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Neutron Spectrum Measurements from Irradiations at NCERC

Kevin R. Jackman, Michelle Mosby, Todd A. Bredeweg, Greg Hutchens, Morgan White

April 2015

Introduction

- Several irradiations have been conducted on assemblies at the National Criticality Experiments Research Center (NCERC) located at the Nevada National Security Site (NNSS)
- Configurations of the assemblies and irradiated materials changed between experiments
- Different metallic foils were analyzed using the radioactivation method by gamma-ray spectrometry to understand / characterize the neutron spectra

COMET/ZEUS

- Epithermal/Fission-spectrum (peaked ~ 500 keV)
- HEU rings raised by hydraulic lift
- Copper reflector (from ZEUS Experiments)
- First critical: August 11th, 2011
- Sample trays (16 foils at reflector; 24 foils in fuel)
- $\sim 10^{12}$ fissions/gram (high-power)



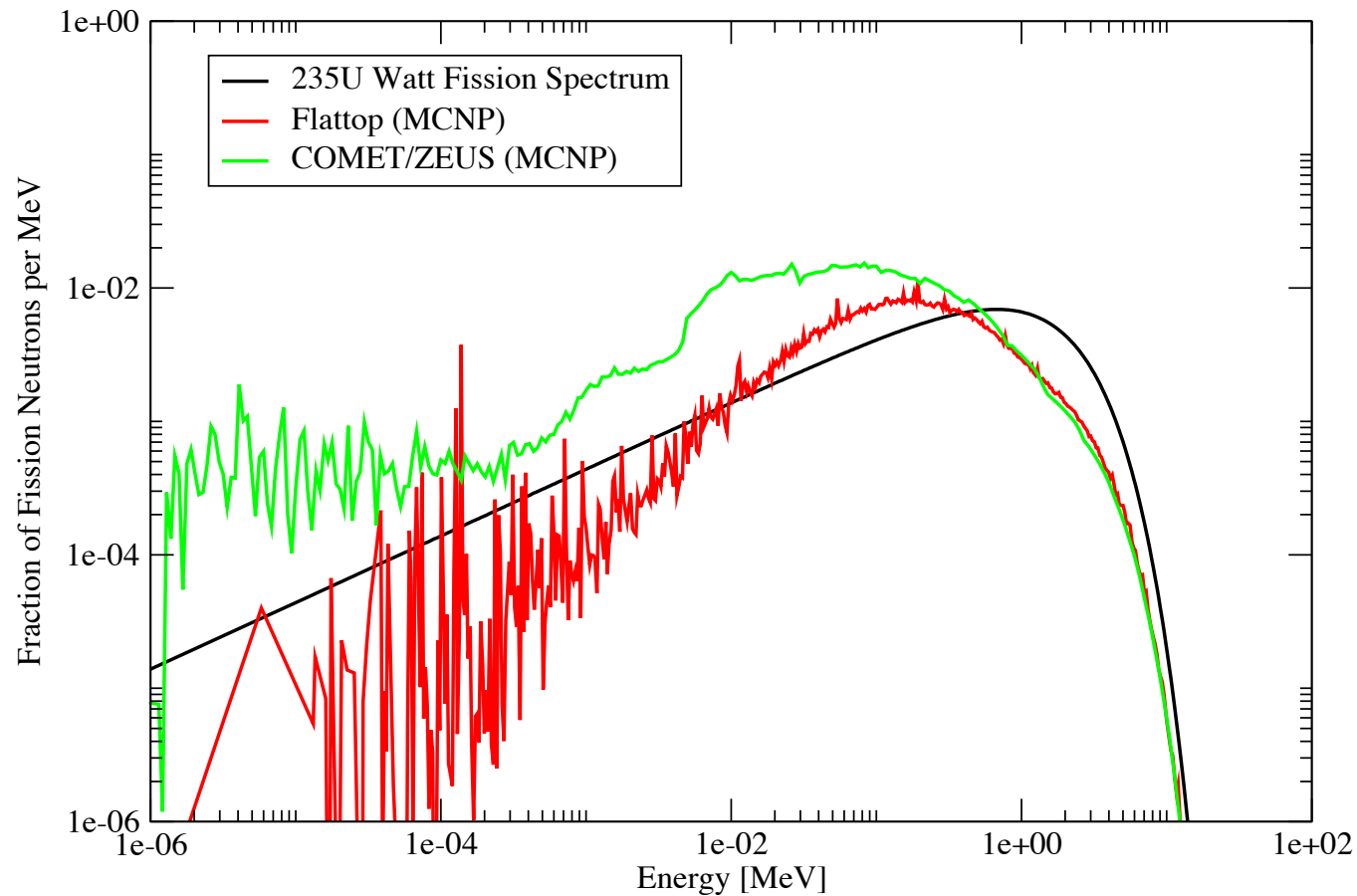
Flattop

- Fast/Fission-spectrum
- HEU and Pu cores with Natural U reflector
- Reactivity increases as parts of spheres brought together and control rods inserted
- Foils inserted in the traverse or glory holes
- $\sim 10^{13}$ fissions/gram (high power)



Fission Neutrons

Prompt Fission Neutron Spectrum Versus Energy



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Ref's: Int'l Handbook of Evaluated Critical Safety Benchmark Experiments (2009),

M. White (LANL), and UCRL-AR-228518.

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Activation Foils Used

Element	Shieldwerx Sets	Reaction(s)	Half-life	Cross-section(s)	Res/Thresh
Al	Fast/Broad	27Al(n, α)24Na	14.95 h	$\sigma \sim 0.15$ b	Thresh ~ 4 MeV
Au	Thermal/Broad	197Au(n, γ)198Au	2.6952 d	$\sigma_t = 98.7$ b; R.I. = 1550 b	4.9 eV
