



U.S. DEPARTMENT OF  
**ENERGY**

**Nuclear Energy**

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# **Implications of the Direct Disposal of Large Dry-Storage System Designs for Repository Design**

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## **U.S. Nuclear Waste Technical Review Board**

Technical Workshop on the Impacts of Dry-Storage Canister Designs on  
Future Handling, Storage, Transportation and Geologic Disposal of Spent  
Nuclear Fuel in the United States

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# Acknowledgments

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# Implications for Repository Design: Outline

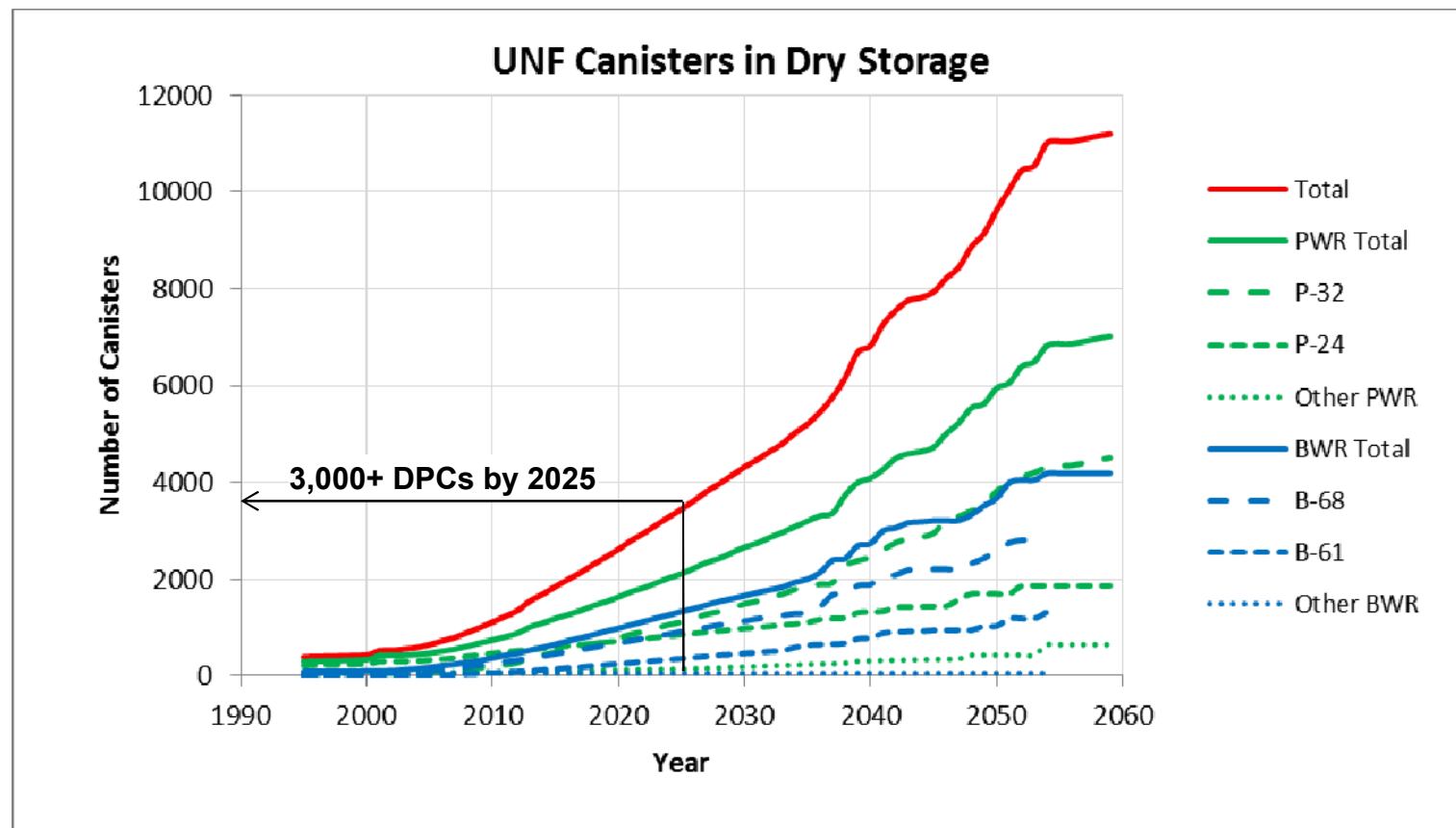
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- **Assumptions**
- **Design Options**
  - Thermal
  - Criticality control
  - Engineering challenges
- **Example Disposal Concepts**
- **Thermal Management Analysis**
- **Reactivity Scoping Analysis (e.g., Maine Yankee)**
- **Preliminary Logistical Analysis**
- **Summary and Conclusion**



# Dry Storage Projection\*

## Accumulation of Canisters (TSL-CALVIN)



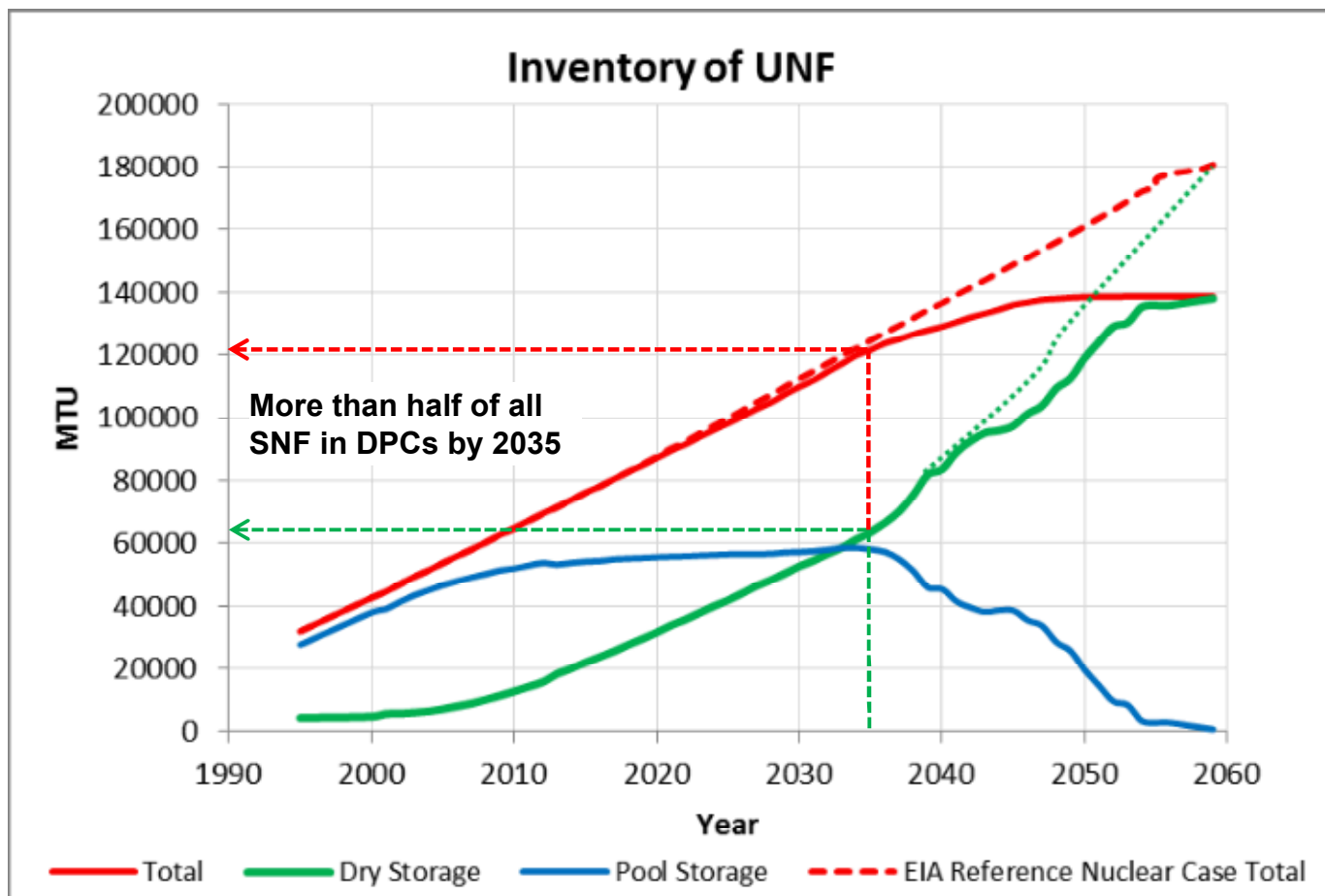
**Assume Presently Used DPC Types, No Fuel Shipments from Existing ISFSIs, and 20-yr Life Extensions for the Currently Operating Reactor Fleet.**

