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Title: Correlated Gammas from MCNPX using CGM

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Correlated Gammas from MCNPX using CGM

T. Wilcox, G. W. McKinney, T. Kawano, J. S. Hendricks, M. L. Fensin

ABSTRACT

Prior to the integration of the Cascading Gamma-ray and Multiplicity (CGM) code, the Monte Carlo Transport code MCNPX lacked the ability to produce correlated secondary particle/gamma emissions. CGM 2.0 was previously linked to MCNPX 2.7.C as a patch file to provide the capability to generate correlated secondary gammas. The previous linking strategy sent excitation energies, from MCNPX to CGM 2.0, to compute correlated secondary emissions for non-fission reactions based only on the lump sum of secondary neutron energies; furthermore, the previous linking strategy did not account for fission reactions. CGM has since been fully integrated into MCNPX 2.7.0 and now has a more comprehensive approach for computing excitation energy as well as a treatment for fission reactions. Excitation energy is now computed using both a lump sum of secondary neutron and charge particle energy, and fission reactions are treated by the LLNL library. This paper details a brief overview of the new integration methodology followed by several test cases highlighting the gamma spectrum and multiplicity distributions, beyond doubles and triples, computed from the CGM/MCNPX code.

Correlated Gammas From MCNPX using CGM

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Outline

- **CGM Code Overview**
- **CGM Integration into MCNPX**
- **Results**
- **Conclusions**
- **Future Work**

Overview - CGM Code

```

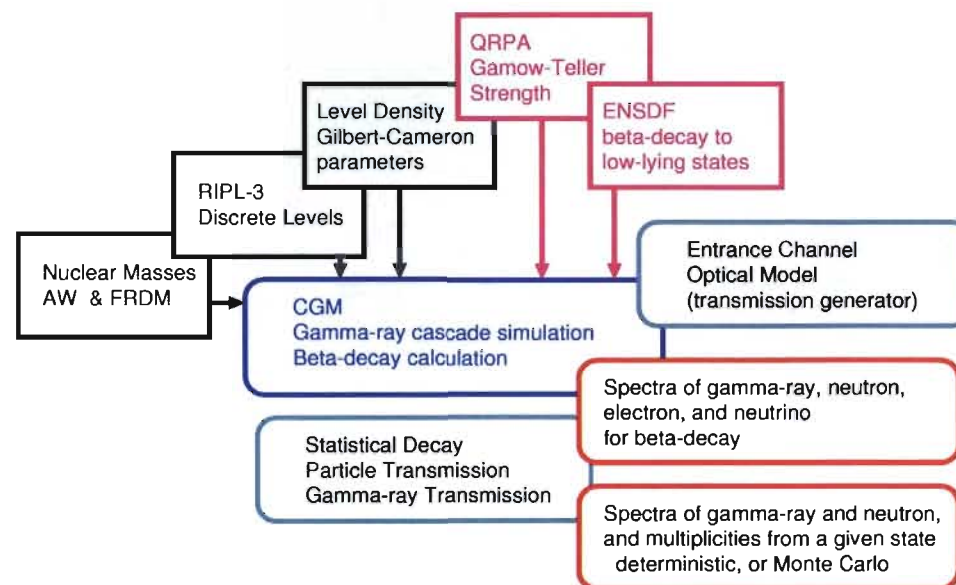
File Edit Options Buffers Tools C++ Help
/*****
/*  CGM
*****/

int main(int argc, char *argv[])
{
//-----
//  Command Line Options

double   exciE = 0.0; // nuclear excitation energy
double   initJ = -1.; // initial state spin, J
int       initP = 0;  // initial state parity. -1 or 1
int       targZ = 0;  // target Z number
int       targA = 0;  // target A number
int       isomr = 0;  // isomeric state for beta decay

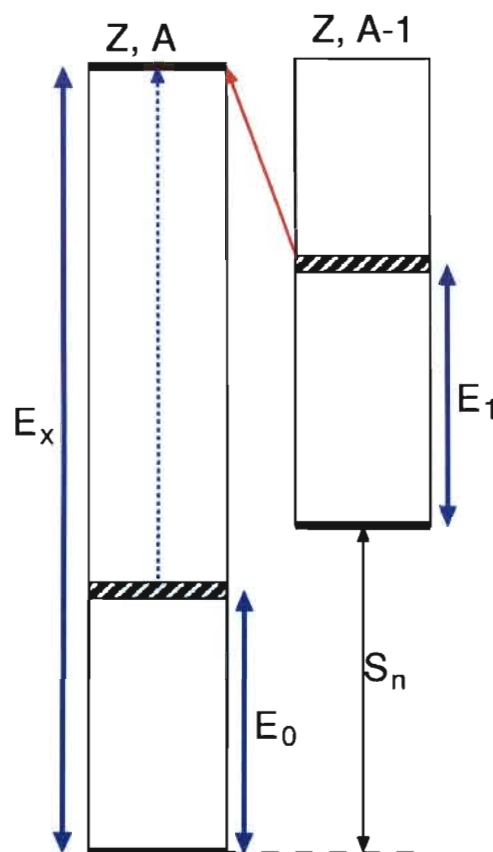
int       p;
unsigned int u=0xffff,c=0;
while((p=getopt(argc,argv,"Z:A:eI:j:p:n:abth"))!= -1){
    switch(p){
        case 'Z': targZ = atoi(optarg); break;
        case 'A': targA = atoi(optarg); break;
        case 'e': exciE = atof(optarg); break;
        case 'I': isomr = atoi(optarg); break;
        case 'j': initJ = atof(optarg); break;
        case 'p': initP = atoi(optarg); break;
        case 'o': o = atoi(optarg); break;
        case 'c': c = c | CALC_MC; break;
        case 'B': c = c | CALC_BETA; break;
        case 'T': c = c | CALC_TRAN; break;
        case '?': cgmHelp(); break;
        case ':': cerr << "ERROR: need a value for option" << p << endl;
        case '\0': break;
    }
}
}
-----
cgm.cpp      24% (82,49)  (C++/1 Abbrev)

