

# An Ion Beam Platform for Screening Materials for Nuclear Reactors

SAND2010-5783C

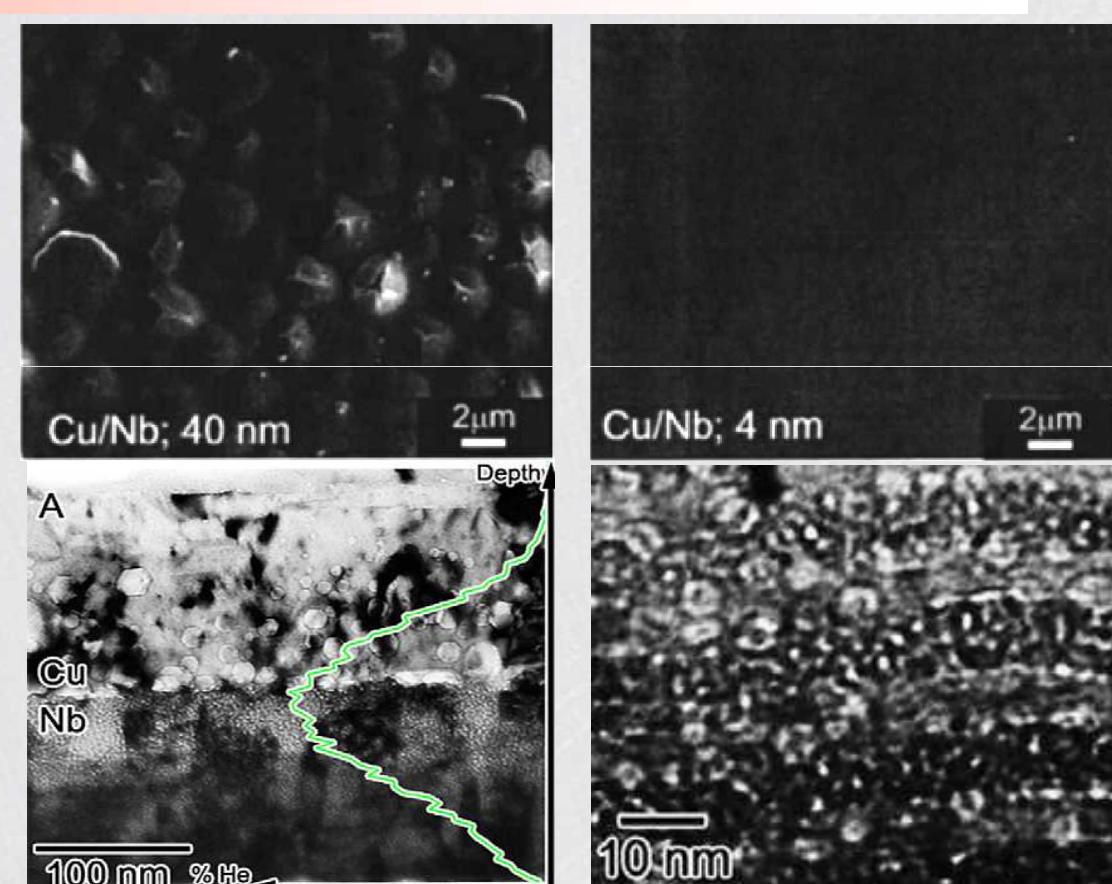
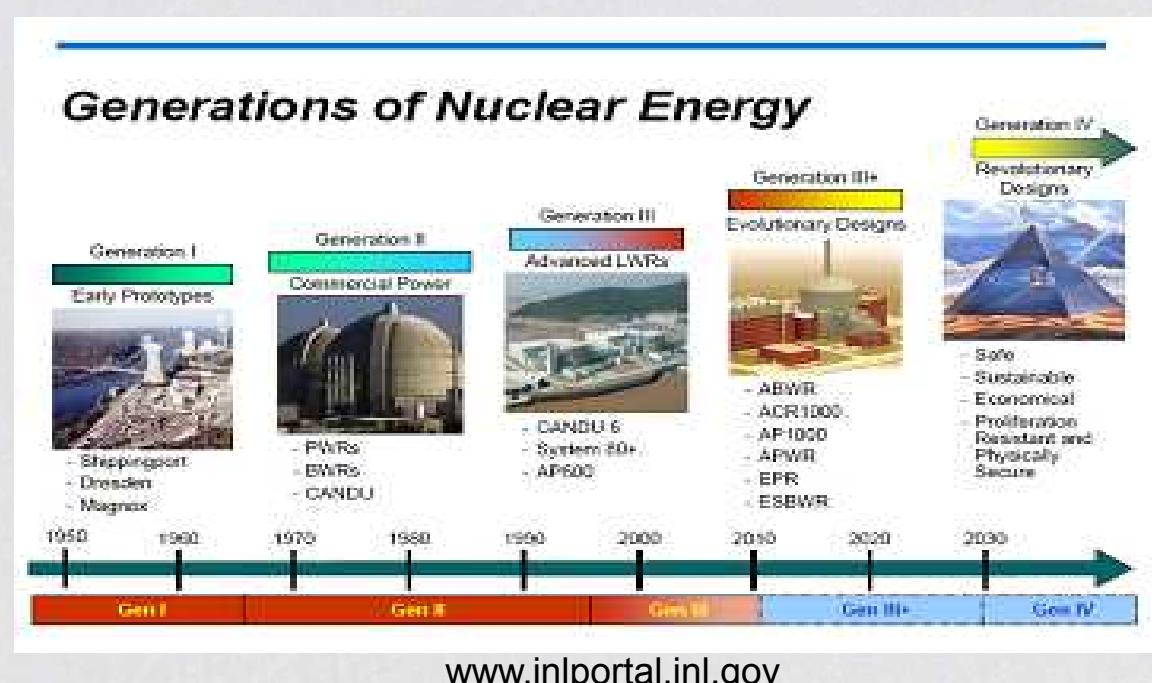
Sandia National Laboratories