Au	Thermal/Broad	197Au(n,2n)196Au	6.617 d	$\sigma \sim 2.2$ b	Thresh ~ 8.5 MeV
Co	Thermal	59Co(n, γ)60Co	5.271 y	$\sigma_t = 37$ b; R.I. = 74 b	132 eV
Co	Thermal	59Co(n,2n)58Co	70.88 d	$\sigma \sim 0.8$ b	Thresh ~ 13 MeV
Co	Thermal	59Co(n,p)59Fe	44.50 d	$\sigma \sim 0.05$ b	Thresh ~ 3 MeV
Cu	Thermal/Fast/Broad	63Cu(n, γ)64Cu	12.701 h	$\sigma_t = 4.5$ b; R.I. = 5.0 b	401 eV; 579 eV
Cu	Thermal/Fast/Broad	63Cu(n, α)60Co	5.271 y	$\sigma \sim 0.045$ b	Thresh ~ 4 MeV
Fe	Thermal/Fast/Broad	54Fe(n,p)54Mn	312.1 d	$\sigma \sim 0.48$ b	Thresh ~ 2 MeV
Fe	Thermal/Fast/Broad	56Fe(n,p)56Mn	2.578 h	$\sigma \sim 0.11$ b	Thresh ~ 5 MeV
Fe	Thermal/Fast/Broad	58Fe(n, γ)59Fe	44.50 d	$\sigma_t = 1.3$ b; R.I. = 1.2 b	230 eV, 359 eV
Mn-Cu	Fast	55Mn(n, γ)56Mn	2.578 h	$\sigma_t = 13.3$ b; R.I. = 14 b	341 eV
Mn-Cu	Fast	55Mn(n,2n)54Mn	312.1 d	$\sigma \sim 0.9$ b	Thresh ~ 10 MeV
Mo	Thermal	98Mo(n, γ)99Mo	2.7476 d	$\sigma_t = 0.13$ b; R.I. = 7.2 b	12 eV; 470 eV
Nb	-	93Nb(n,n')93mNb	16.1 y	$\sigma \sim 1.8$ b	Thresh ~ 2.5 MeV
Nb	-	93Nb(n,2n)92Nb	10.13 d	$\sigma \sim 1.3$ b	Thresh ~ 9 MeV
Ni	Fast/Broad	58Ni(n,2n)57Ni	35.6 h	$\sigma \sim 0.08$ b	Thresh ~ 13 MeV
Ni	Fast/Broad	58Ni(n,p)58Co	70.88 d	$\sigma \sim 0.6$ b	Thresh ~ 2 MeV
Ni	Fast/Broad	60Ni(n,p)60Co	5.271 y	$\sigma \sim 0.15$ b	Thresh ~ 4 MeV
Sc	Thermal/Broad	45Sc(n, γ)46Sc	83.81 d	$\sigma_t = 17$ b; R.I. = 12 b	460; 3.3e3 eV
Ti	Fast/Broad	46Ti(n,p)46Sc	83.81 d	$\sigma \sim 0.3$ b	Thresh ~ 4 MeV
Ti	Fast/Broad	47Ti(n,p)47Sc	3.349 d	$\sigma \sim 0.15$ b	Thresh ~ 1 MeV
Ti	Fast/Broad	48Ti(n,p)48Sc	43.7 h	$\sigma \sim 0.06$ b	Thresh ~ 5 MeV
V	Fast/Broad	51V(n, α)48Sc	43.7 h	$\sigma \sim 0.02$ b	Thresh ~ 7 MeV
W	Thermal	186W(n, γ)187W	23.9 h	$\sigma_t = 38$ b; R.I. = 490 b	18.8 eV