```



- **CGM was derived from CoH3 code for**
 - Studying beta-delayed neutron and gamma emission
 - An event generator for the transport code MCNP
 - A Monte Carlo approach to the prompt fission neutron spectrum

Overview – Emission Probability



■ Gamma-ray emission

$$P(\varepsilon_\gamma)dE_0 = \frac{T_\gamma(E_x - E_0)\rho(Z, A, E_0)}{N}dE_0$$

■ Neutron emission

$$P(\varepsilon_n)dE_1 = \frac{T_n(E_x - S_n - E_1)\rho(Z, A-1, E_1)}{N}dE_1$$

where $T_{n,\gamma}$ are the transmission coefficients, $\rho(Z, A, E)$ is the level density, and the normalization N is given

$$N = \int_0^{E_x} T_\gamma(E_x - E_0)\rho(Z, A, E_0)dE_0 + \int_0^{E_x - S_n} T_n(E_x - S_n - E_1)\rho(Z, A-1, E_1)dE_1$$

- Integration performed only for spin and parity conserved states
- At low excitation energies, discrete level data is used (RIPL-3)

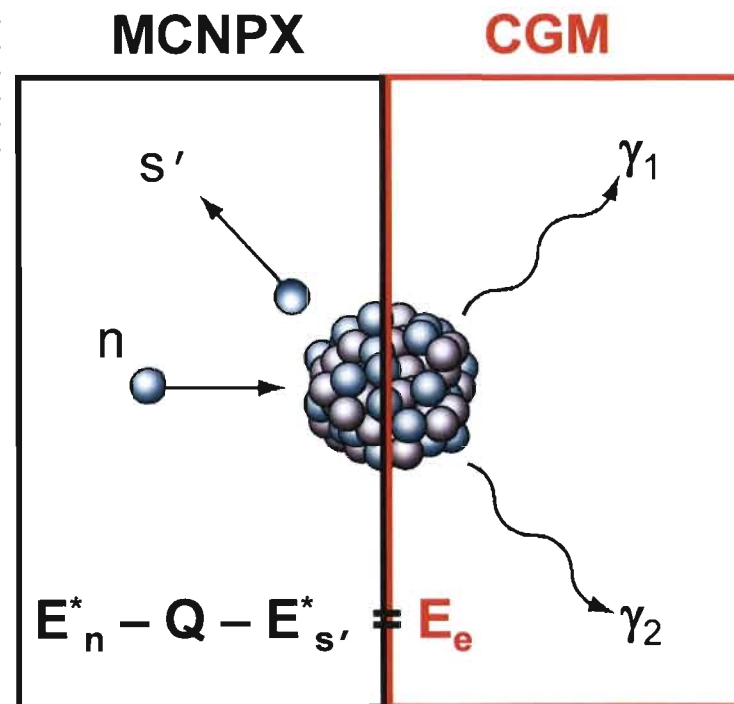
UNCLASSIFIED

Overview - Monte Carlo CGM Version 3.4

```
// $Id: cgm.cpp,v 1.8 2010/11/15 23:19:05 kawano Exp $
/*****
/**
/**  C G M  :  Cascading Gamma-ray and Multiplicity
/**                      Version 3.4      Sinope (2011)
/**                      T. Kawano
/**
/**  History
/**  1.0  2008 May   (Io)      Hauser-Feshbach for gamma spectra
/**  2.0  2008 Jul. (Europa)   Gamma cascade by Monte Carlo
/**  -    2008 Dec. (Ganymede) CoH3beta branch full Hauser-Feshbach code
/**  3.0  2009 Sep. (Amalthea) Backport version from CoH3-Callisto
/**  3.1  2010 Feb. (Himalia)  Beta decay enhanced
/**  3.2  2010 Apr. (Elara)    Monte Carlo recovery version
/**  3.3  2011 Jan. (Pasiphae) Multiple neutron emission version
/**  3.4  2011 Jun. (Sinope)   Monte Carlo with neutron emission version
/**
/**                      Los Alamos National Laboratory
/**
/**
/*****/
```

