



U.S. DEPARTMENT OF  
**ENERGY**

**Nuclear Energy**

SAND2015-4410C

**An Integrated Approach to Closing the  
Technical Data Gap for High Burnup  
Spent Fuel Performance during Normal  
Conditions of Transport (NCI)**

**IAEA International Conference on  
Management of Spent Fuel From  
Nuclear Power Reactors**

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## UFD Storage & Transportation

# Objectives of the US DOE Used Fuel Disposition (UFD) Storage and Transportation R&D Program:

*Contribute to:*

- 1. the technical bases to demonstrate used fuel integrity for extended storage periods*
- 2. the technical bases for fuel retrievability and transportation after long term storage*
- 3. the technical basis for transportation of high burnup fuel*

### **Focus of this presentation:**

**Progress and status on the ability for high burnup spent fuel to maintain its integrity during Normal Conditions of Transport.**



# Can high burnup irradiated rods withstand Normal Conditions of Transport (NCT)?

***Assembly shaker  
and truck tests:***  
*measured NCT loads on  
fuel rods*



***Modeling:***  
*models can predict loads  
on rods for other  
transport configurations  
or environments*



***Irradiated rod  
testing:***  
*material properties to  
compare with measured  
NCT loads*





# Can high burnup irradiated rods withstand NCT?

## Cladding and fuel behavior of high burnup used fuel under NCT loading

*Fundamentally:*

***applied stress/strain < material strength***

*Requires determination of:*

- *Applied loads on the fuel*
- *Measured strains*
- *Calculated stresses*

*Requires determination of:*

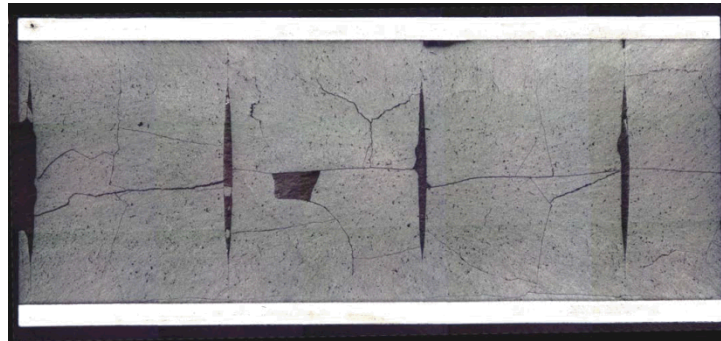
- *Material properties*
  - *Yield/ultimate strength*
  - *Ductility*
  - *Fracture toughness*
- *Constitutive relationships*
- *Pellet-clad interaction*



## Material properties:

- Cladding and fuel response to cyclic loadings
- Pellet/clad interaction (PCI)

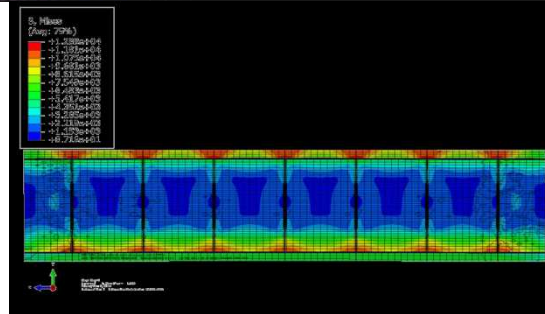
Note: NRC initiated and funded development of test apparatus, procedures and data collecting. Funding for FY14 efforts was shared by NRC and DOE.



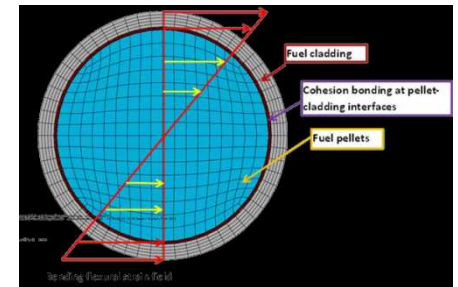
### Pellet-Clad & Pellet-Pellet Bonding



**CIRFT test apparatus  
developed at ORNL**  
CIRFT: Cyclic Integrated  
Reversible-bending Fatigue Tester



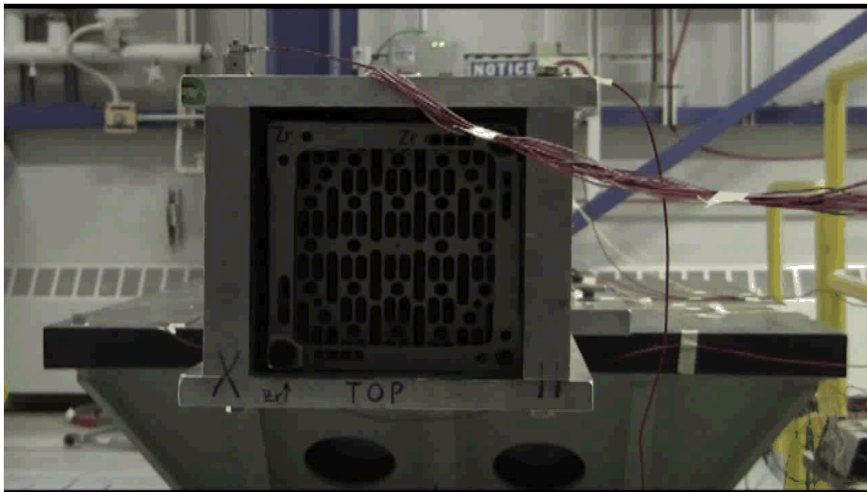
**In-cell measured displacements  
and applied moment provides  
necessary data for  
strain/stress calculations**