\*Simulation with TSL-CALVIN: Nutt et al. (2012)



# Spent Fuel Projection\*

## Accumulation in Dry Storage & Pools



**Assume 20-yr Life Extensions for the Currently Operating Reactor Fleet.**

\*Simulation with TSL-CALVIN: Nutt et al. (2012)



# DPC Direct Disposal Study Assumptions and Conditions

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## ■ Key Assumptions for This Analysis

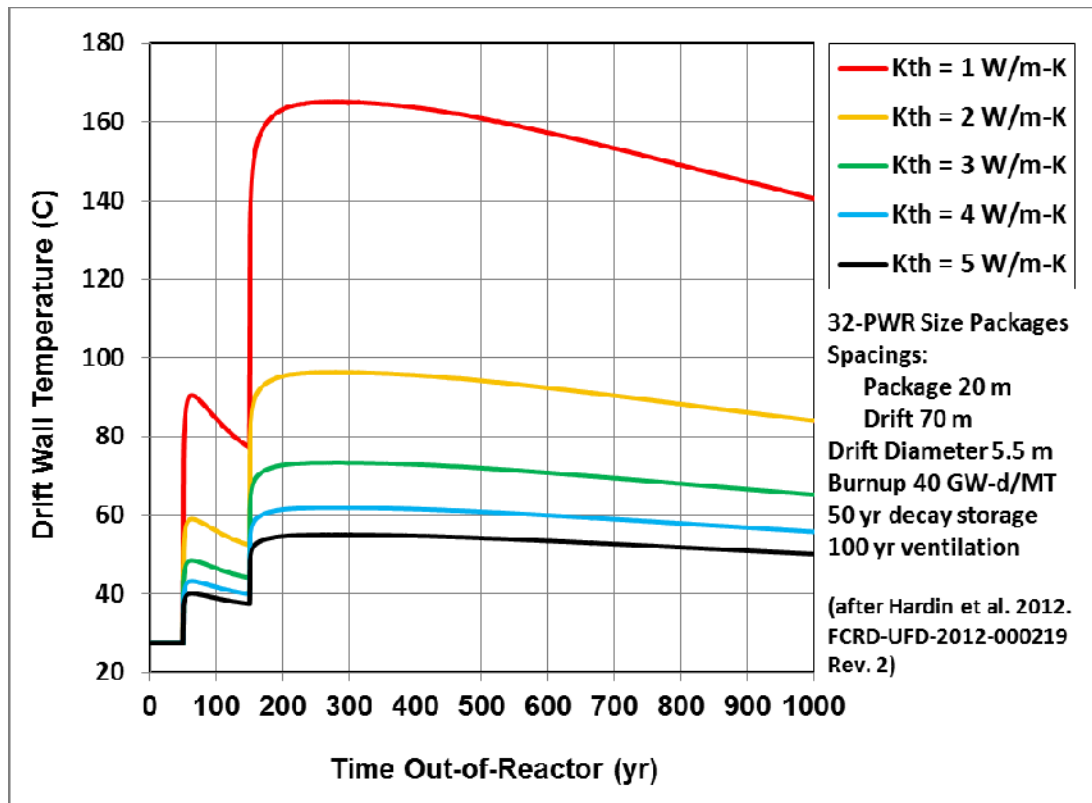
- Complete disposal (repository panel closure) at/before fuel age of 150-years out-of-reactor
- Fuel and canister condition will be suitable for transport and disposal, 50 to 100 years out-of-reactor
- Regulatory context similar to 40CFR197 and 10CFR63 (e.g., probabilistic treatment of features, events & processes)



# Design Options for Thermal Management

## ■ For a Given Waste Package Capacity and SNF Burnup:

- Choice of host rock
  - Salt
  - Hard rock
  - Sedimentary (clay-rich)
- Repository spacings
- Surface decay storage duration
- Ventilation
- Use of backfill



**Effect of rock  $K_{th}$  on drift wall temperature for a typical 32-PWR, average burnup case.**



# Design Options for Nuclear Reactivity Control

## ■ Disposal Environment

- Groundwater availability
- Salinity
- Package integrity

## ■ Moderator Exclusion

- Package integrity

## ■ Moderator Displacement

- Fillers (e.g., boron carbide loaded grout)

