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Title: Los Alamos National Laboratory Support for Commercial U.S. Production
of 99Mo without the Use of Highly Enriched Uranium

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Los Alamos National Laboratory Support for Commercial U.S. Production of ^{99}Mo without the Use of Highly Enriched Uranium

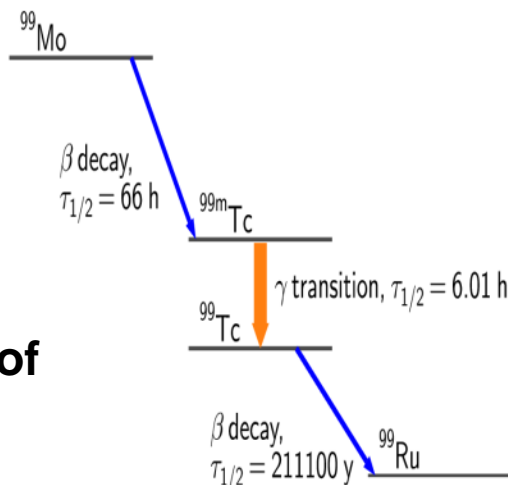
Gregory E. Dale

Los Alamos National Laboratory

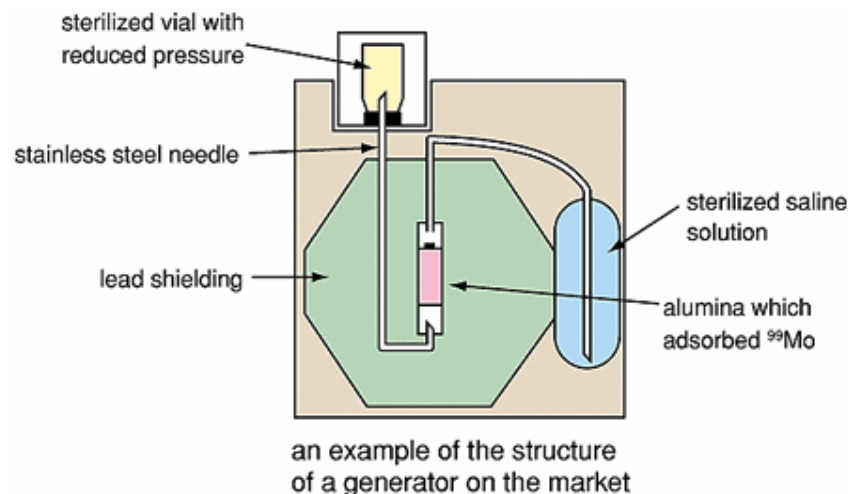


Use and Distribution of ^{99m}Tc

- ^{99m}Tc is used for diagnostic imaging.
 - Decays with a 141 keV internal transition (IT) γ .
 - 6-hour half life.
- ^{99m}Tc is the daughter product of the longer lived ^{99}Mo .
 - 66-hour half life.
 - Allows distribution using radionuclide generators.
- The source of over 99% of the world's supply of ^{99m}Tc from ^{99}Mo is produced from the fissioning of ^{235}U .
- Most of this production uses Highly Enriched Uranium (HEU) targets.

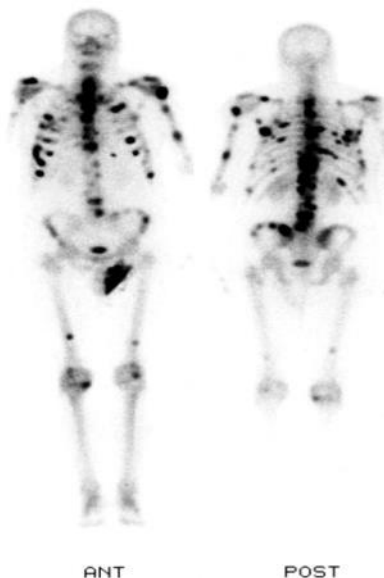


Integrated SPECT/CT Imager

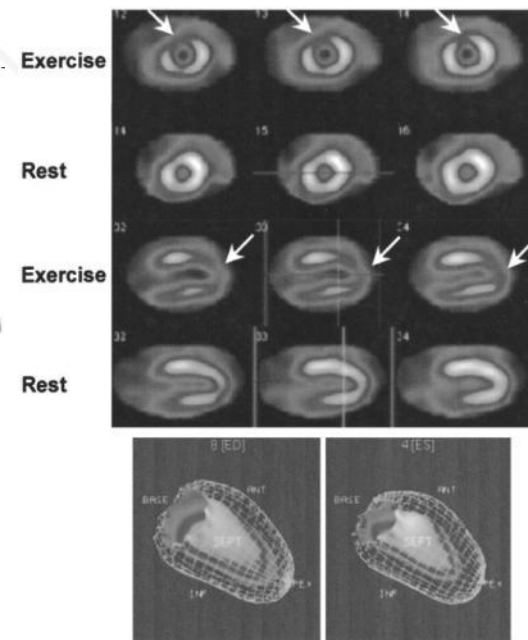


Diagnostic Imaging with ^{99m}Tc

- ~80% of diagnostic imaging procedures in nuclear medicine use ^{99m}Tc , ~50,000 procedures every day in the US.
- The isotope's short half-life and excellent binding properties make it uniquely suited for medical imaging procedures.
- Also due to this short half-life, it must be continuously produced to meet the medical community's requirements.
- Recent production disruptions have seriously impacted the supply of this critical medical radioisotope.



Bone Imaging~ 16%



Heart Imaging ~60%

Global Production of ^{99}Mo

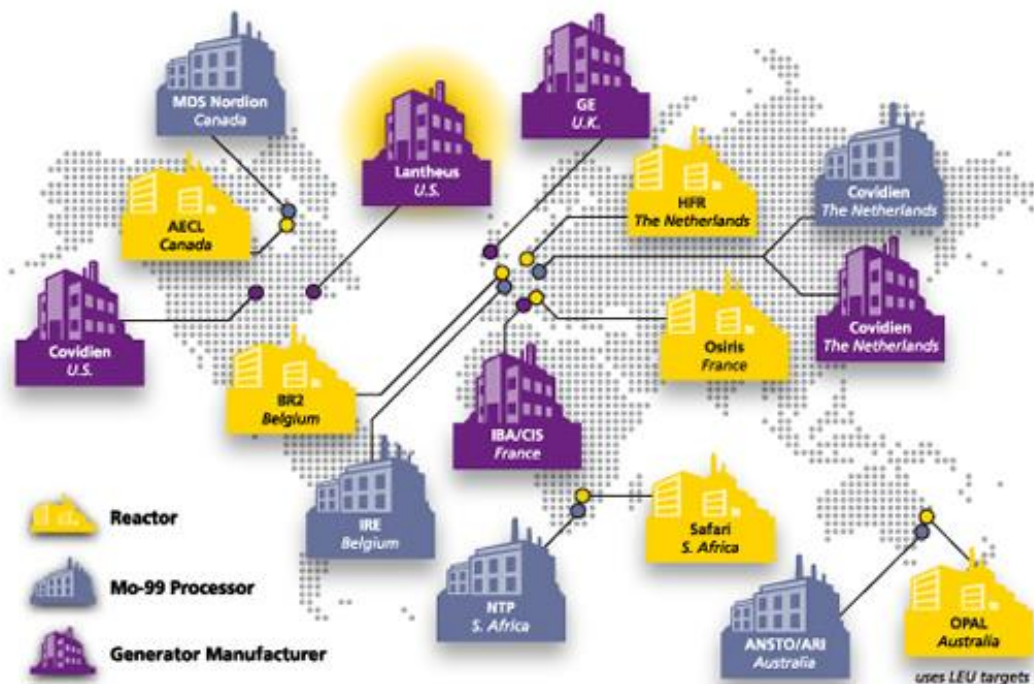
Past major suppliers of ^{99}Mo to the US:

NRU, Chalk River (60%)

HFR, Petten (40%)

NRU and HFR produce 95% of the world supply of ^{99}Mo .

Current Global State of ^{99}Mo Production
<http://www.radiopharm.com/SupplyUpdate/index.html>



- ^{99}Mo is currently produced in nuclear reactors as a fission product from HEU targets (fission yield is 6%).
- There is currently a critical shortage of ^{99}Mo in the US.
 - The US is completely reliant on foreign suppliers for ^{99}Mo .
 - Two major production reactors are currently offline.
 - The crisis is likely to continue or worsen due to the age of the major production reactors and the lack of alternatives.

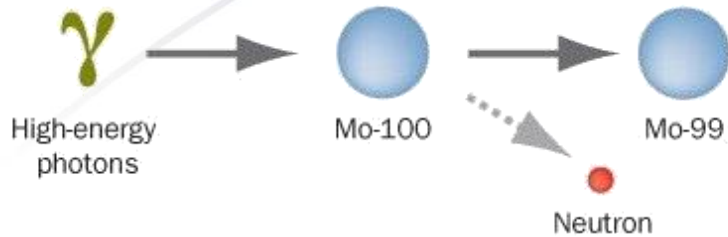
M³ Effort in ⁹⁹Mo Production

- **NNSA Office of Materials Management and Minimization (M³) effort for production of ⁹⁹Mo.**
 - Develop a reliable, domestic, commercial supply of ⁹⁹Mo that avoids a single point-of-failure and does not require the use of HEU.
 - The M³ focus is on commercial production. Federal facilities should not be used for production or distribution. The efforts of the national laboratories are for engineering design support and proof of concept demonstrations.
- **M³ is working on implementing projects to demonstrate the viability of non-HEU based technologies for large-scale production, which include the following technology pathways:**
 - LEU targets (mostly foreign production support)
 - LEU solution reactors (B&W)
 - Neutron capture (GE/Hitachi on hiatus)
 - Accelerator production.
 - DT accelerator driven U solution (Shine/Morgridge)
 - Electron accelerator production (NorthStar)

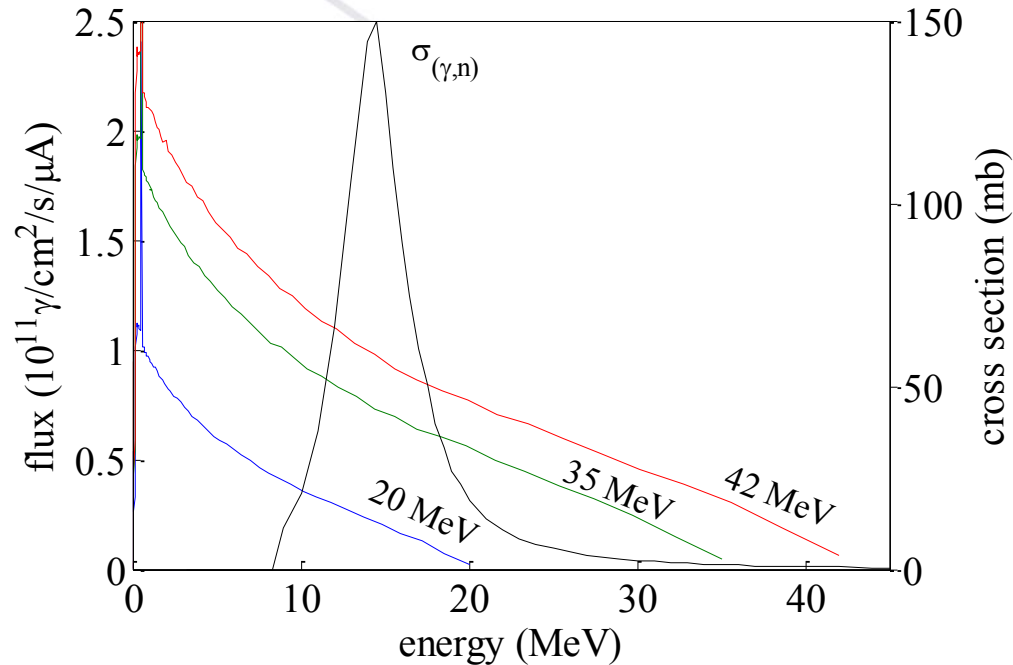
LANL Support for Domestic ^{99}Mo Production

- As part of the M³ Program, LANL is supporting:
 - NorthStar Medical Radioisotopes with the electron accelerator production of ^{99}Mo from $^{100}\text{Mo}(\gamma, n)^{99}\text{Mo}$.
 - Shine Medical Technologies with the production of fission product ^{99}Mo from a DT accelerator driven subcritical uranium salt solution.

NorthStar Electron Accelerator Production



- The NorthStar process uses an electron accelerator to create a high flux of bremsstrahlung photons in enriched ^{100}Mo targets to create ^{99}Mo through the photonuclear reaction $^{100}\text{Mo}(\gamma, n)^{99}\text{Mo}$.
 - Reaction threshold is 9 MeV.
 - Peak cross section is 150 mb at 14.5 MeV.
- We are exploring electron beams in the 35-42 MeV range.



Average bremsstrahlung photon spectra produced with 20, 35, and 42 MeV electron beams in a Mo target compared to the photonuclear cross section of ^{100}Mo .

