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Title: Differential Die-Away Self-Interrogation

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# Differential Die-Away Self-Interrogation

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**NGSI Spent Fuel Inter-Lab Meeting, December 16-17, 2013**



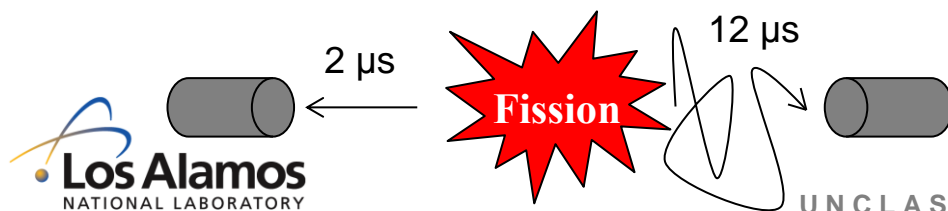
# Outline

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- How DDSI works
- Conceptual design
- FY13 achievements
- Upcoming work

## How DDSI Works

- Neutrons from spontaneous fission isotopes in spent fuel slow down as they scatter through a spent fuel pool, and induce fission in fissile isotopes. This increases an otherwise difficult to measure signal
- We record the time of arrival of each neutron (list-mode) to perform neutron coincidence counting. A coincidence is detection of two correlated neutrons and it is used to detect spontaneous and induced fission events
- Use list-mode data to produce a Rossi-alpha distribution (RAD), which is a histogram of times between captures of two neutrons
- RAD allows for flexible post-processing; traditional shift-register technique requires pre-selection of gate of interest



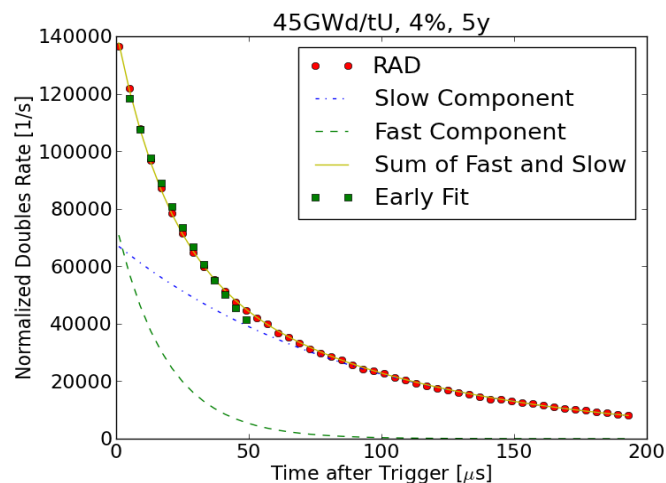
Time between captures = 10  $\mu$ s, add to histogram

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Slide 2

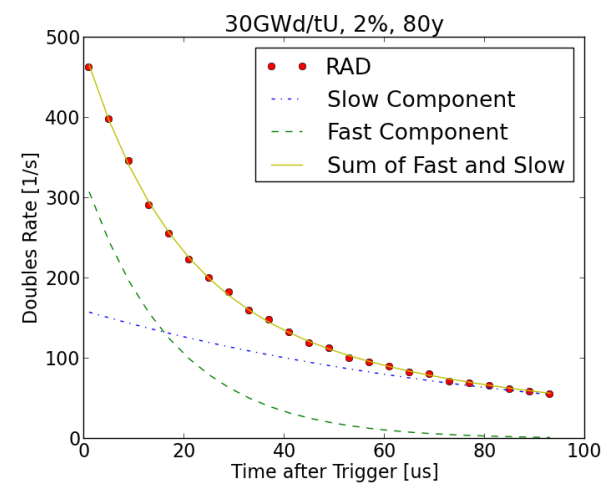
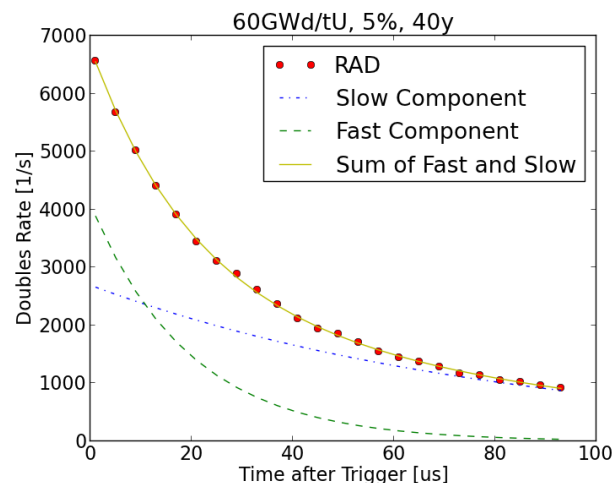
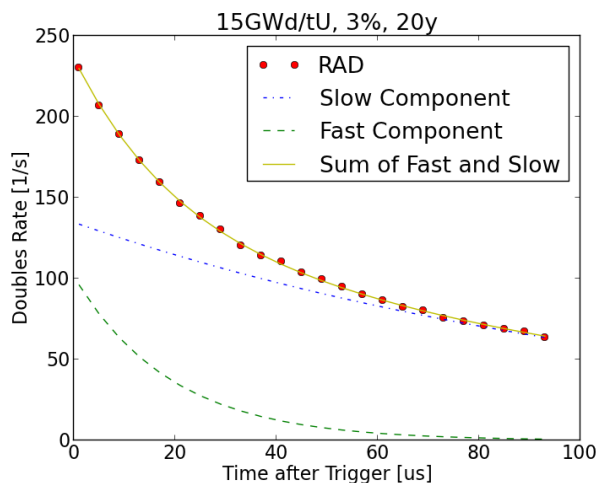
# How DDSI Works

- RAD can be broken down into two components
  - *Fast Component* composed of neutrons arriving from the same fission event
  - *Slow Component* composed of neutrons arriving from different fission events in the same chain
- Fast and slow components from broken down RAD help us characterize spent fuel assemblies; they reflect fissile/fertile content