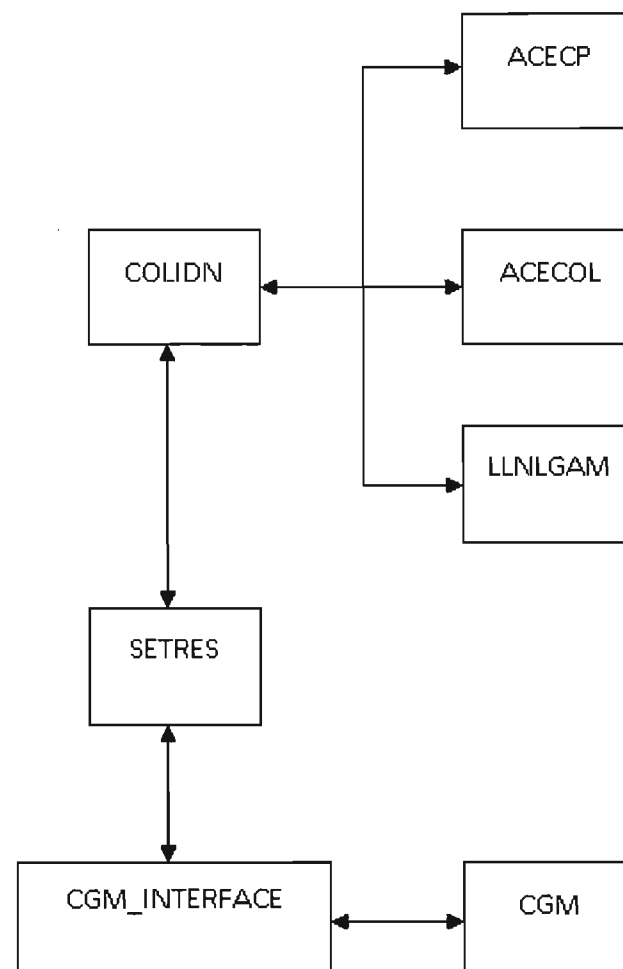
```
#ifndef CGM
int main(int argc, char *argv[])
#else
extern "C"
{
void cgm(int, int, float, float *);
}
void cgm(int nz, int na, float ee, float rr[9])
#endif
{
//-----
//      Command Line Options

double      exciE = 0.0; // nuclear excitation energy
double      initJ = -1.; // initial state spin, J
int          initP = 0;  // initial state parity, -1 or 1
int          targZ = 0;  // target Z number
int          targA = 0;  // target A number
int          isomr = 0;   // isomeric state for beta decay
unsigned long nsim = 0;   // number of Monte Carlo sampling
```



CGM Integration into MCNPX

- **COLIDN determines**
 - Charged secondary particles
 - Number of secondary neutrons
 - Fission vs. non-fission
- **ACECP**
 - Produces secondary charged particles
 - Energy of charged particles
- **ACECOL**
 - Calculates total energy of non-fission neutrons
- **LLNLGAM**
 - Uses LLNL library for fission reactions
- **SETRES**
 - Determines residual nucleus
- **CGM_INTERFACE**
 - Calculates residual excitation energy (E_e)
 - Passes E_e and residual ZA to CGM
 - Receives gamma energies from CGM



Results

■ Gamma Spectrum and Multiplicities

- 14 MeV neutrons on Mg-24
- Thermal neutrons on Ni-61
- 14 MeV neutrons on Ni-61
- Thermal neutrons on Ge-73
- 14 MeV neutrons on W-183
- Thermal neutrons on W-183

