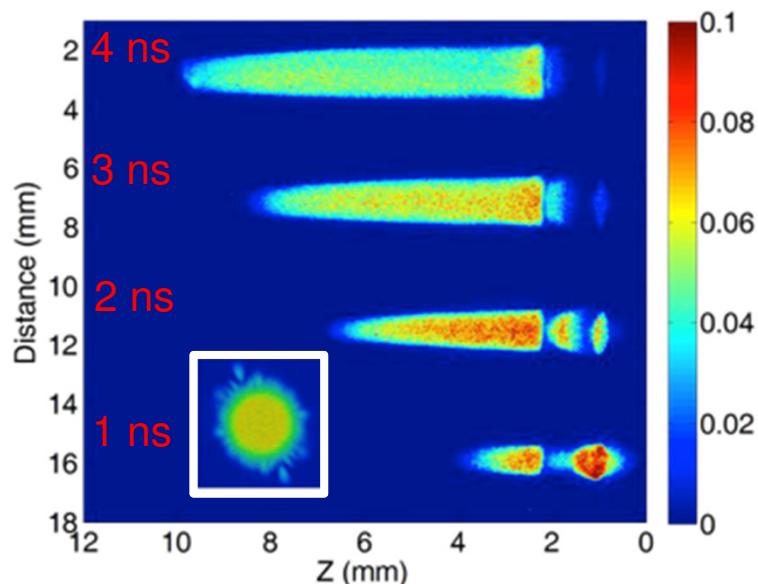
Preheat: Poor beam quality is contributing to poor laser-fuel coupling; future experiments will use conditioned beam



Z-Beamlet currently does not use any beam smoothing techniques adopted by the laser community

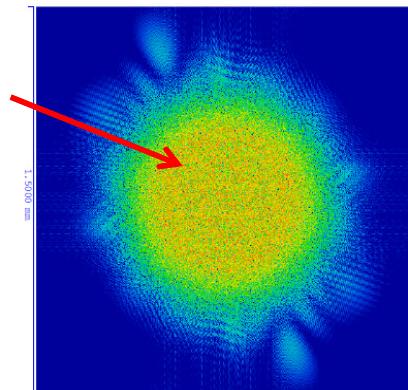


4 ns/3.1 kJ, 2 μ m LEH, no prepulse **with DPP** (SNL Omega-EP data)

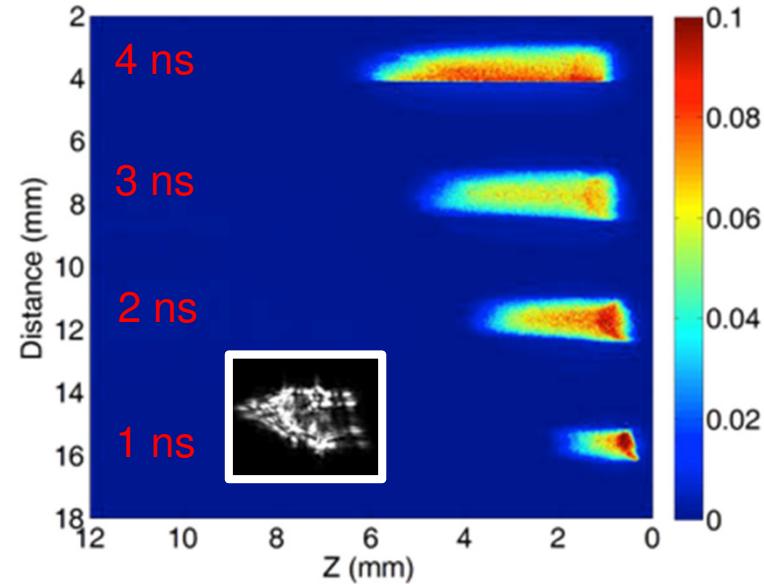


OMEGA-EP
750um DPP

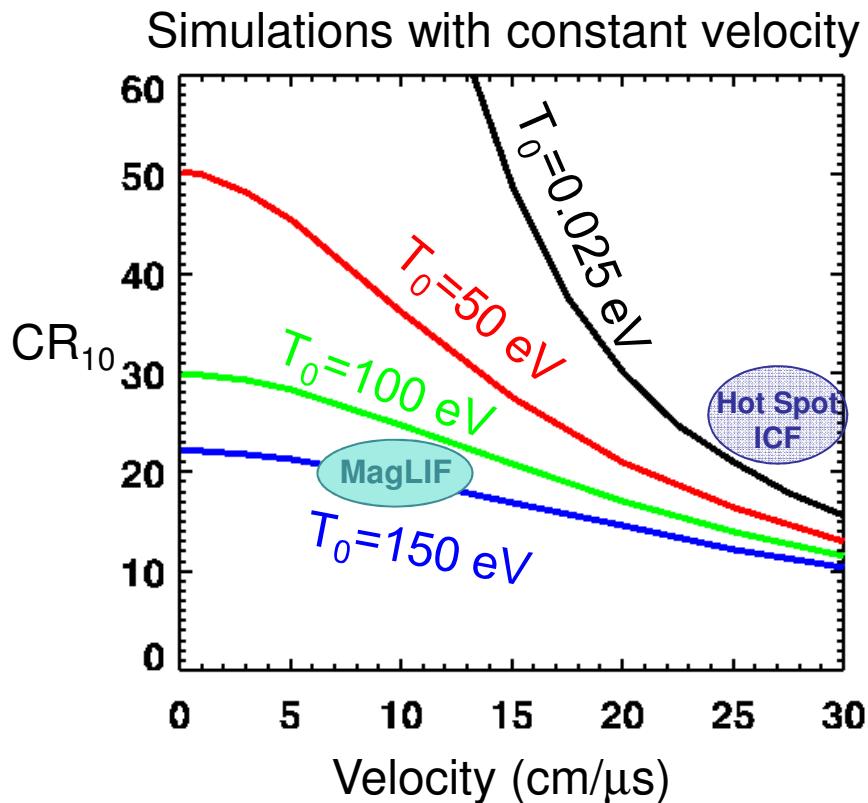
ZBL: No DPP
(representative)



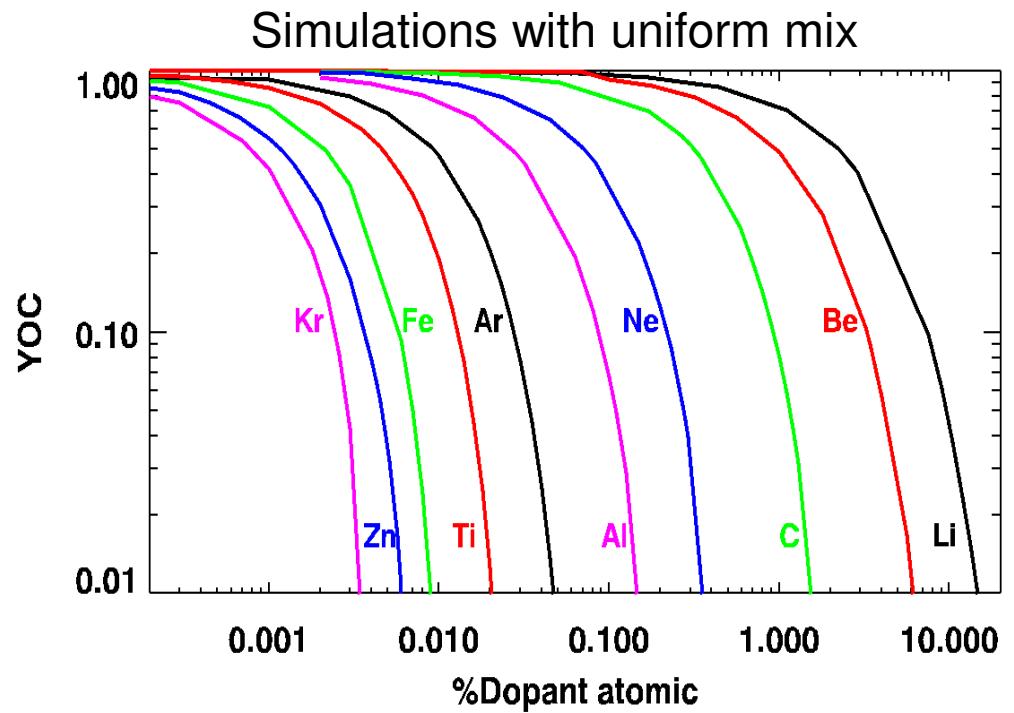
4 ns/2.93 kJ, 2 μ m LEH, no prepulse **without DPP** (SNL Omega-EP data)



Preheat and implosion: Preheating the fuel allows slower, more stable implosions and lower convergence ratios than traditional ICF, but the preheated plasma is sensitive to mix



CR_{10} = Convergence Ratio (R_0/R_f) needed to obtain $T = 10 \text{ keV}$ with no radiation or thermal conduction losses

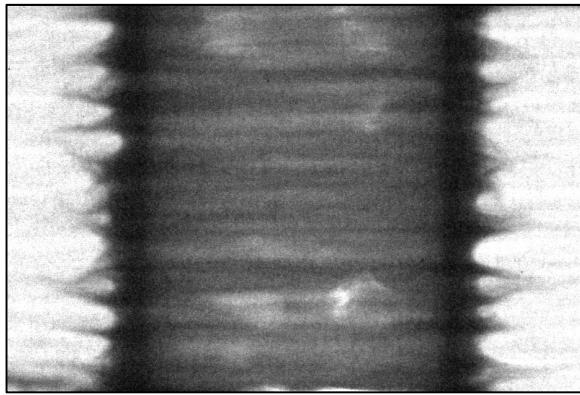


While magnetization reduces conduction losses, the long preheat stage is susceptible to radiative losses from mix, so we can't use dopants to diagnose preheat on integrated experiments.