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Background (SAND-II-SNL)

- SAND-II-SNL Spectrum Unfolding / Iterative Perturbation Method
 1. Activities are calculated based on current iteration of neutron spectrum (flux) using cross-section libraries in code
 2. Calculated activities are compared with measured activities to provide correction factors for each foil
 3. Weighting function (as a function of energy) is obtained for each foil based on sensitivity (differential cross-section times differential flux)
 4. Foil weighting functions are applied to an averaging procedure to obtain average correction factors at each energy (based on calculated/measured activity and relative contribution to flux at given energy)
 5. Average correction factors are applied to current iteration of neutron spectrum (flux) to obtain the next iterative spectrum

Note: flux for steady-state, fluence for pulsed reactors

Ref's: AFWL-TR-67-41 (1967), SAND93-3957 (1994).

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Background (SAND-II-SNL)

$$A_i^{[k]} = \sum_{j=1}^m \bar{\sigma}_{i,j}^{[k]} \Phi_j^{[k]} \prod_{r=1}^3 e^{-N_r * X_r * \bar{\sigma}_{r,j}}$$



j is energy index

i is foil index

k is iteration index

N is foil cover nuclei density

X is foil cover thickness

σ_r is foil cover removal cross-section

σ_i is foil reaction cross-section

Φ is integral flux

bar above means energy averaged

$$w_{i,j}^{[k]} = \frac{\frac{1}{2} (A_{i,j}^{[k]} + A_{i,j-1}^{[k]})}{A_i^{[k]}}$$



$j = 2, 3, \dots, m$

$$C_j^{[k]} = \frac{\sum_{i=1}^n w_{i,j}^{[k]} \ln \left(\frac{A_i}{A_i^{[k]}} \right)}{\sum_{i=1}^n w_{i,j}^{[k]}}$$



$$\varphi_i^{[k+1]} = \varphi_i^{[k]} e^{(C_j^{[k]})}$$

Ref's: AFWL-TR-67-41 (1967), SAND93-3957 (1994).