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LABORATORY DIRECTED RESEARCH & DEVELOPMENT

## Problem

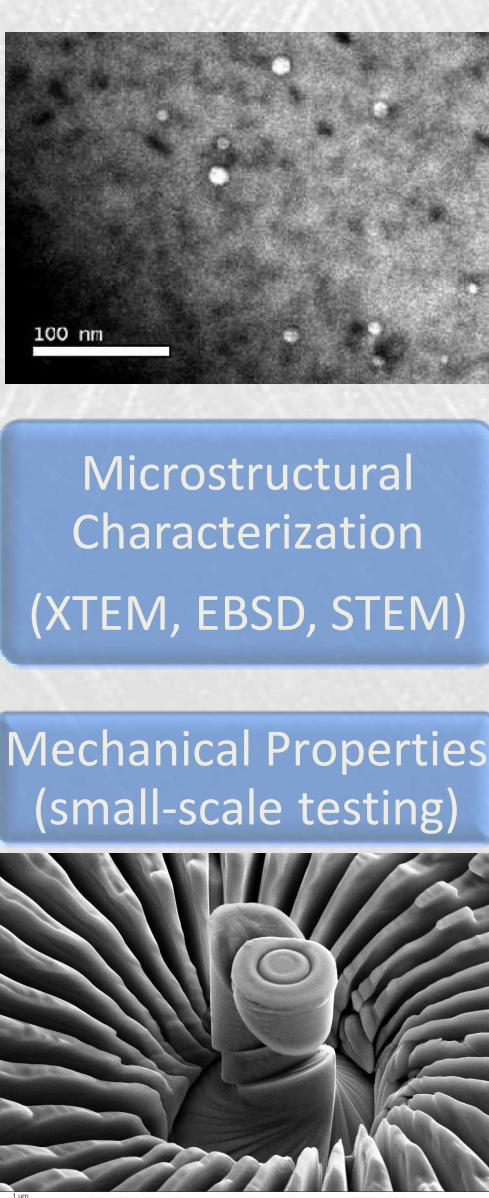
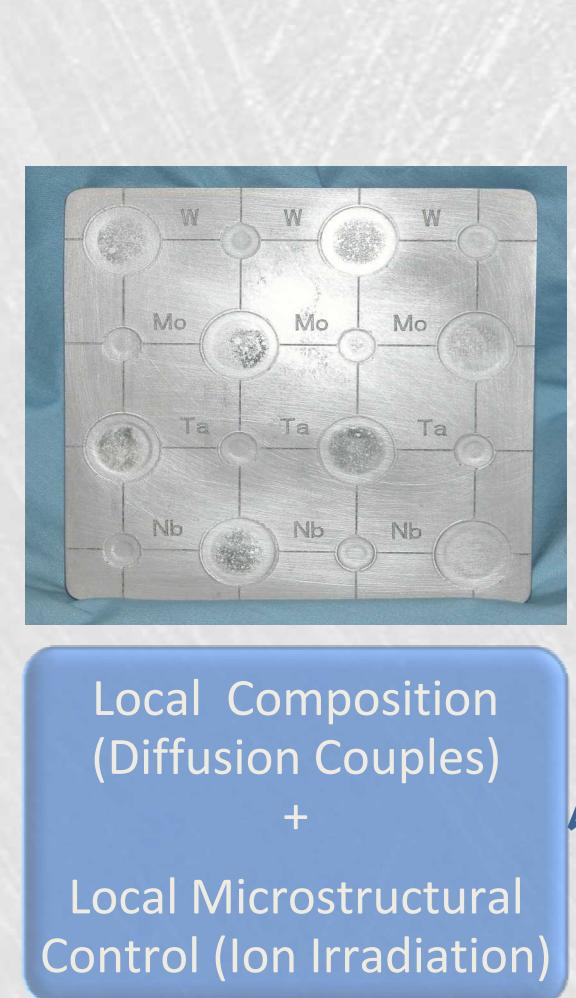


Hattar et al. Scripta Mat. 2007

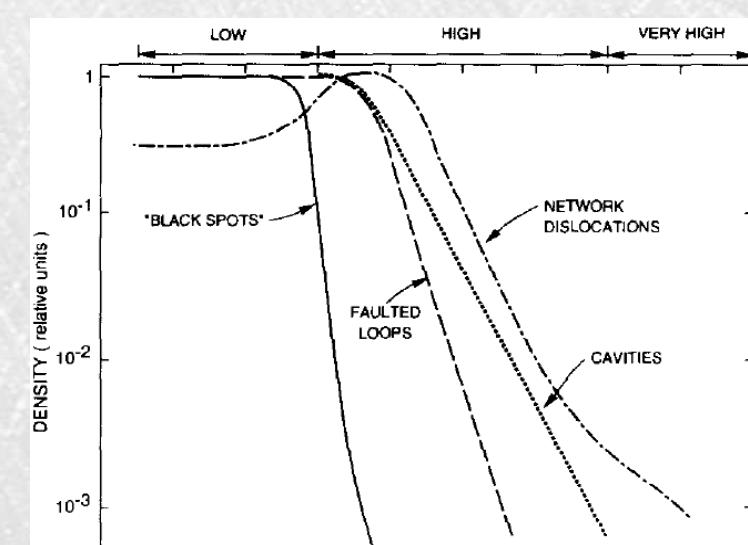
- All future reactor designs require advancements in radiation tolerant materials
- Many materials systems are being considered
- Interface engineering is providing a potential solution