NorthStar Support Focus Areas

- Production and Thermal Tests at ANL
- Target Design and Testing
 - Target thermal performance
 - Production and radionuclide inventory
- Subsystem Development and Testing
 - Beam diagnostics
 - Target cooling system
 - Control systems
- Production Facility Design Support
 - Local target shielding
 - Beam line design
 - Target removal and conveyance

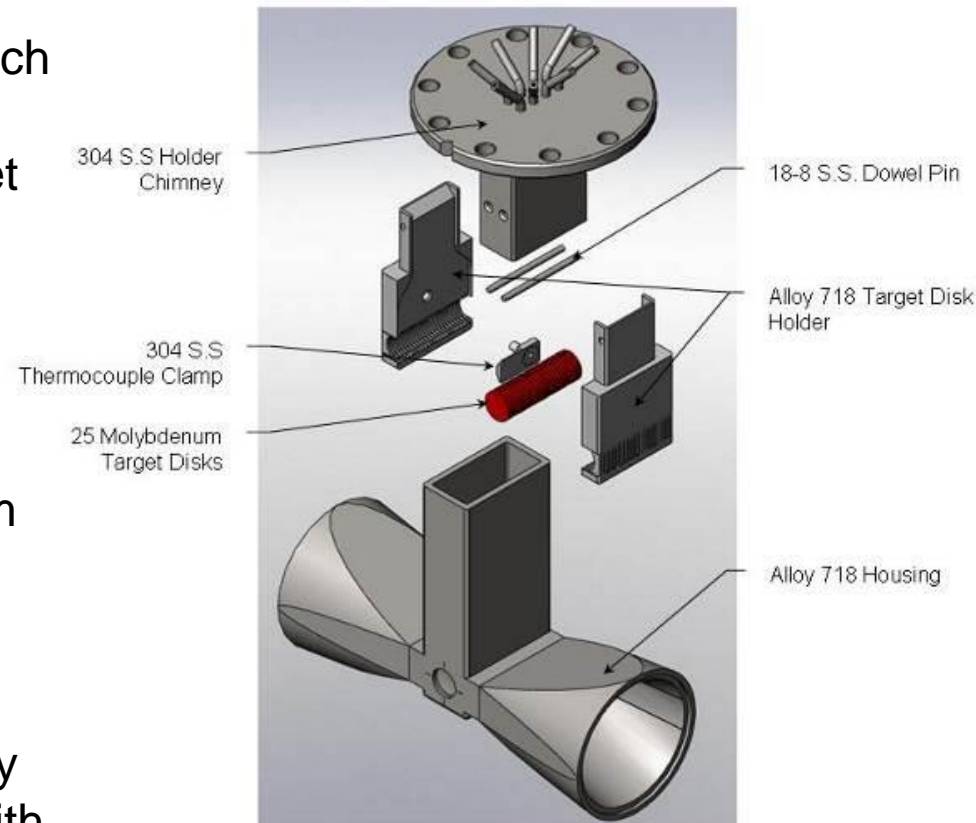
Scaled Accelerator Tests at ANL



Date	Test
April 2010	Water cooled target test using natural Mo targets, produced 236 μCi of ^{99}Mo .
May 2010	Water cooled target test using natural Mo targets, produced 377 μCi of ^{99}Mo .
July 2010	Water cooled production test using enriched ^{100}Mo targets, produced 10.5 mCi of ^{99}Mo .
April 2011	Once through gaseous helium cooled thermal test using natural Mo targets, 145 μCi of ^{99}Mo .
March 2012	Closed loop gaseous helium thermal test using natural Mo targets.
April 2014	Closed loop gaseous helium thermal test using natural Mo targets.
January 2015	35 and 42 MeV thermal tests at 13 kW and 7 kW, respectively. ~ 5 mm FWHM beam.
January 2015	Production Test 1: 42 MeV, 19 hours, 4.8 kW
March 2015	Production Test 2: 42 MeV, 19 hours, 7 kW
March 2015	Production Test 3: 42 MeV, 19 hours, 6 kW
May 2015	Production Test 4: 35 MeV, 24 hours, ~ 8 kW
September 2015	7 Day Production Test: 42 MeV, 156 hours, 8 kW

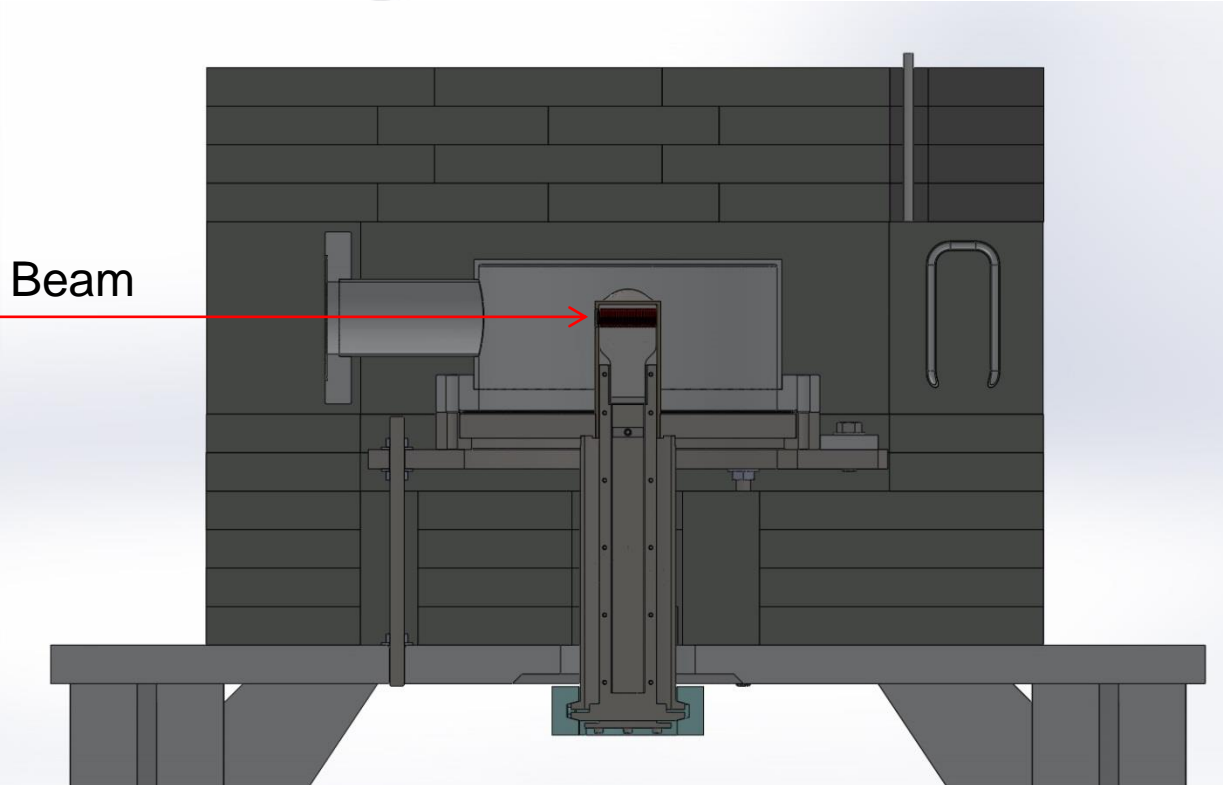
Present Target Design Concept

- Multiple Mo-100 disks held in a holder with gaps in between for coolant flow.
- The beam strikes the target from each end. Exploits target symmetry and doubles production for a given target mass.
- No x-ray converter, x-rays are produced in the Mo-100 target directly.
- The ANL test target disks are 12 mm in diameter, 1 mm thick, with 1 mm gaps, and there are 25 disks in a target.
- Production facility disks are currently 29 mm in diameter, 0.5 mm thick, with 0.25 mm gaps.



NorthStar Target Testing at ANL

Target Side View

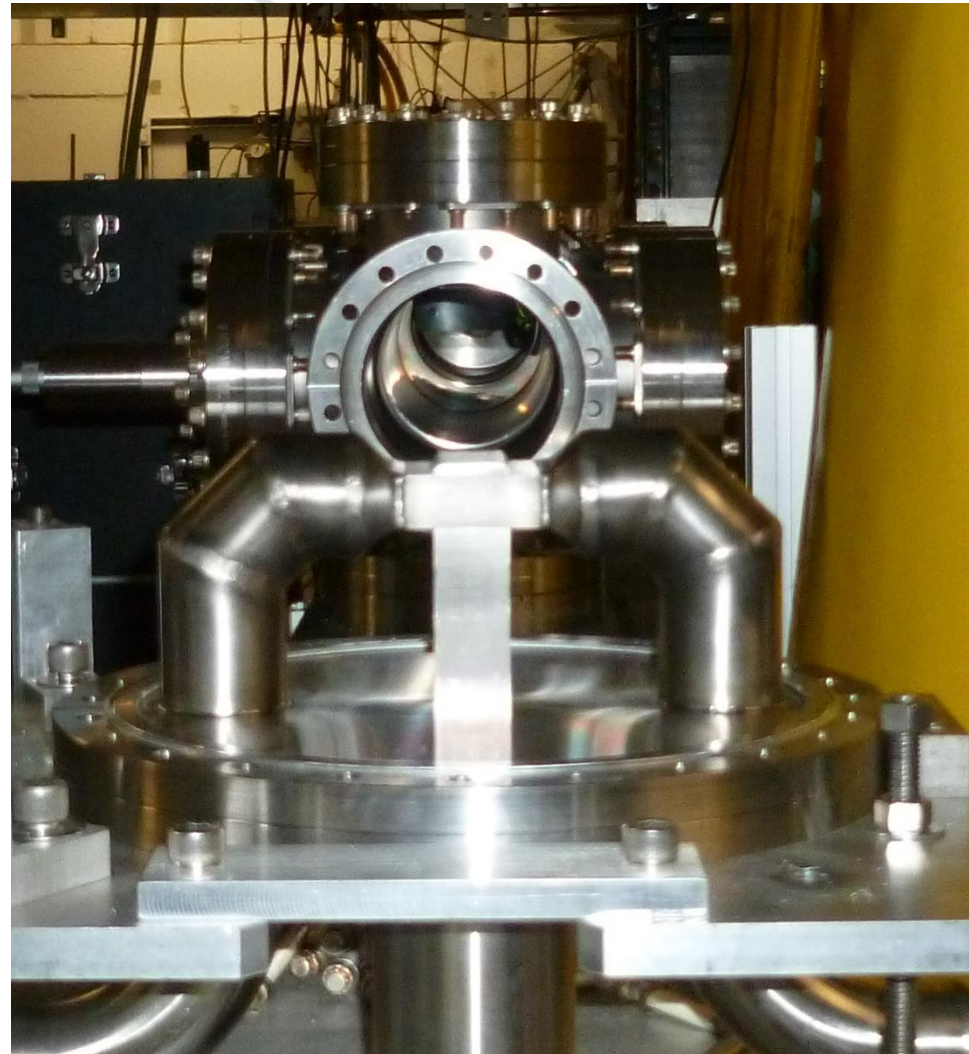
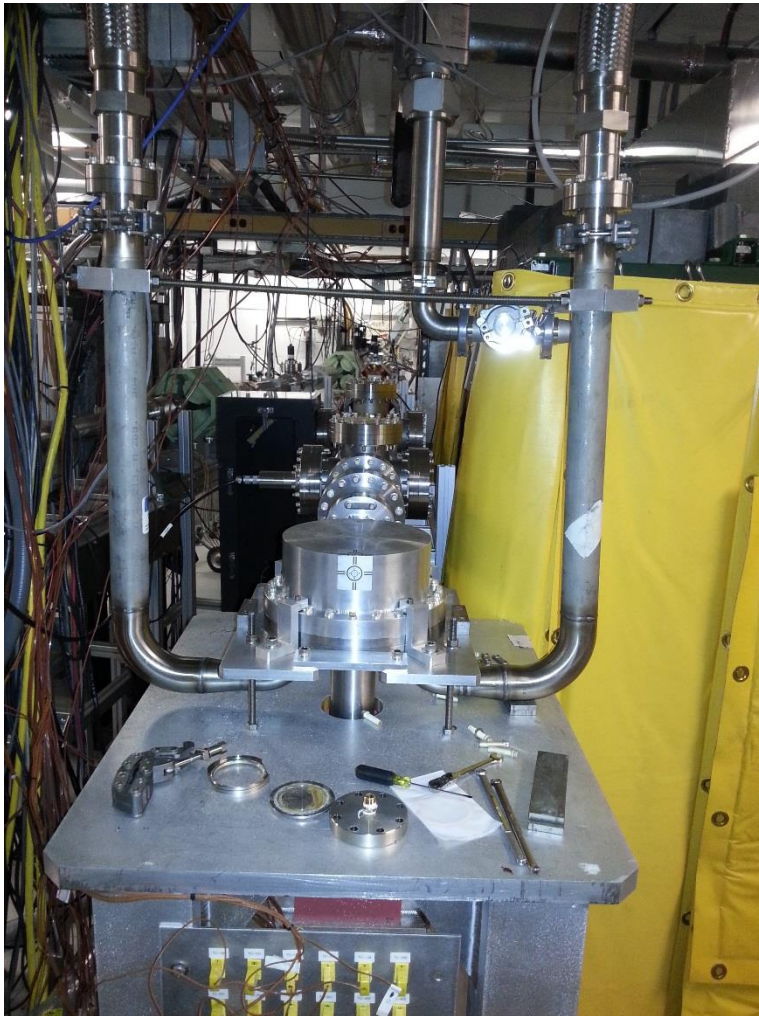