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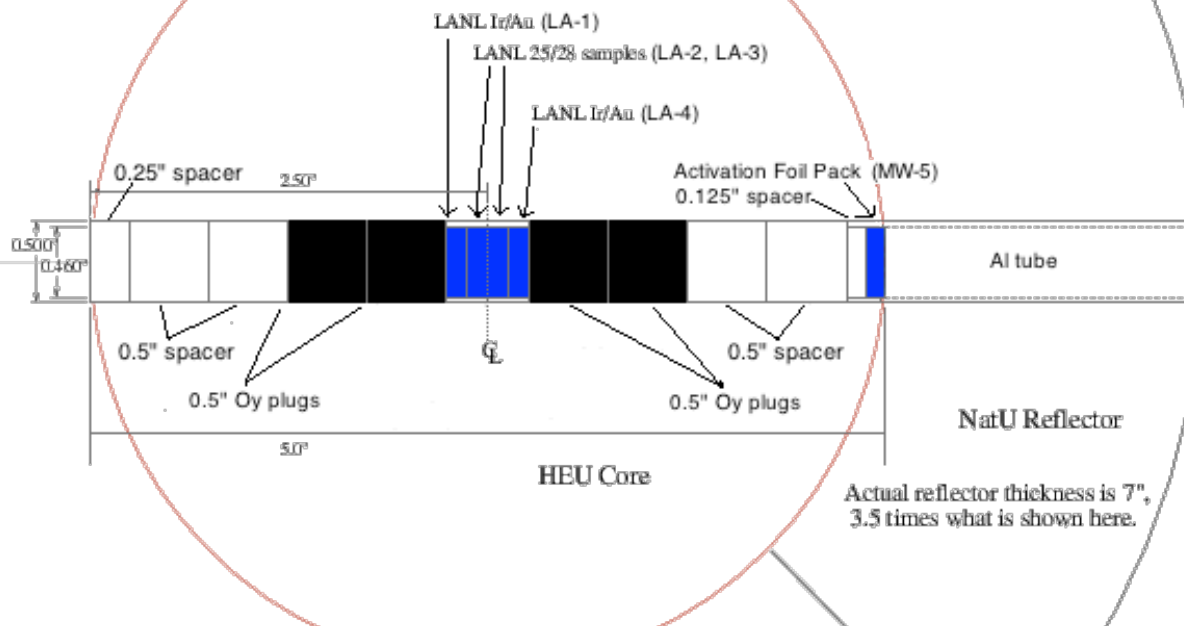
COMET/ZEUS Experiments

- Four irradiations on COMET/ZEUS
 - SEP 2011 targets at bottom reflector
 - Au
 - SEP 2012 – targets at bottom reflector
 - Al-Au (0.67% + 1 Cd covered), Cu, Co, Mn-Cu, Nb, and Ti
 - DEC 2012 – targets near center of core
 - Al-Au (0.67% + 1 Cd covered), Mo, W, Sc, Ni, Fe, V, and Ti
 - AUG 2013 – targets near center of core
 - Al-Au (0.67% + 1 Cd covered), Cu, W, Sc, Ni, Fe, and Ti