Advanced materials and the need for rapid testing often requires new experimental testing techniques

## Approach



Microstructural Characterization (XTEM, EBSD, STEM)



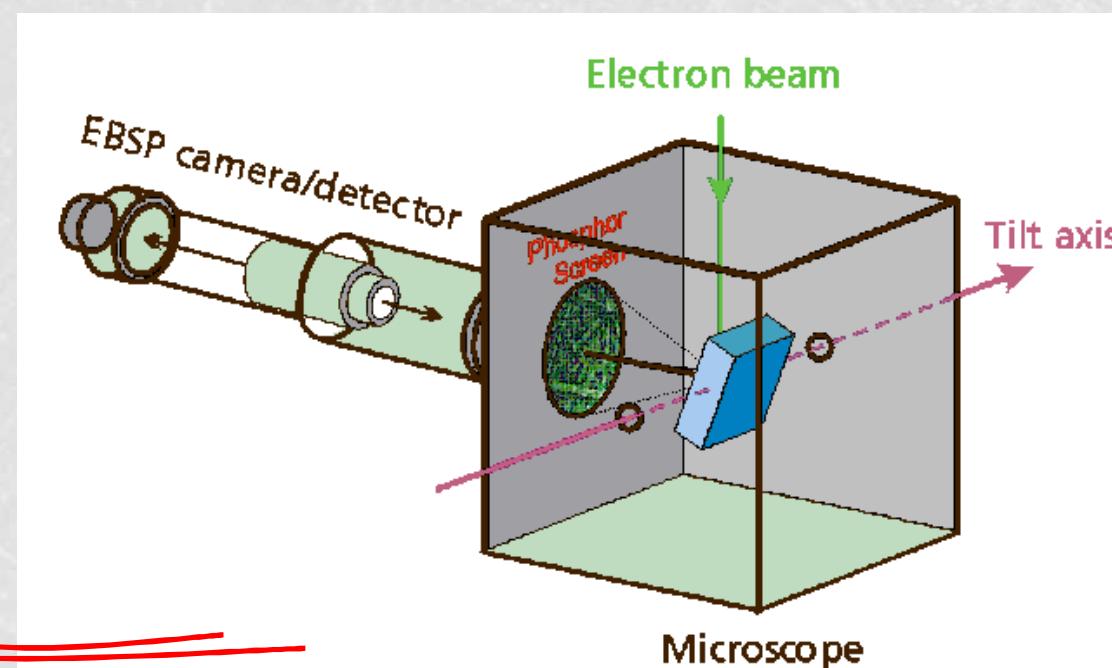
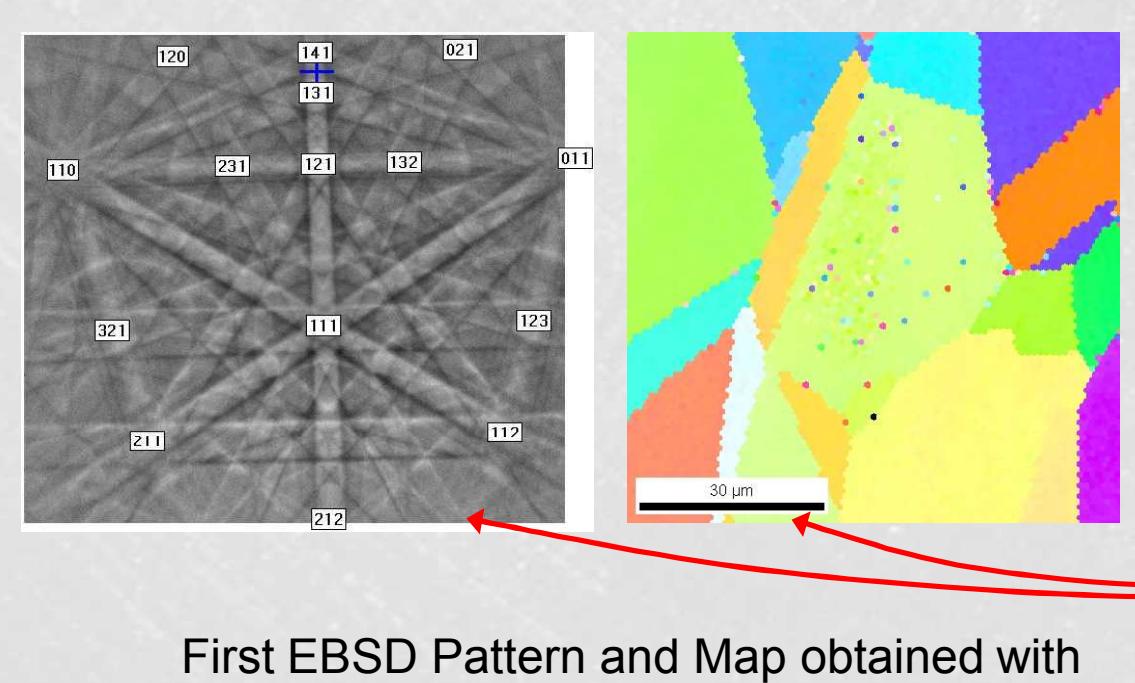
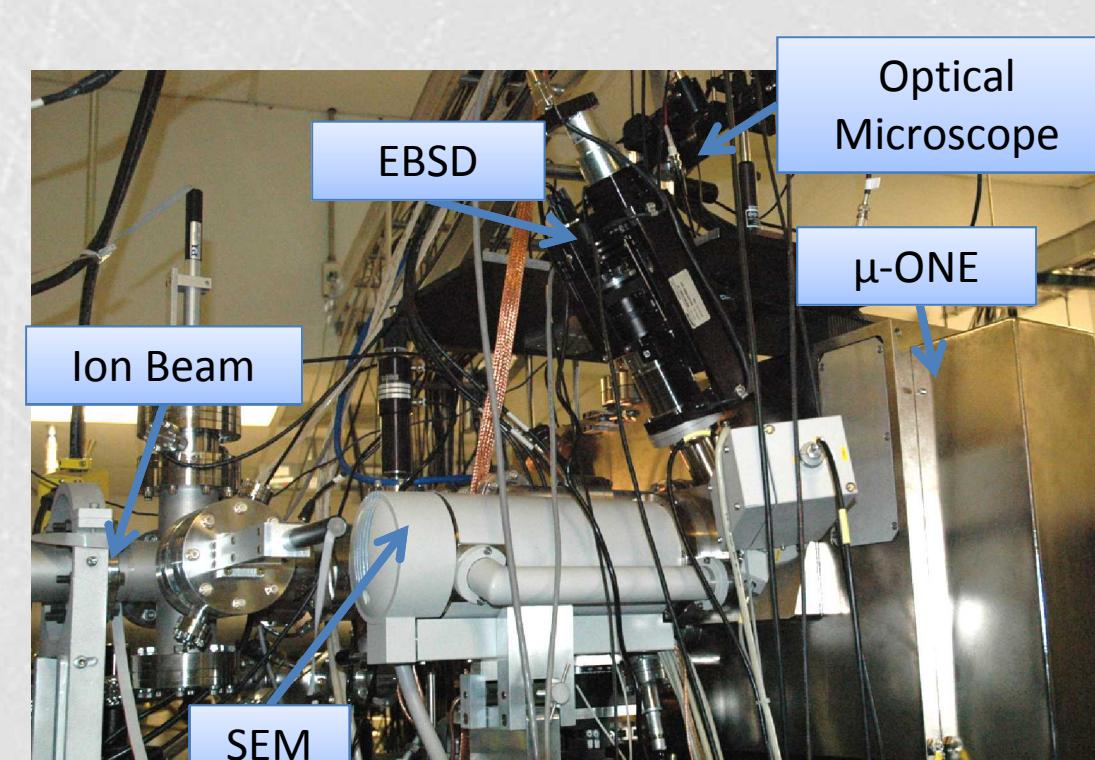
Mechanical Properties (small-scale testing)