Results - Example MCNPX Input Deck

test CGM for 14 mev n plus 24Mg

1 1 -1.738 -1 imp:n,p,h,a,d=1

2 0 1 imp:n,p,h,a,d=0

1 so 1

mode n p h a d

phys:n 7j 2 \$ Use CGM

cut:n 2j 0 0 \$ Force analog capture

m1 12024.70c 1

lca 7j -2

sdef par=n erg=14

f1:p 1

e1 1e-3 999i 20.0

f21:p 1

e21 1e-3 99i 20.0

c

nps 1e7

New 8th entry on the PHYS:N card

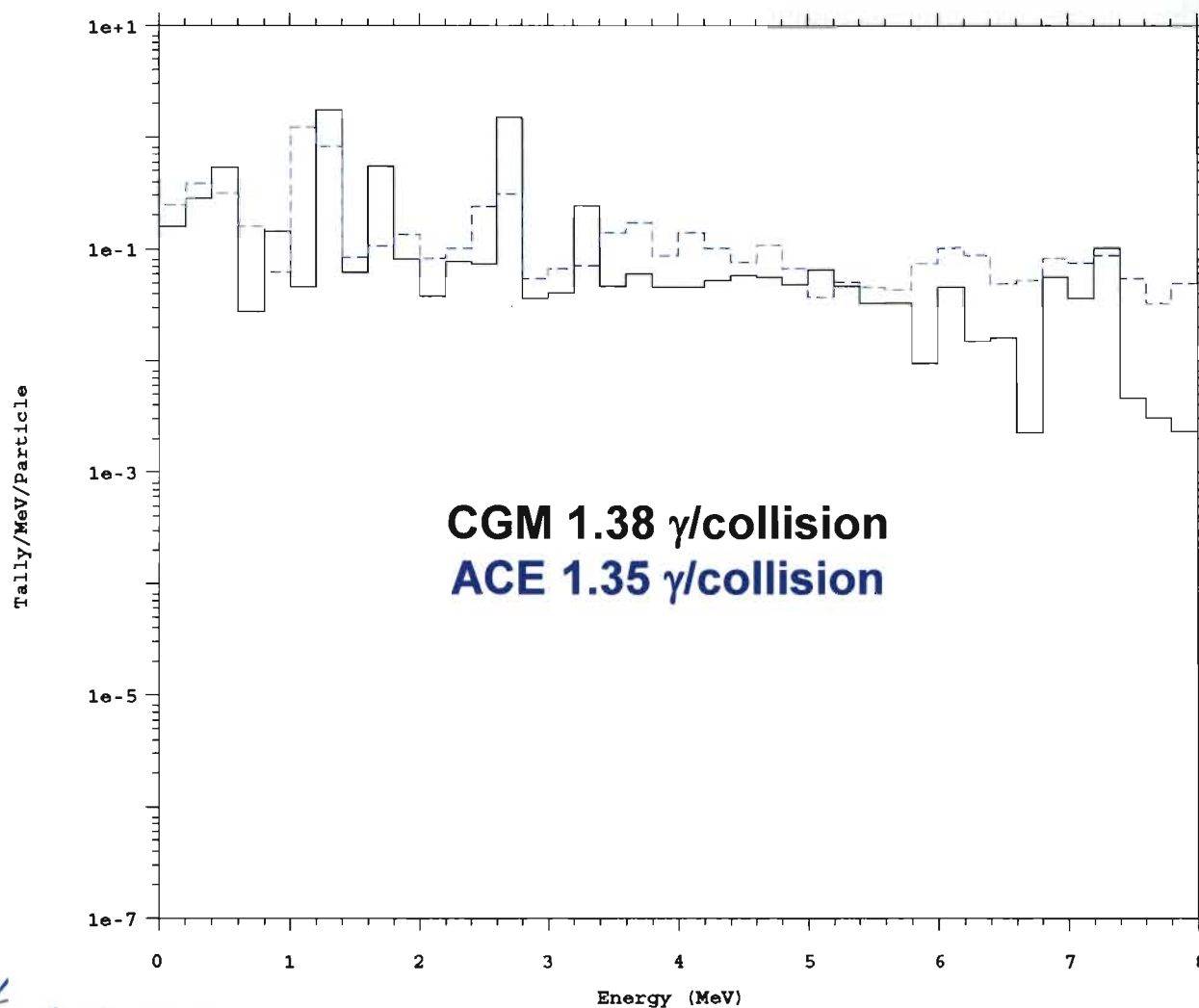
Neutron induced photon production
method (table data energy range)

= 0 : no gamma production from
neutrons;

= 1: ACE library gamma production
(DEFAULT);