Single sided target for thermal and production tests at ANL

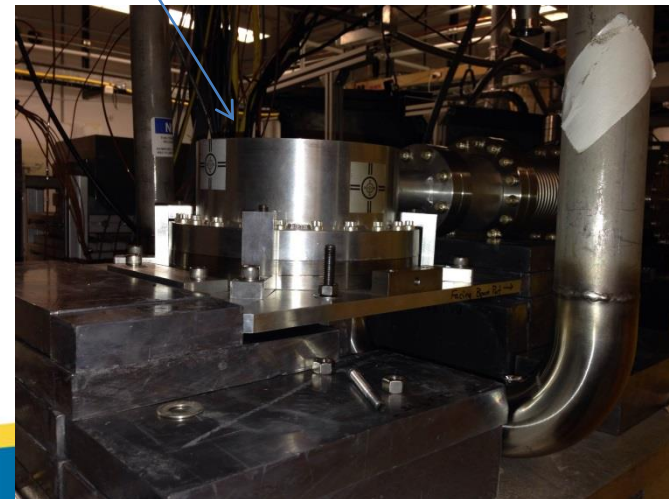
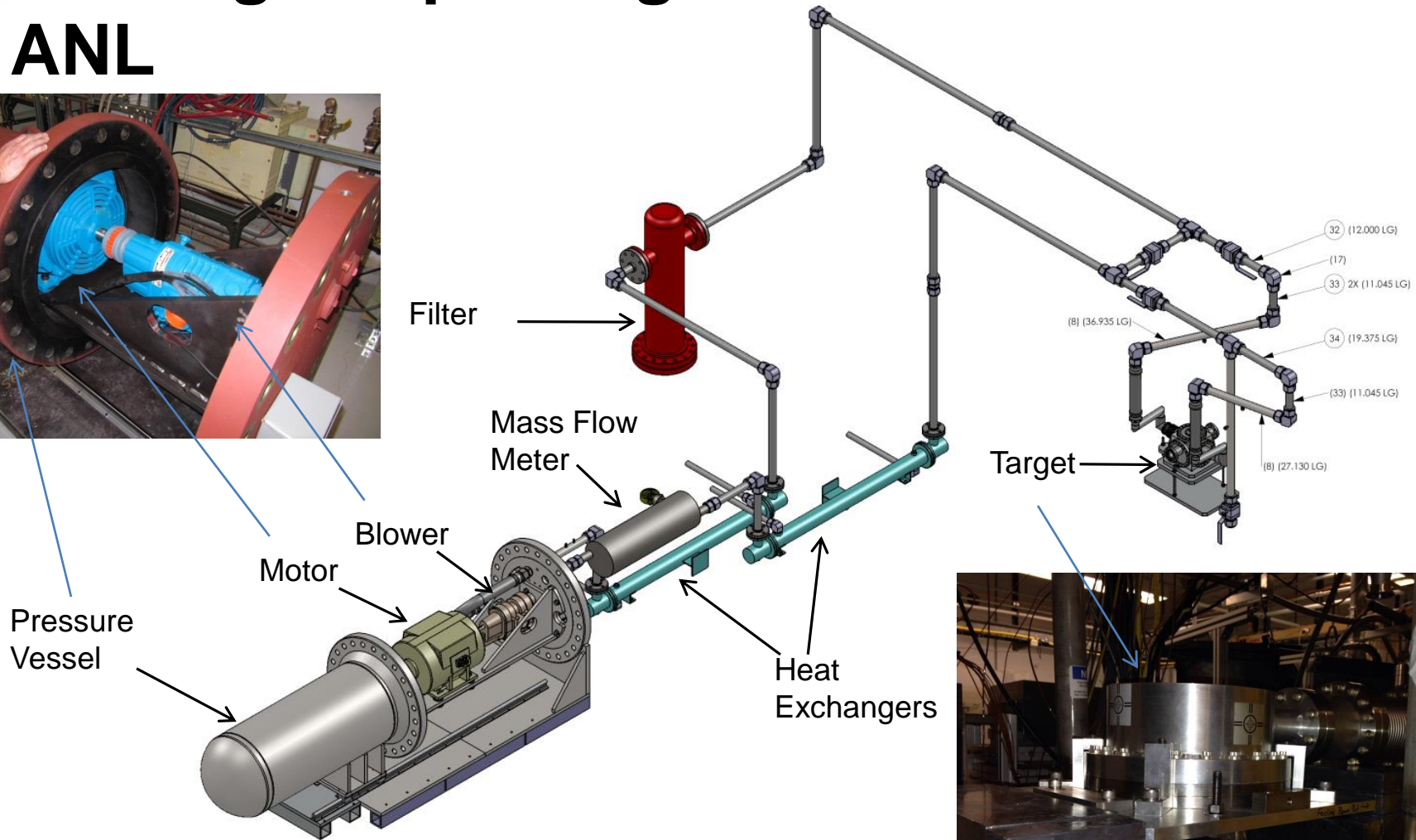
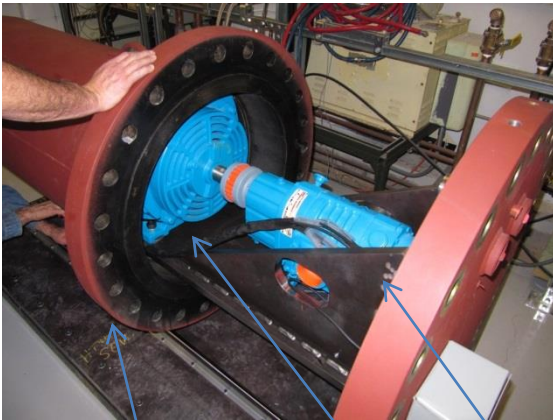


12 mm diameter, 1 mm thick disks, 25 disks in a target for testing at ANL.

April 2014 Thermal Test Setup



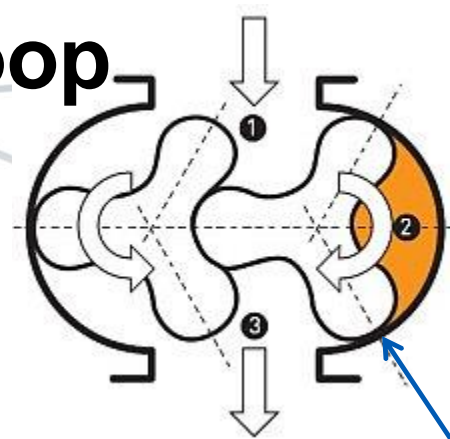
NorthStar Target and Helium Cooling Loop Design Installed at ANL



Gaseous Helium Flow Loop Using a Roots Blower

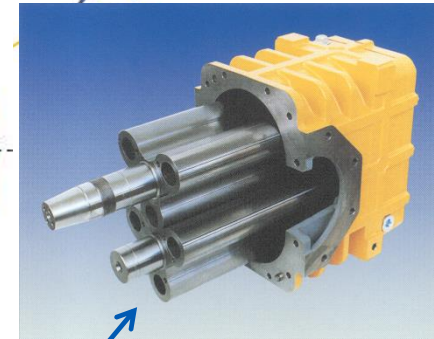
The roots blower is used to move the He through the loop and across the targets. The PV is used to increase the base pressure of the system to 300 psi.

Pressure Vessel (PV)