Validating Comparison to Neutron Irradiation Experiments + Investigation into new materials

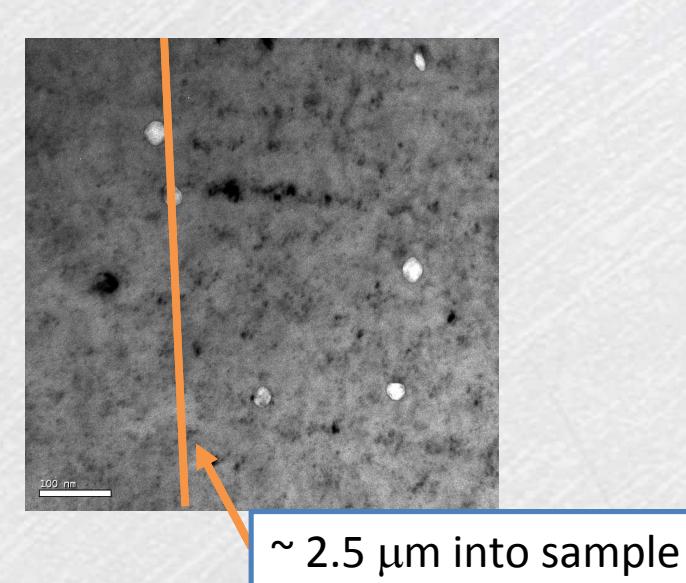
### Micrometer Resolution Optical, Nuclear, and Electron Microscope

- Improvements have been made to micro-ONE to permit rapid characterization of ion beam damage during implantation of H, He, or heavy ions at currents up to 10,000 ions/s.
  - Updating and making operations a SEM run in parallel
  - Addition of electron back scattered (EBSD) detector