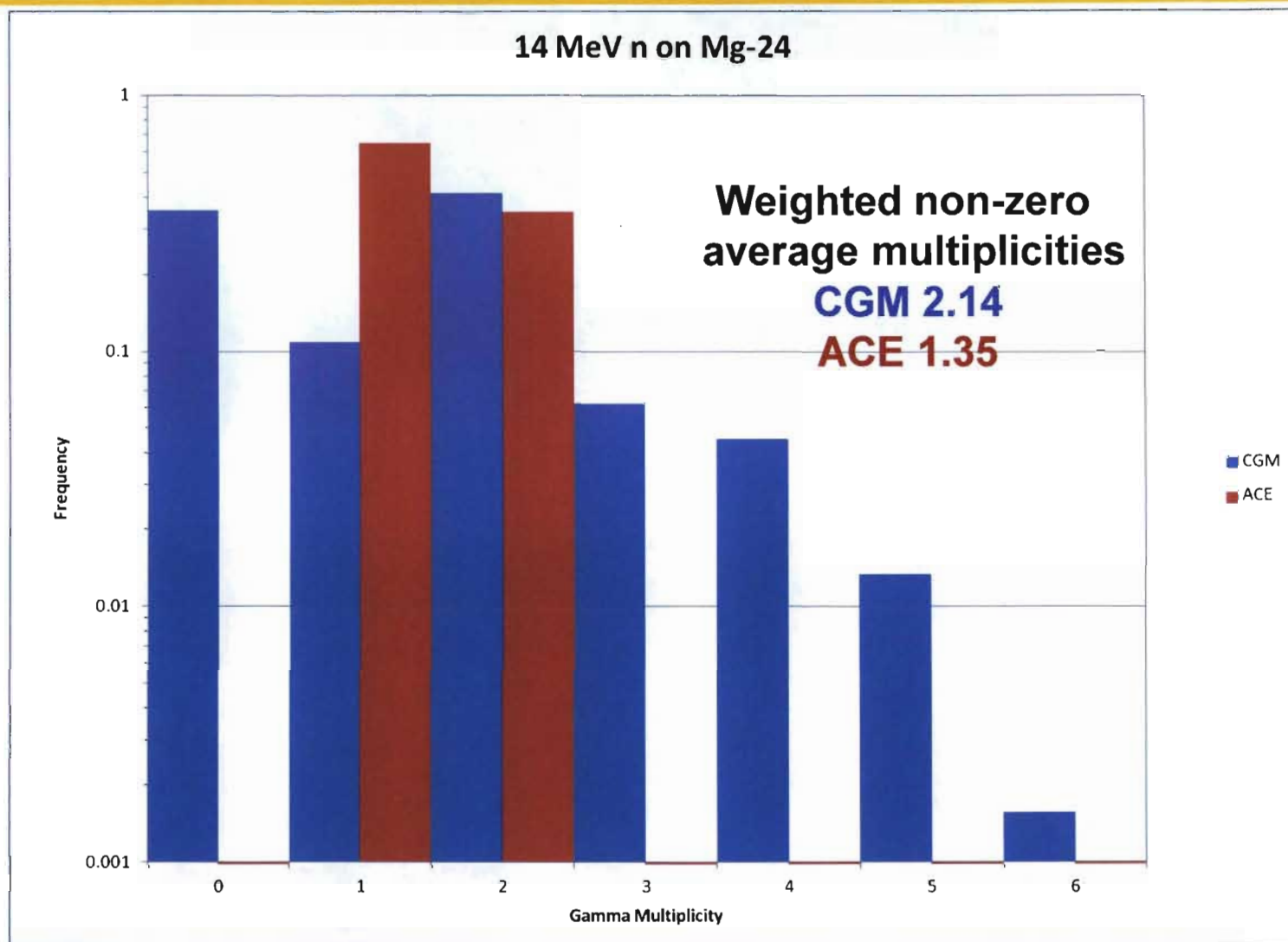
= 2 : CGM correlated gamma
production. This turns on LLNL fission
(5th entry on the PHYS:N card set to 5)