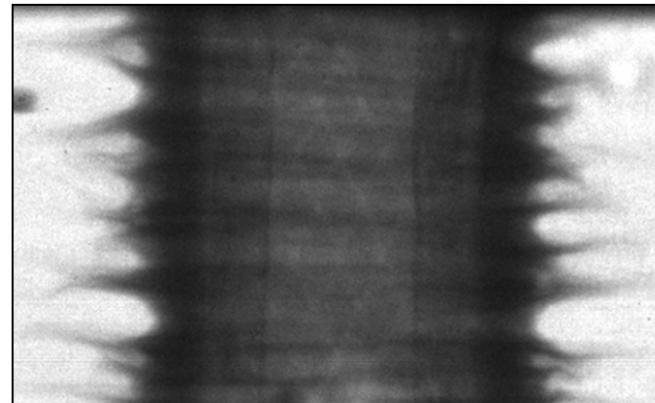
Magnetization and implosion: The axial magnetic field and dielectric coatings can effectively mitigate the growth of Magneto-Rayleigh-Taylor instabilities



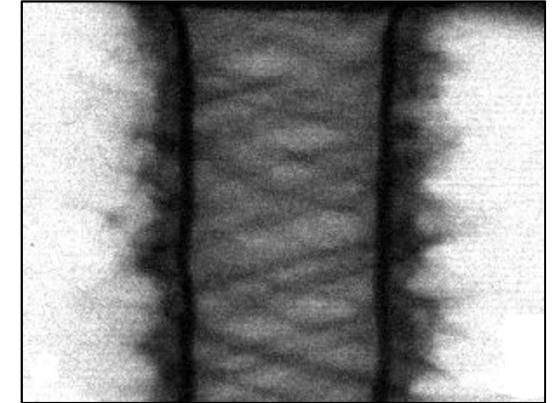
Axially-polished liner



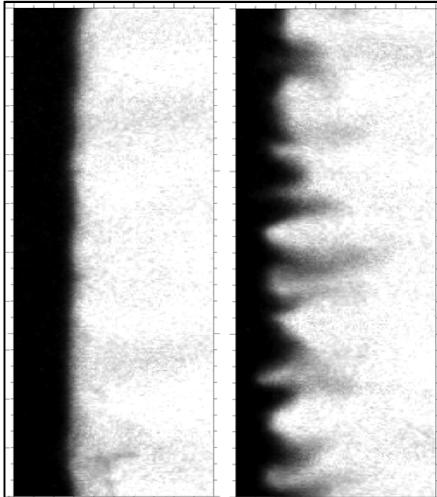
Helically perturbed liner



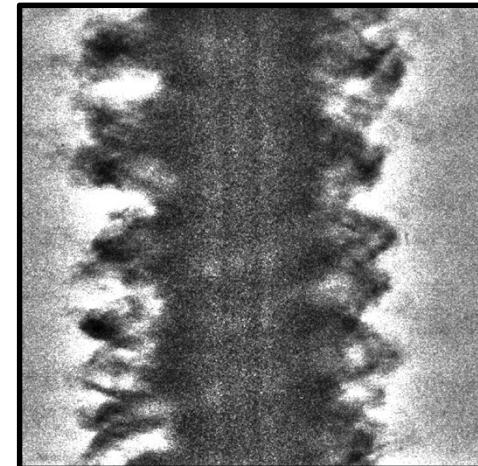
Magnetized liner



Coated Uncoated

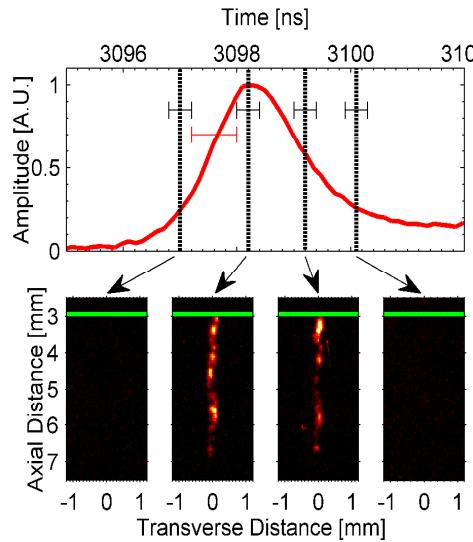


CH coatings on metal liners suppress the electro-thermal instability, the dominant MRT seed for MagLIF



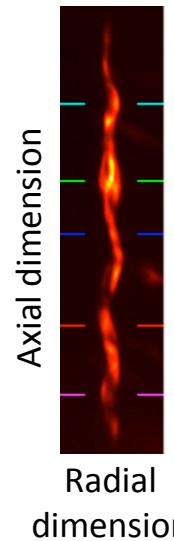
A CH-coated, magnetized Be liner has a stable inner surface at $CR \sim 20$

Stagnation: Extensive x-ray and neutron diagnostics resolve plasma conditions in space, energy, and time