Flattop Experiments

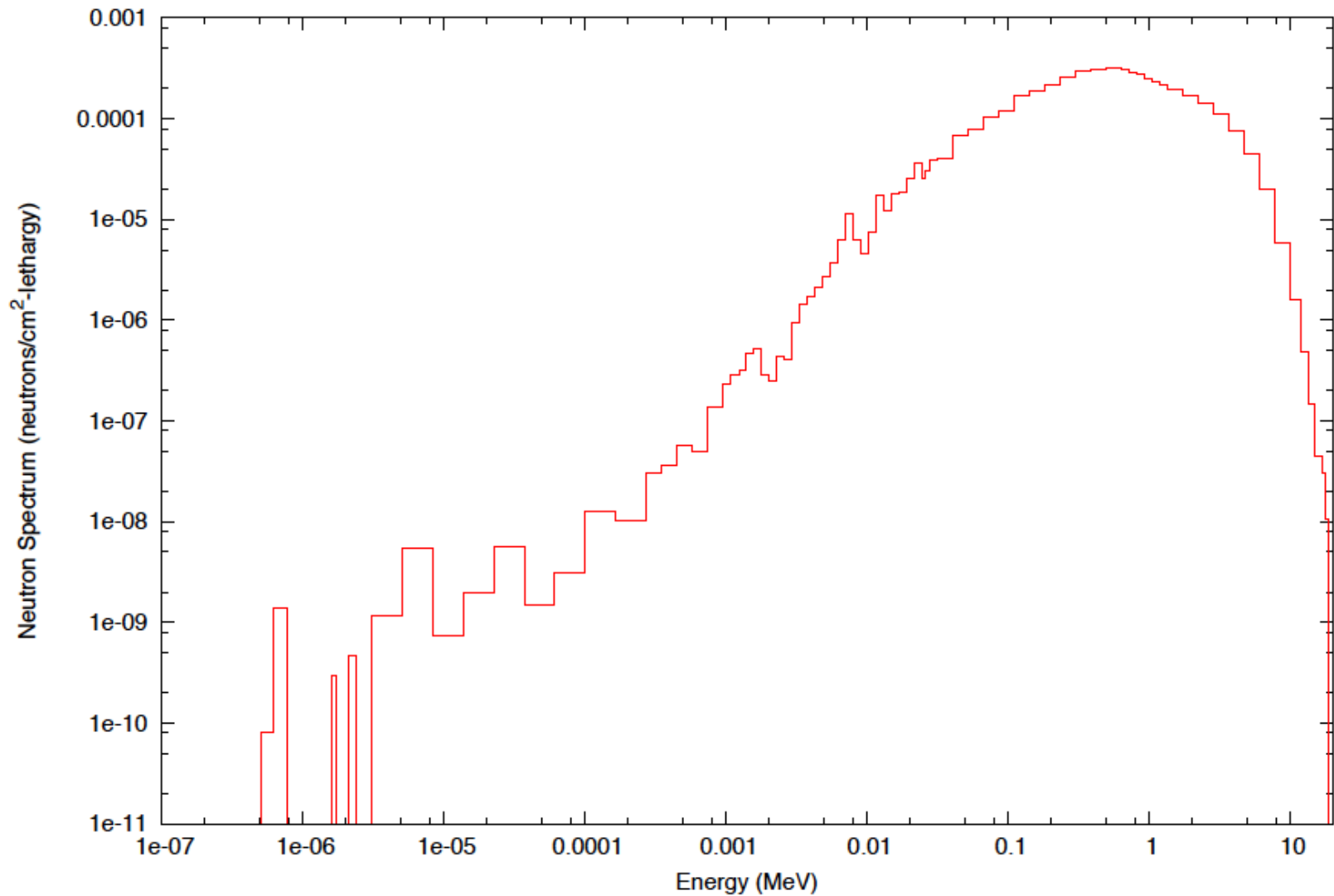
- Three irradiations on Flattop
 - JUN 2013 – fuel plugs on one side
 - Sc, Ti, Ni, and Cu at core / reflector interface
 - JUL 2014 – fuel plugs symmetric
 - Sc, Ti, Ni, and Cu at core / reflector interface
 - AUG 2014 – fuel plugs on one side
 - Sc, Ti, Ni, and Cu at core / reflector interface