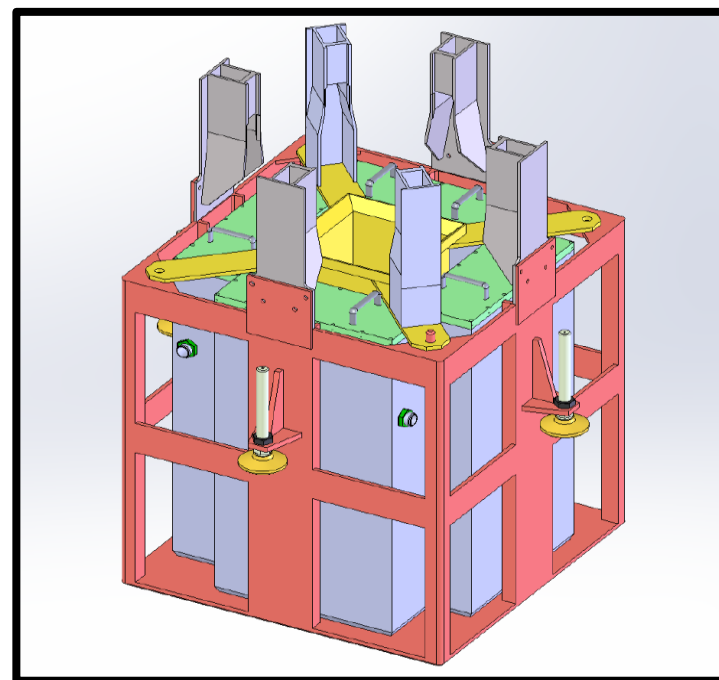
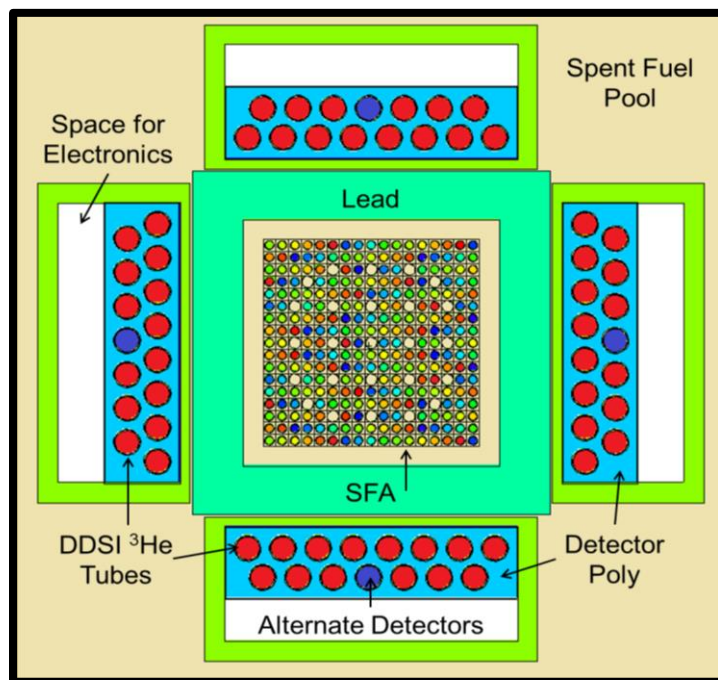
# How DDSI Works

- Die-away time of the slow component along with the relative magnitudes of both components shape RAD and reflect system multiplication



# Design

- 56  $^3\text{He}$  tubes, 4 detector pods, lead shield, centered assembly



- Air-filled insert for BWR assemblies

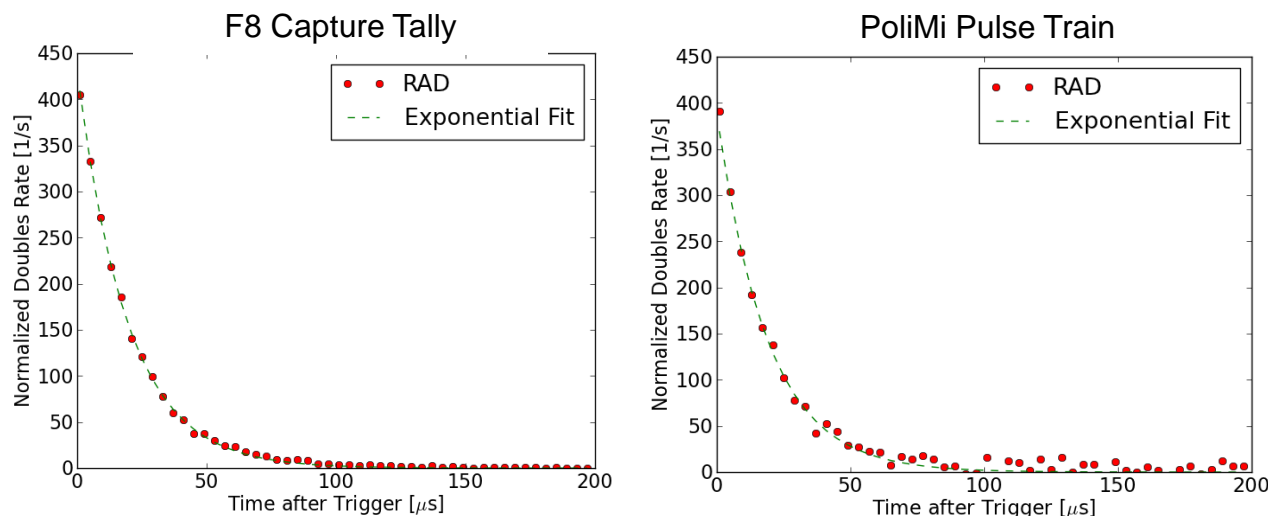
## FY13: SFL Exploration

- We utilize 44 SFL2a assemblies to observe changes in parameters such as multiplication, fissile/fertile masses, fast/slow component shape
    - Initial Enrichment (IE): 2%, 3%, 4% and 5%
    - Burnup (BU): 15, 30, 45 and 60 GWd/tU
    - Cooling time (CT): 1, 5, 20, 80 years
- } = 44 SFAs



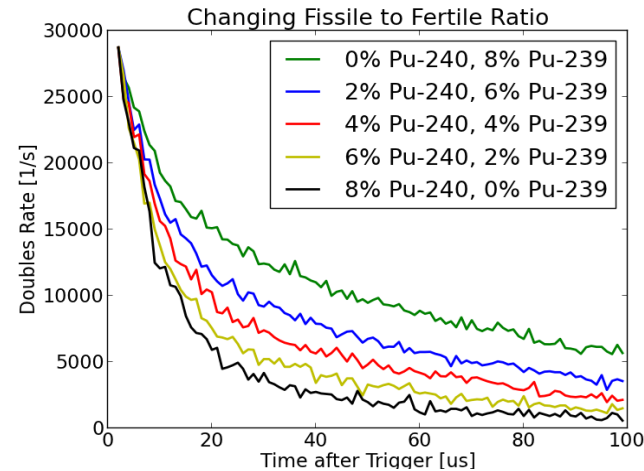
## FY13: Pulse Train Simulation

- MCNPX PTRAC capability and MCNPX-PoliMi used to produce pulse trains including accidentals
- Algorithms developed to build Rossi-alpha distributions from pulse trains
- Very good agreement between pulse train RAD and F8 capture tally RAD production (MCNPX approximation, free of accidentals)