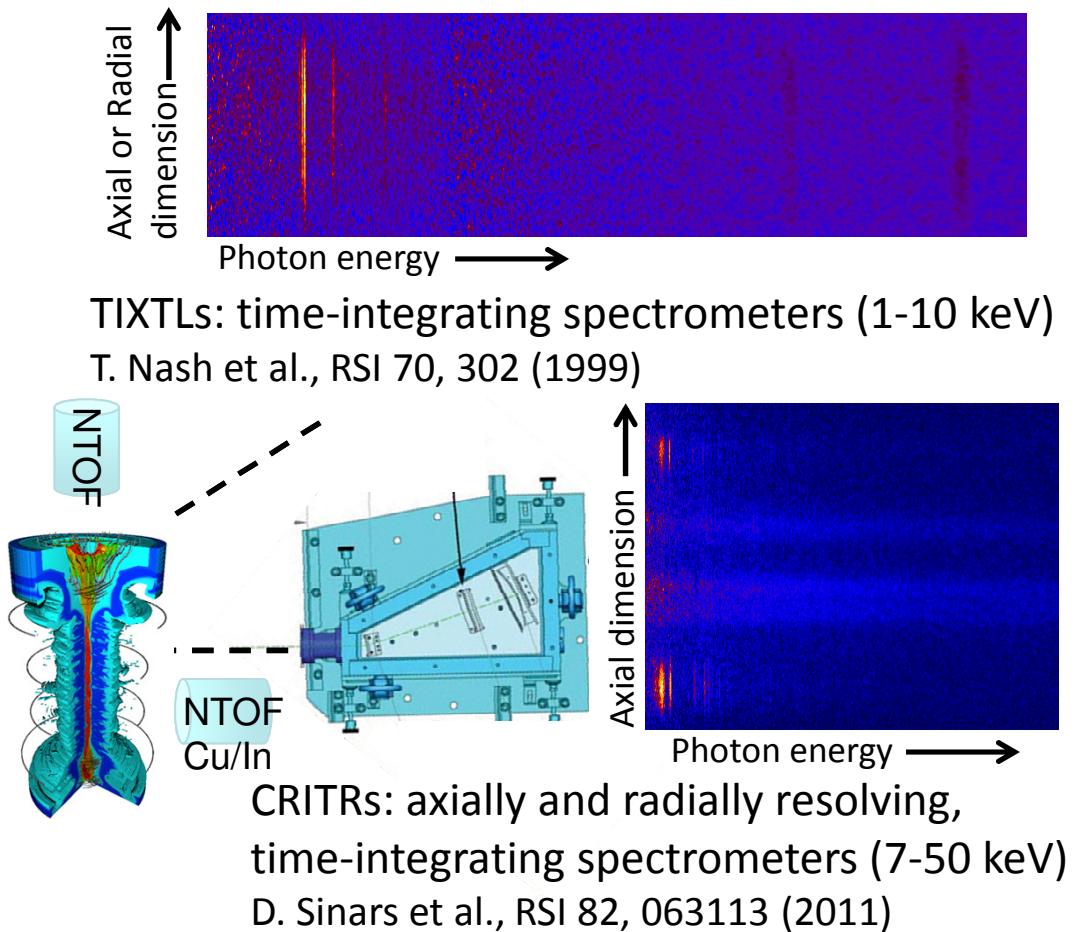


Filtered PCDs and SiDs: time-resolved x-ray powers

MLM/filtered pinholes: time-gated x-ray images
M. Gomez

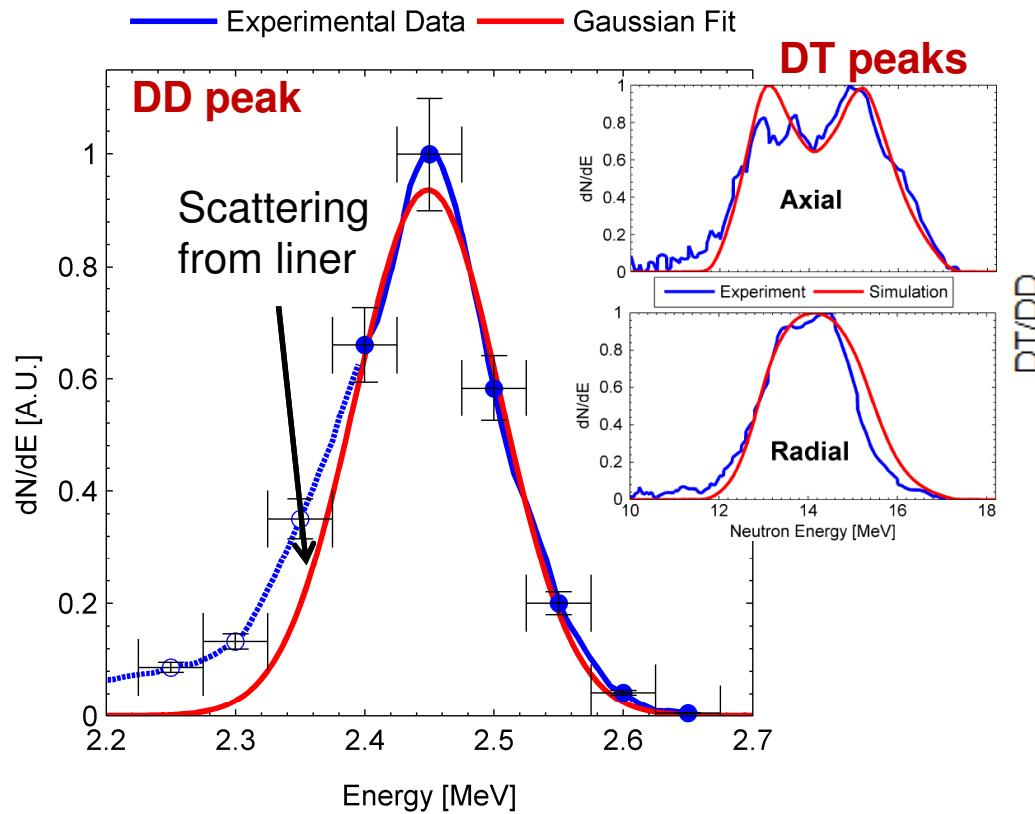


Crystal imager: time-integrated hard x-ray image
E. Harding

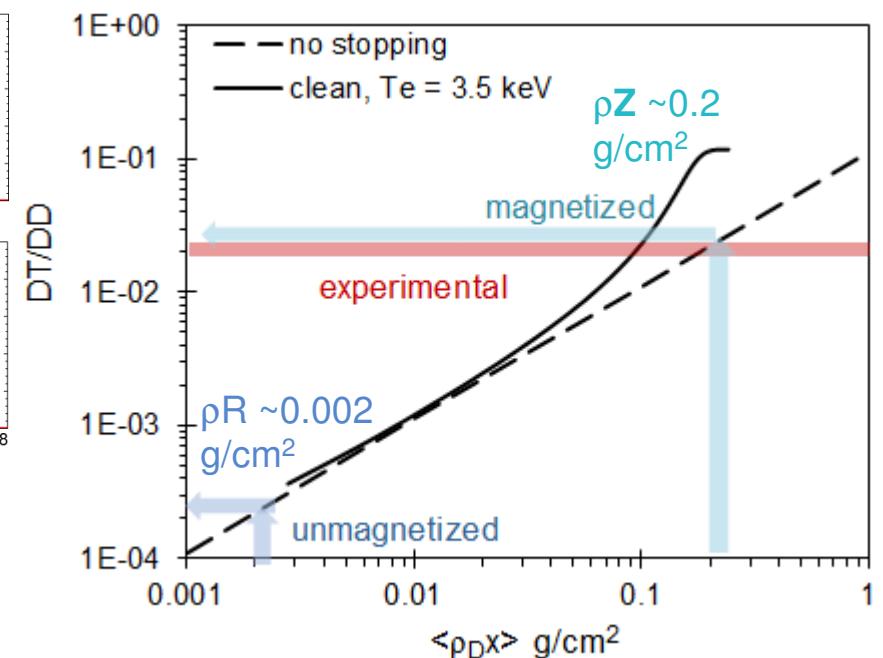


Measuring MagLIF's ~ 30 J x-ray yields is challenging compared to the few-MJ x-ray yields of many Z experiments

Stagnation: neutron data indicate $T_{ion} \sim 2$ keV and effective confinement of both fuel and 1 MeV tritons

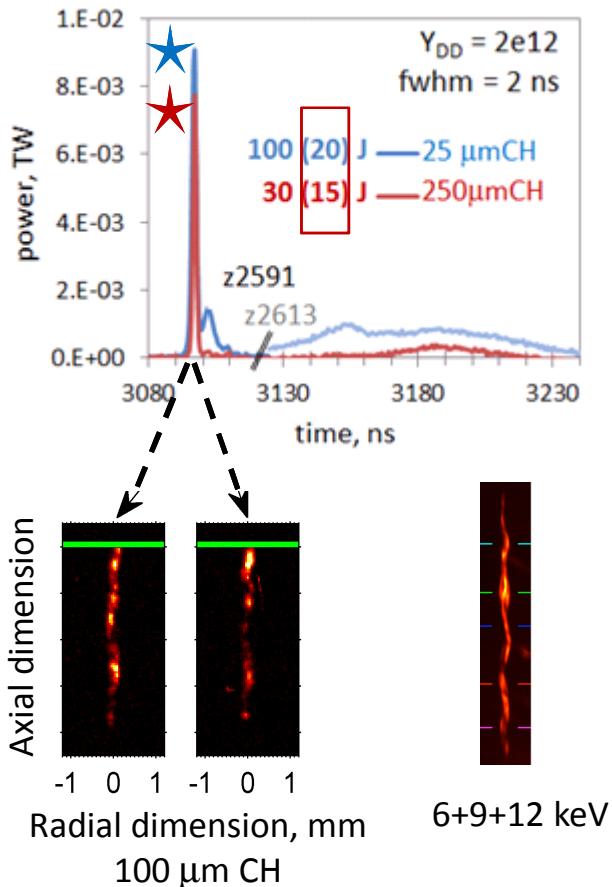


Neutron time-of-flight data measure D-D neutrons, indicating $T_{ion} \sim 2$ keV and $\rho R_{liner} > 0.9 \text{ cm}^2/\text{g}$



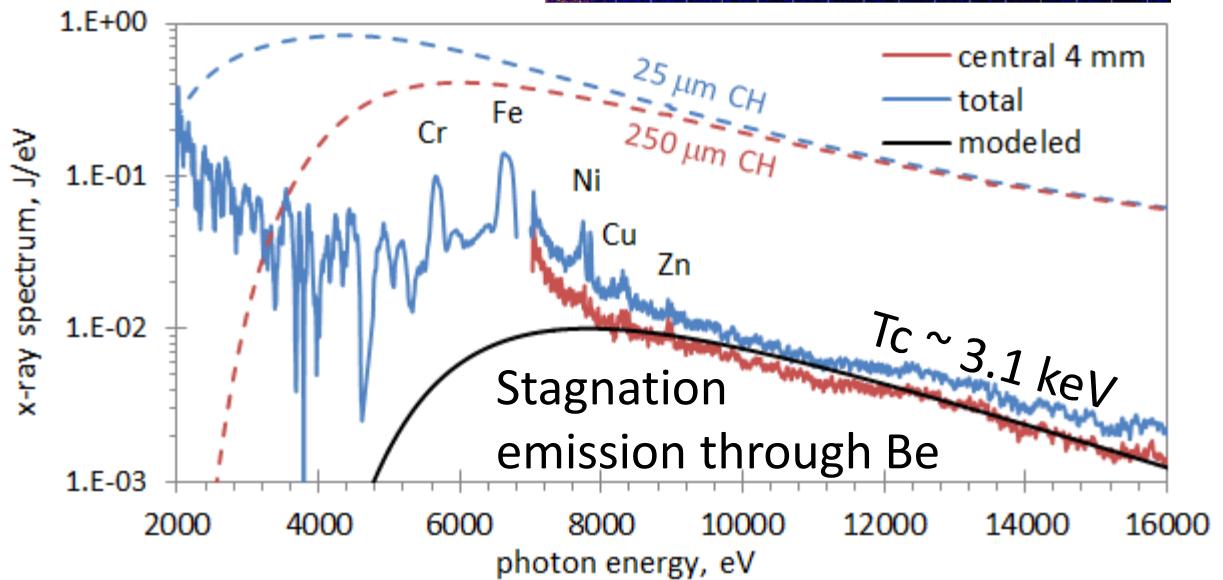
High “secondary” DT neutron yields from the interaction of 1 MeV tritons generated by DD reactions – and asymmetry in the DT spectra – indicate $BR \sim 0.4 \text{ MG-cm}$.