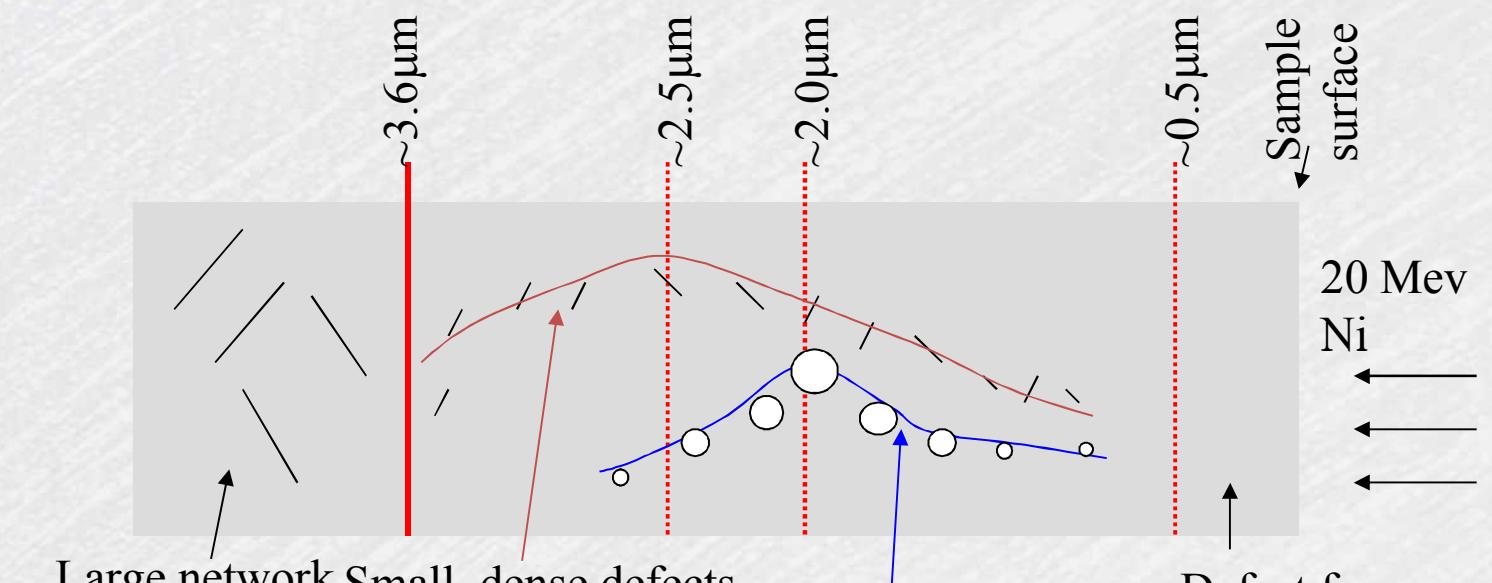


Micro-ONE now permits parallel imaging of changes in microstructure: grain size, phase transformations

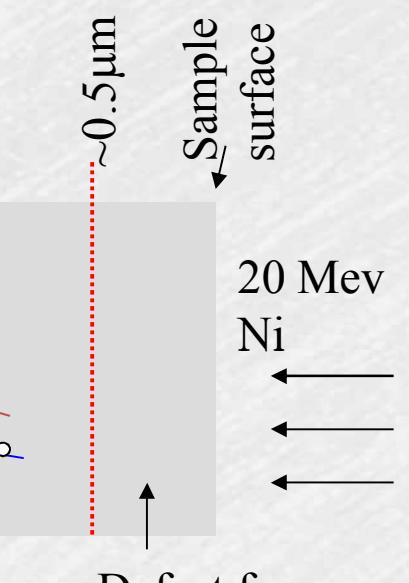
## Results



~2.5 μm into sample

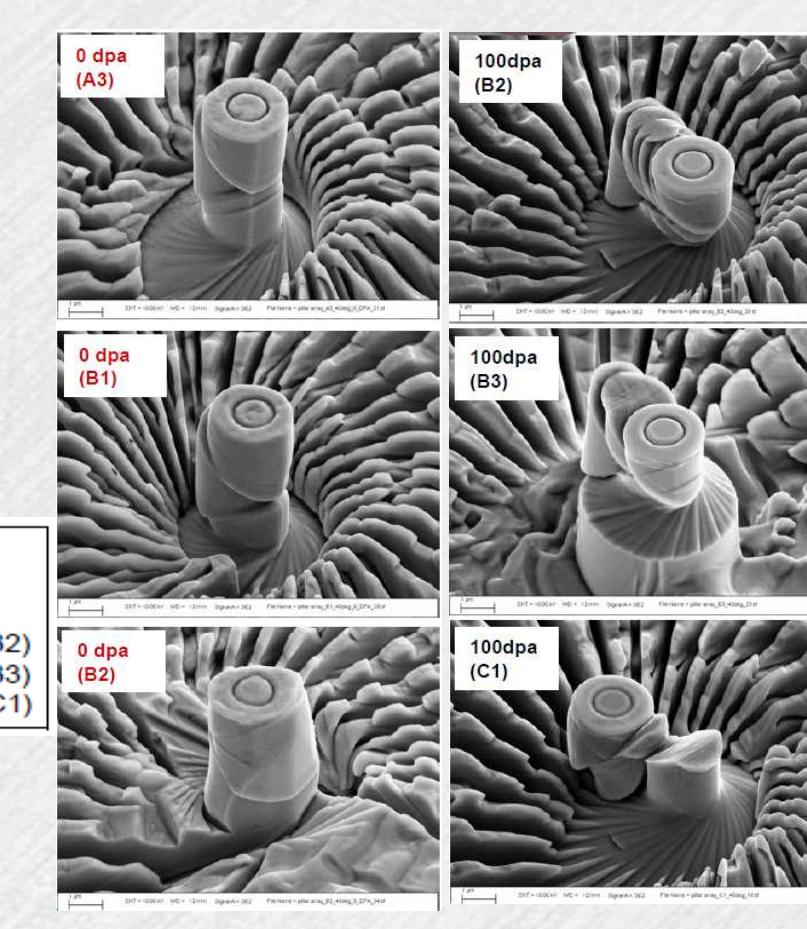
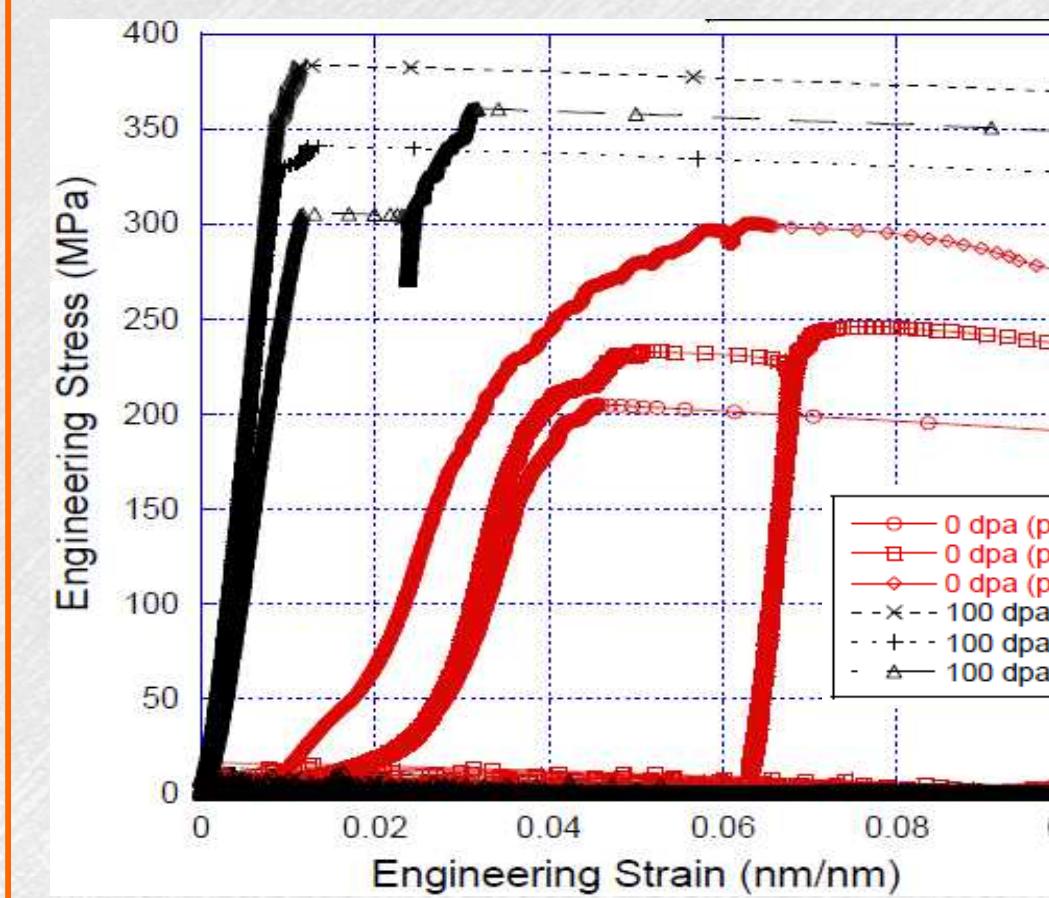