**Due to PCI, fuel provides  
structural support  
to applied load**



# Normal Conditions of Transport: Fuel Assembly Loadings



***2013 Shaker Table Test***

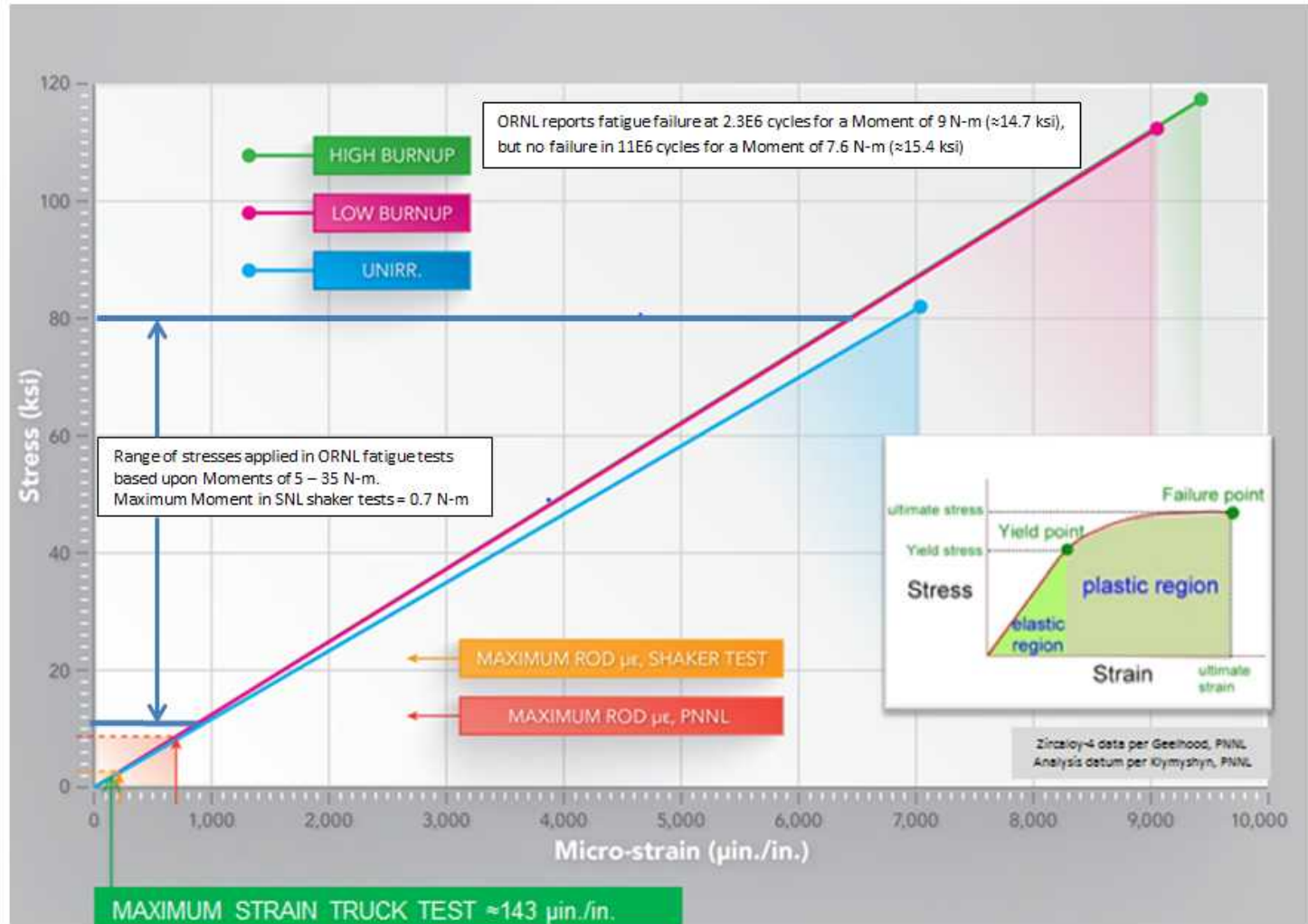


***2014 Over-the-road Test***



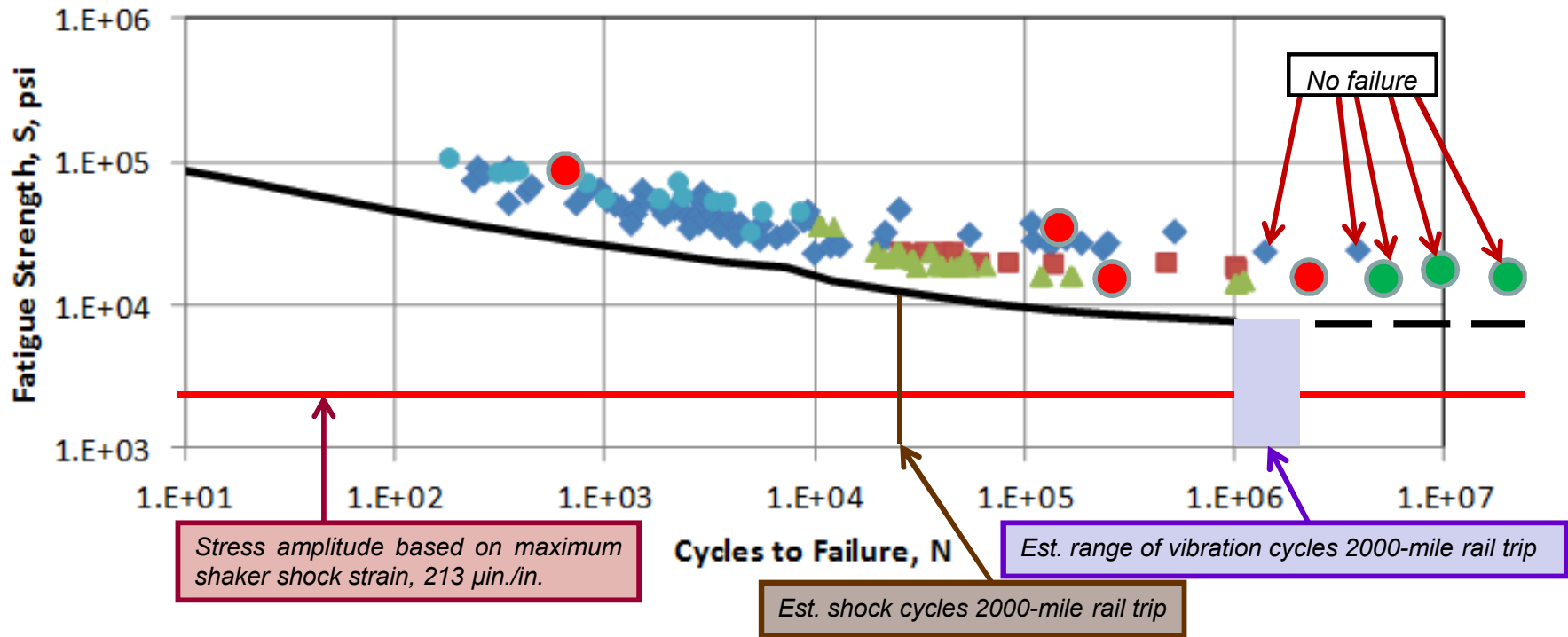


# Measured strains very low relative to elastic limit of Zircaloy





# *NCT vibrations unlikely to result in fatigue failure*



Fatigue design curve ( — ): O'Donnel and Langer, "Fatigue Design Basis for Zircaloy Components," Nucl. Sci. Eng. 20, 1, 1964. (cited in NUREG-0800, Chapter 4)

Data plot courtesy of, PNNL  
The large circles are courtesy of ORNL





## Nuclear Energy

- The strains measured on the rods in the surrogate PWR assembly NCT test program were in the micro-strain levels
- Strains on irradiated fuel rods during NCT may be less than strains measured on the unirradiated Zircaloy-4
- Preliminary investigations suggest that HBU spent fuel will maintain its integrity when subjected to NCT loading conditions

### *Anecdotal evidence from transport experience in France:*

- More than **75,000 LWR used fuel assemblies** transported  
(from France, Japan, Germany, Belgium, Switzerland, the Netherlands, Italy)
- About **7,500 loaded casks** with LWR assemblies