Stagnation: Combining information from all x-ray diagnostics constrains T_e , ρ_{fuel} , and ρR_{Be}

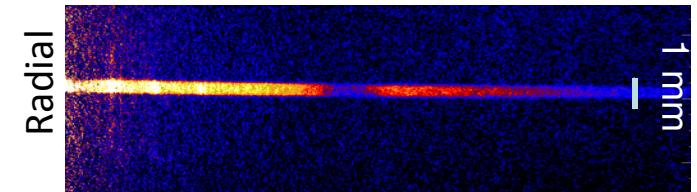
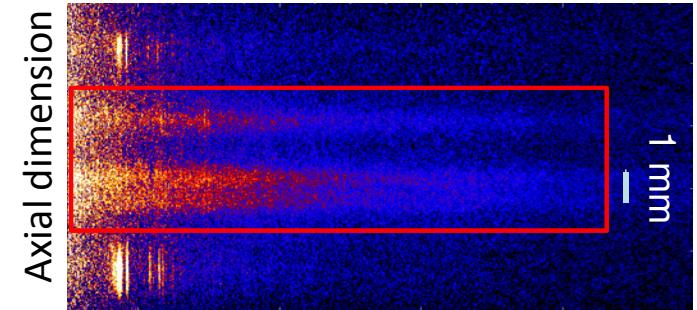


Images constrain stagnation volume:
 $R \sim 50 \mu\text{m}$, $Z \sim 4 \text{ mm}$

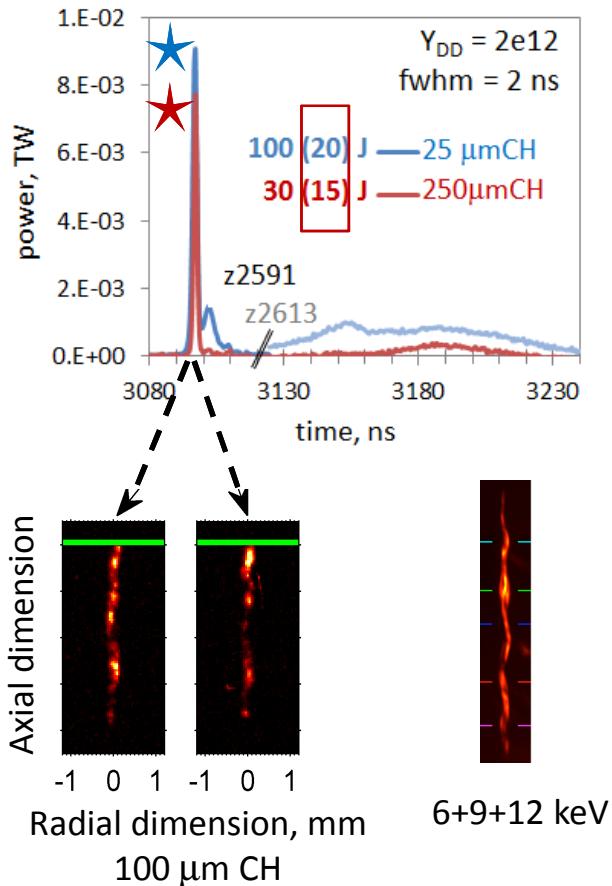
Powers constrain density and Be absorption:
 $\rho_D = 0.4 \text{ g/cm}^3$ (55%)
 $\text{Be } \rho R \sim 0.9 \text{ g/cm}^2$



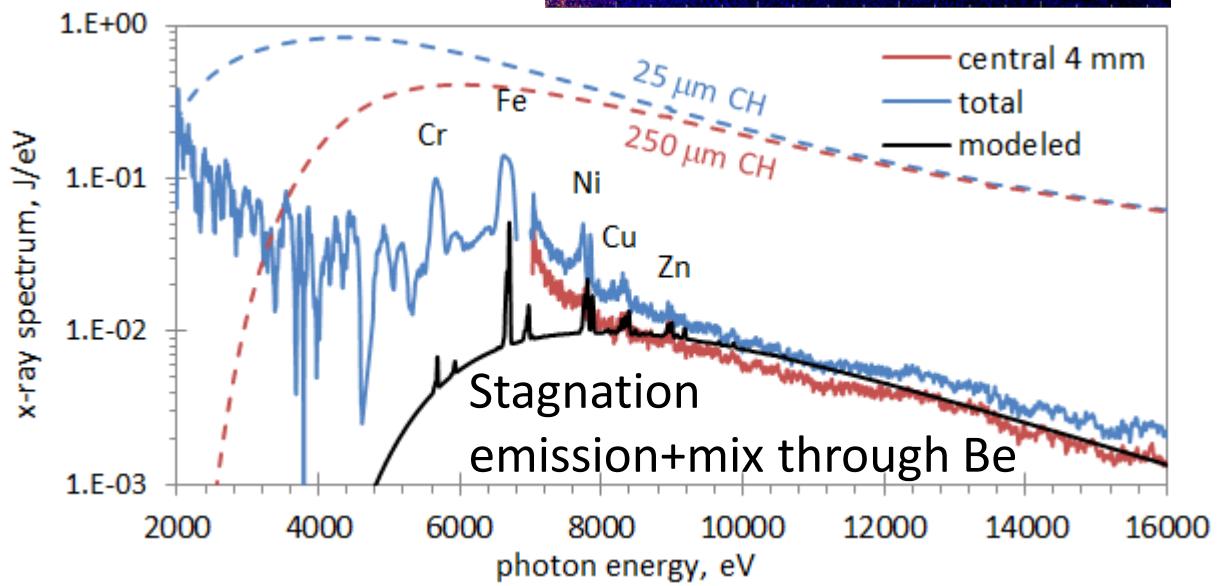
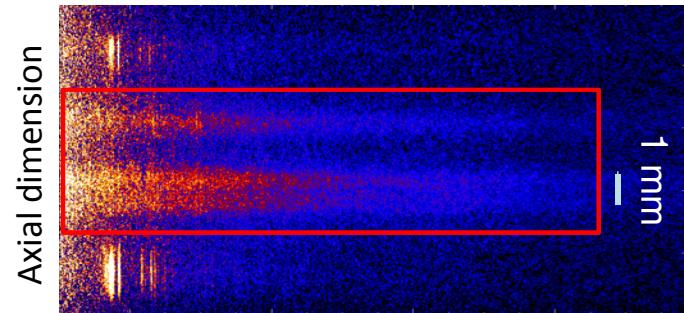
X-ray spectrum constrains stagnation temperature:
 $T_c \sim 3.1 \text{ keV}$



Stagnation: detailed spectroscopic measurements also constrain the amount of mix – but not yet its origin

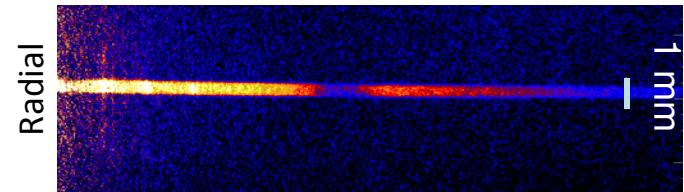


Powers constrain density and Be absorption:
 $\rho_D \sim 0.25 \text{ g/cm}^3$ (35%)
 $\text{Be } \rho R \sim 0.9 \text{ g/cm}^2$



Images constrain stagnation volume:
 $R \sim 70 \mu\text{m}$, $Z \sim 4 \text{ mm}$

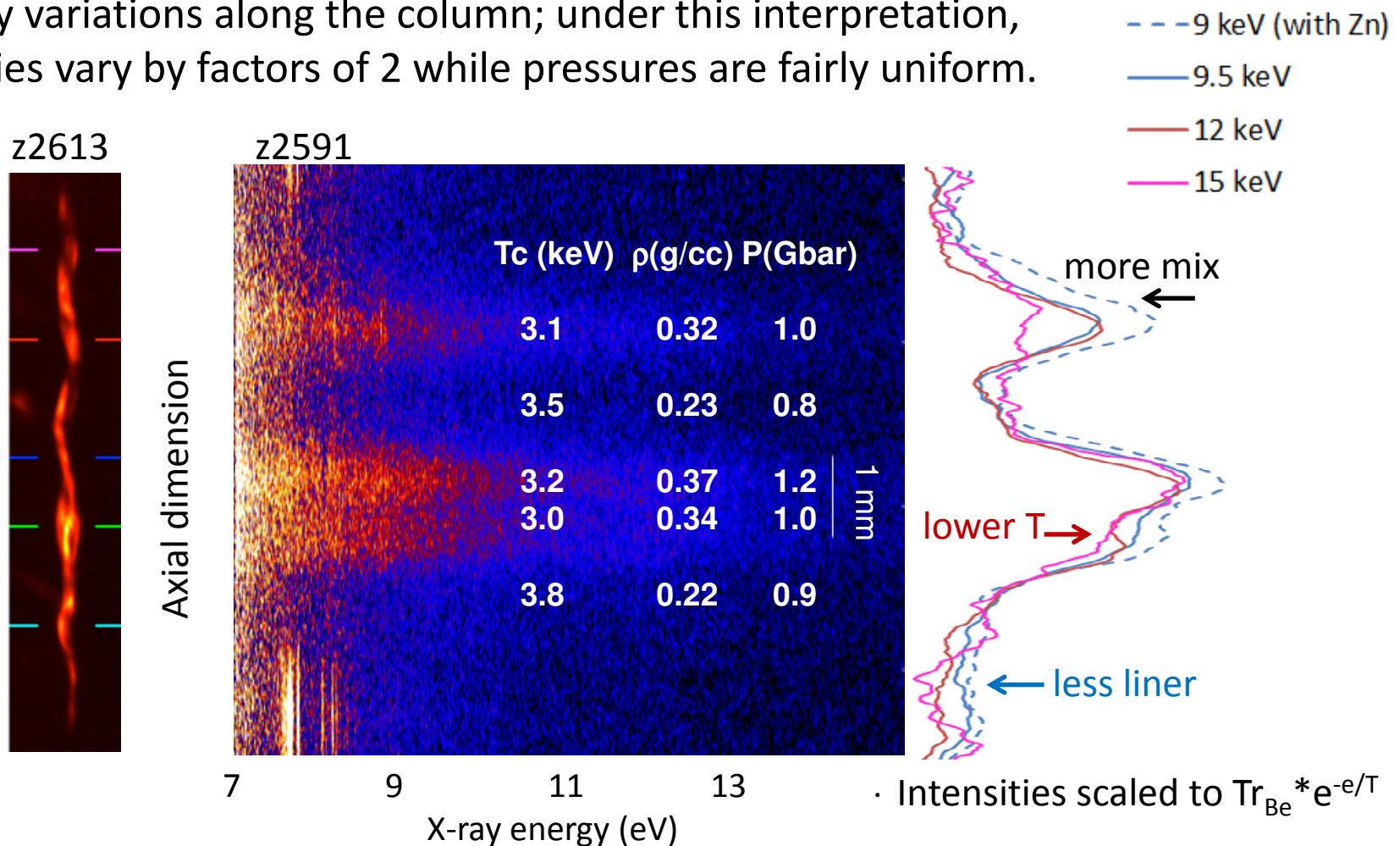
Spectrum constrains mix:
 $f_{\text{Be}} \sim 5\%$ or $f_{\text{Al}} \sim 0.04\%$



Stagnation: A closer look at axially-resolved x-ray spectra indicates ~20% axial pressure variations



Gross variations in axial intensities are most likely due to density variations along the column; under this interpretation, densities vary by factors of 2 while pressures are fairly uniform.