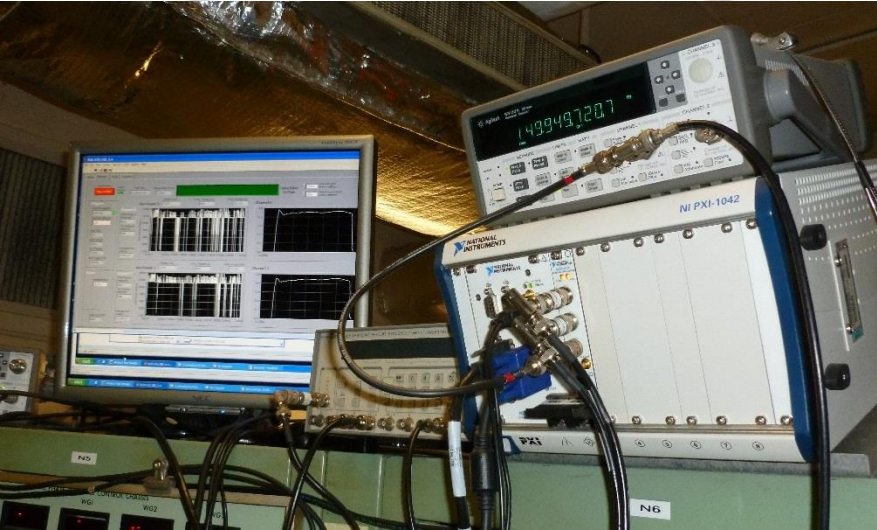


Electric motor

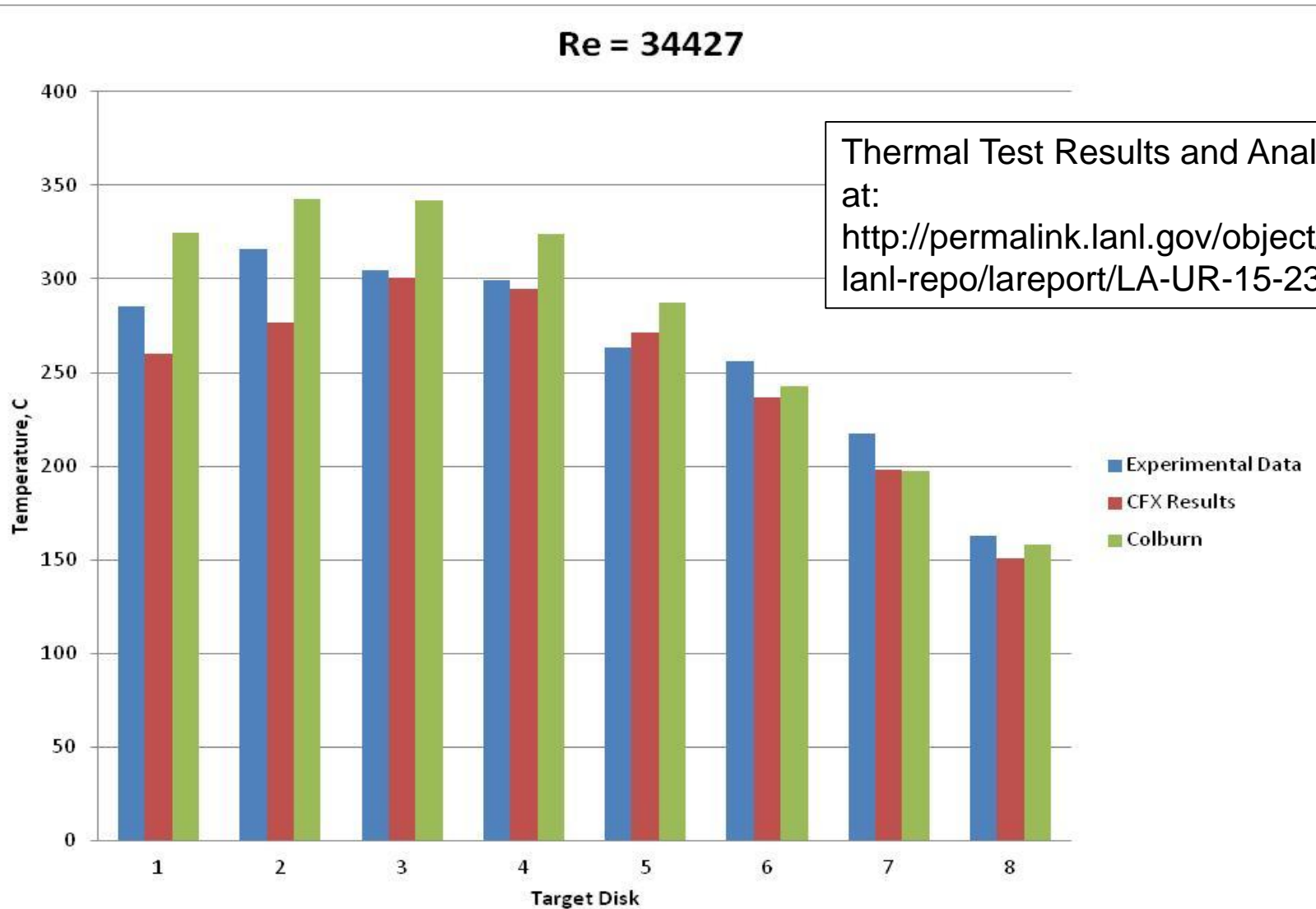
Roots Blower



January 2015 Thermal Test



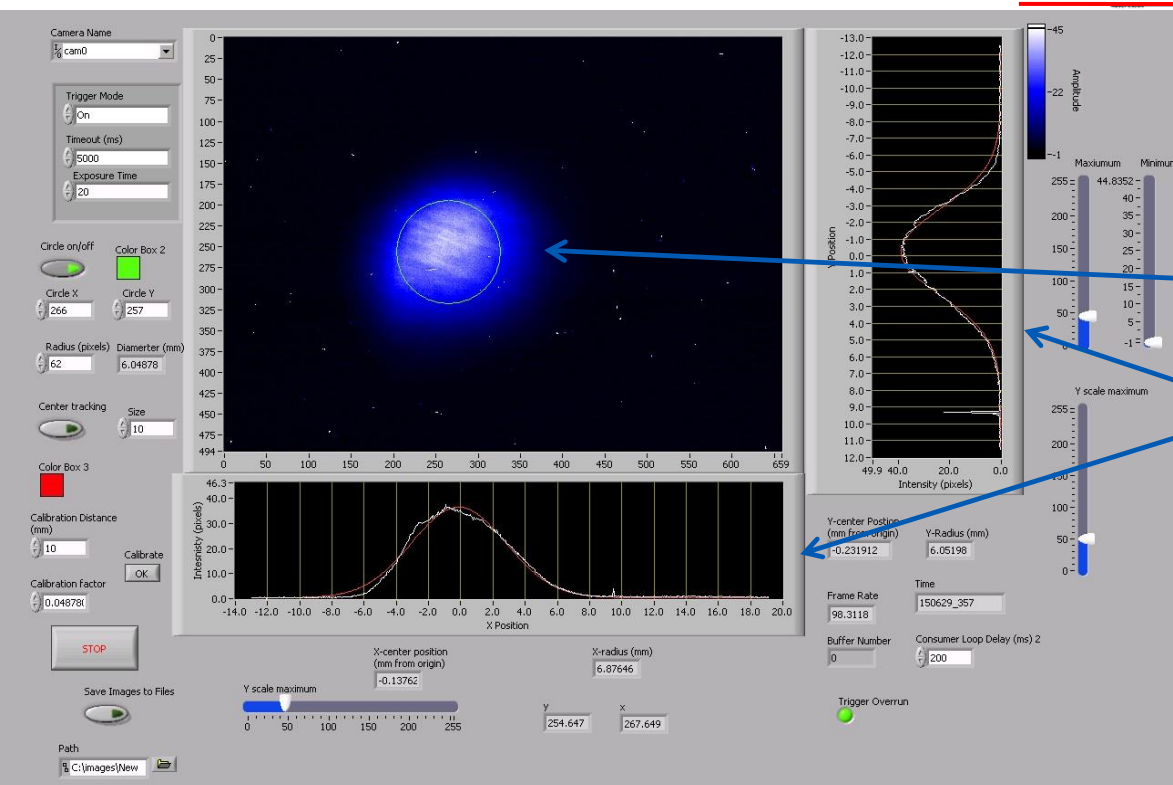
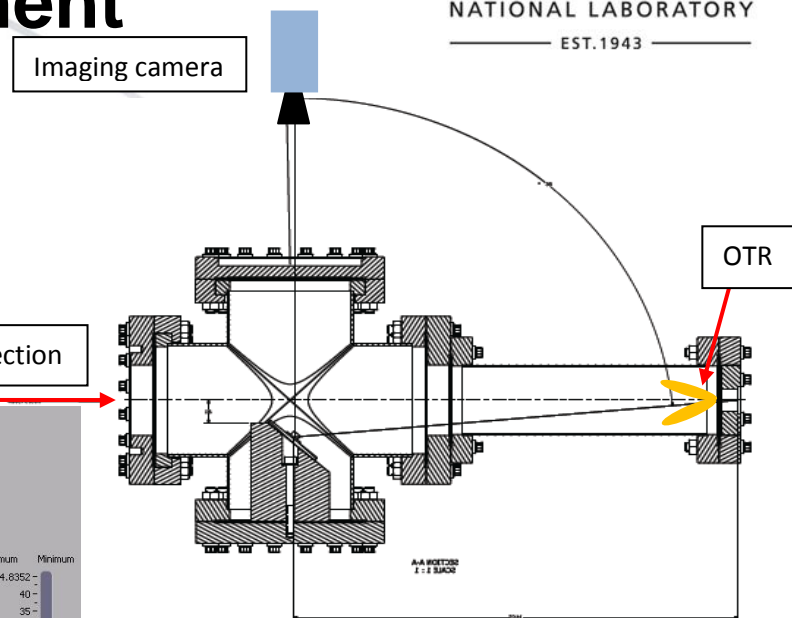
Target Thermocouple Data from the January 2015 Thermal Test 35 MeV, 13 kW beam, 290 psi inlet, 97 g/sec



Thermal Test Results and Analysis available at:
<http://permalink.lanl.gov/object/tr?what=info:lanl-repo/lareport/LA-UR-15-23134>

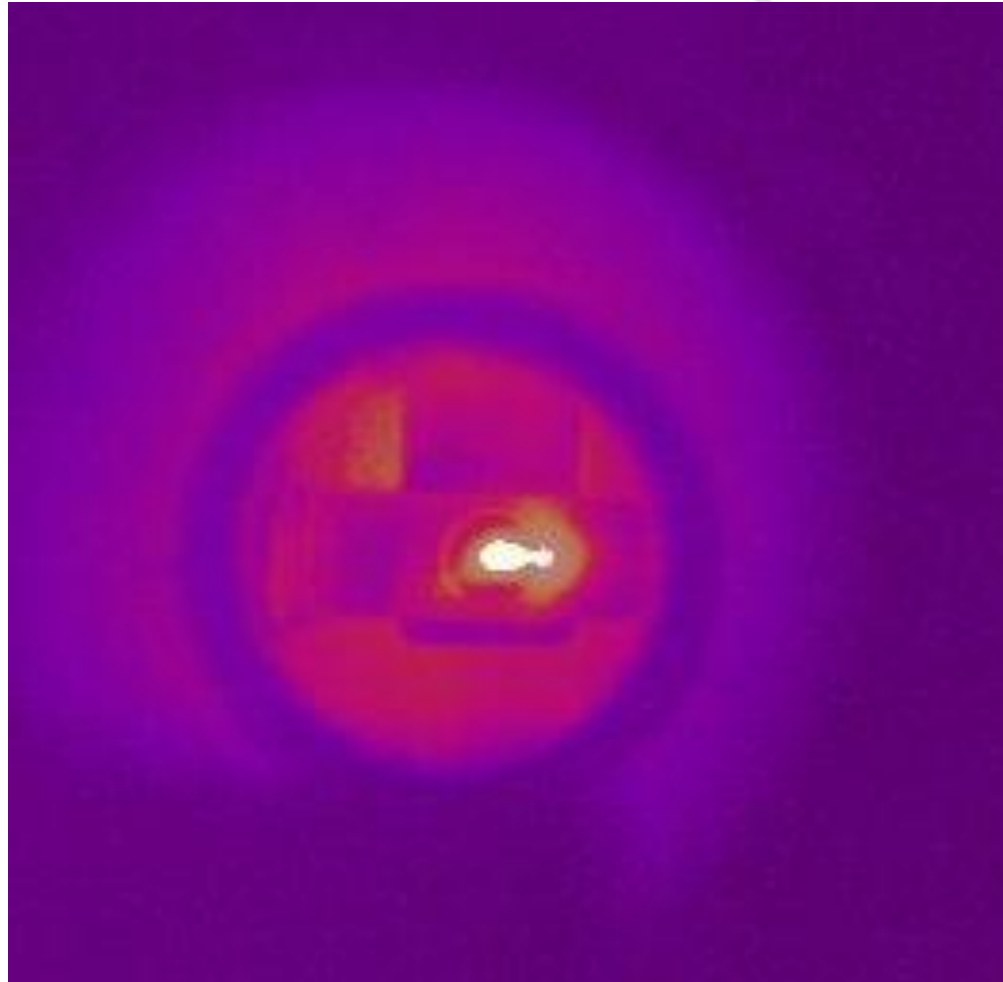
Optical Transition Radiation (OTR) Beam Position and Profile Measurement

To address the beam steering and beam profile monitoring, LANL developed an OTR system looking at the exit window for measuring the ANL beam profile and position during the irradiation.



Beam image
x and y profile measurements

IR Camera Image



ENDF Photonuclear Cross Sections

ENDF/B-VII and TENDL-2012 libraries

ENDF: 163 targets and 4335 reactions

Elements in ENDF shown in blue in the figure below.

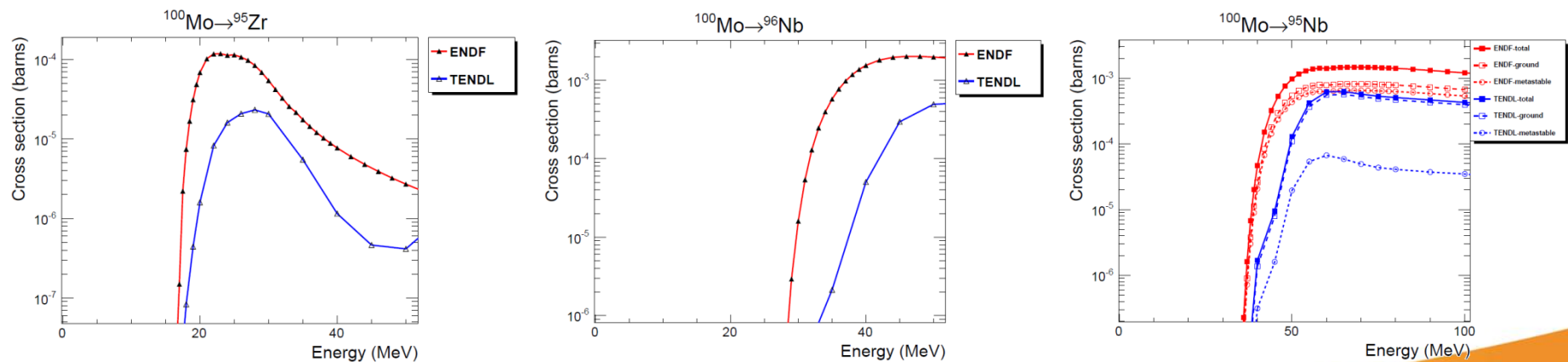
TENDL: 2400 targets and 301,565 reactions

0 n	1 H																	2 He		
	3 Li	4 Be													5 B	6 C	7 N	8 O	9 F	10 Ne
	11 Na	12 Mg													13 Al	14 Si	15 P	16 S	17 Cl	18 Ar
	19 K	20 Ca	21 Sc	22 Ti	23 V	24 Cr	25 Mn	26 Fe	27 Co	28 Ni	29 Cu	30 Zn	31 Ga	32 Ge	33 As	34 Se	35 Br	36 Kr		
	37 Rb	38 Sr	39 Y	40 Zr	41 Nb	42 Mo	43 Tc	44 Ru	45 Rh	46 Pd	47 Ag	48 Cd	49 In	50 Sn	51 Sb	52 Te	53 I	54 Xe		
	55 Cs	56 Ba	57 La	72 Hf	73 Ta	74 W	75 Re	76 Os	77 Ir	78 Pt	79 Au	80 Hg	81 Tl	82 Pb	83 Bi	84 Po	85 At	86 Rn		
	87 Fr	88 Ra	89 Ac	104 Rf	105 Db	106 Sg	107 Bh	108 Hs	109 Mt	110 Ds	111 Rg									
				58 Ce	59 Pr	60 Nd	61 Pm	62 Sm	63 Eu	64 Gd	65 Tb	66 Dy	67 Ho	68 Er	69 Tm	70 Yb	71 Lu			
				90 Th	91 Pa	92 U	93 Np	94 Pu	95 Am	96 Cm	97 Bk	98 Cf	99 Es	100 Fm	101 Md	102 No	103 Lr			

From <http://www.nndc.bnl.gov/sigma/>

Libraries and Side Reactions

- TENDL cross sections are generally lower than ENDF
- More side reaction and impurity activation analysis in future
- Photonuclear metastable production not in ENDF
- Half-lives: Zr-95 64 d, Nb-96 23 h, Nb-95 35 d, Nb-95m 3.6 d



Potassium Photonuclear Evaluation

From T. Kawano, "Evaluation of photo-induced reactions on $^{39,41}\text{K}$,"
LA-UR 14-20531