Flattop Irradiation – JUL 2014



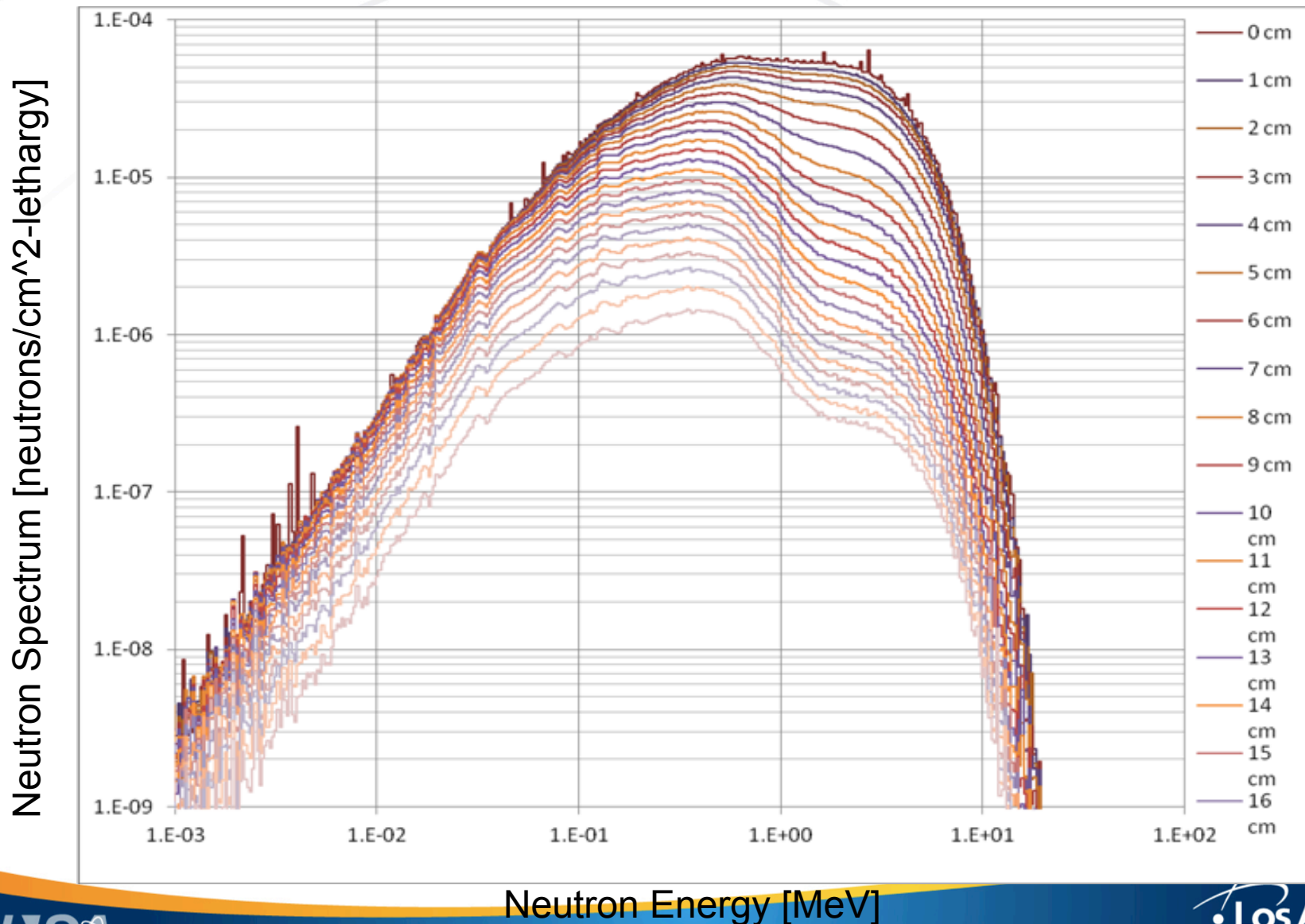
MCNP Calculations for COMET/ZEUS

Neutron Spectrum for IER 163
(September 6, 2012)