Stagnation: An isobaric assumption for radial gradients improves agreement with the collection of x-ray and neutron data and is consistent with degraded-preheat simulations

Degraded simulation: burn averages

$$\begin{aligned}\rho_D &= 0.4 \text{ g/cm}^3 \\ R &= 65 \mu\text{m} \\ z &= 4 \text{ mm} \\ t_{\text{burn}} &= 1.6 \text{ ns} \\ T &\sim 3 \text{ keV}\end{aligned}$$

$$\rho r_{\text{liner}} = 0.9 \text{ g/cm}^2$$

$$Y_{\text{DD}} = 2-4 \times 10^{12}$$

Measured neutron data:

$$\begin{aligned}Y_{\text{DD}} &= 2 \times 10^{12} \\ T_i &= 2.5 \text{ keV}\end{aligned}$$

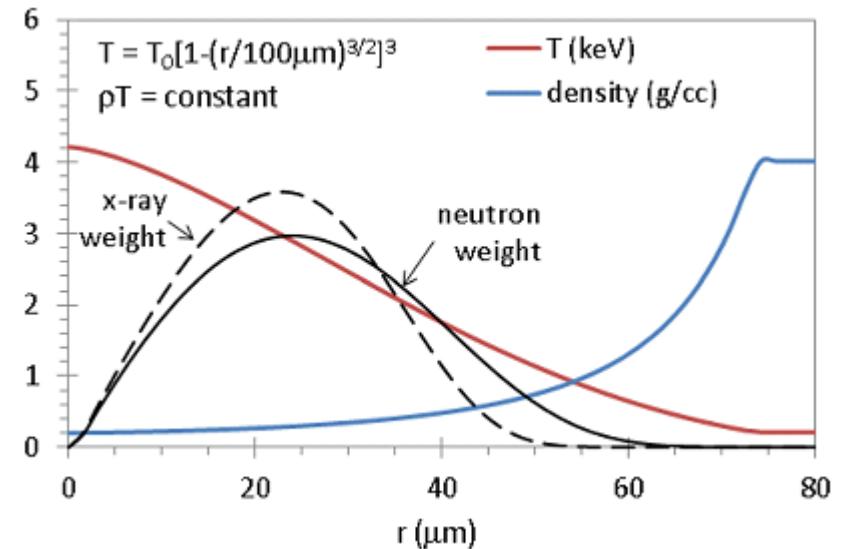
X-ray analysis with “cartoon” model

$$\begin{aligned}\rho_D &\sim 0.3 \text{ g/cm}^3 \\ R &= 70 \mu\text{m} \\ z &= 4 \text{ mm} \\ t_{\text{burn}} &= 2 \text{ ns} \\ T_e &= 3.1 \text{ keV}\end{aligned}$$

$$\rho r_{\text{liner}} = 0.9 \text{ g/cm}^2$$

$$Y_{\text{DD}} = 6 \times 10^{12}$$

Radial gradients in isobaric model*



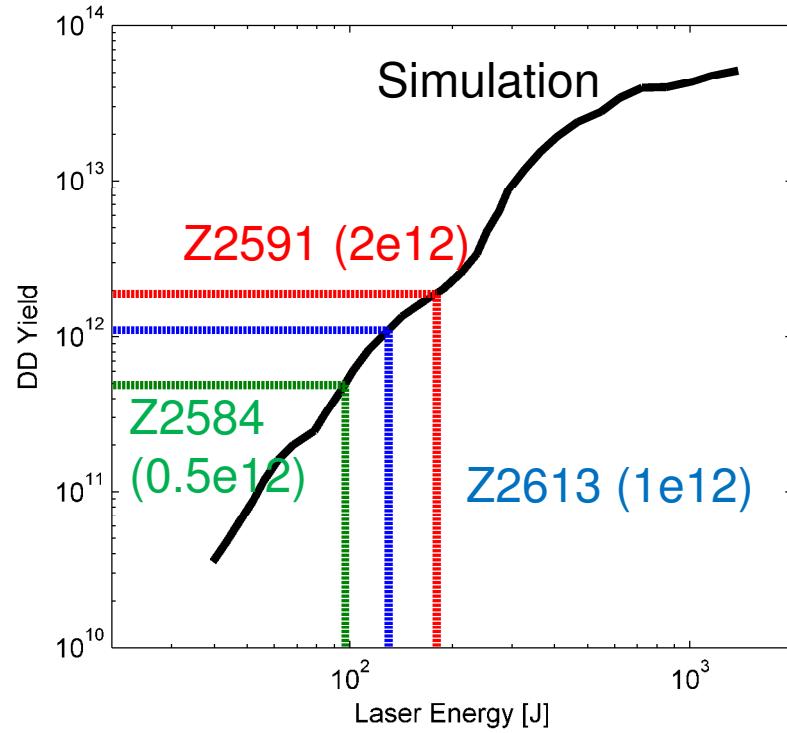
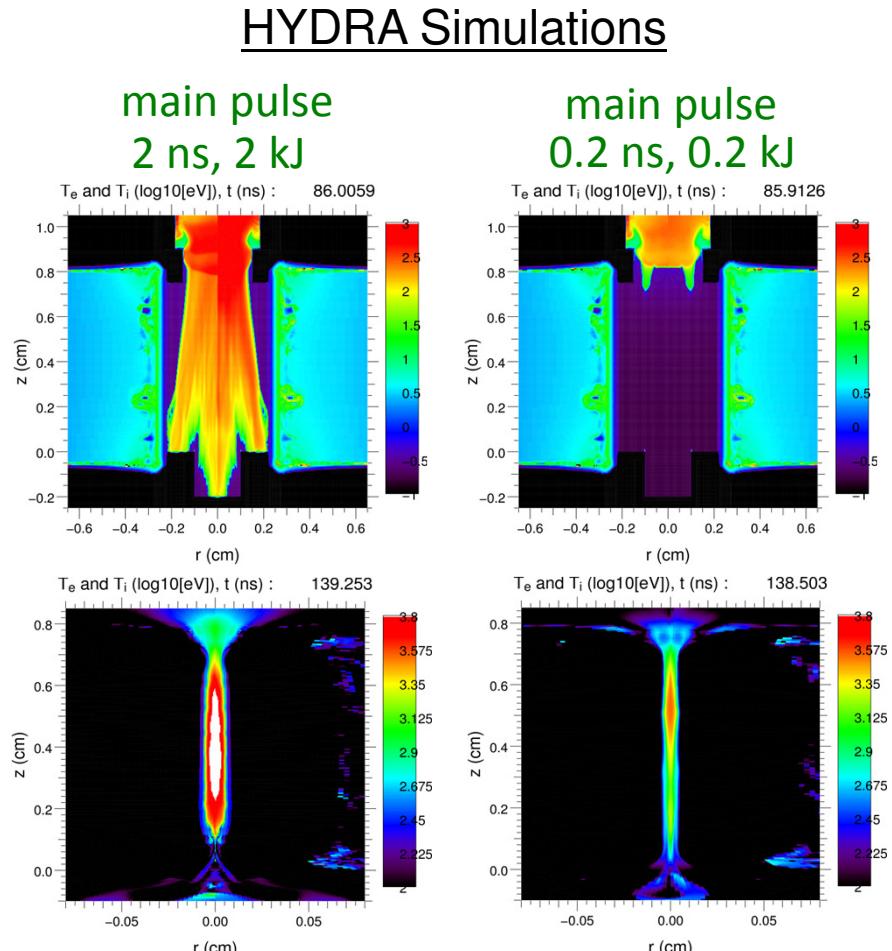
Synthetic diagnostics:

Neutrons (sample $\sqrt{T_i}$) $\langle T_i \rangle = 2.5 \text{ keV}$

X-rays (sample $\partial j / \partial \varepsilon$) $\langle T_e \rangle = 3.1 \text{ keV}$

$$Y_{\text{DD}} = 2 \times 10^{12}$$

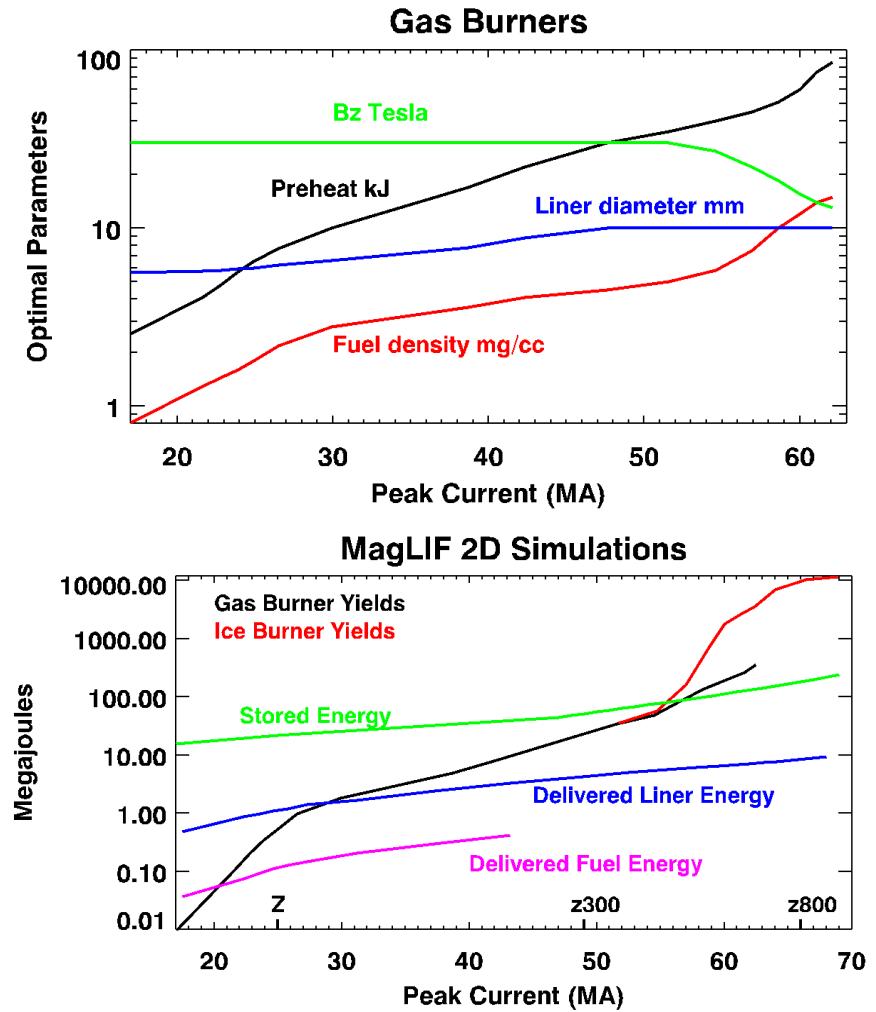
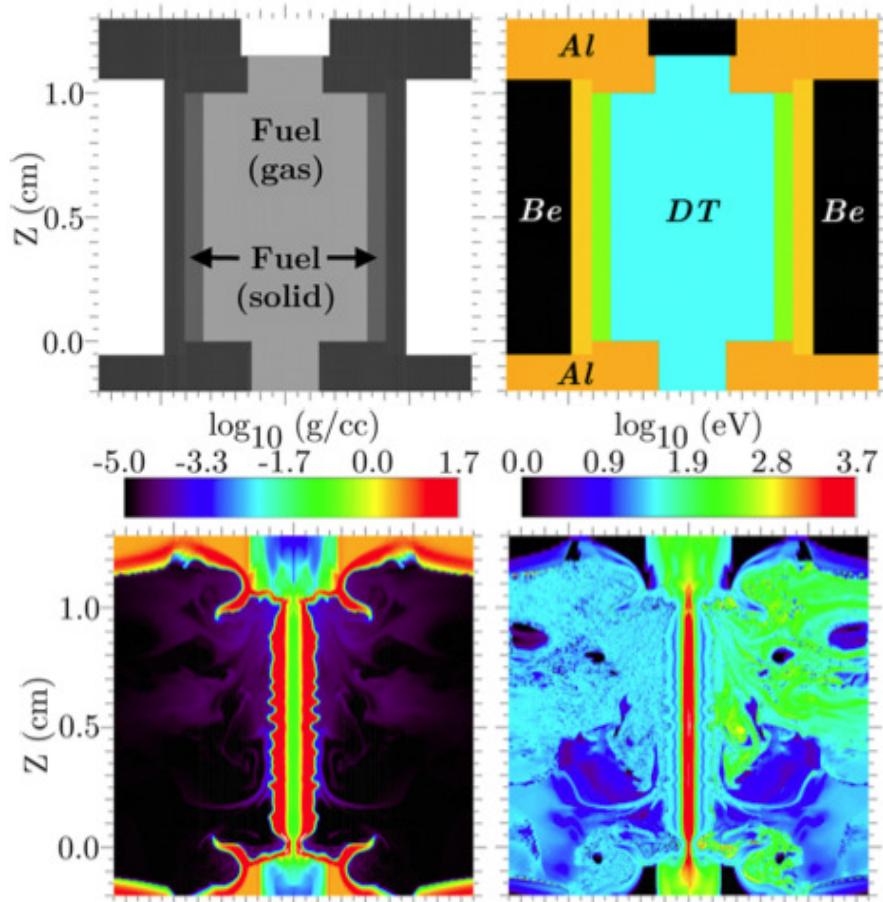
Simulations and scaling: Degraded-preheat simulations are consistent with 10^{12} DD yields; improving the preheat (without increasing mix) would increase yields > tenfold



Simulations with 100-200 J preheat match both yields and the detailed parameters measured in the experiments (temperature, shape, BR)

A.B. Sefkow *et al.*, Phys. Plasmas (2014).

Simulations and scaling: alpha heating and ignition are possible on a future facility using a cryogenic DT layer and substantial preheat—we are testing the physics of these targets on Z today

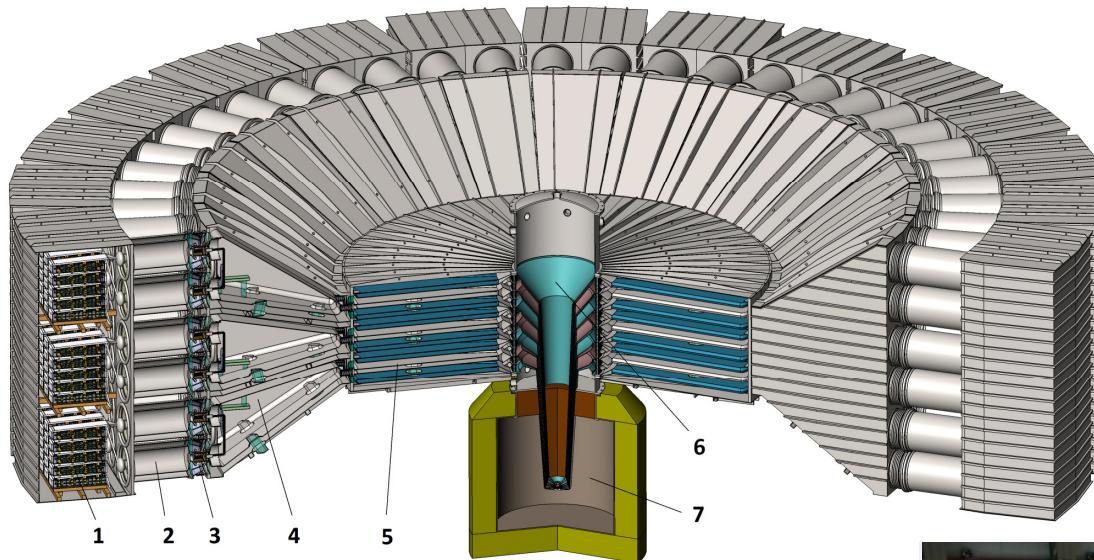


We are engaging with multiple collaborators to help us improve our understanding



- **U. Rochester:** Collaboration on “mini-MagLIF” will examine MHD modeling of magnetic flux loss; preheat studies are underway
- **LLNL:** Scaled preheat NIF experiments are planned along with code workshops and development; extensive collaboration on diagnostics
- **NRL:** Developing validation tools (Noh, Nernst) and radiation transport models
- **Universities** (Cornell, U. Mich, U. Nevada): Implosion validation experiments are underway along with active development of extended MHD models
- **Private companies:** Voss Scientific and Prism Computational Sciences are developing PIC/Hybrid PIC and experimental design and diagnostic tools
- **International Collaborators:** Fundamental work on transport properties at U. Kiel – more interaction would be welcome!