## ■ Reactivity Analysis Methodology

- Burnup credit, as-loaded, degradation cases

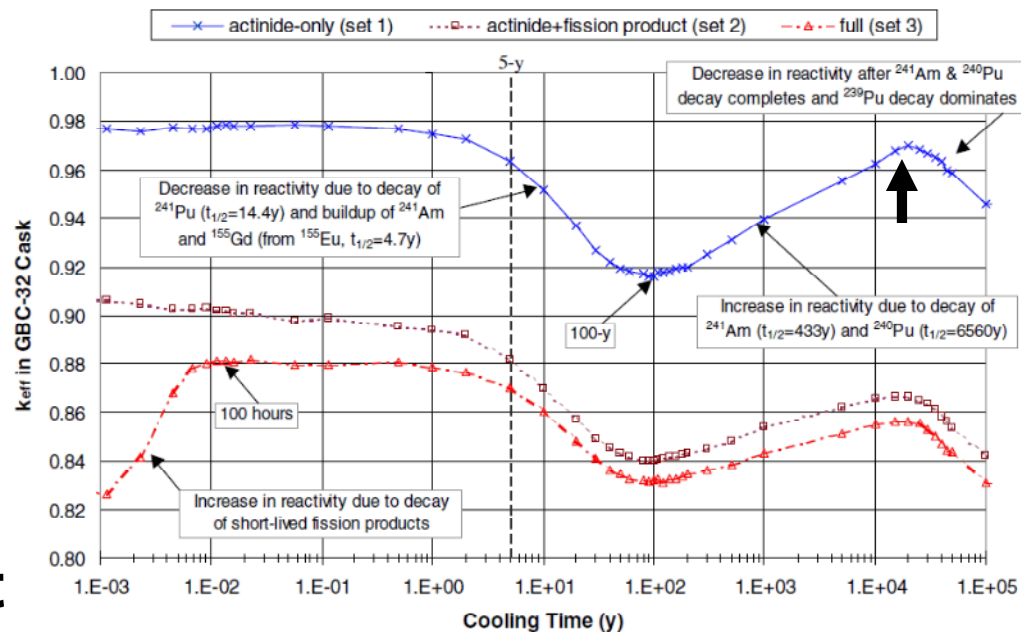


Figure source:  
Wagner and Parks 2001. NUREG/CR-6781.(Fig. 3)  
Generic burnup credit 32-PWR cask  
PWR fuel (4% enriched, 40 GW-d/MT burnup)





# Design Options to Address Engineering Challenges

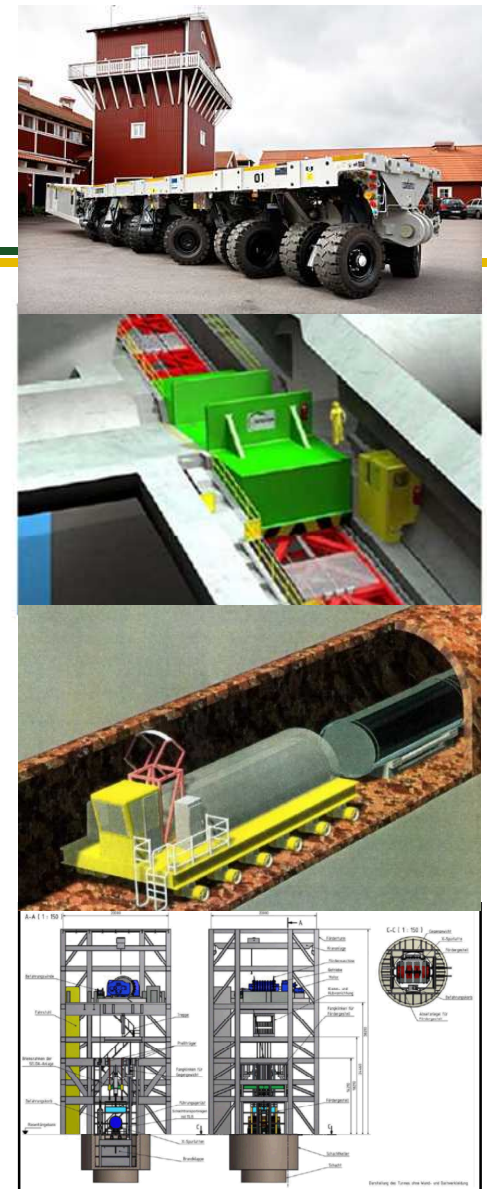
## ■ Handling/Packaging Using Current Practices

## ■ Surface-Underground Transport

- Heavy shaft hoist
- Spiral ramp ( $\leq 10\%$  grade for rubber-tires)
- Linear ramp ( $>10\%$  possible with funicular)
- Shallow ramp ( $\leq 2.5\%$  for standard rail)

## ■ Opening Stability Constraints

- Salt (a few years with minimal maintenance)
- Hard rock (50 years or longer)
- Sedimentary (50 years may be feasible; longer may require special geologic settings)



Sources:

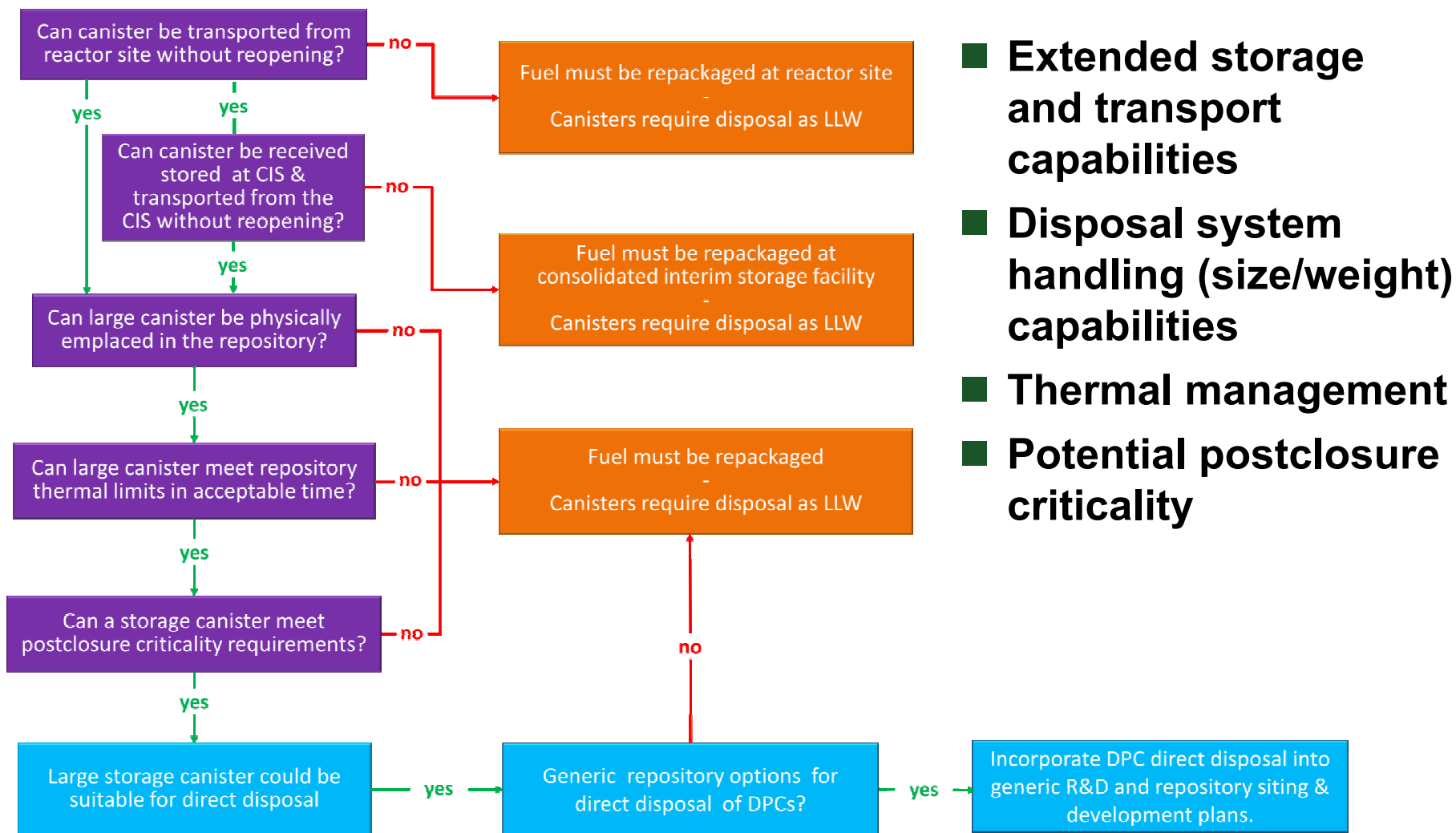
Fairhurst 2012

[www.wheelift.com](http://www.wheelift.com)