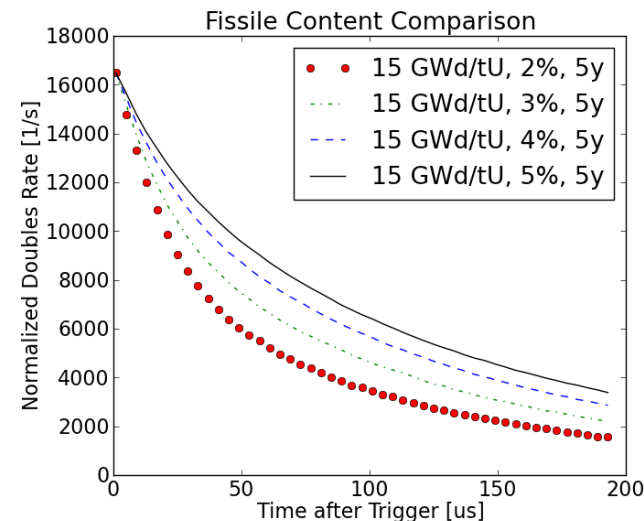


# FY13: RAD Dependence on Fissile/Fertile

- The shape of the RAD is strongly affected by the fissile/fertile material
- Relative magnitude of the slow and fast components change with addition of fissile or fertile material, and change the shape of the RAD as a result
- Longer, slower die-away of slow component with more fissile material and fewer neutron absorbers



Homogenized  
Assemblies

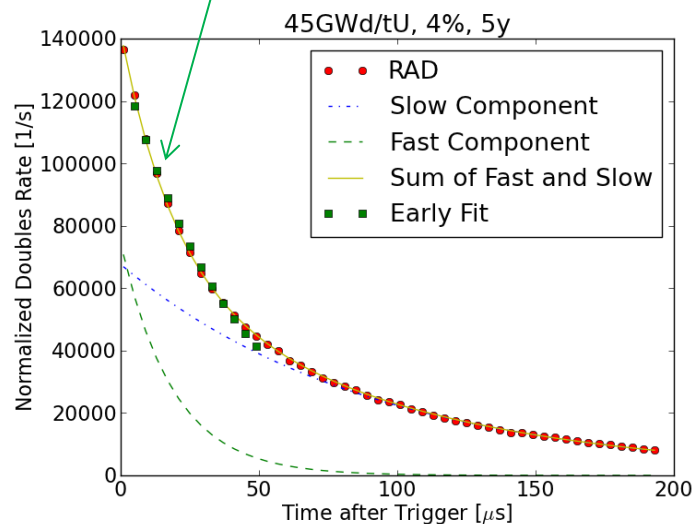


SFL-2a  
Assemblies

## FY13: Determining Multiplication

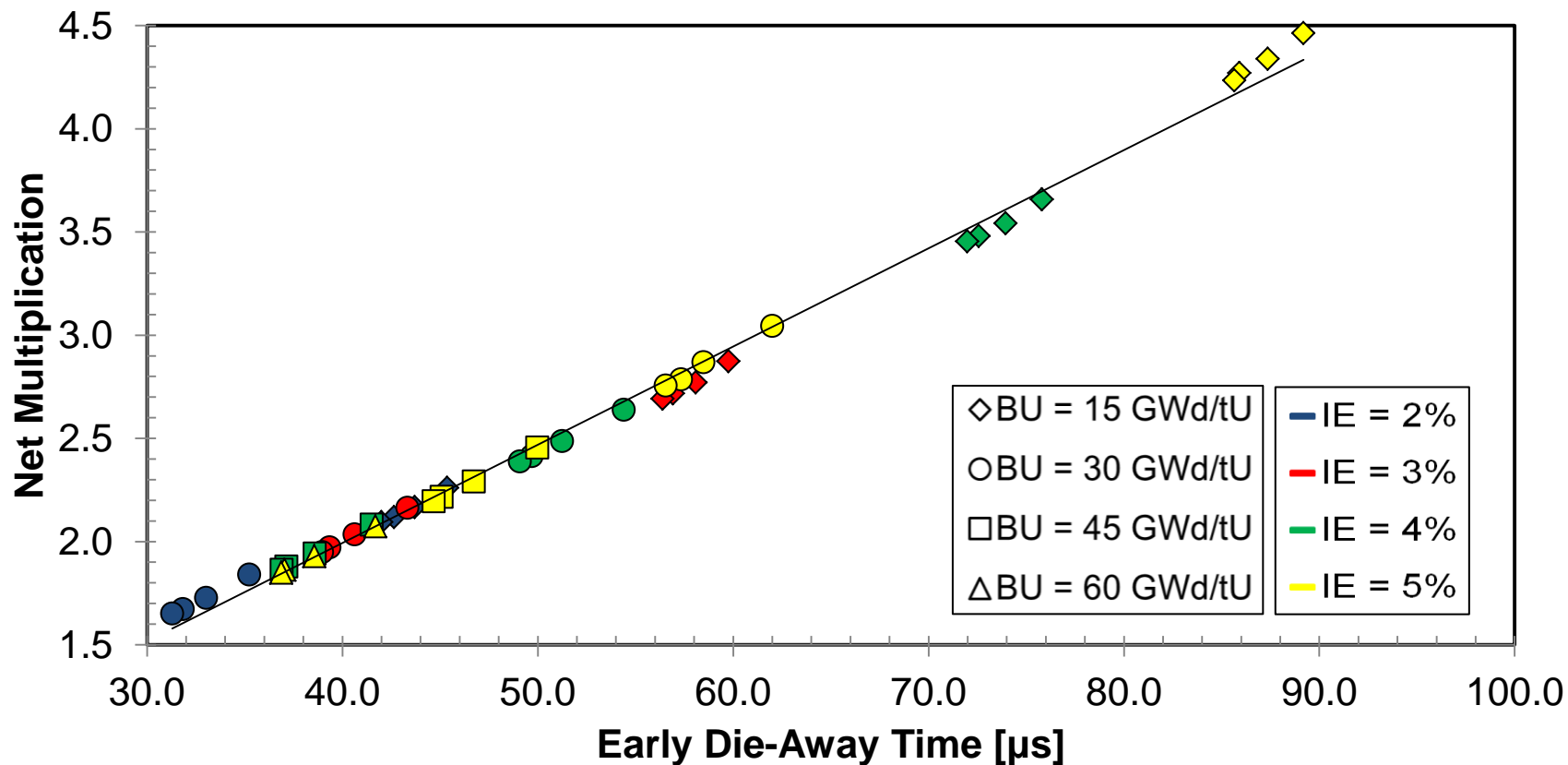
- Ratio of doubles count rate in a late gate over total counts (singles) has been used in the past, and is sensitive to  $(\alpha, n)$  neutrons
- We found that a single exponential fit to the early time-domain of the RAD correlates very well with multiplication

### Early die-away time domain



- Insensitive to neutrons from other sources besides spontaneous fission
- Improvement upon previously used method of Doubles/Singles to determine multiplication