- La Hague reprocessing plant has received **15,156 assemblies ... with burn-up greater than 45GWd/tU** (from EDF).  
[Fuel shipped with relatively short cooling times]
- No assemblies have ever been damaged during transport
- Experience and gap analysis on used fuel and dry cask components behaviour in transport and storage

*Herve ISSARD, TN International - ESCP International subcommittee meeting, Tokyo 2014*

AREVA TN information EDF website: [http://www.edf.com/fichiers/fckeditor/Commun/En\\_Direct\\_Centrales/Nucleaire/General/Notes\\_Info/Note\\_info\\_transport\\_comb\\_dechets\\_nucl\\_082010.pdf](http://www.edf.com/fichiers/fckeditor/Commun/En_Direct_Centrales/Nucleaire/General/Notes_Info/Note_info_transport_comb_dechets_nucl_082010.pdf)



## Conclusions

- Real progress is being made on understanding the response of high burnup spent fuel to Normal Conditions of Transport.
- Given observed loading data, mechanical property data, and rod stiffness response characteristics, there are positive indications that high burnup spent fuel will maintain its integrity during NCT.
- Work is on-going to further address technical issues:
  - The 30-cm drop will be assessed analytically to estimate spent fuel response to this specific condition
  - Consistency and continuity of PCI along the length of the spent fuel rod
  - Assessment of PCI for low burnup fuel