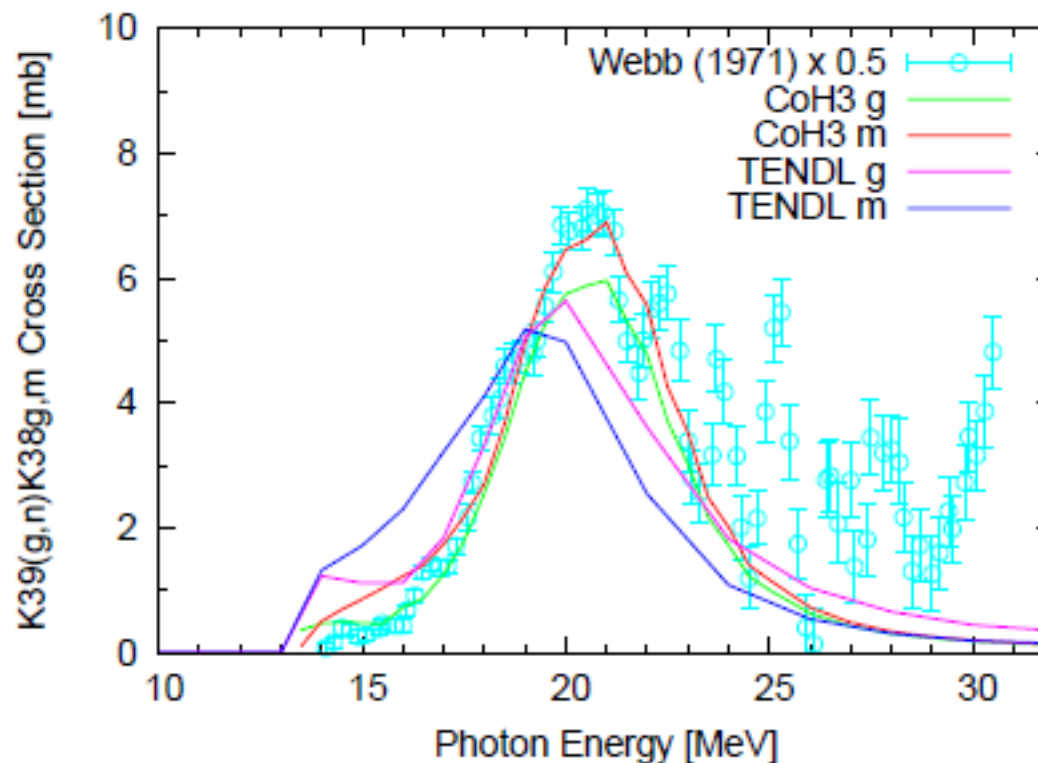
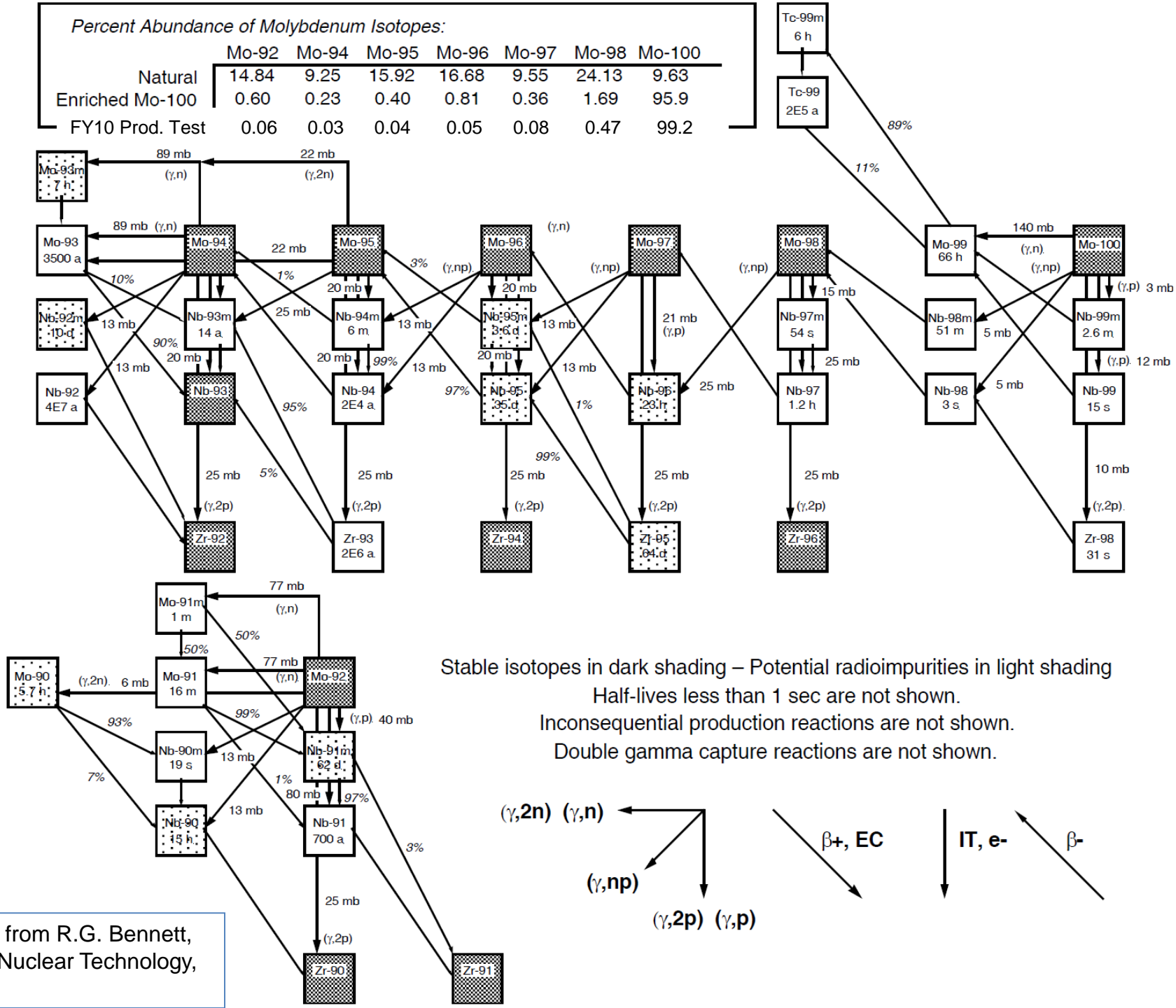


FIG. 3. Calculated ^{38m}K and ^{38g}K production cross sections for the photo-induced reaction on ^{39}K , compared with the experimental data (meta-state), as well as the evaluated data in TENDL.

Percent Abundance of Molybdenum Isotopes:

	Mo-92	Mo-94	Mo-95	Mo-96	Mo-97	Mo-98	Mo-100
Natural	14.84	9.25	15.92	16.68	9.55	24.13	9.63
Enriched Mo-100	0.60	0.23	0.40	0.81	0.36	1.69	95.9
FY10 Prod. Test	0.06	0.03	0.04	0.05	0.08	0.47	99.2



Stable isotopes in dark shading – Potential radioimpurities in light shading
 Half-lives less than 1 sec are not shown.
 Inconsequential production reactions are not shown.
 Double gamma capture reactions are not shown.

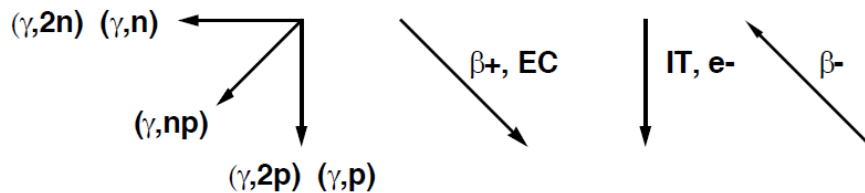
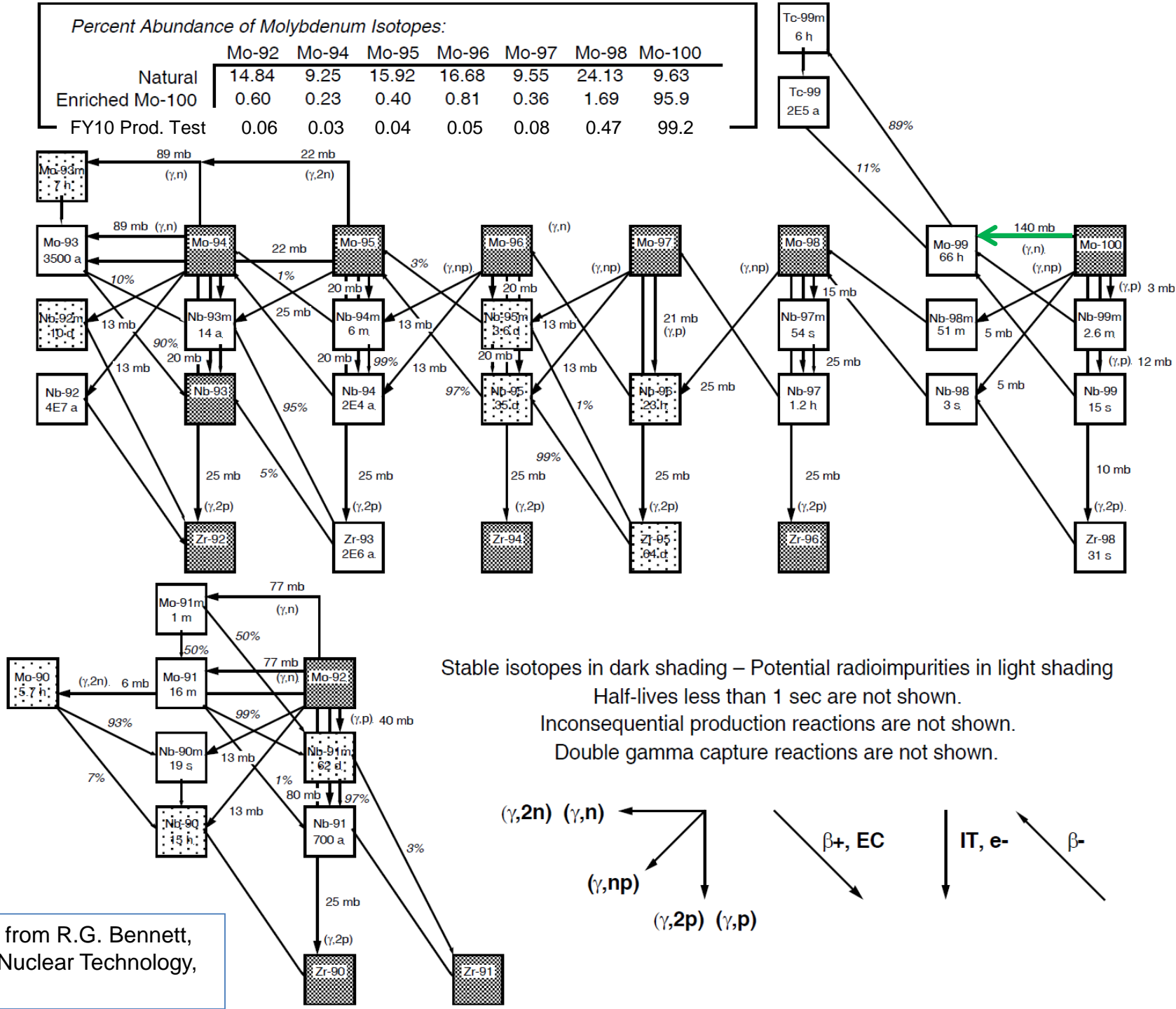