Nieder-Westermann et al. 2013



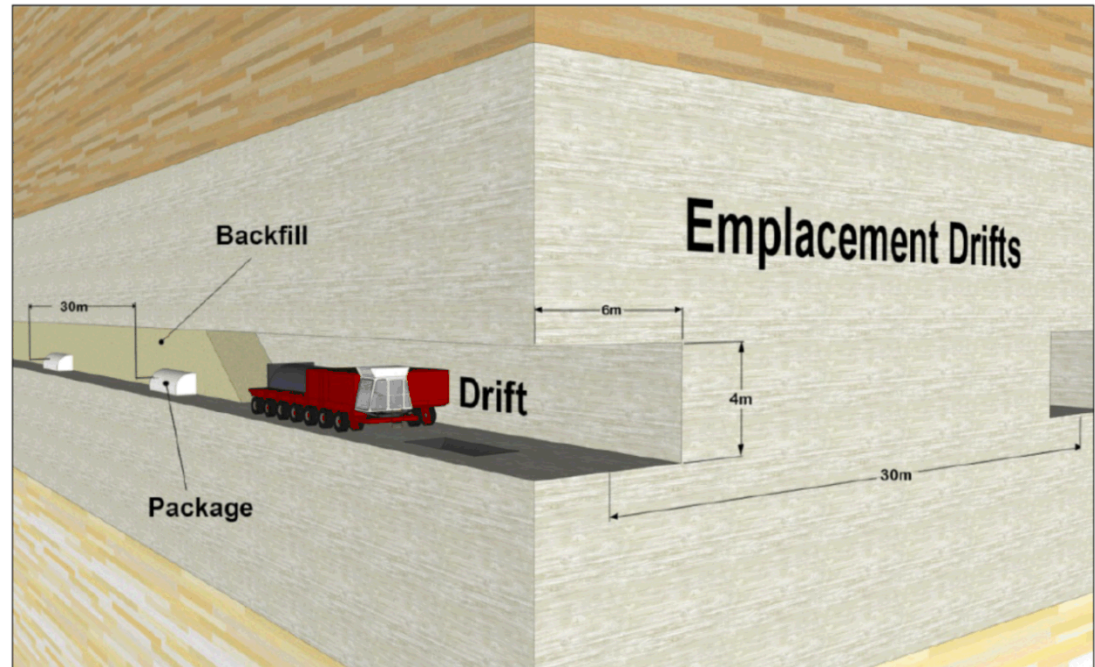
# Path to Direct Disposal of SNF in Dual-Purpose Canisters





# Example: Generic Concept for a SNF Repository in Salt

- 32-PWR size or larger
- Emplace SNF at 50 to 70 years out-of-reactor (OoR)
- Crushed salt backfill at emplacement
- Bedded or domal salt
- Shaft or ramp access (~175 MT payload with shielding and cart)
- Handling equipment and shaft/ramp conveyance development needed

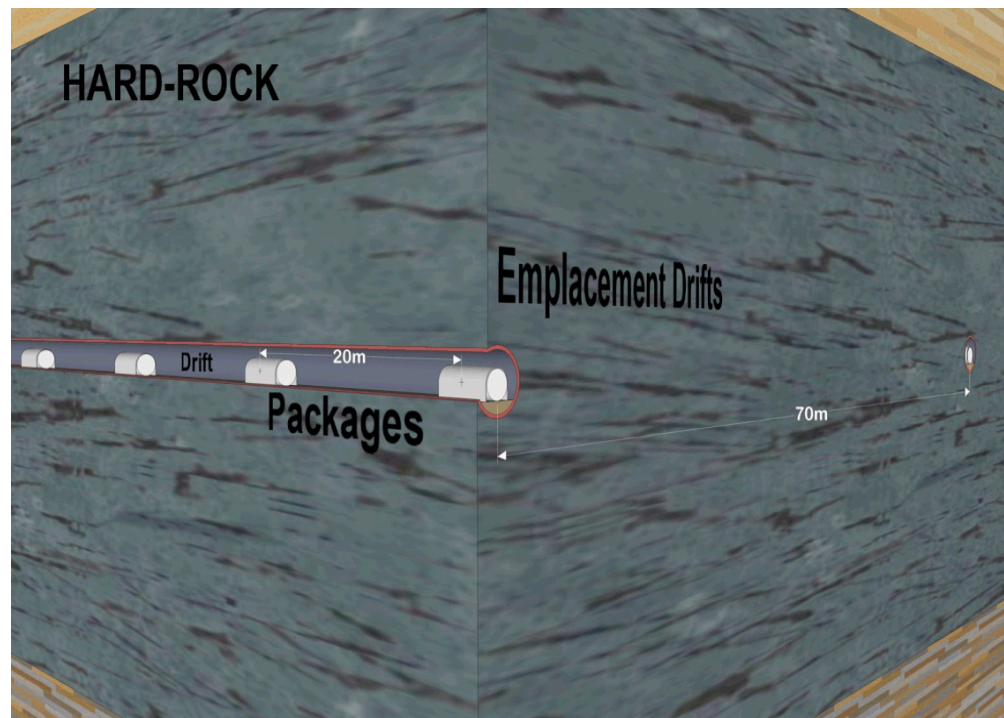


Source: Hardin et al. 2013. FCRD-UFD-2013-000171 Rev. 0)



# Example: Hard-Rock Unbackfilled Open Concept

- 32-PWR size or larger
- Emplace SNF at 50 to 100 years OoR
- Ventilate up to 50 yr, closure at <150 years OoR
- Unbackfilled, for unsaturated settings (or include backfill for saturated settings)
- Corrosion resistant waste packaging
- Additional engineered barriers may be installed at closure (e.g., drip shields)
- Long-term opening stability can be expected through the postclosure thermal peak