# FY13: Determining Multiplication



## FY13: Determining Multiplication

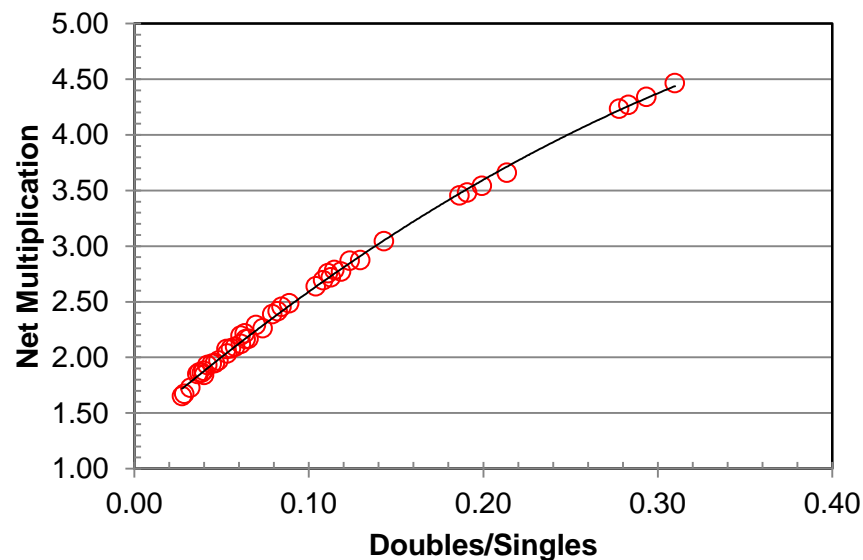
- The worst case for ( $\alpha,n$ ) influence is the 15 GWd/tU, 5% IE, 80 year cooled assembly with the highest alpha ratio of all SFL2a assemblies considered
- Alpha ratio =  $\frac{(\alpha,n)neutrons}{SF\ neutrons}$

	SF Source Only	( $\alpha,n$ ) Source Only	Combined Sources
$T_{early} [\mu s]$	86	86	86
D/S	0.28	0.18	0.21