Results – 14 MeV Neutrons on Mg-24^{LA-UR}

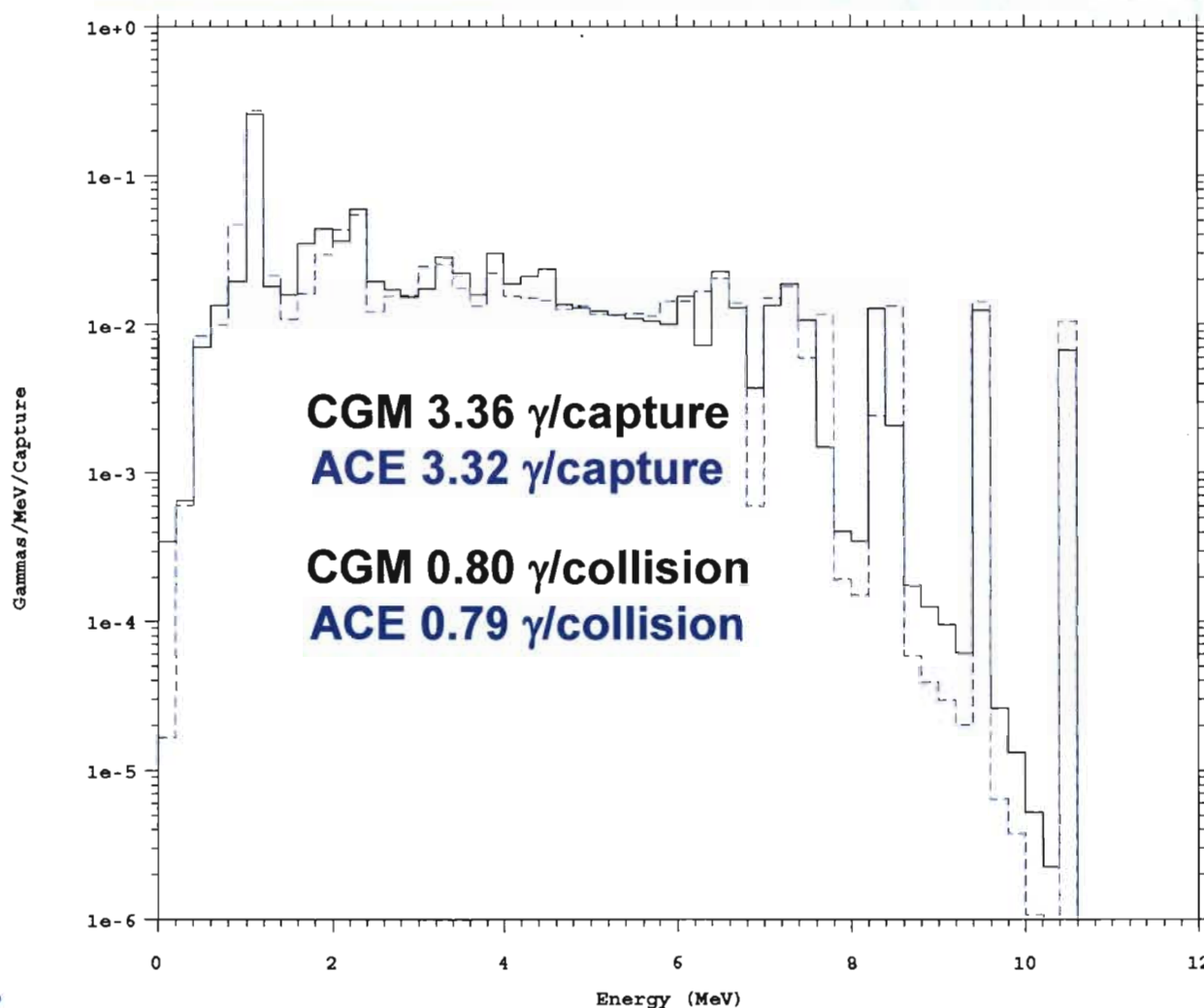


```
mcnpX      70CGM
probid: 09/09/11 08:57:08
tally      21
p
nps        10000000
bin normed
runtpe = n14_mg24_cgm.r
dump       2
f  Surface      1
d  Flag/Dir     1
u  User         1
s  Segment      1
m  Mult         1
c  Cosine       1
e  Energy       *
t  Time         1
_____ CGM
----- ACE
```