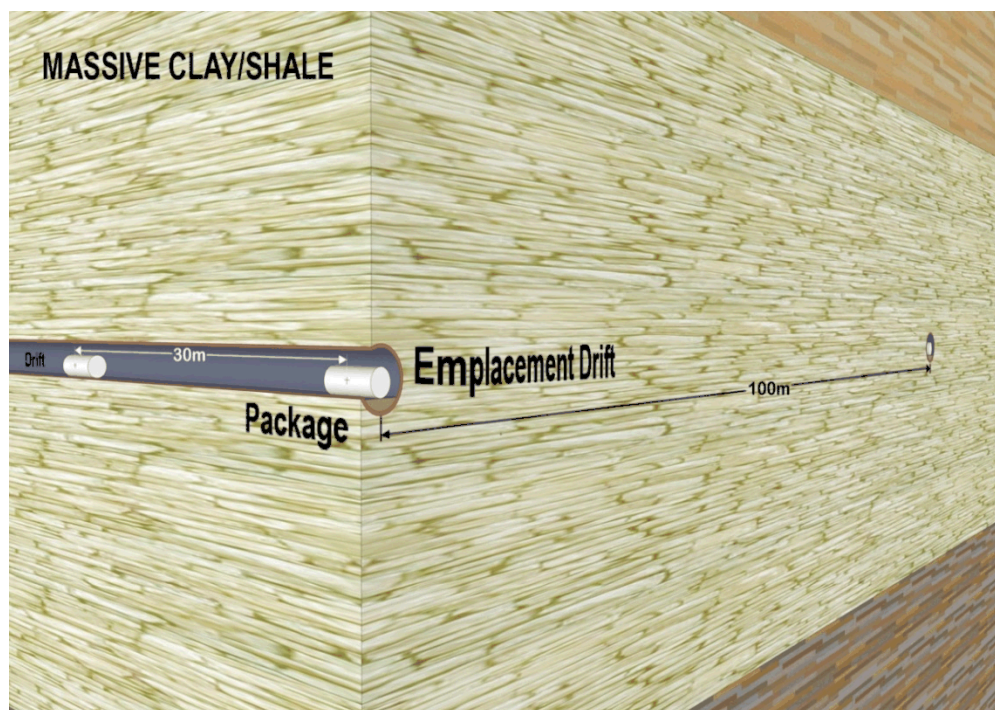


Source: Hardin et al. 2013. FCRD-UFD-2013-000171 Rev. 0.



# Example: Sedimentary Backfilled Open Concept

- **Massive, soft clay/shale**
- **In-drift emplacement**
- **32-PWR size or larger**
- **Emplace SNF at 50 to 100 years OoR**
- **Backfilling at closure (peak backfill  $T \gg 100^{\circ}\text{C}$ )**
- **Closure at 100 to  $>200$  years OoR (limited by host rock)**
- **Possible local heating of host rock  $>100^{\circ}\text{C}$**
- **Steel or corrosion resistant waste packaging as needed**

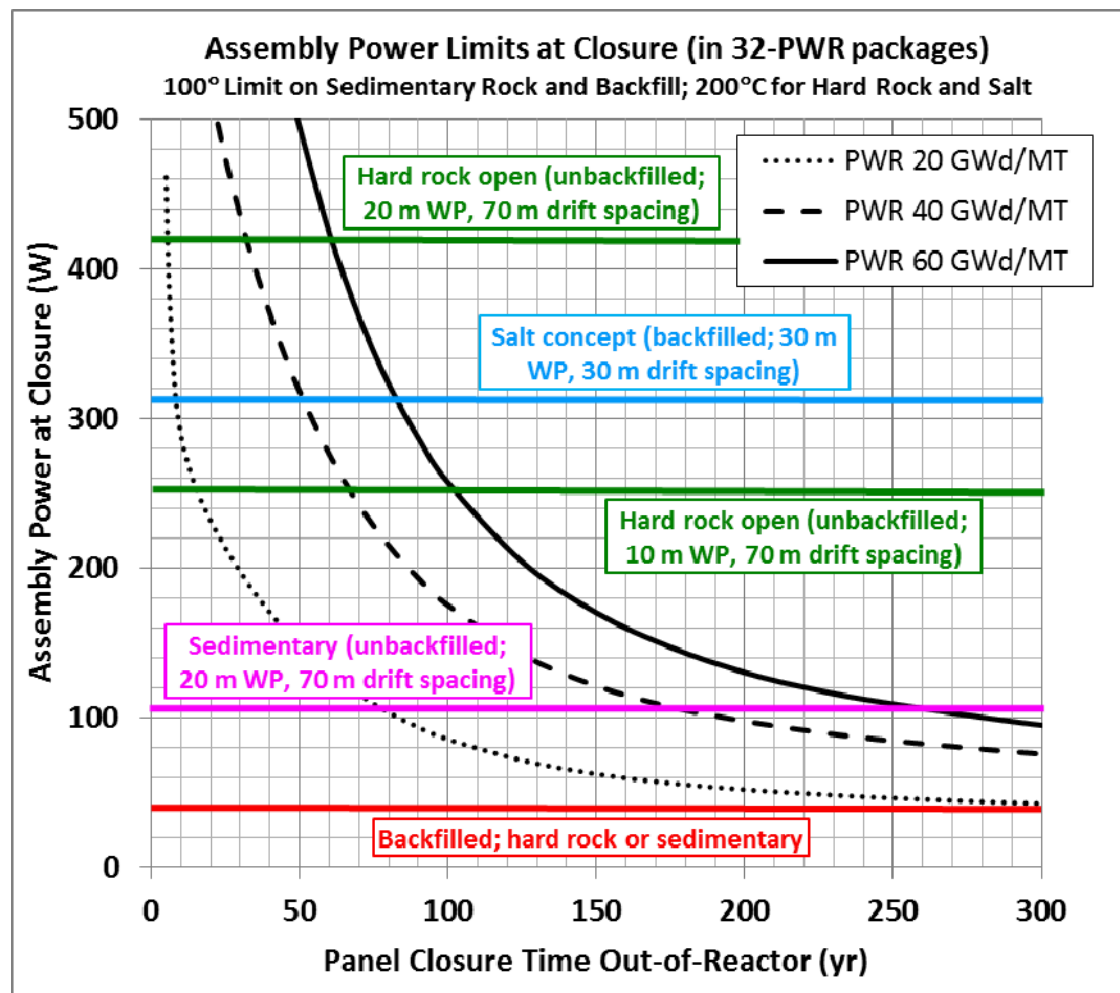


Source: Hardin et al. 2013. FCRD-UFD-2013-000171 Rev. 0.



# Time to Repository Panel Closure for Representative Disposal Concepts

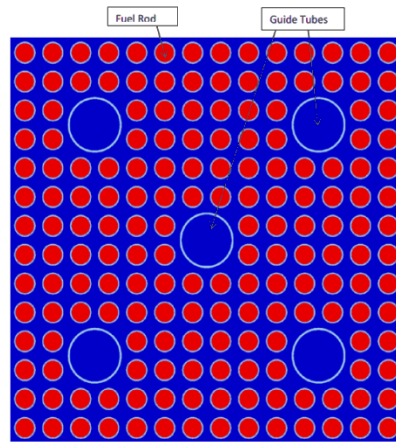
## 32-PWR size packages



Based on: Hardin et al. 2013. *Collaborative Report on Disposal Concepts*. FCRD-UFD-2013-000170 Rev. 0.