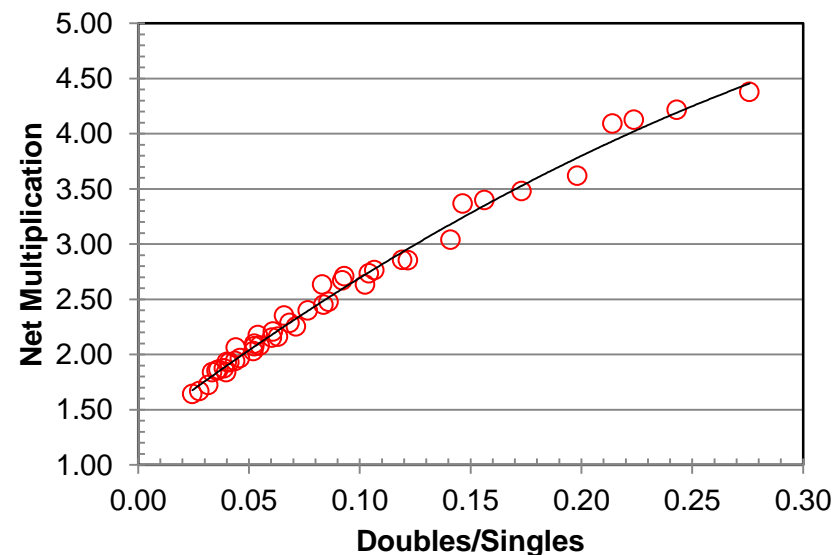
# FY13: Determining Multiplication

- D/S changes with SF or ( $\alpha,n$ ) source

SF Source



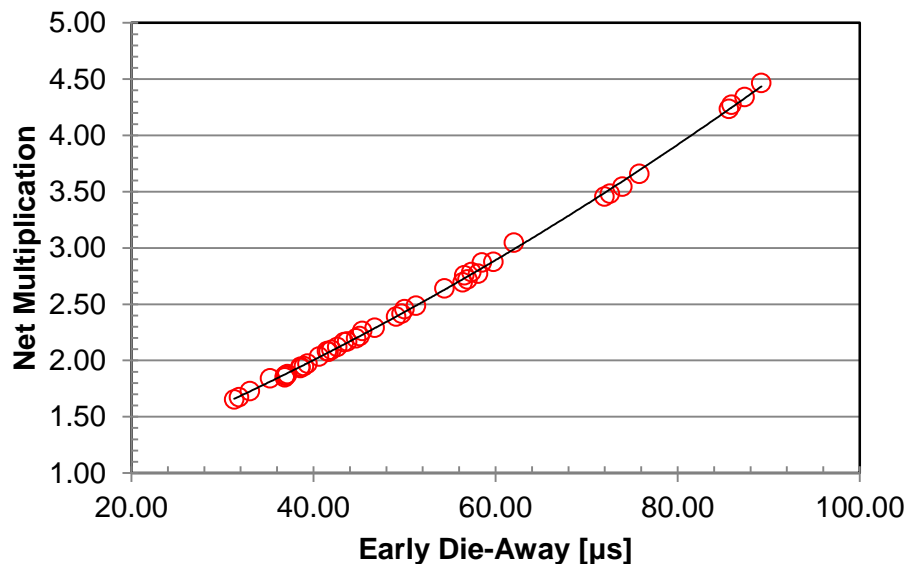
SF and ( $\alpha,n$ ) Sources



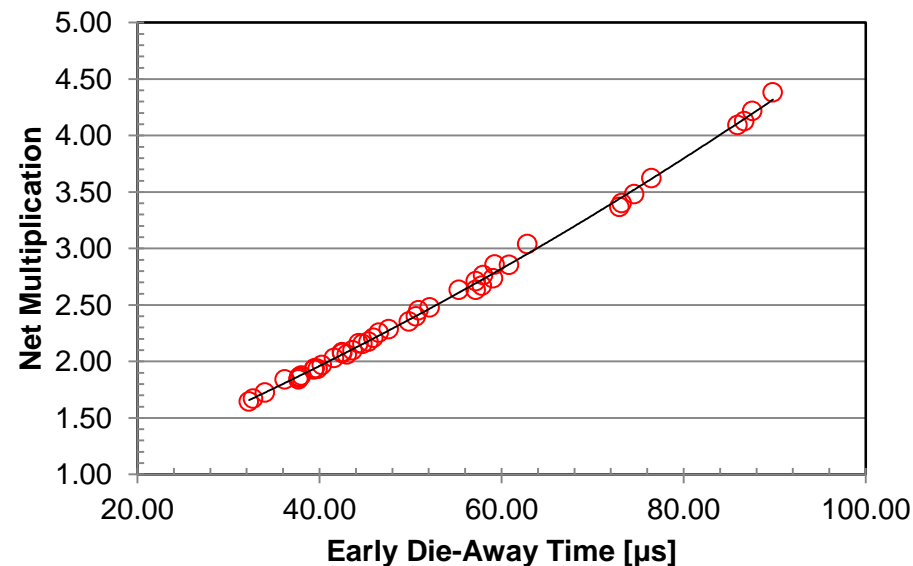
# FY13: Determining Multiplication

- Early die-away **does not change** with SF or ( $\alpha$ ,n) source

SF Source Only



SF and ( $\alpha$ ,n) Sources



## FY14: Total Pu Mass Determination

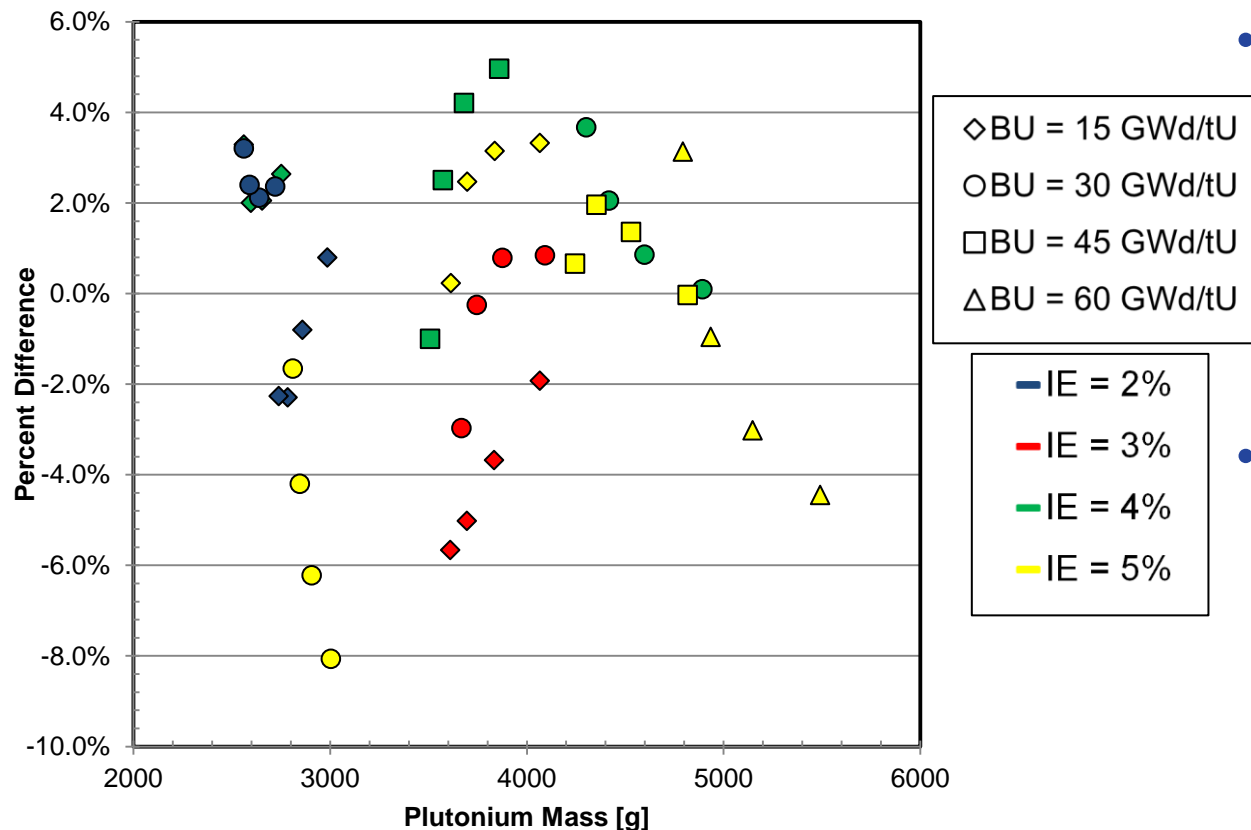
- Building upon DDA work on the same topic, we apply the same physics but utilize the early die-away time instead of multiplication, eliminating two calibration constants

- $$M_{pu} = c(CT) \frac{\tau_{early} + 2e}{\tau_{early} + e} \left( \frac{PN}{\tau_{early} + 2e} \right)^{d(CT)}$$

- Compute calibration constants c, d, and e
- Determine total Pu mass with mean variation of 2.5%



# FY14: Total Pu Mass Determination



- For some IE/BU combinations, longer CT are more accurate. Others, short CT are more accurate.
- When Multiplication is used instead of early die-away, we find the same calibration constant as DDA

## Upcoming work

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- Simulate assemblies to be measured at CLAB
- Carry out experiments to validate simulations
- Work toward initial enrichment and burnup determination
- Partial defect detection with SFL6
- Simulate SFL 3 and 4 to verify multiplication and total Pu correlations for more unusual fuel assemblies
- Utilize isotope-specific pulse trains from homogenized assemblies to further develop discrimination ratio

# Acknowledgements

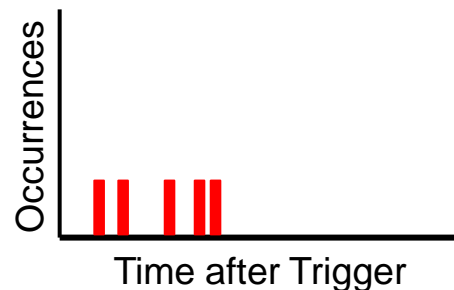
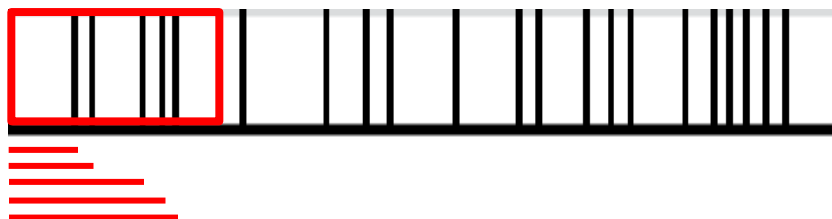
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- This material is based upon work supported by the U.S. Department of Homeland Security under Grant Award Number, 2012-DN-130-NF0001-02. The views and conclusions contained in this document are those of the authors and should not be interpreted as necessarily representing the official policies, either expressed or implied, of the U.S. Department of Homeland Security.*

## Bonus: Rossi-Alpha Distributions (RAD)

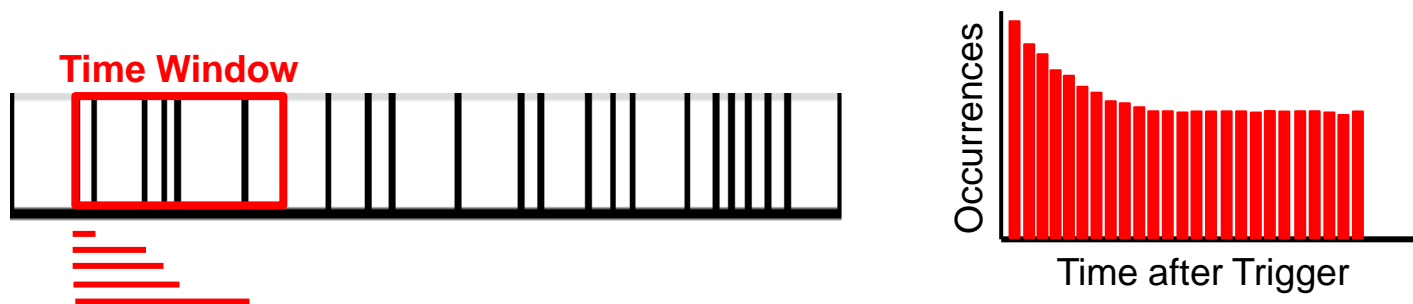
- By measuring the time of detection of neutrons following each trigger, a histogram is created that reflects correlations in the system