There is growing international interest in pulsed-power ICF



Russian Facility (Baikal)

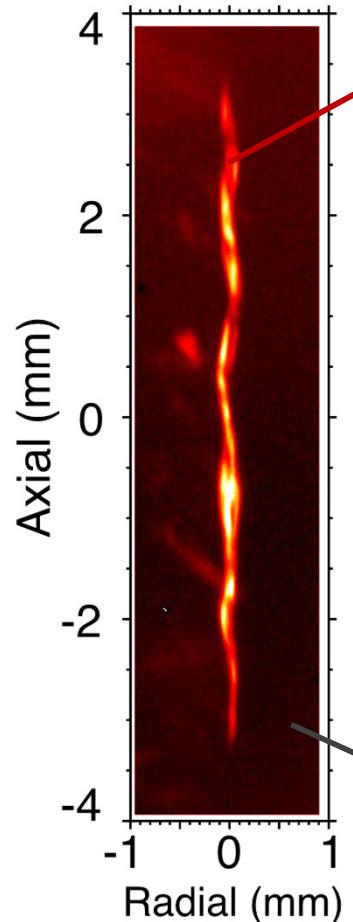
- 50 MA
- 150 ns
- 100 MJ ($4 \times Z$)
- Stated goal: **25 MJ fusion yield**
- Scheduled for completion in 2019, funding is secured and there is activity
- If it works, they will have this capability before any realistic scenario for Z-300

Operating Chinese Facility (PTS)

- 8 MA
- 100 ns
- 8 MJ ($1/3 \times Z$)
- Successfully duplicating previous published work worldwide
- Building a 1 ns, 1 kJ laser facility like Z-Beamlet!
- Currently evaluating LTD and Marx-based architectures



MagLIF experiments are extensively diagnosed, demonstrating significant promise for Magnetized Inertial Fusion



X-ray image of a MagLIF plasma that produced 10^{12} DD neutrons

Stagnating plasma
 $\langle T \rangle = 3 \pm 0.5 \text{ keV}$
 $T(r) = T_0[1 - (r/R)^2]$
 $t_{\text{burn}} = 1.5 \pm 0.5 \text{ ns}$
 $f_{\text{Be}} = 5 \pm 3 \%$
 $z = 5 \pm 2 \text{ mm}$
 $R = 50 \pm 20 \mu\text{m}$
 $\rho_D = 0.3 \pm 0.1 \text{ g/cm}^3$
 $P(z) = 1 \pm 0.2 \text{ Gbar}$
 $\rho R \sim 1.5 \text{ mg/cm}^2$
 $BR \sim 0.4 \text{ MG}\cdot\text{cm}$

Confining liner
 $\rho r_{\text{liner}} = 0.9 \text{ g/cm}^2$

Magnetized Liner Inertial Fusion (MagLIF) has the potential to produce high fusion yields by exploiting:

- 1) magnetic confinement that relaxes required pressures 100x (present experiments trap ~40% of fast fusion products)
- 2) a highly efficient driver delivering ~1% of its stored energy *to the fuel*
- 3) Symmetric drive and slow, low-convergence implosions that are robust against instabilities

Many researchers are contributing to the MagLIF effort:



T.J. Awe, C.J. Bourdon, G.A. Chandler, P.J. Christenson, M.E. Cuneo, M. Geissel, M.R. Gomez, K.D. Hahn, S.B. Hansen, E.C. Harding, A.J. Harvey-Thompson, M.H. Hess, C.A. Jennings, B. Jones, M. Jones, R.J. Kaye, P.F. Knapp, D.C. Lamppa, M.R. Lopez, T. Nagayama, M.R. Martin, R.D. McBride, L.A. McPherson, J.S. Lash, K.J. Peterson, J.L. Porter, G.A. Rochau, D.C. Rovang, C.L. Ruiz, S.E. Rosenthal, M.E. Savage, P.F. Schmit, A.B. Sefkow, D.B. Sinars, S.A. Slutz, I.C. Smith, W.A. Stygar, R.A. Vesey, E.P. Yu

Sandia National Laboratories, Albuquerque, NM

B.E. Blue, D.G. Schroen, K. Tomlinson

General Atomics, San Diego, CA

M.C. Herrmann, D. Ryutov

Lawrence Livermore National Lab, Livermore, CA

R. Betti, E. M. Campbell, J. Davies, M. Wei

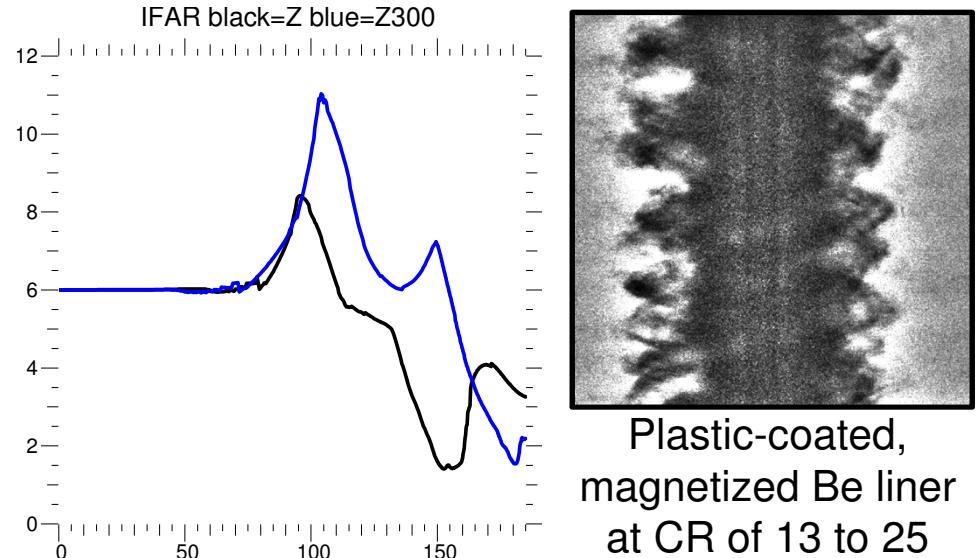
Laboratory for Laser Energetics, U. Rochester

Questions?



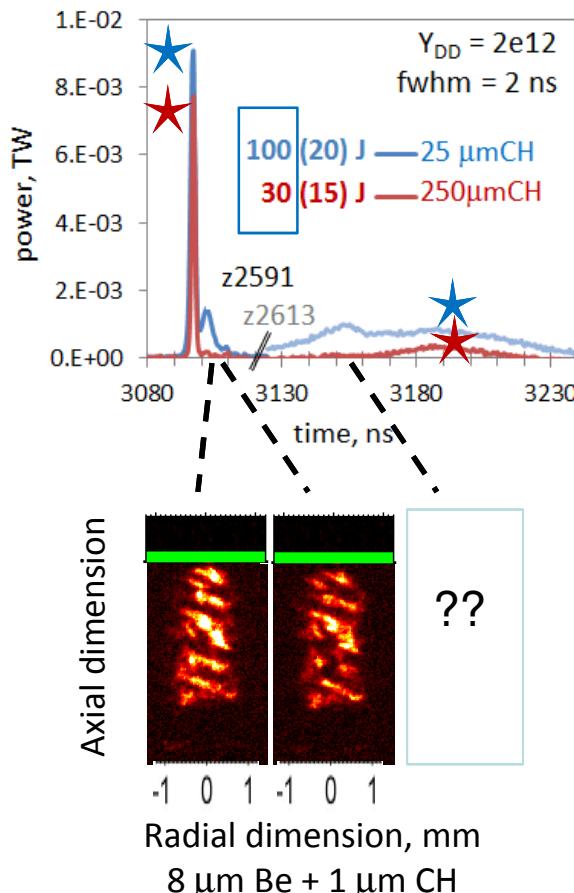
Relative to the primary ICF approach, MagLIF uses a robust fuel compression method and (largely) untested magneto-inertial fusion principles

Metric	X-ray Drive on NIF	100 kJ MagLIF on Z
Pressure	~140-160 Mbar	26 MA at 1 mm is 100 Mbar
Force vs. Radius	Goes as R^2	Goes as $1/R^2$
Peak velocity	350-380 km/s	70-100 km/s
Peak IFAR	13-15 (high foot) to 17-20 (low foot)	8.5
Hot spot CR	35 (high foot) to 45 (low foot)	25
Volume Change	43000x to 91000x (high & low foot)	625x
Fuel rho-R	>0.3 g/cm ²	~0.003 g/cm ²
Liner rho-R	n/a	>0.3 g/cm ²
BR	n/a	>0.5 MG-cm
Burn time	0.15 to 0.2 ns	1 to 2 ns
T _{ion}	>4 keV	>4 keV



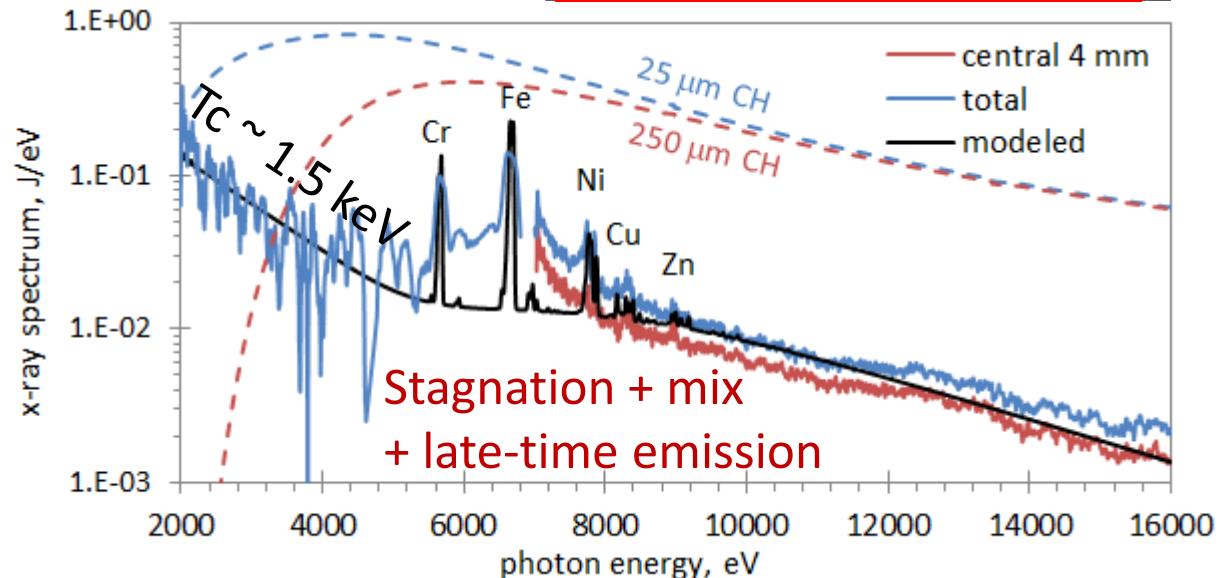
- Magnetic drive is fundamentally different than x-ray or laser-driven ablation
- By traditional ICF implosion metrics MagLIF is very conservative, though different physics
- Reaching fusion conditions relies on largely untested MIF principles
 - Long stagnation time (2 ns) → more susceptible to high-Z contamination
 - Magnetic suppression of heat transport