# Numerical Model of TSC-24 Canister, Maine Yankee



- Numerical Model of TSC-24 Canisters (31 analyzed)
- ORNL Database SNF-ST&DARDS
- Software/Data
  - SCALE code system (ORNL 2011)
  - Details: see Clarity and Scaglione (2013)
- Also Analyzed: 26 canisters at Sequoyah (MPC-32 type)

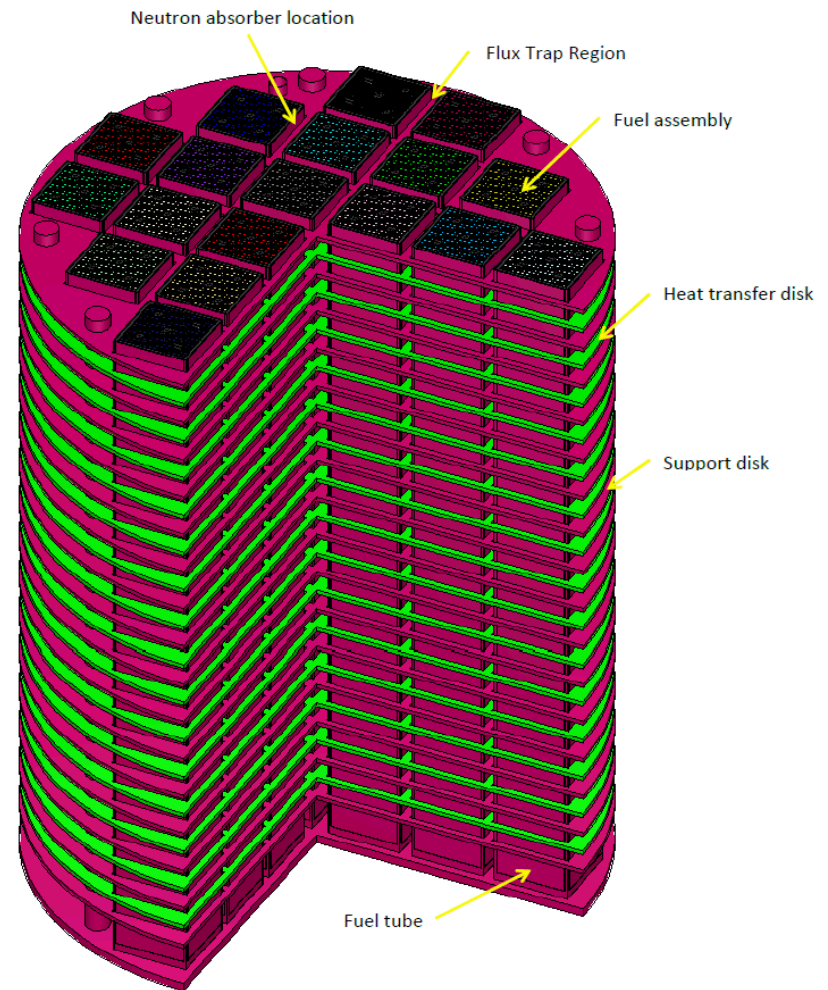
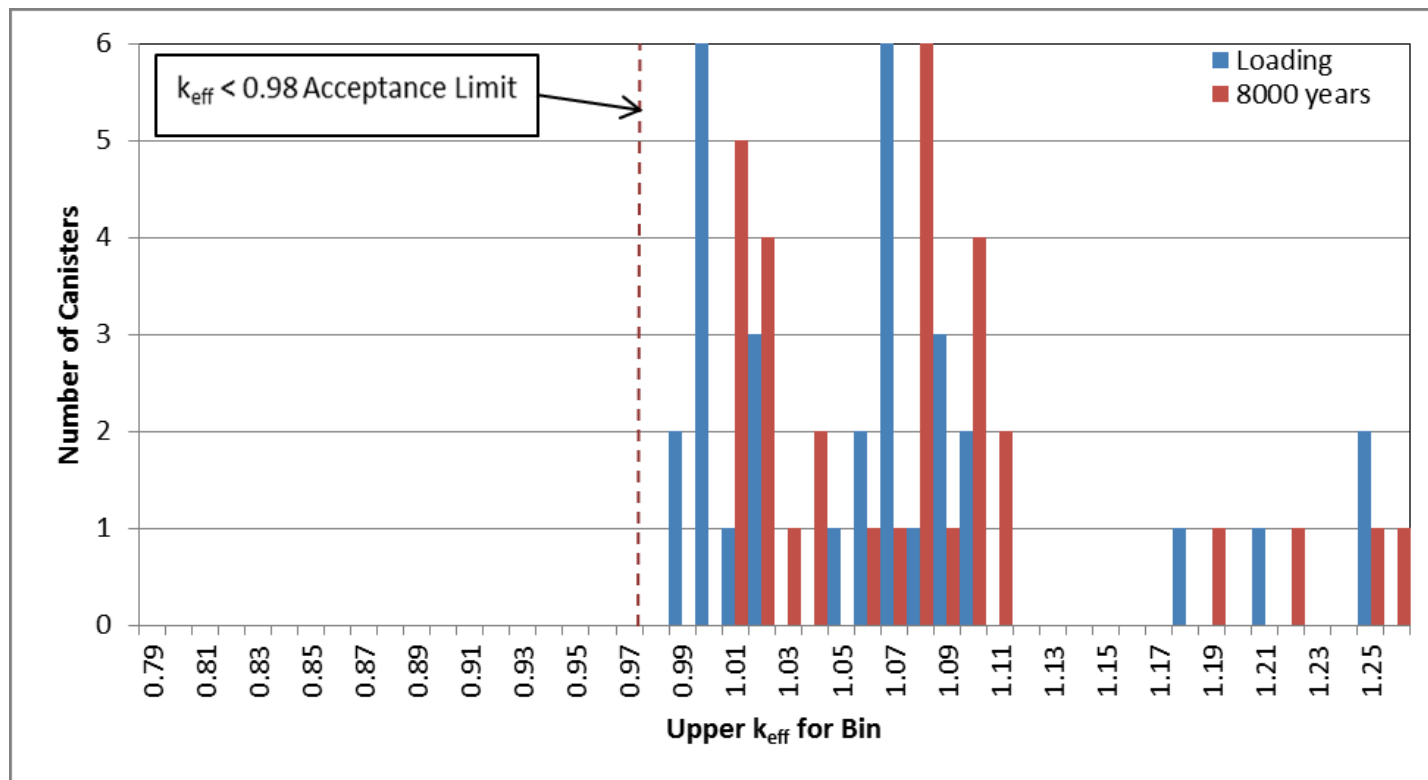


Figure 8-3. Cut-away view of the TSC-24 design basis KENO model



# Reactivity Scoping Analysis Results, Maine Yankee

- Degraded basket case (and loss of absorber), flooded with fresh water
- Analyzed as-loaded, with burnup credit