Large network Small, dense defects dislocations & dislocation loops Void distribution



Defect free Region

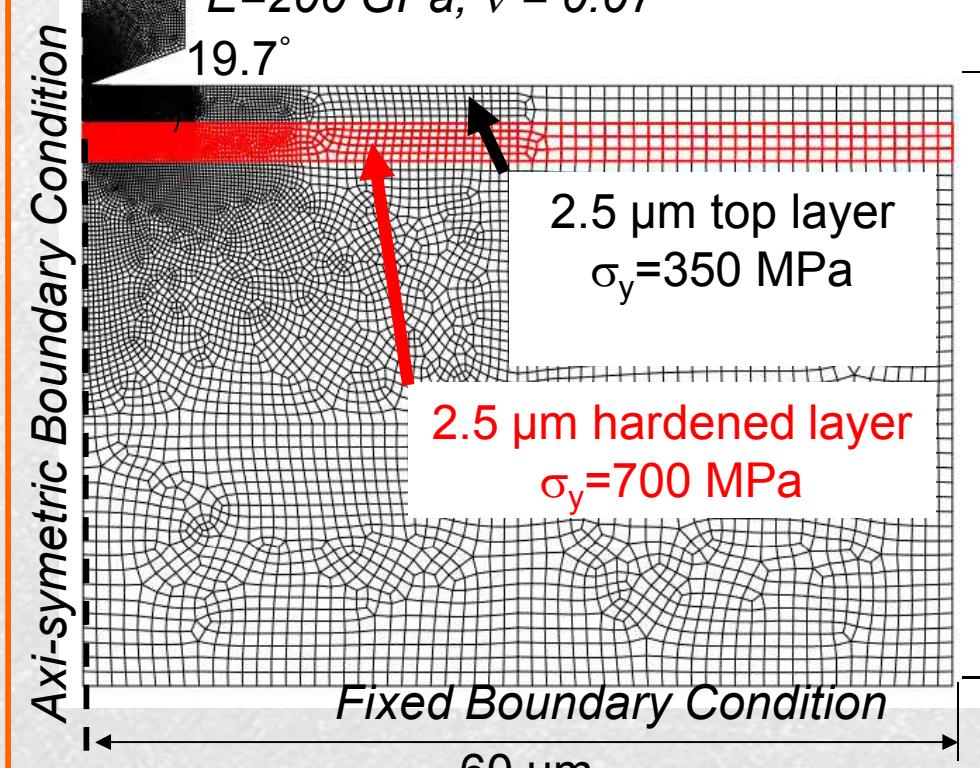
316 SS samples were irradiated at 400°, 500° and 600°C by 20 MeV Ni ions to a maximum peak dpa of ~100 at about 3.5 μm. The FIB was used for TEM sample preparation.