Late-time emission contributes to low-energy spectral region

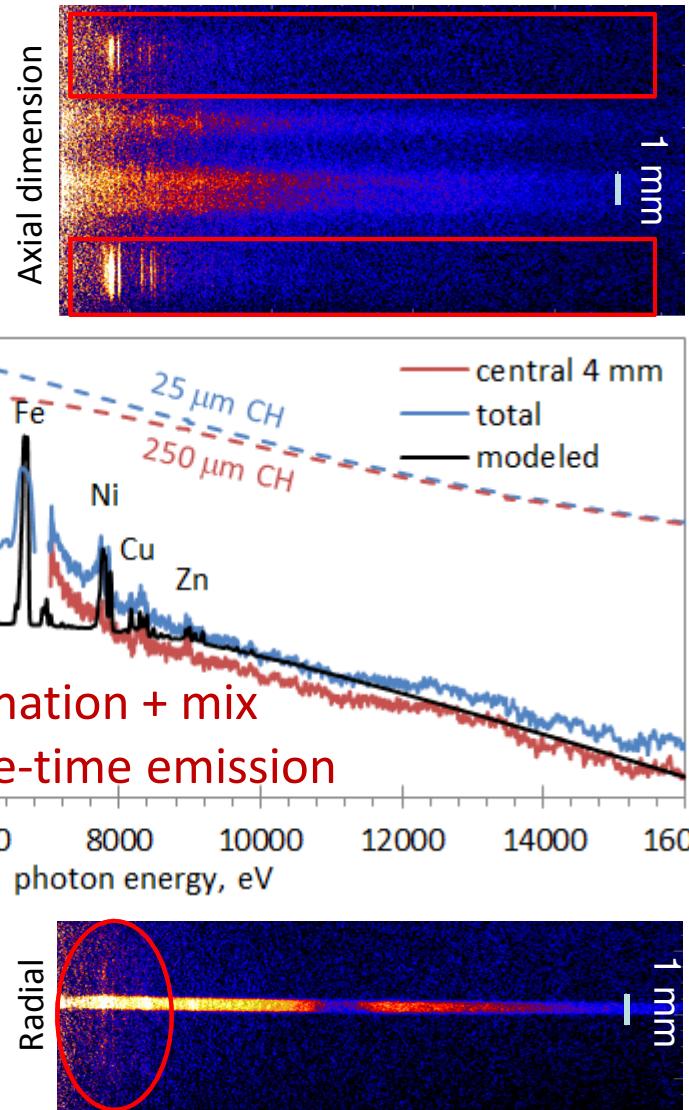


Images constrain late-time emission area:
 $R \sim 1 \text{ mm}$, $Z \sim 8 \text{ mm}$

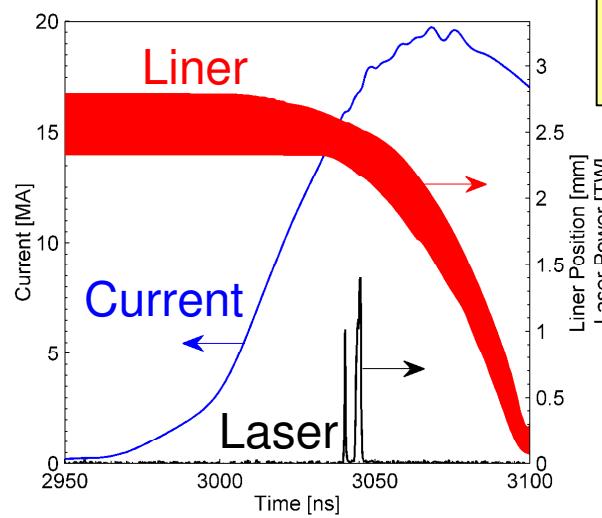
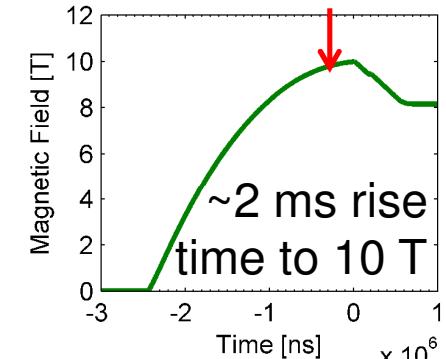
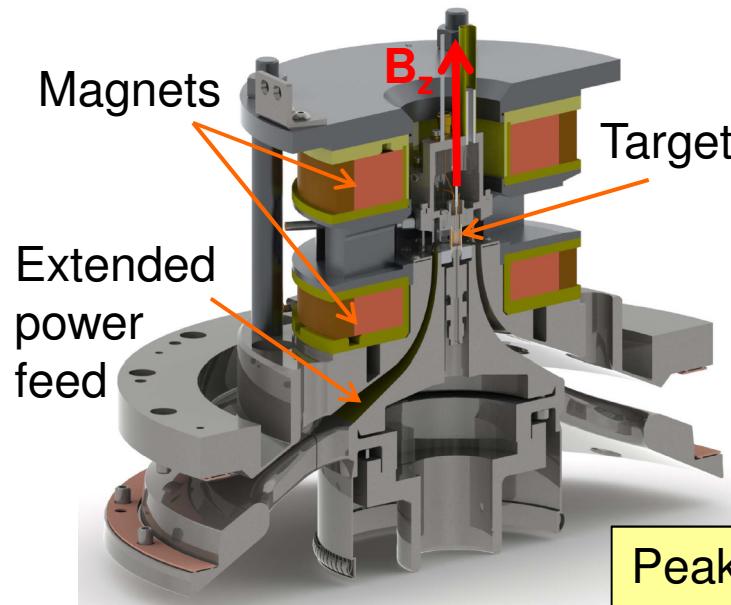
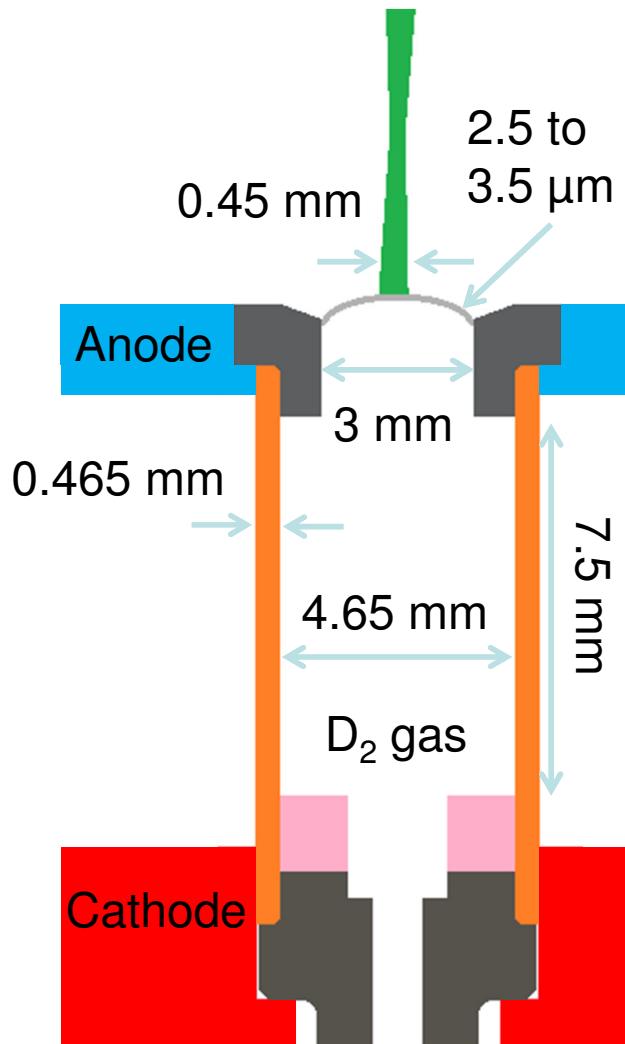
Powers constrain participating mass:
 $\rho R_{\text{Be}} \sim 1 \mu\text{g/cm}^2$
 $\rho R_{\text{stainless}} \sim 10 \text{ ng/cm}^2$



Spectrum constrains temperature:
 $T_e \sim 1.5 \text{ keV}$



The initial experiments used 10 T, 2.5 kJ laser energy, and 19-20 MA current to drive a D₂ filled (0.7 mg/cc) Be liner



Peak current is 19 MA
Magnetic field is 10 T
Total laser energy is 2.5 kJ