Results - 14 MeV Neutrons on Mg-24



Results – Thermal Neutrons on Ni-61

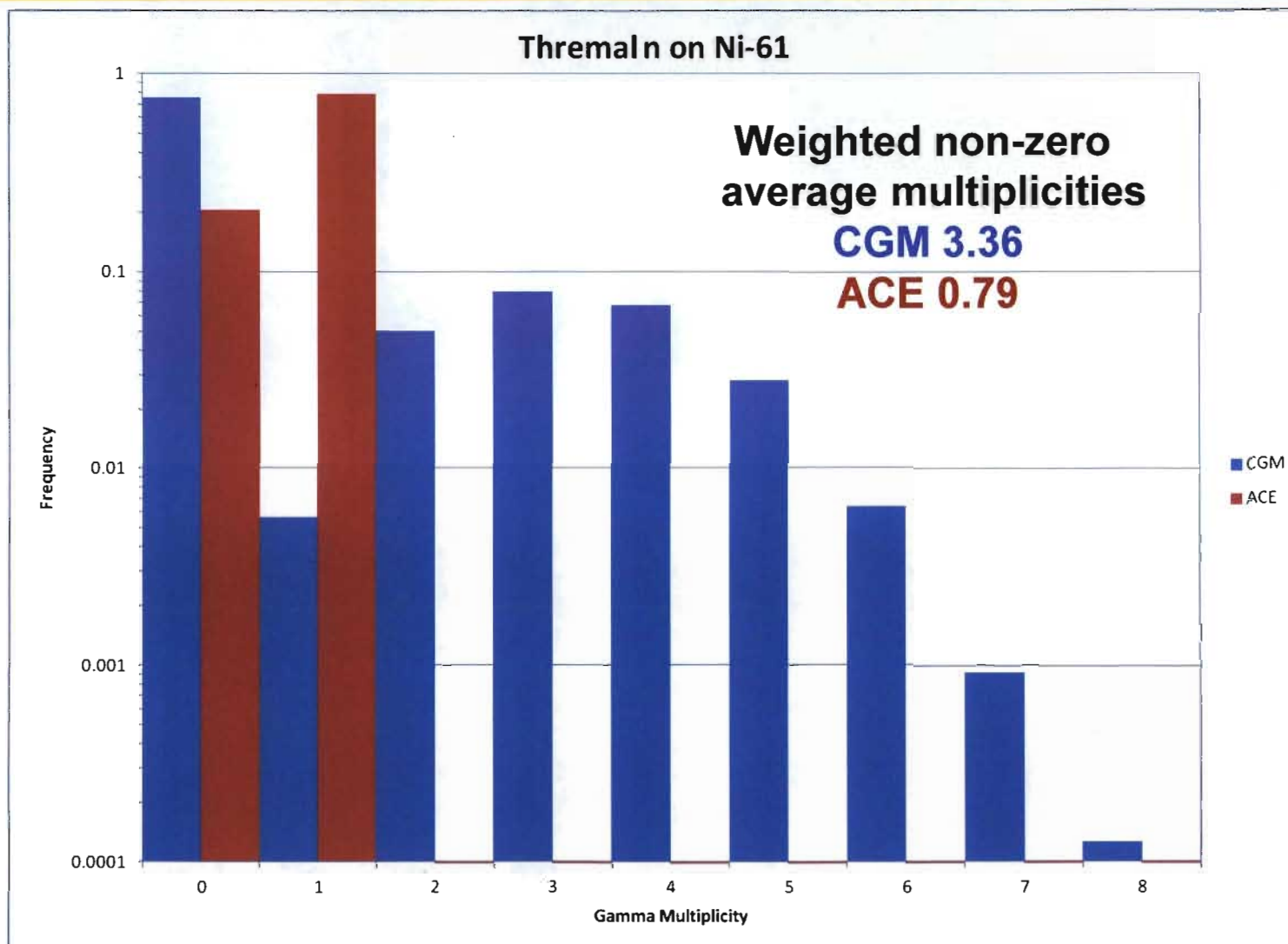


```
mcnpx      70CGM
probid: 10/02/11 09:13:20
tally      21
P
nps        10000000
bin normed
mctal = tn_61Ni_cgm.m

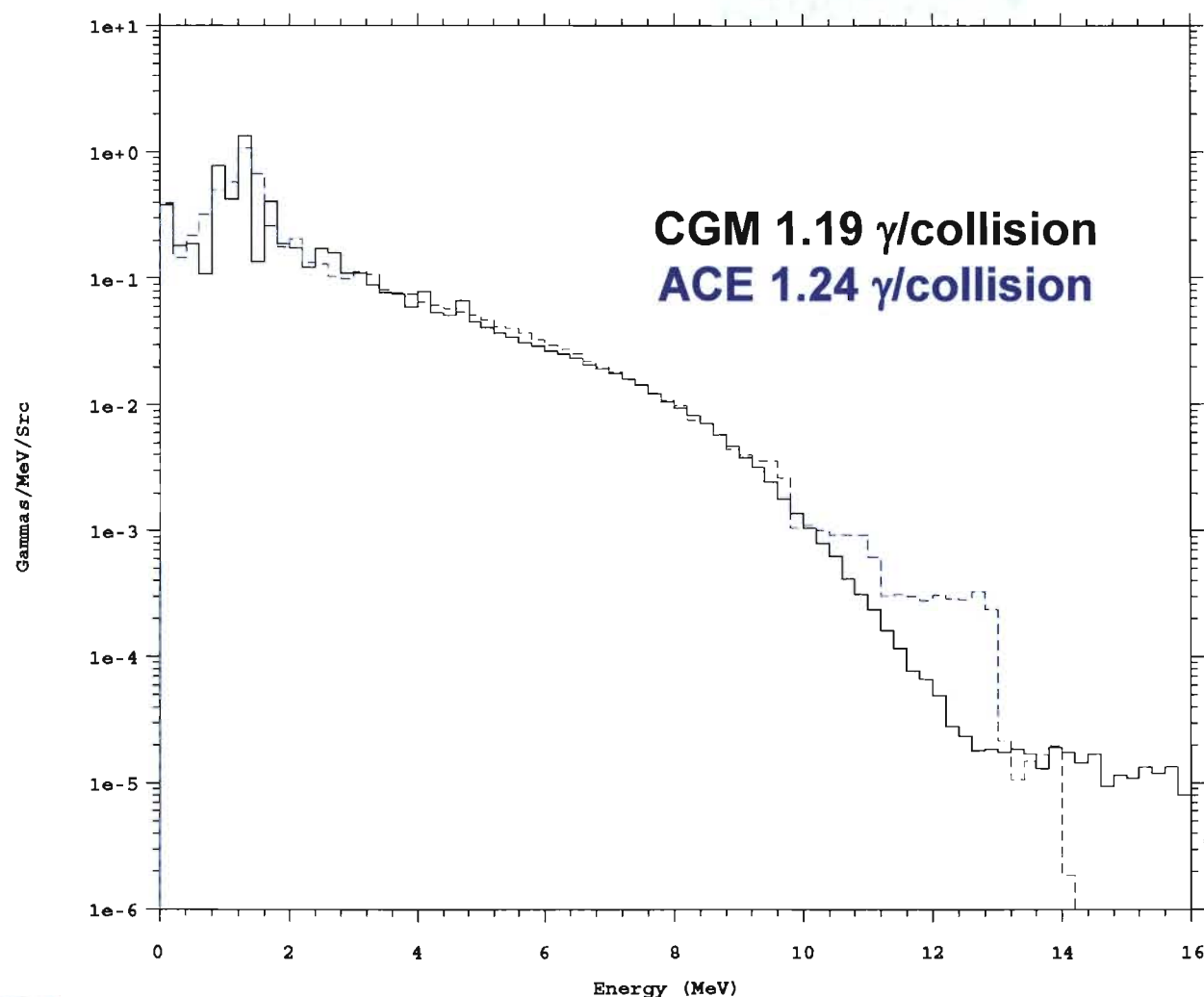
f  Surface      1
d  Flag/Dir     1
u  User         1
s  Segment      1
m  Mult         1
c  Cosine       1
e  Energy       *
t  Time         1

— CGM
- - - ACE
```