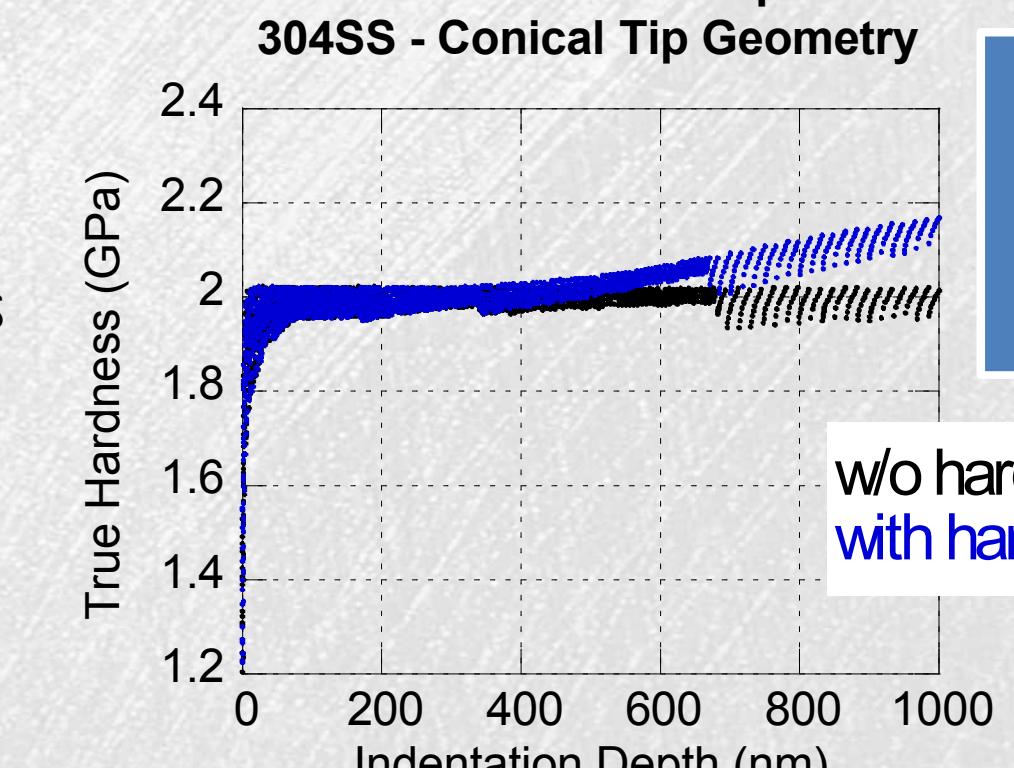
Results from ion irradiated and FIB prepared Cu micropillars indicates that ion beam irradiation can be used to determine the mechanical properties of ion irradiated volumes.

Disp. Boundary Condition Applied

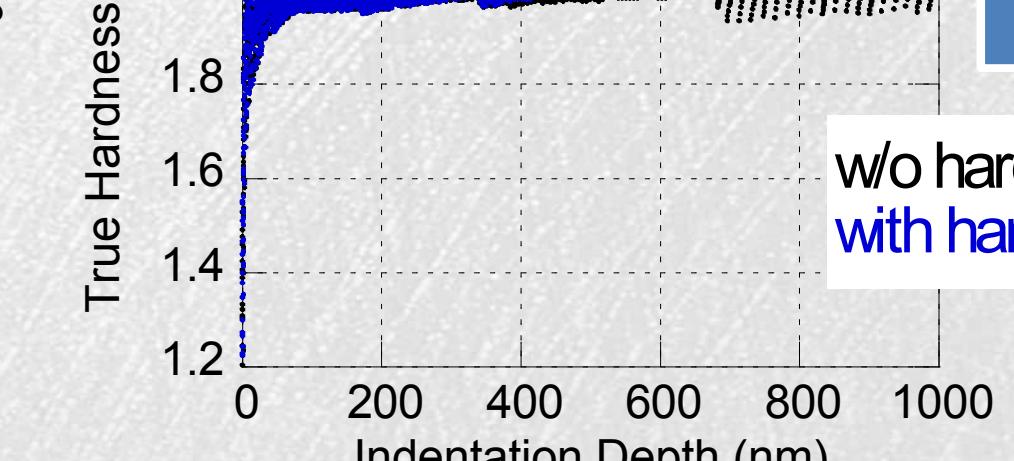
Tip: Diamond-  $E=1141 \text{ GPa}$ ,  $v=0.07$   
Substrate: 304L Stainless Steel  $E=200 \text{ GPa}$ ,  $v=0.07$



Hardness vs. Depth Simulated Indentation Experiments 304SS - Conical Tip Geometry