Time Window



## Bonus: Rossi-Alpha Distributions (RAD)

- By measuring the time of detection of neutrons following each trigger, a histogram is created that reflects correlations in the system

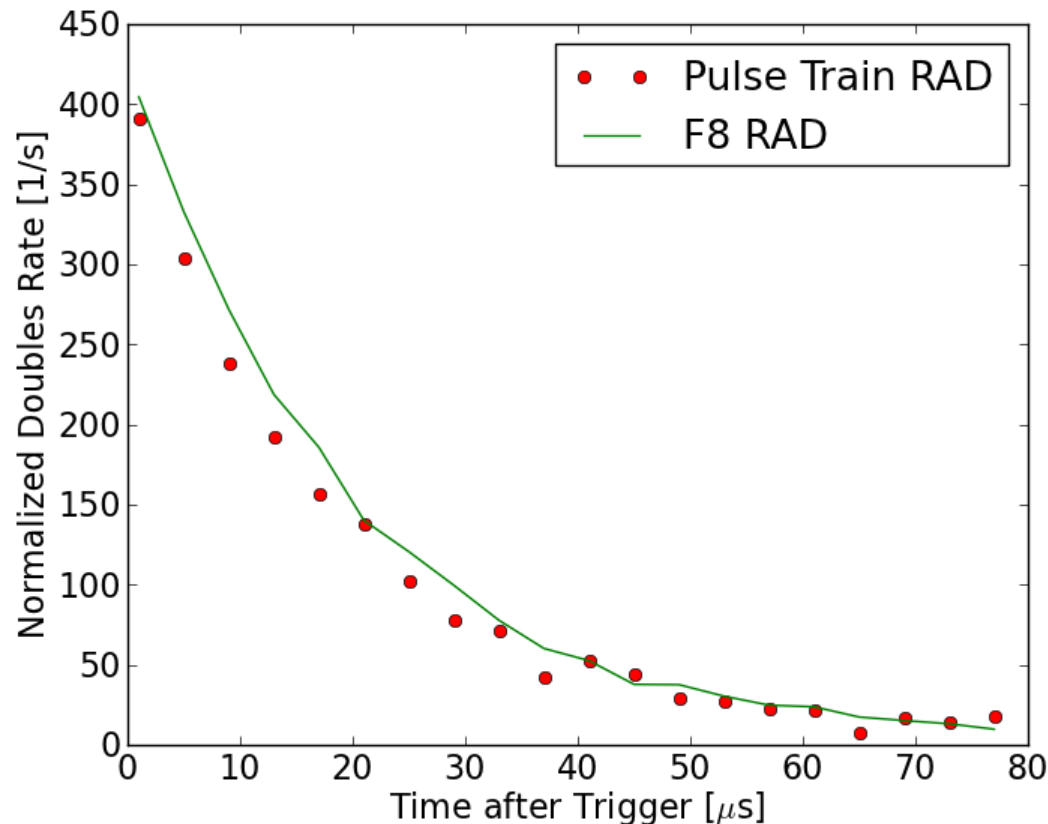


- This process repeats until adequate statistics are obtained
- If the arrival of the pulses is random, a flat distribution is created, but if correlations exist, the familiar shape of a RAD is produced

## Bonus: Simulating Rossi-Alpha Distributions

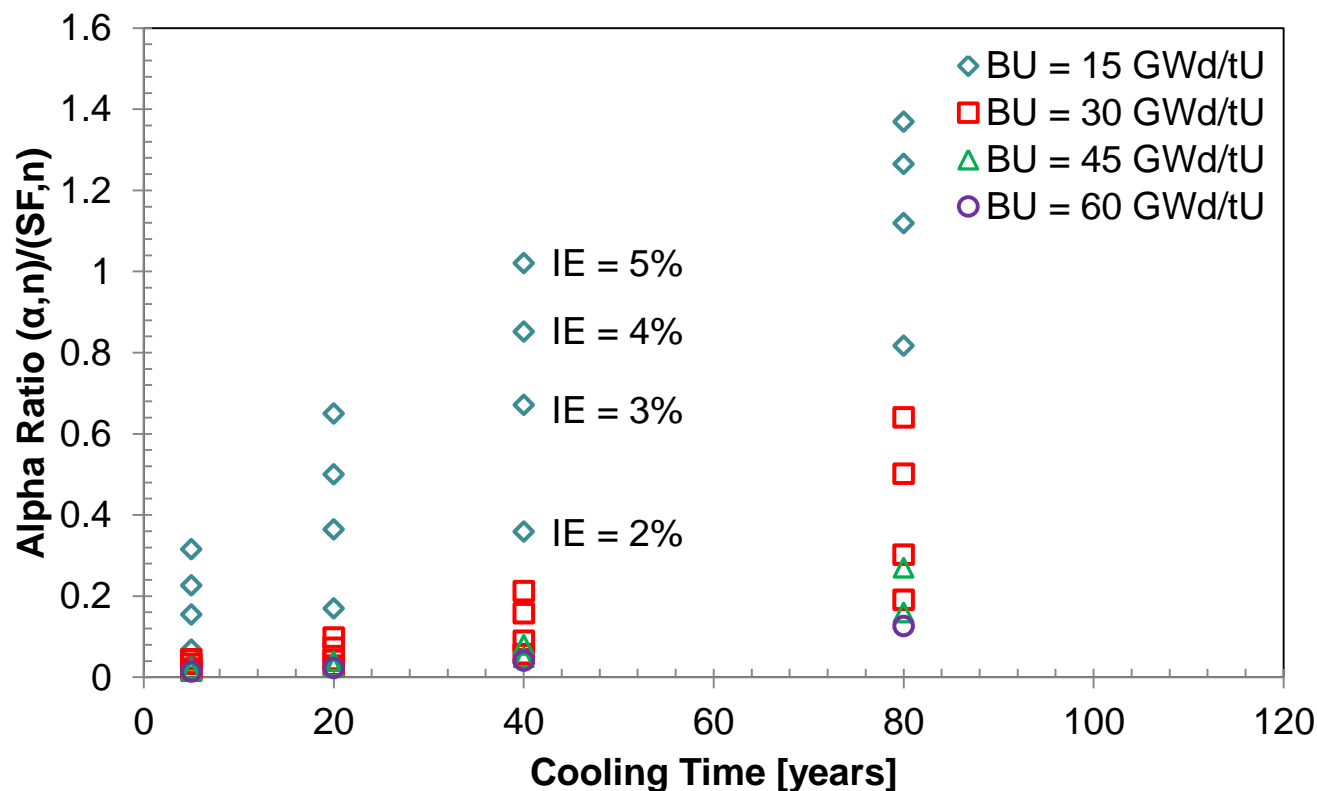
- Rossi-alpha distribution (RAD) is a histogram of times of coincident neutron captures following a trigger neutron capture
- In real life, this is filled with accidentals
- Simulate pulse train with PTRAC or PoliMi to make a RAD with accidentals, dead time, etc...
- Simulate RAD with MCNPX tally of structured, sequential “coincidence” gates that count neutron arrivals at certain times after trigger neutron. One history at a time = no accidentals