Figure from R.G. Bennett, et al., Nuclear Technology, 1999

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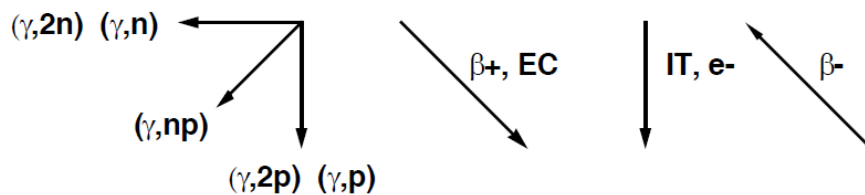
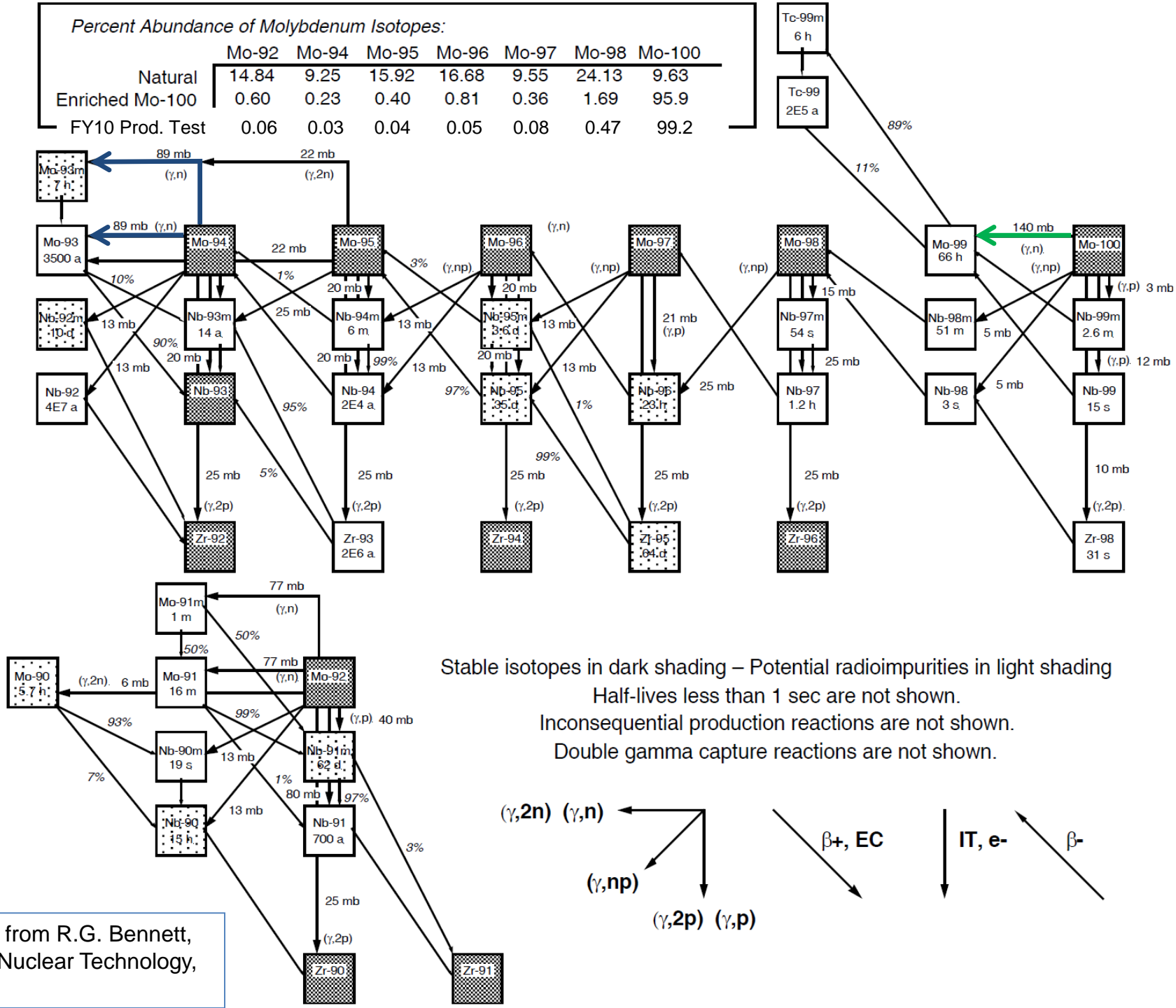


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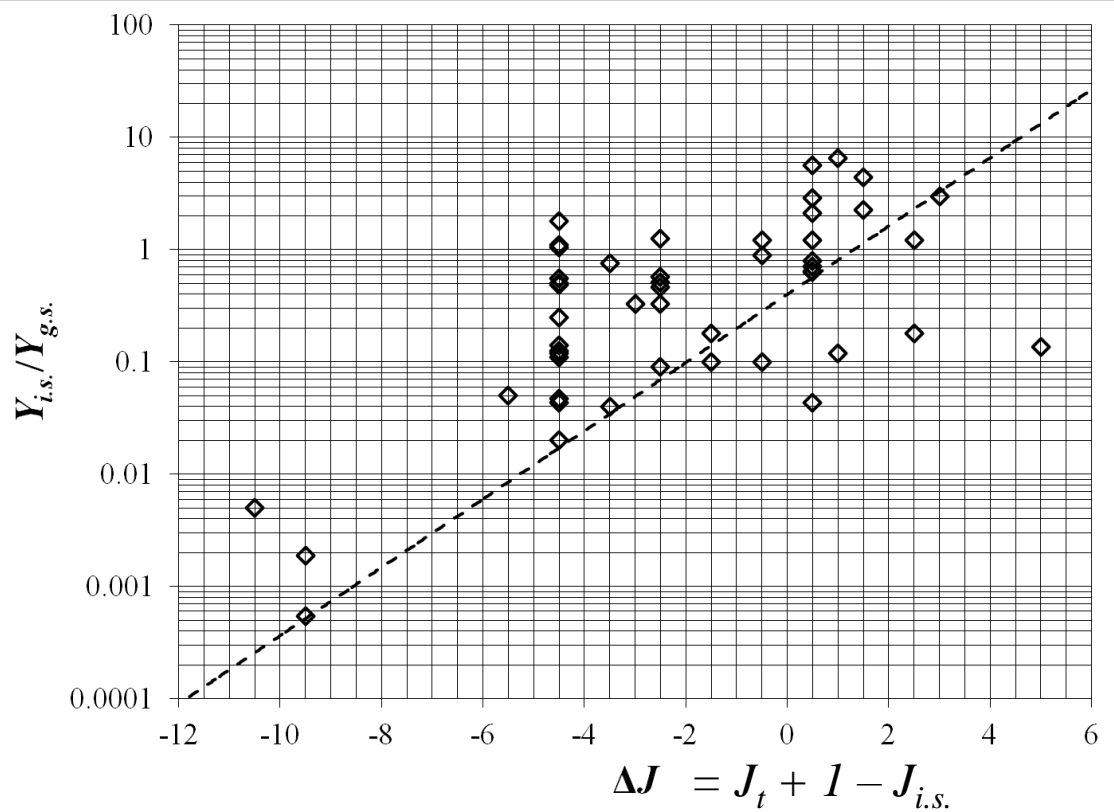
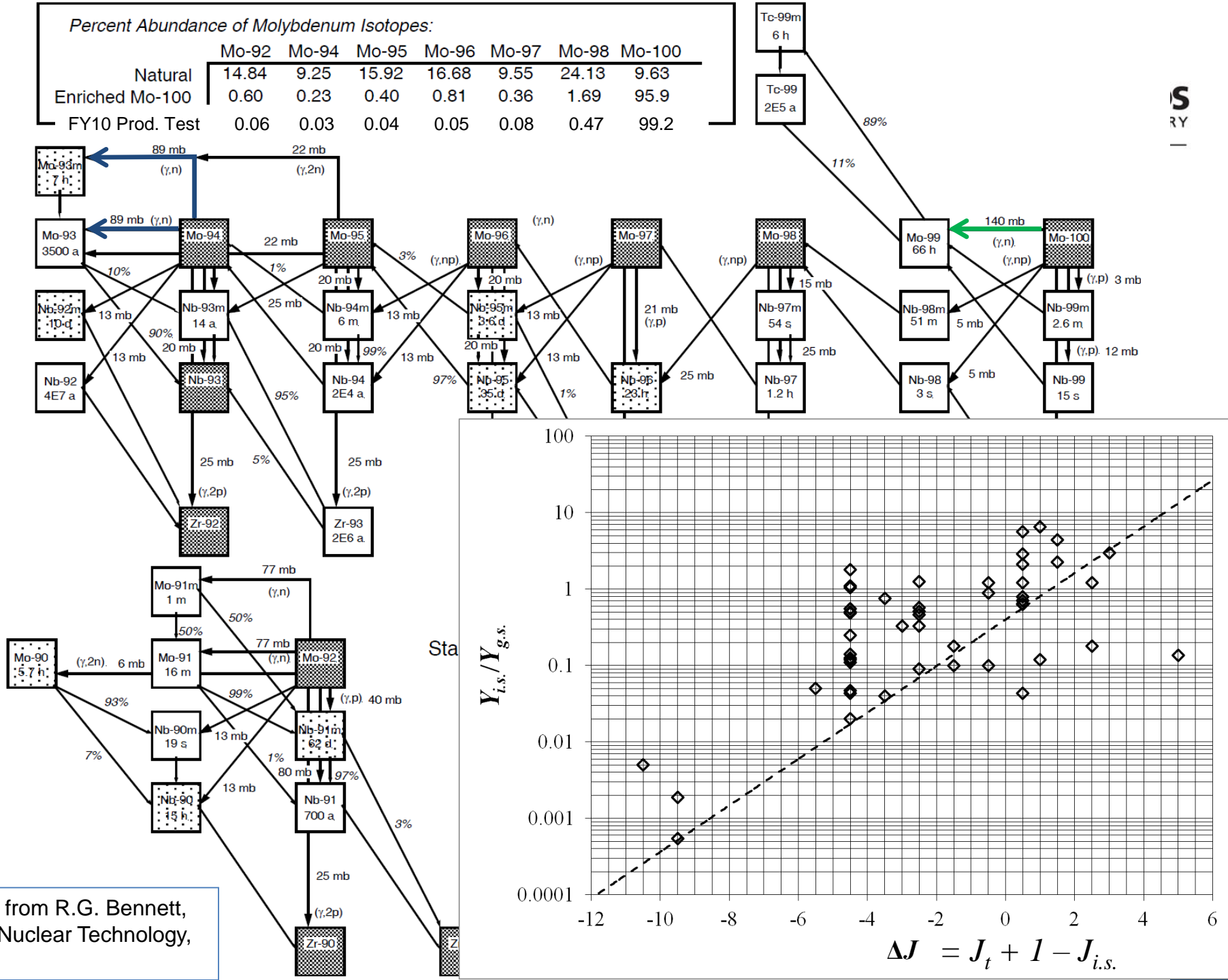


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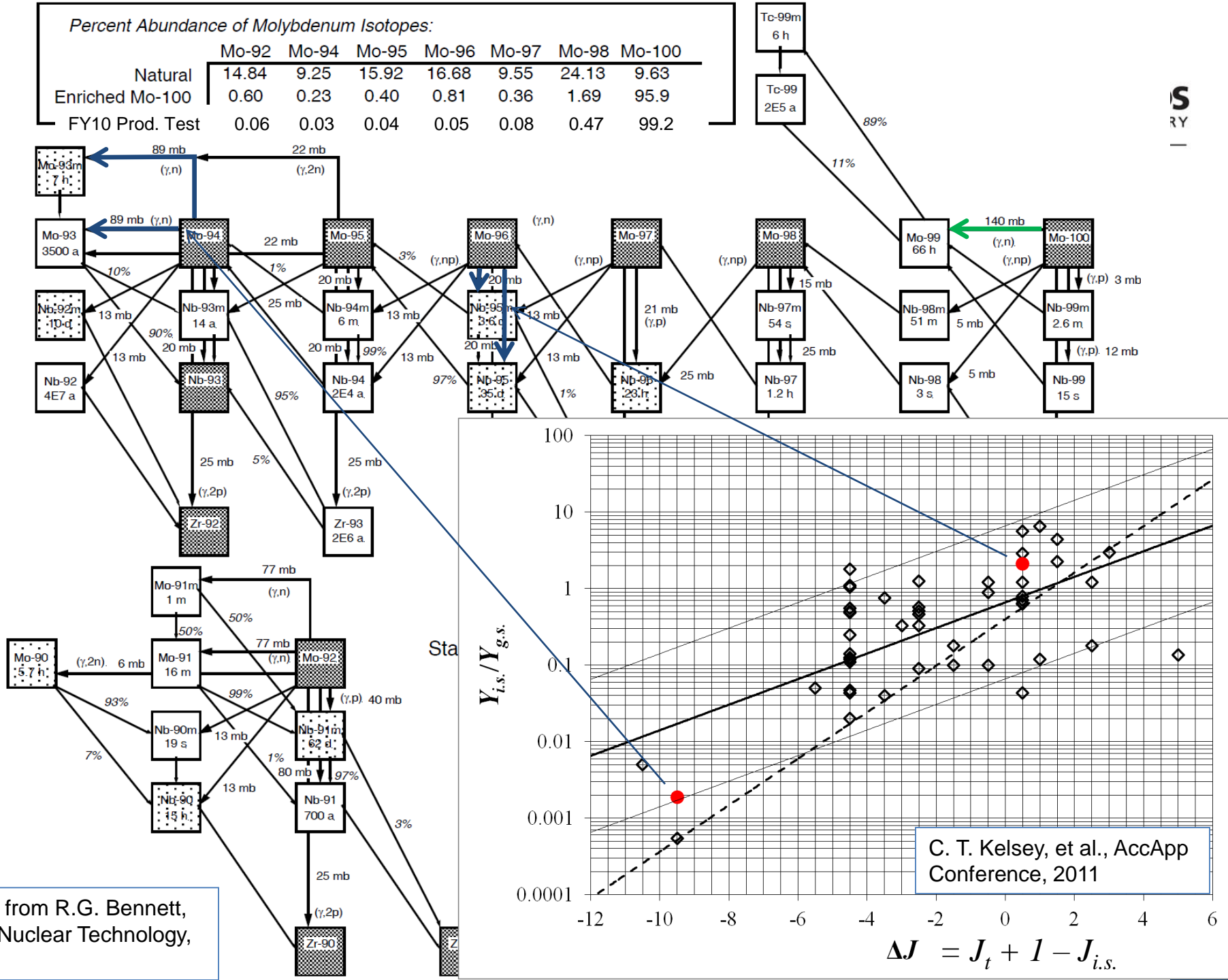
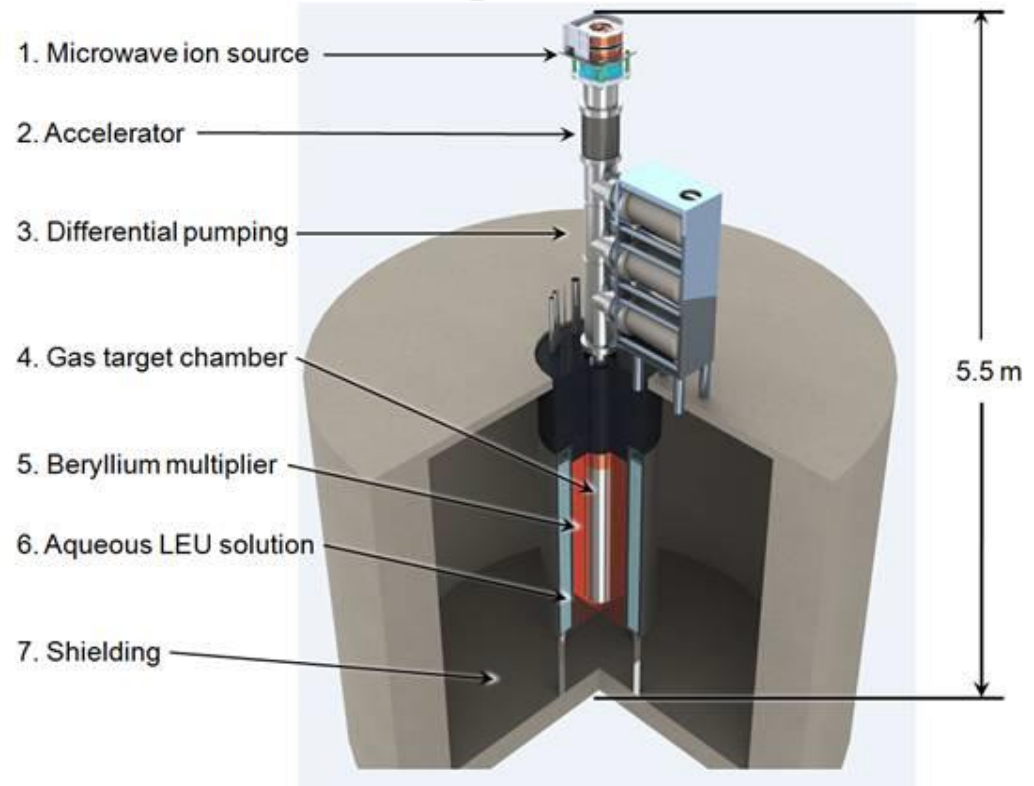