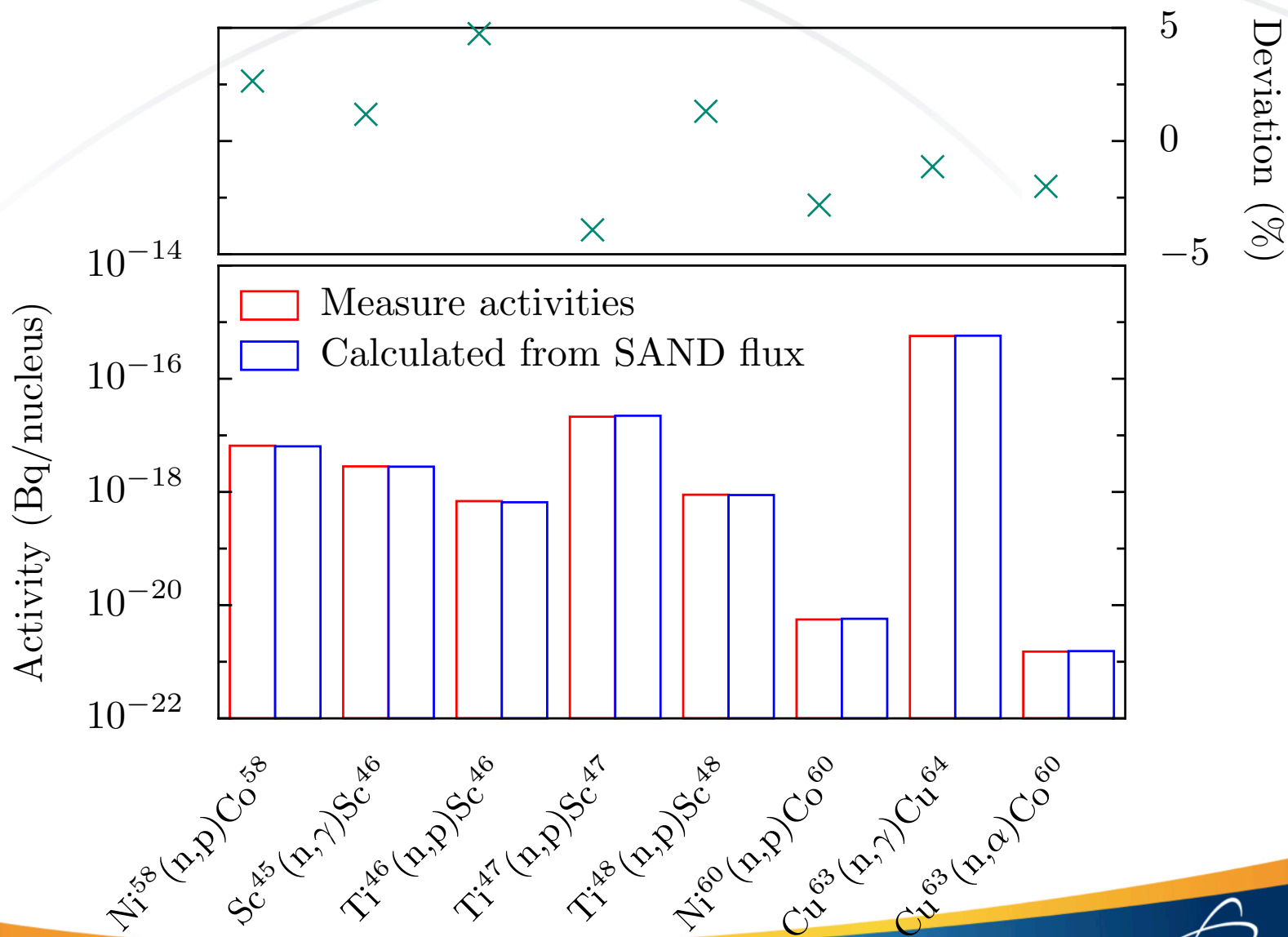
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MCNP Calculations for Flattop