## Bonus: Simulating Rossi-Alpha Distributions



- Agreement point-by-point is not excellent, die-away agreement is better
- 19.4  $\mu$ s for F8 tally vs. 19.0  $\mu$ s for PoliMi pulse train

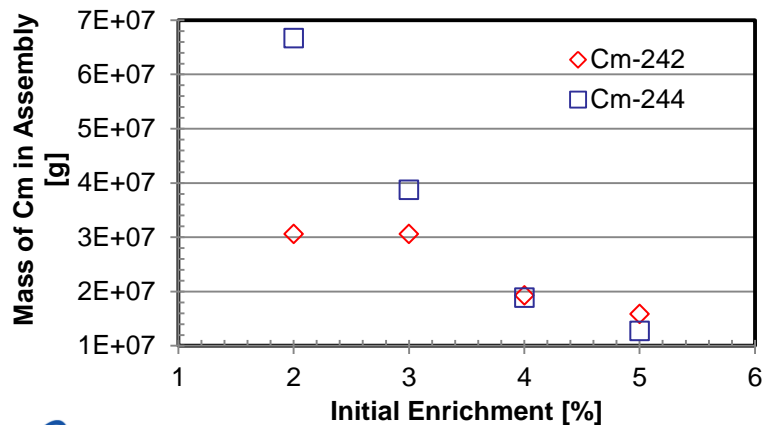
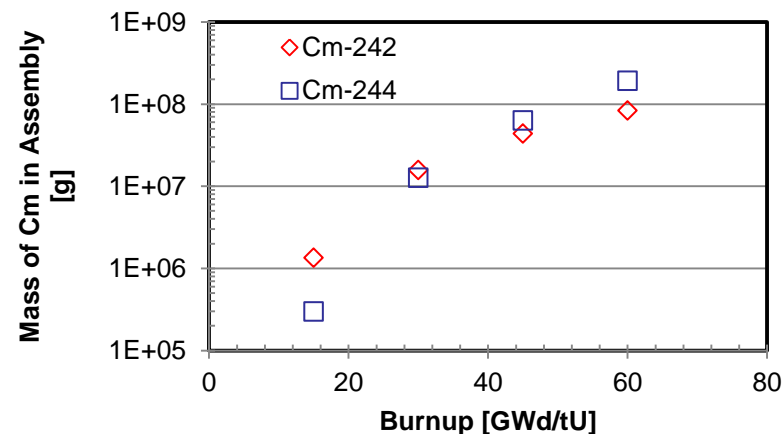
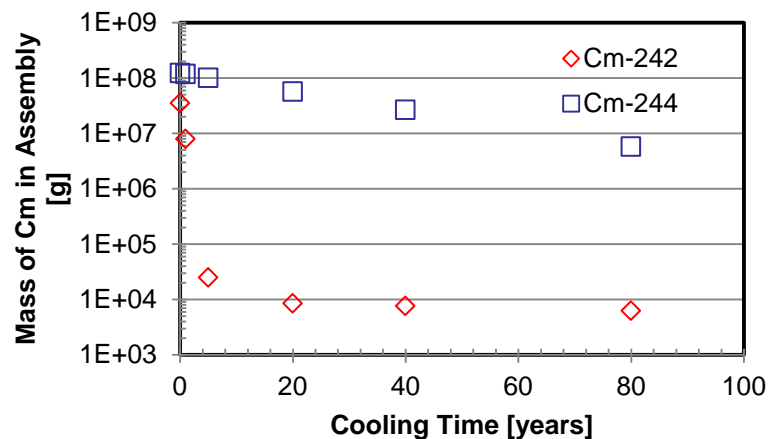
## Bonus: Alpha Ratio for SFL-2a



- Highest alpha ratio observed in cases with long CT, low BU, and high IE
- A.K.A. unusual cases