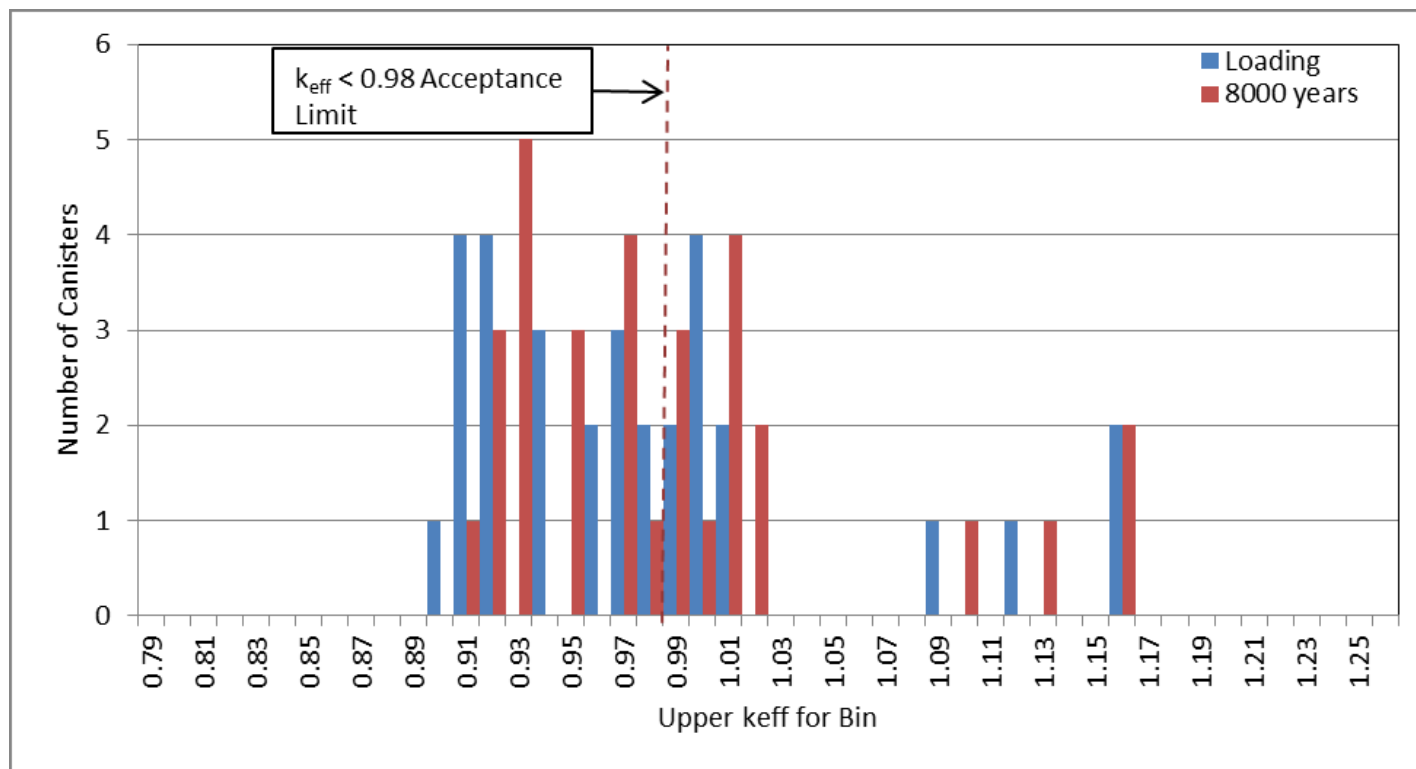
Source: Clarity, J. and J. Scaglione 2013. ORNL/LTR-2013/213.





# Reactivity Scoping Analysis Results, Maine Yankee

- Degraded basket case (and loss of absorber), flooded with 1 molal NaCl brine
- Analyzed as-loaded, with burnup credit



Source: Clarity, J. and J. Scaglione 2013. ORNL/LTR-2013/213.



# Preliminary Logistical Analysis of DPC Direct Disposal Scenarios

## ■ Objectives:

- Forecast when DPCs loaded with SNF from the existing fleet could be emplaced in a repository, for emplacement thermal power limits of 4, 6, 8, 10, and 12 kW/canister
- Project repository acceptance (throughput) rates
- Estimate the incremental costs that would be required to store DPCs at a centralized interim storage (CIS) facility for cooling
- Compare with estimates of the cost to re-package the SNF into smaller canisters for disposal

**Use TSL-CALVIN code, developed originally for Yucca Mountain repository studies, adapted with additional features to generic studies (Nutt et al. 2012).**

Source: Nutt, W.M. 2013. *Preliminary System Analysis of Direct Dual Purpose Canister Disposal*. FCRD-UFD-2013-000184.



# Preliminary Logistical Analysis: Assumptions

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## ■ Modeling from the Present Until Repository Closure:

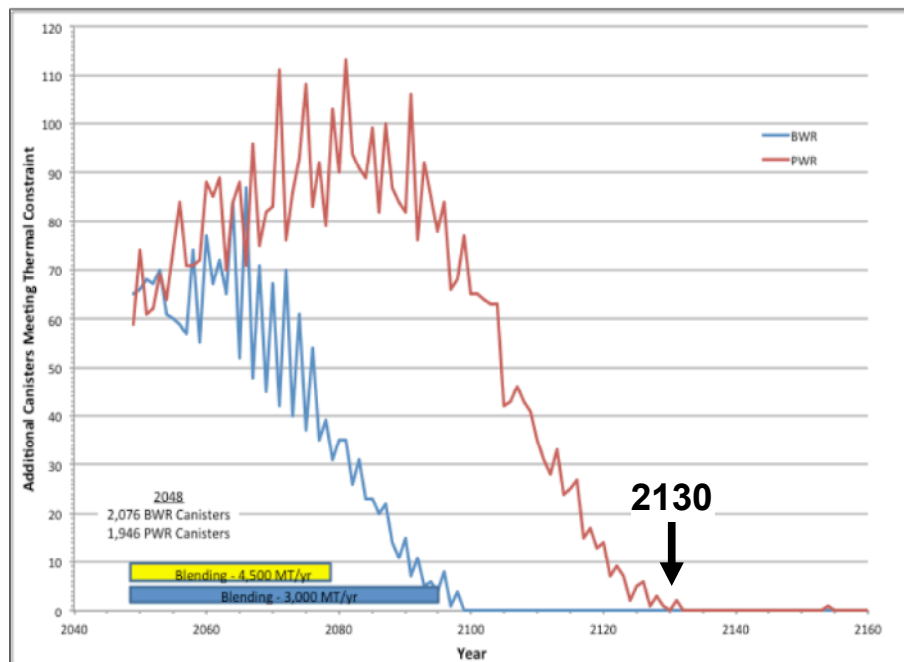
- SNF will be generated at all currently operating power plants, with 20-year life extensions, and gradual increases in burnup.
- All SNF would be put in dry storage as plants are decommissioned.
- Shipment of DPCs from reactor sites to an CIS would begin in 2025.
- A repository would open and begin to emplace DPCs underground in 2048.
- Once the repository is operating, DPCs cool enough for disposal would be shipped from reactor sites or from the CIS.

Source: Nutt, W.M. 2013. *Preliminary System Analysis of Direct Dual Purpose Canister Disposal*. FCRD-UFD-2013-000184.