Figure from R.G. Bennett, et al., Nuclear Technology, 1999

LANL Support for SHINE Medical Technologies

SHINE Medical Technologies will produce fission product ^{99}Mo in a subcritical accelerator driven low enriched uranium salt solution



SHINE Support Activities

- System Modeling & Design Support
 - System dynamics and reactivity modeling
 - Thermal hydraulics modeling
 - Gas nozzle design for the accelerator target.
- Irradiations and Separations Chemistry
 - Measurement and control of the total uranium concentration
- Evaluation of the Tritium Recycle Loop and Associated Systems (in partnership with SRNL)
- Zr Clad DU Target Fabrication
 - For the ANL photoneutron target for the mini-SHINE experiment.

Thermal Hydraulic Modeling

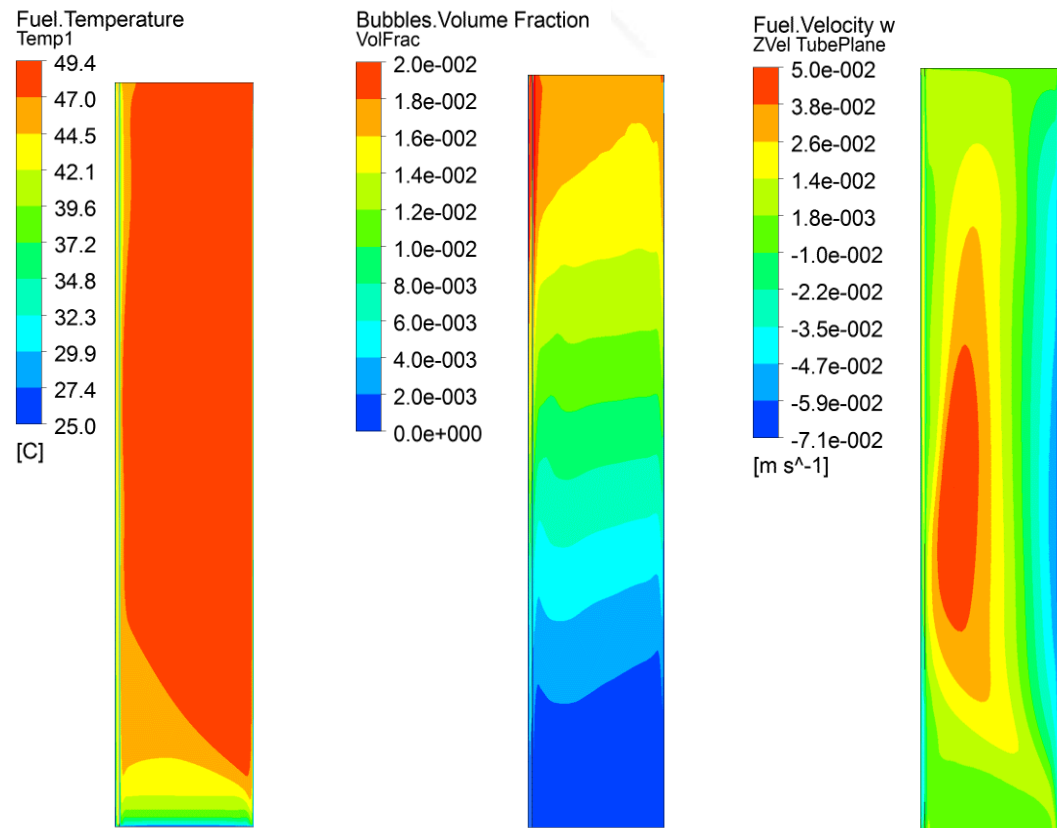
Fluid dynamics model developed to calculate steady state temperature and void fraction profiles for an externally cooled fuel solution vessel.

Computational fluid dynamics calculations performed in Ansys FLUENT.

- Heat transfer by natural convection enhanced by bubble generation
- Non-uniform volumetric heat and bubble generation profiles
- Temperature-dependent fuel and gas properties

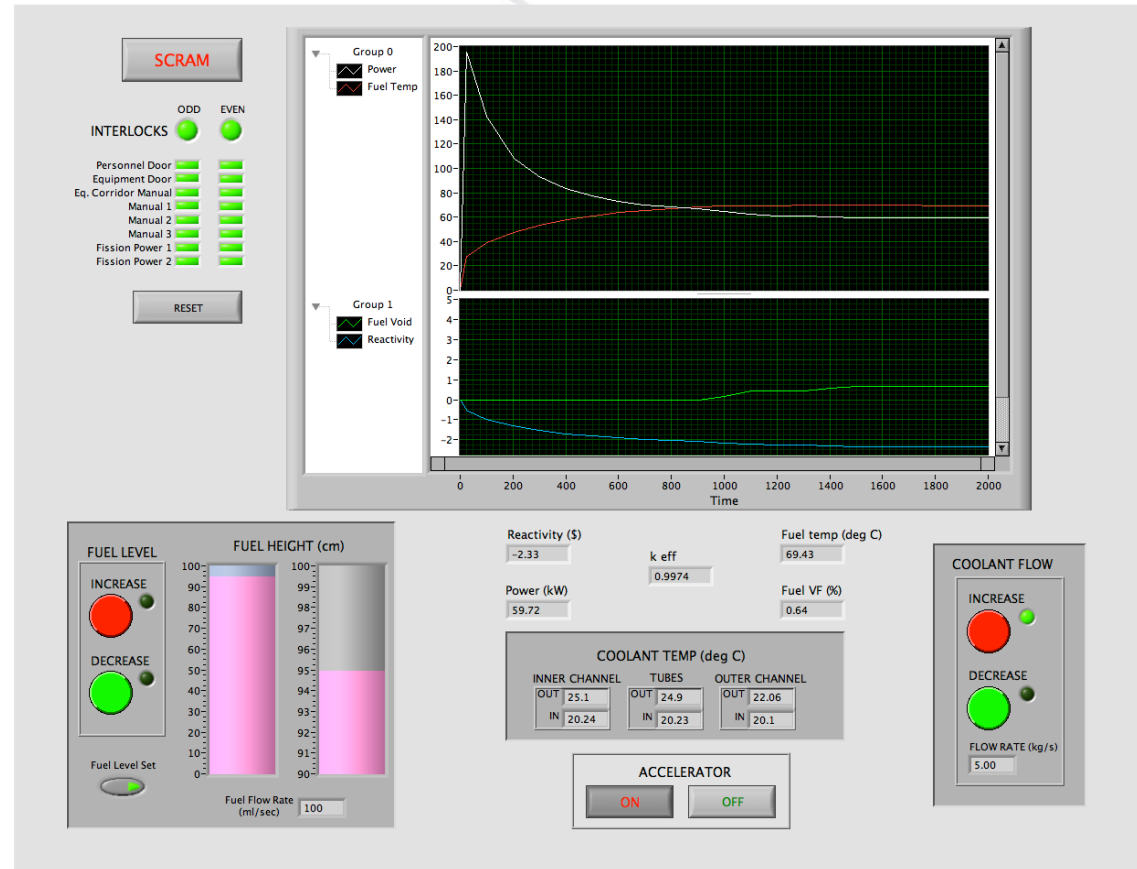
Currently iterating with reaction simulations to obtain steady-state solutions for various conditions.

- Results will be used to improve heat transfer calculations in system model.

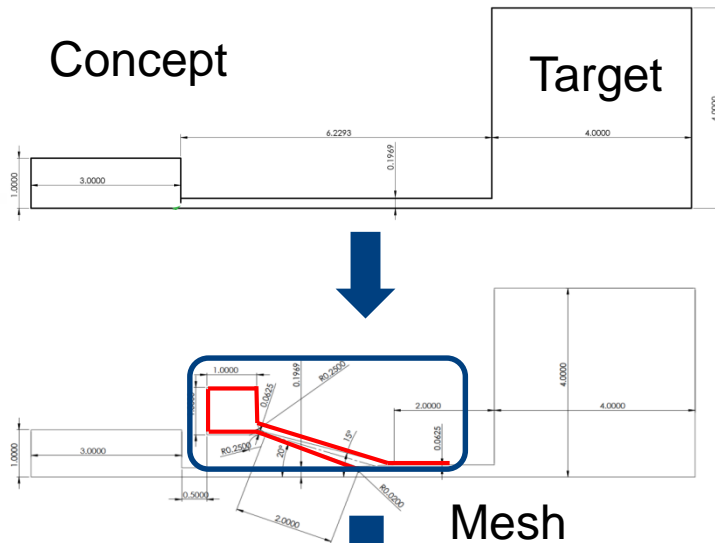


Simulator for Accelerator-Driven System

- Implemented in LabVIEW
- Functional controls
- Strict operational sequencing and protocols enforced
- Data displays derived from DSS model
- Aids in human factors engineering of controls design
- With companion “Instructor’s Screen” may be used for operator training in start-up, normal and off-normal events



Purpose is to reduce the leakage of gas from the target to the accelerator vacuum



- Hydrogen
- Deuterium

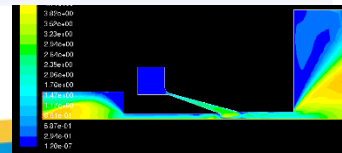
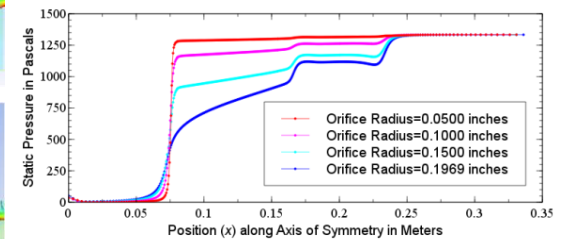