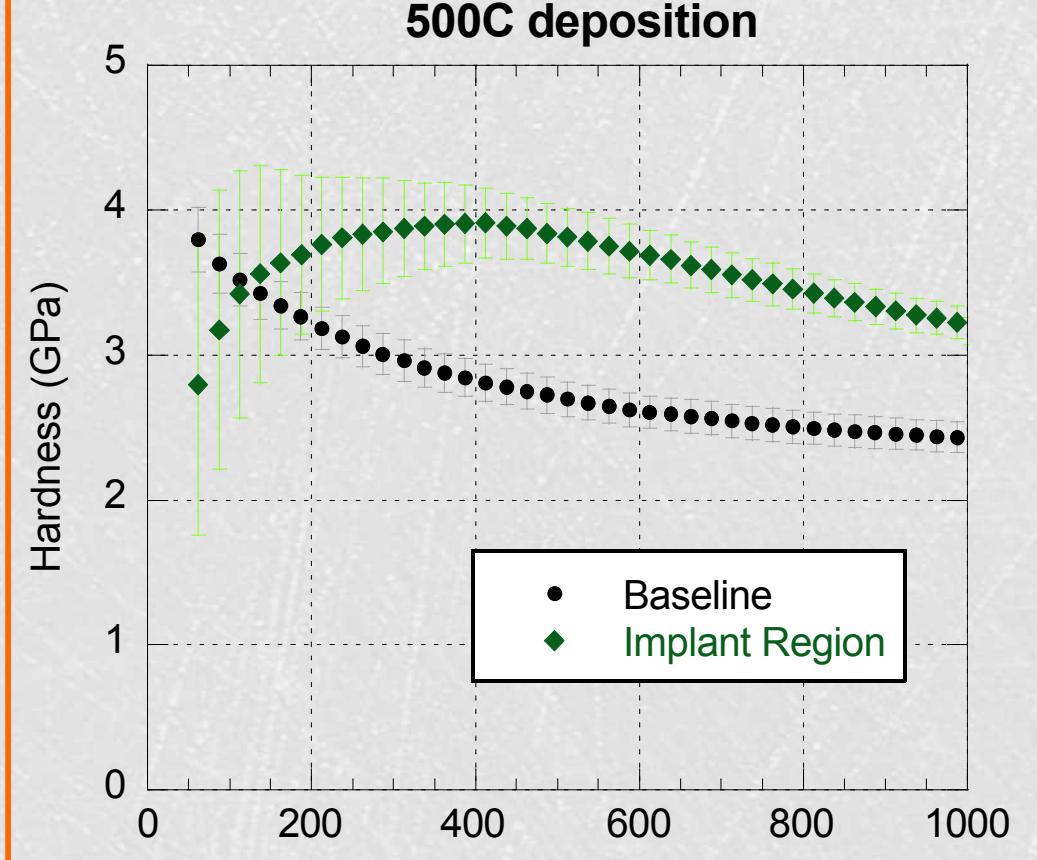


Finite Element Modeling indicates that radiation damage is identifiable

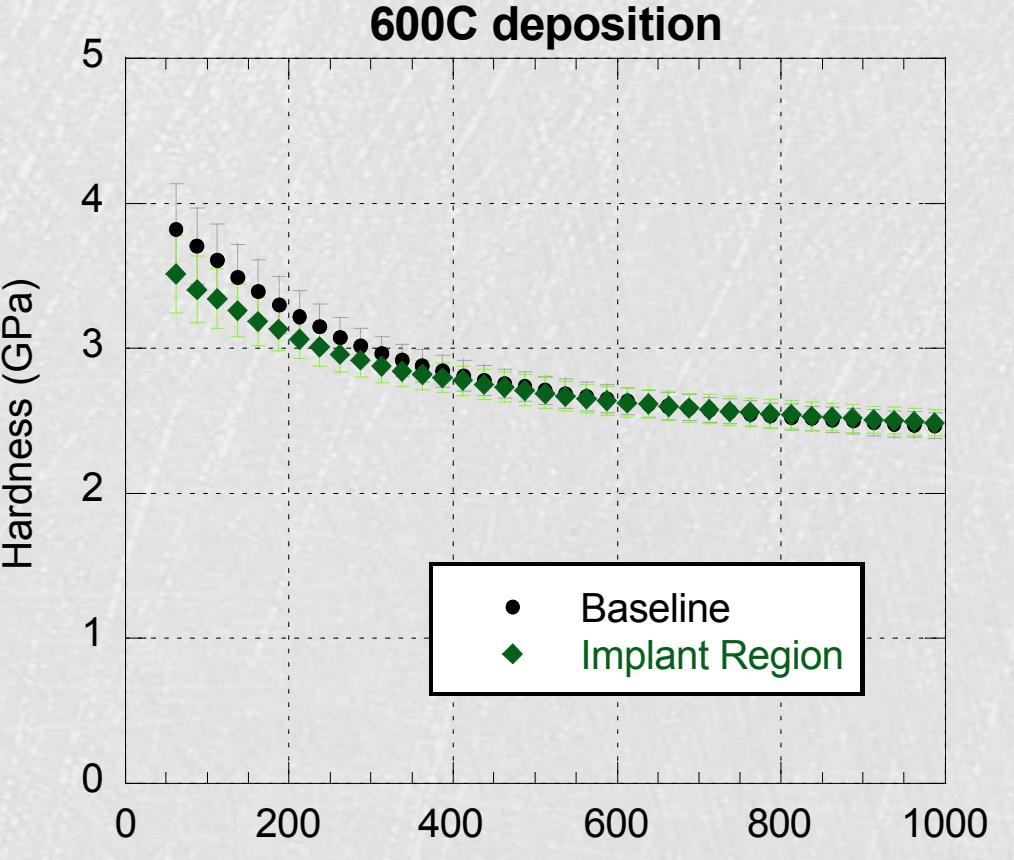


w/o hardened subsurface with hardened subsurface

Hardness vs. Indentation Depth Ion-Implant experiment on Stainless Steel 500C deposition



Hardness vs. Indentation Depth Ion-Implant experiment on Stainless Steel 600C deposition



Implantation at 600 C is dominated by diffusion of defects not seen in the 400 C and 500 C dominated by a high density of point defects.

## Significance

If a combinatorial approach to rapidly test the radiation damage produced to emulate neutron damage is developed, it will significantly enhance:

- First-order validation method for advanced cladding and structural materials for the next generation nuclear reactors.
- Rapid method to characterize and identify radiation tolerant materials
- Greater fundamental understanding of property-microstructure-processing relationship of materials in the extreme temperature and irradiation environment.
- Emergence of Sandia as a cutting-edge facility for ion-based simulation of radiation damage.