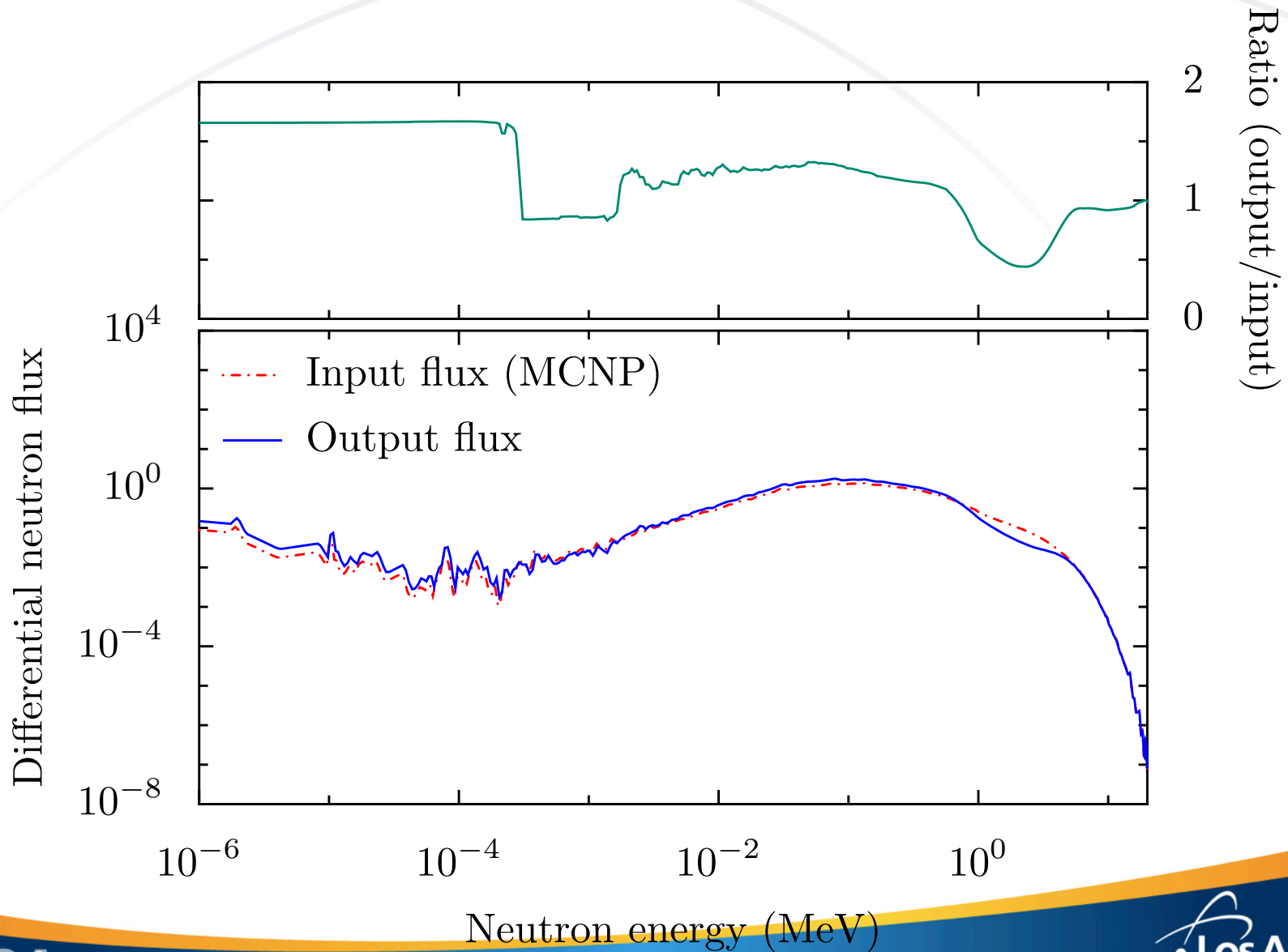


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Results for Flattop



Results for Flattop (cont'd)



Improvements

- Incorporate IRDF 2002 cross-section library into SANDII-SNL (update SNL library) and potentially IRDFF library (up to 60 MeV)
- Incorporate > 20 MeV reactions using data from TALYS / TENDL for LANL Isotope Production Facility (IPF) neutron spectrum characterization (100 MeV)
- Evaluate use of alternate statistical techniques (e.g. maximum likelihood) and least-squares adjustment codes (LSL-M2, STAY'SL) for neutron spectrum unfolding / adjustment

Conclusions

- MCNP simulated spectra agree with experimental measurements
 - Limited by statistics at low-energies
 - Some activation foils have low activities
- Experiments will continue to be conducted to understand / characterize the neutron spectra of assemblies at NCERC

Acknowledgements

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