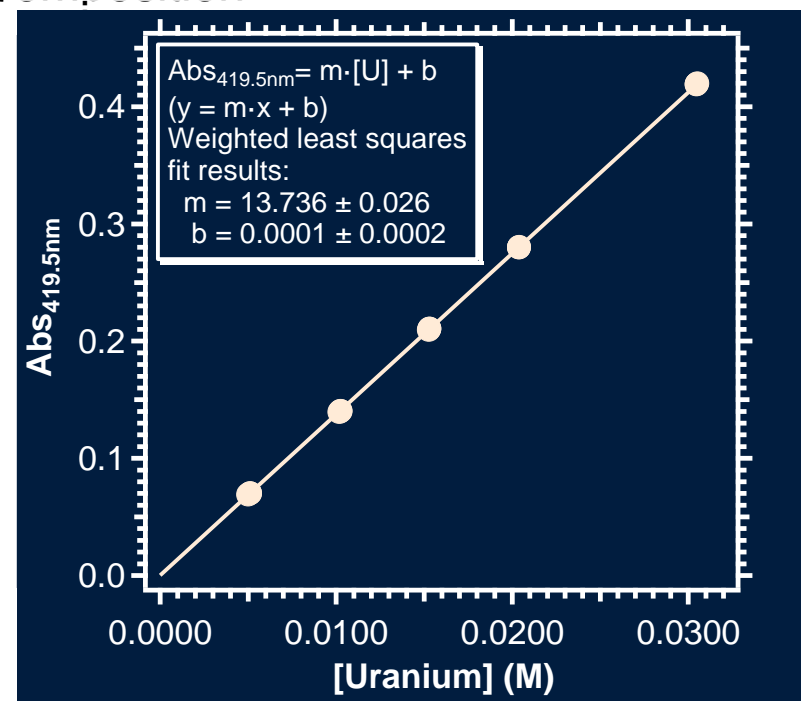
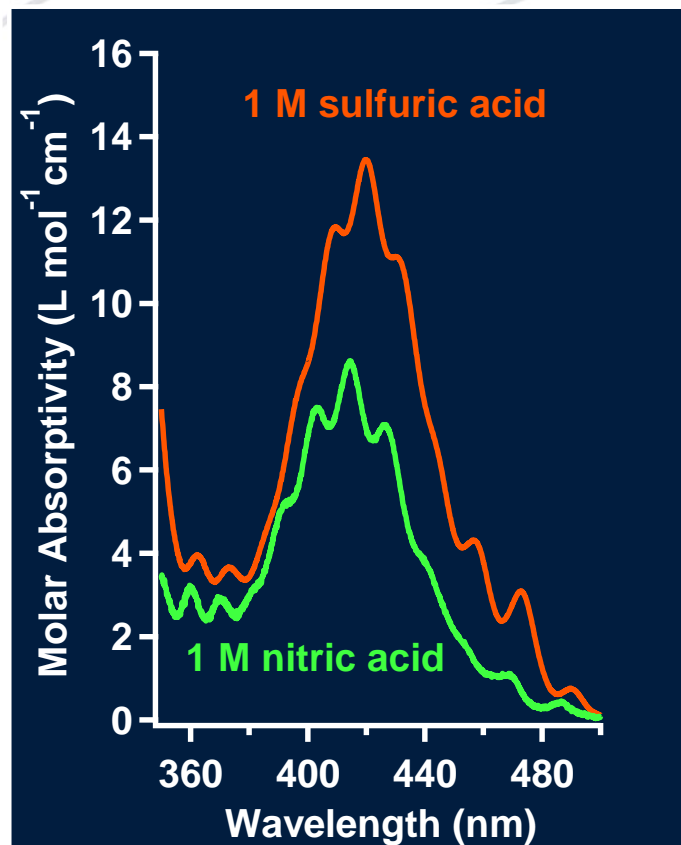
Figure 1 is a line graph showing the mass flow rate into vacuum (in g/s) as a function of nozzle pressure (in Torr) for two different target pressures: 10 Torr (blue line) and 30 Torr (red line). The x-axis ranges from 0 to 800 Torr, and the y-axis ranges from 0 to 8 g/s. The 30 Torr target curve starts at a high mass flow rate of approximately 8.5 g/s at 50 Torr, drops sharply to a minimum of about 0.8 g/s at 120 Torr, and then gradually increases to about 2.8 g/s at 750 Torr. The 10 Torr target curve starts at approximately 3.2 g/s at 50 Torr, drops to a minimum of about 0.7 g/s at 100 Torr, and then gradually increases to about 2.8 g/s at 750 Torr. The two curves converge at higher pressures, around 400 Torr and above.

Nozzle pressure (Torr)	Mass flow rate (g/s) - 10 Torr target	Mass flow rate (g/s) - 30 Torr target
50	3.2	8.5
100	0.7	5.8
150	1.1	3.5
200	1.3	1.5
250	1.5	1.4
300	1.6	1.4
400	1.8	1.8
500	2.1	2.1
600	2.4	2.4
750	2.8	2.8



Uranium Concentration Measurement

- Uranyl absorption spectroscopy – can be applied to uranium concentration measurement in solution
- A small aliquot of sample (e.g. 100 μL) diluted in excess of 1 M H_2SO_4 (2000 μL)
- λ_{max} (peak max, nm) and ϵ (molar absorptivity) vary with chemical composition



Fabrication of Zr clad DU Disks

DU Disk
(actually, SS
surrogate in the
picture)

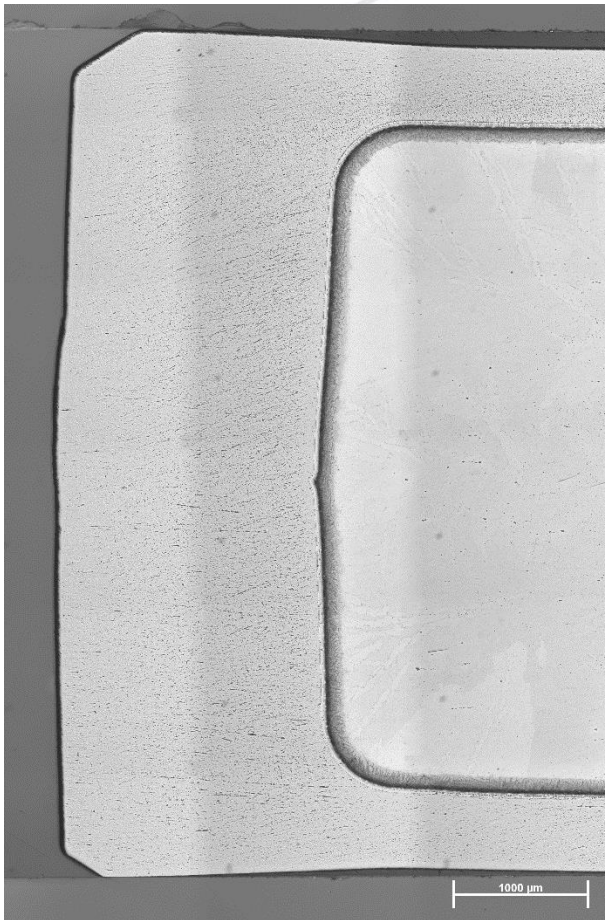
Zr Clad



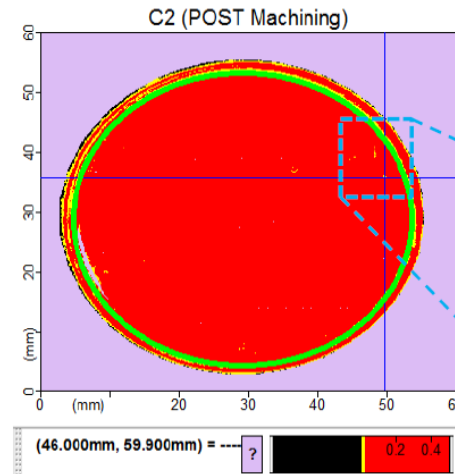
Completed thick disks

The Zr cladding was ebeam welded and then the cans were HIP bonded to create good thermal contact between the DU and Zr.

Characterization of the Zr Clad DU Disks

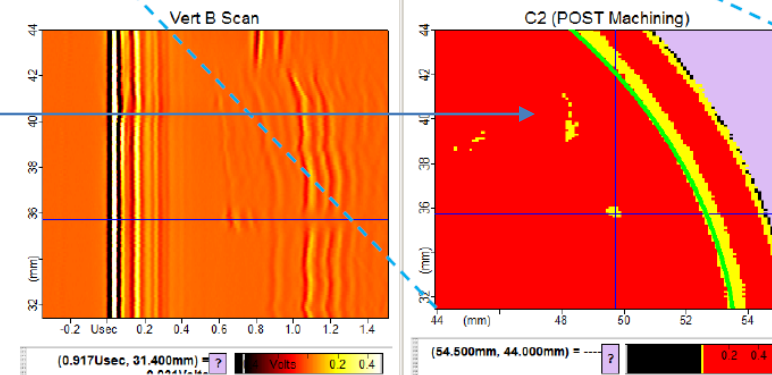
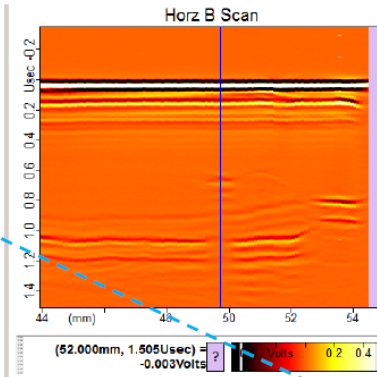
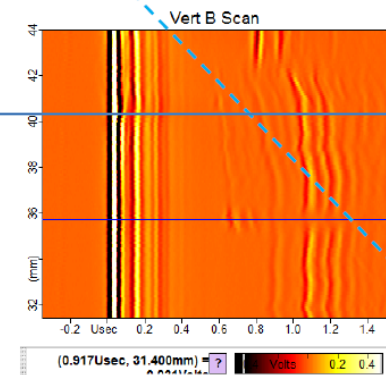
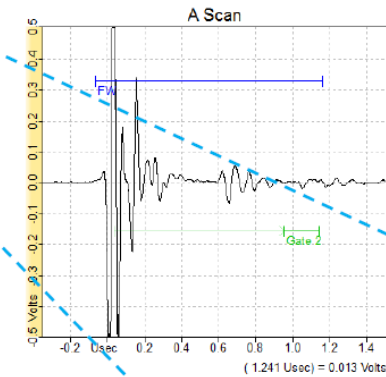


Optical Image of a sectioned test piece



Assembly C2 – Post Machining

- Very small disbonded region observed near rim in upper right quadrant
- No appreciable difference noted from pre-machined sample

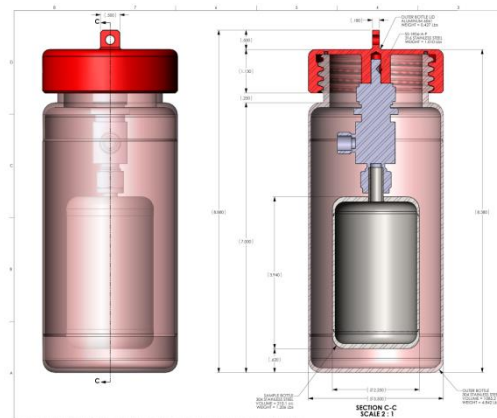


Ultrasonic test results of the final machined component.

Experimental Validation

Low Enriched Uranium Irradiations

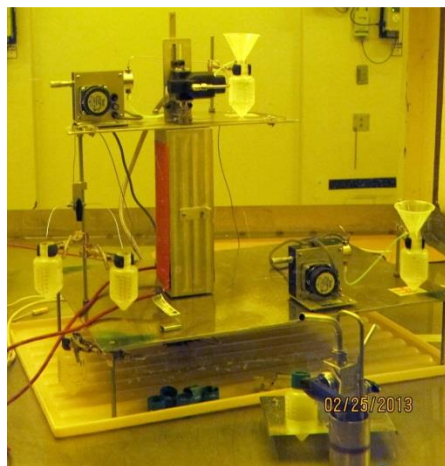
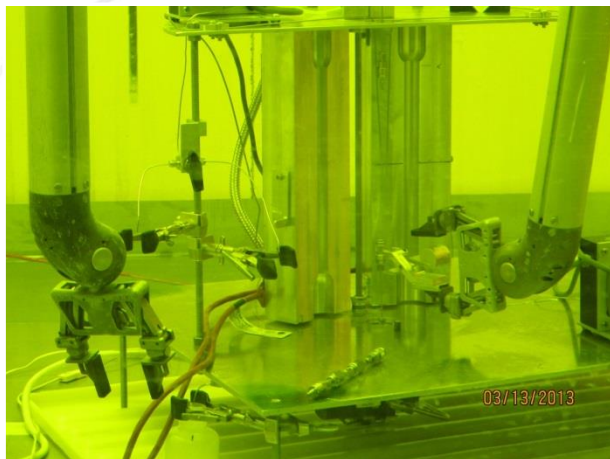
Generation of *ca.* 1 mCi ^{99}Mo



Experimental Validation Low Enriched Uranium Irradiation Generation of 70 mCi ^{99}Mo



Experimental Validation Separations Chemistry



Summary

- LANL is partnering closely with NNSA and the other national laboratories to help the commercial domestic production of ^{99}Mo without the use of HEU.
- Under the M³ ^{99}Mo Program, we are currently supporting NorthStar Medical Radioisotopes and SHINE Medical Technologies.
- Leveraging the unique capabilities of the National Laboratories to increase the production of ^{99}Mo .