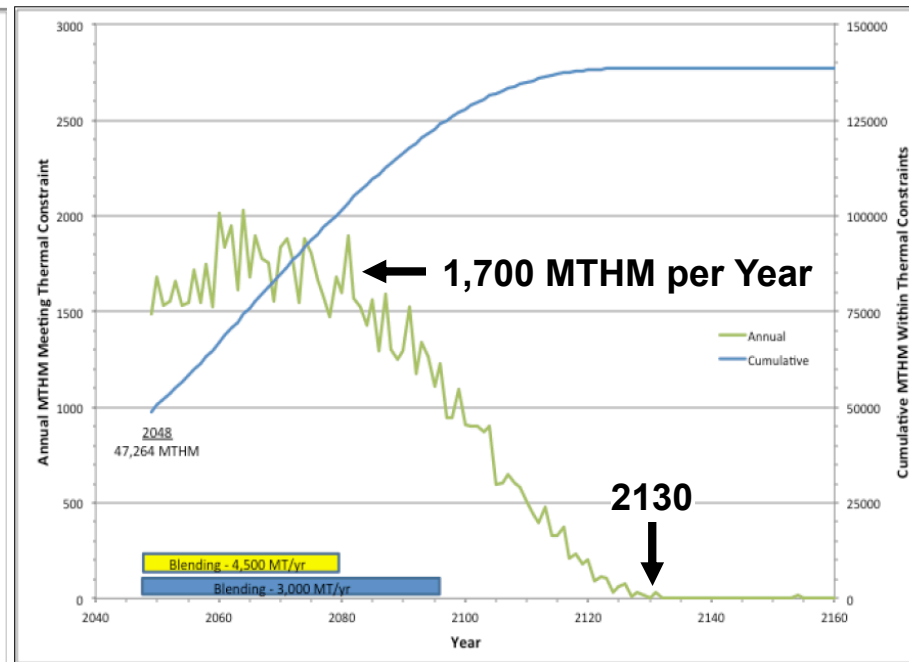


# Preliminary Logistical Analysis: 10 kW Emplacement Power Limit

- 10 kW is a typical emplacement power limit for salt disposal
- Emplacement operations would be substantially done by 2130
- Additional ventilation time would be needed for hard rock (up to 50 yr) and sedimentary (100 to >200 yr) concepts



Number of canisters per year, vs. calendar year



SNF emplaced per year (MTHM), vs. calendar year

Source: Nutt, W.M. 2013. FCRD-UFD-2013-000184.



# Implications for Repository Design Summary and Conclusions

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## ■ Summary of Design Options

- Thermal, criticality, and engineering challenges

## ■ Example Disposal Concepts for DPC-Based Waste Packages

- Salt (backfilled at emplacement)
- Hard rock (unsaturated/unbackfilled or saturated/backfilled)
- Sedimentary (clay-rich)

## ■ Thermal Results

- Repository panel closure <150 yr fuel age out-of-reactor (salt and hard rock, and low-to-moderate burnup SNF in sedimentary)
- For sedimentary settings and higher burnup SNF: need some combination of longer repository operations, local heating of host rock > 100°C, and larger repository spacings
- Backfill temperature potentially >> 100°C (if used)



# Implications for Repository Design Summary and Conclusions, cont.

## ■ Reactivity Scoping Results

- Reactivity margin available with burnup credit analysis, as-loaded assembly information
- Preliminary results show some, but not all, DPCs could be sub-critical for the degraded cases as defined
- Saline water ( $^{35}\text{Cl}$ ) provides significant absorption
- Other options (e.g., fillers) are being investigated

## ■ Preliminary Logistical Result

- At 10 kW power limit, emplacement could be complete at 2130, with average emplacement rate of 1,700 MTHM/yr

***Preliminary results indicate DPC direct disposal could be technically feasible, at least for certain concepts. They also suggest that cost savings might be realized compared to re-packaging, although further analysis is needed. Feasibility evaluation and related R&D activities are planned to continue.***



# References

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- Wagner and Parks 2001. *Recommendations on the Credit for Cooling Time in PWR Burnup Credit Analyses*. NUREG/CR-6781. Oak Ridge National Laboratory, Oak Ridge, TN. ORNL/TM-2001/272.



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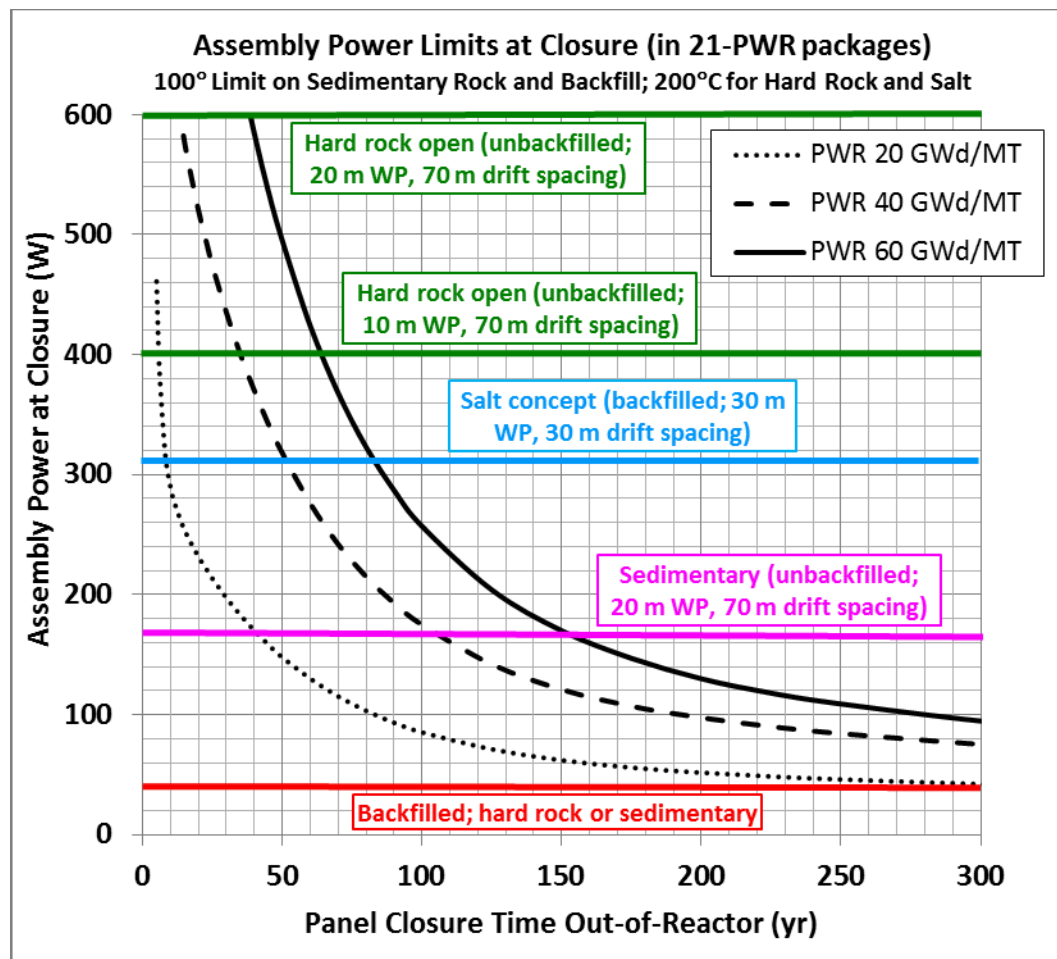
# Backup





# Time to Repository Panel Closure for Representative Disposal Concepts

## 21-PWR size packages



Based on: Hardin et al. 2013. *Collaborative Report on Disposal Concepts*. FCRD-UFD-2013-000170 Rev. 0.