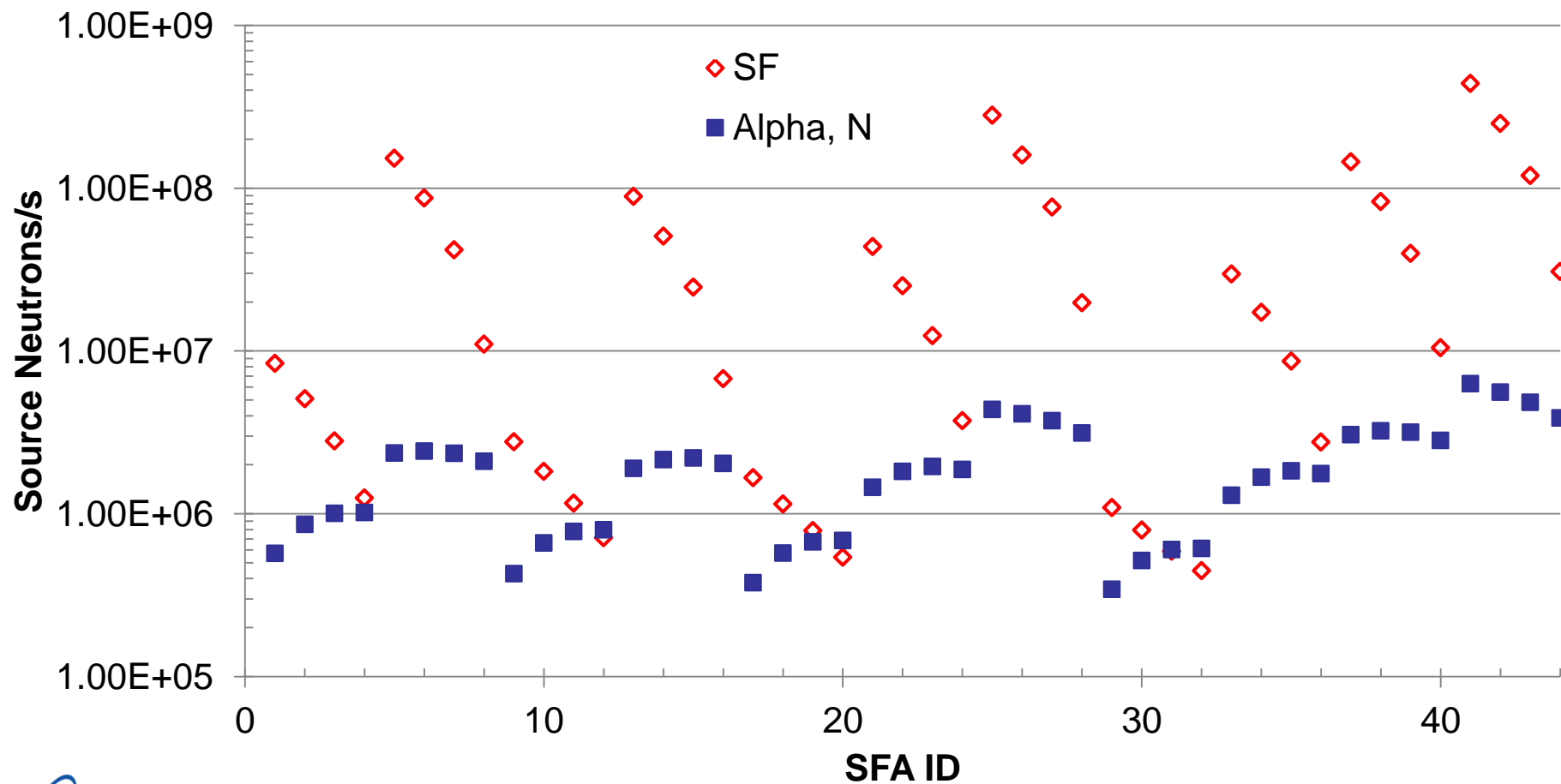


# Bonus: Alpha Ratio for SFL-2a

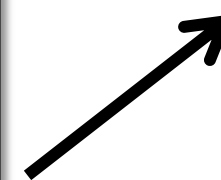
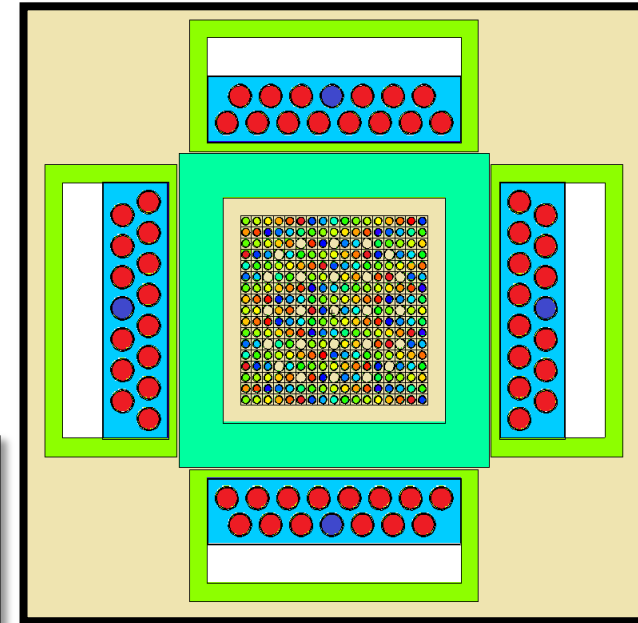
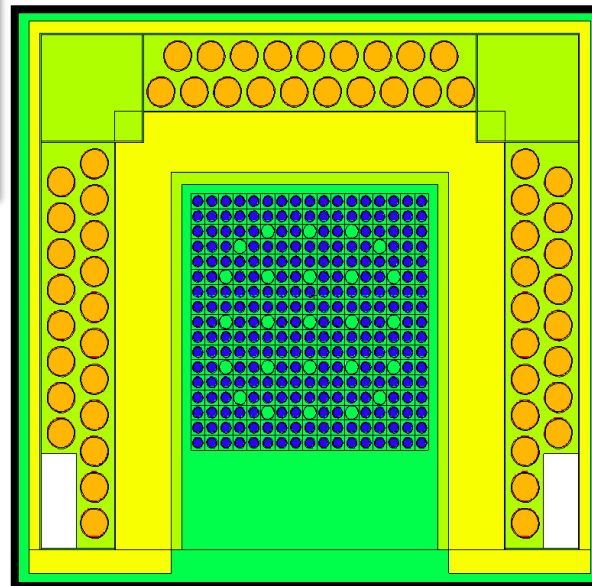
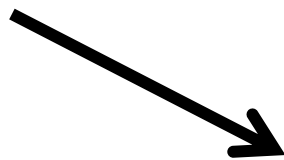
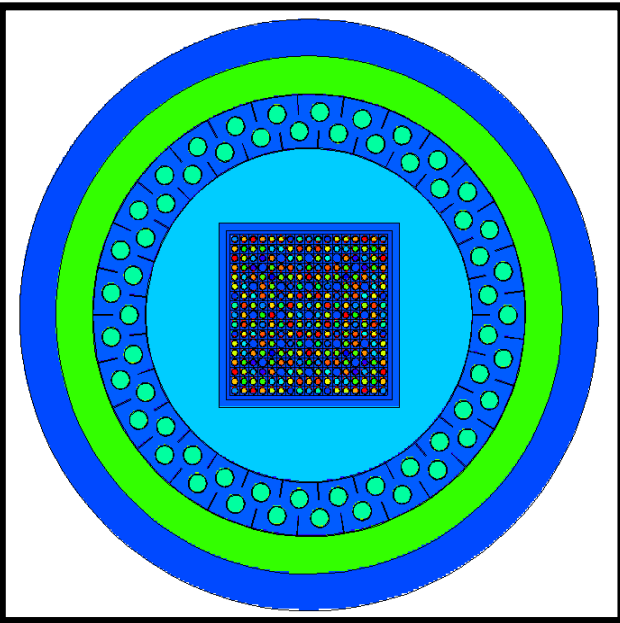


- LEAST Cm in cases with long CT, low BU, and high IE (same unusual cases)

## Bonus: Alpha Ratio for SFL-2a



# Bonus: Evolution of Design



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# Bonus: PTRAC Example Output

History	Time [shakes]	Cell	Source	SF	1 <sup>st</sup> fission	2 <sup>nd</sup> fission	3 <sup>rd</sup> fission	Last fission
6	7.25954E+04	1045	3	96244	94239	92235	94239	92238
6	1.40081E+04	1075	3	96244	94239	0	0	94239
33	5.50695E+02	1025	1	96244	0	0	0	96244
73	2.04089E+03	1075	1	96244	94239	0	0	94239
89	1.87530E+04	1045	2	96244	94239	92235	94241	92235
141	6.46086E+03	1075	1	96244	94241	94239	92235	92235
200	3.10552E+03	1005	2	96244	94239	94239	0	94239
239	1.00913E+03	1025	3	96244	0	0	0	96244
244	1.28950E+03	1045	2	96244	94239	0	0	94239
289	3.89959E+02	1025	3	96244	0	0	0	96244
334	2.21057E+04	1045	1	96244	94239	94239	94239	92235
341	5.21562E+04	1005	3	96244	94239	94239	0	94239
385	9.81907E+02	1075	1	96244	0	0	0	96244
396	1.21175E+04	1075	3	96244	92235	92235	94239	94239
396	2.67270E+04	1045	3	96244	92235	92235	92235	92235
396	1.09711E+03	1025	1	96244	94241	0	0	94241
396	8.16233E+02	1025	1	96244	94241	0	0	94241