Results – Thermal Neutrons on Ni-61



Results – 14 MeV Neutrons on Ni-61

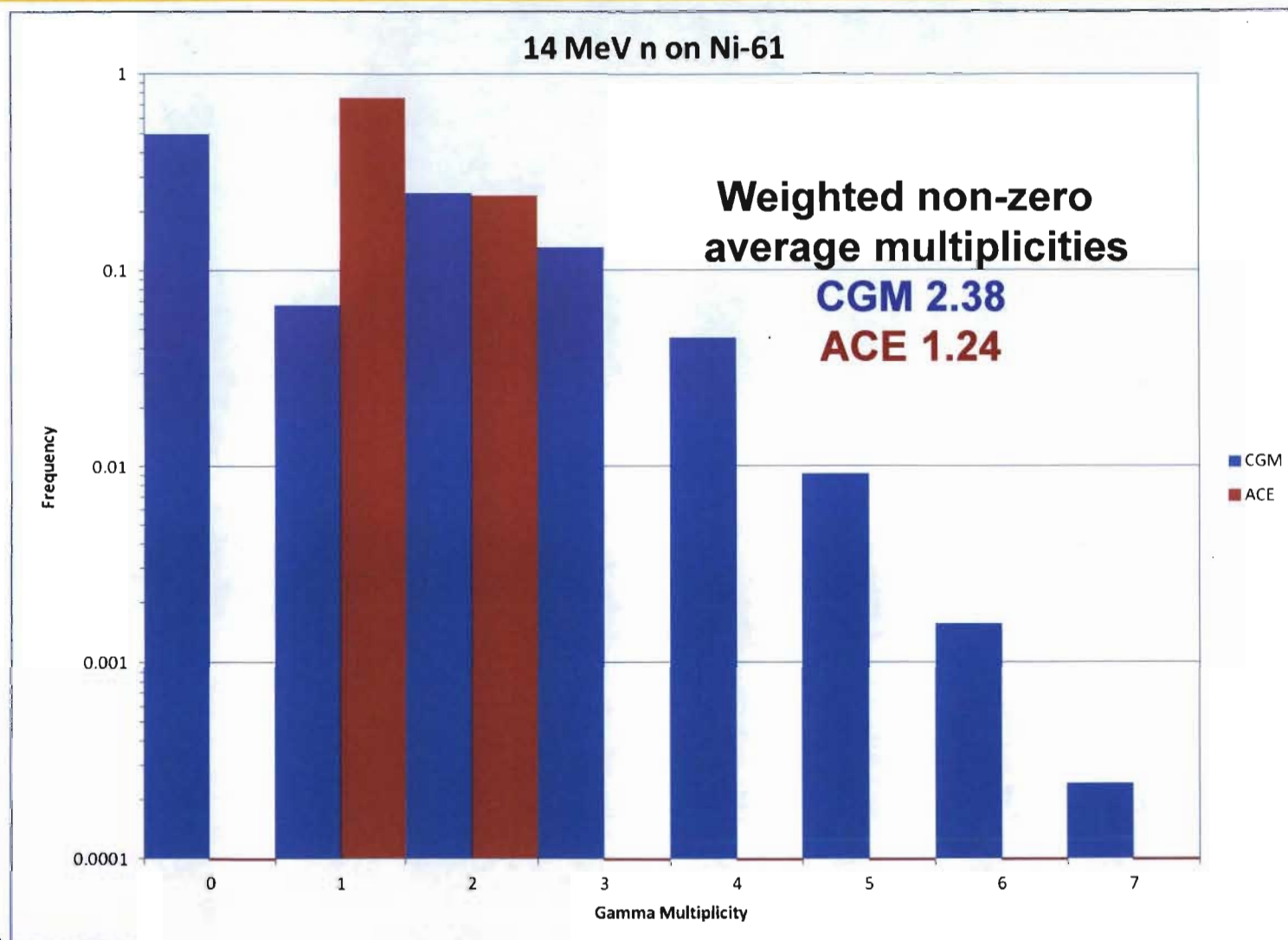


```
mcnpx          70CGM
probid: 10/01/11 07:17:34
tally      21
p
nps          10000000
bin normed
mctal = n14_61Ni_cgm.m

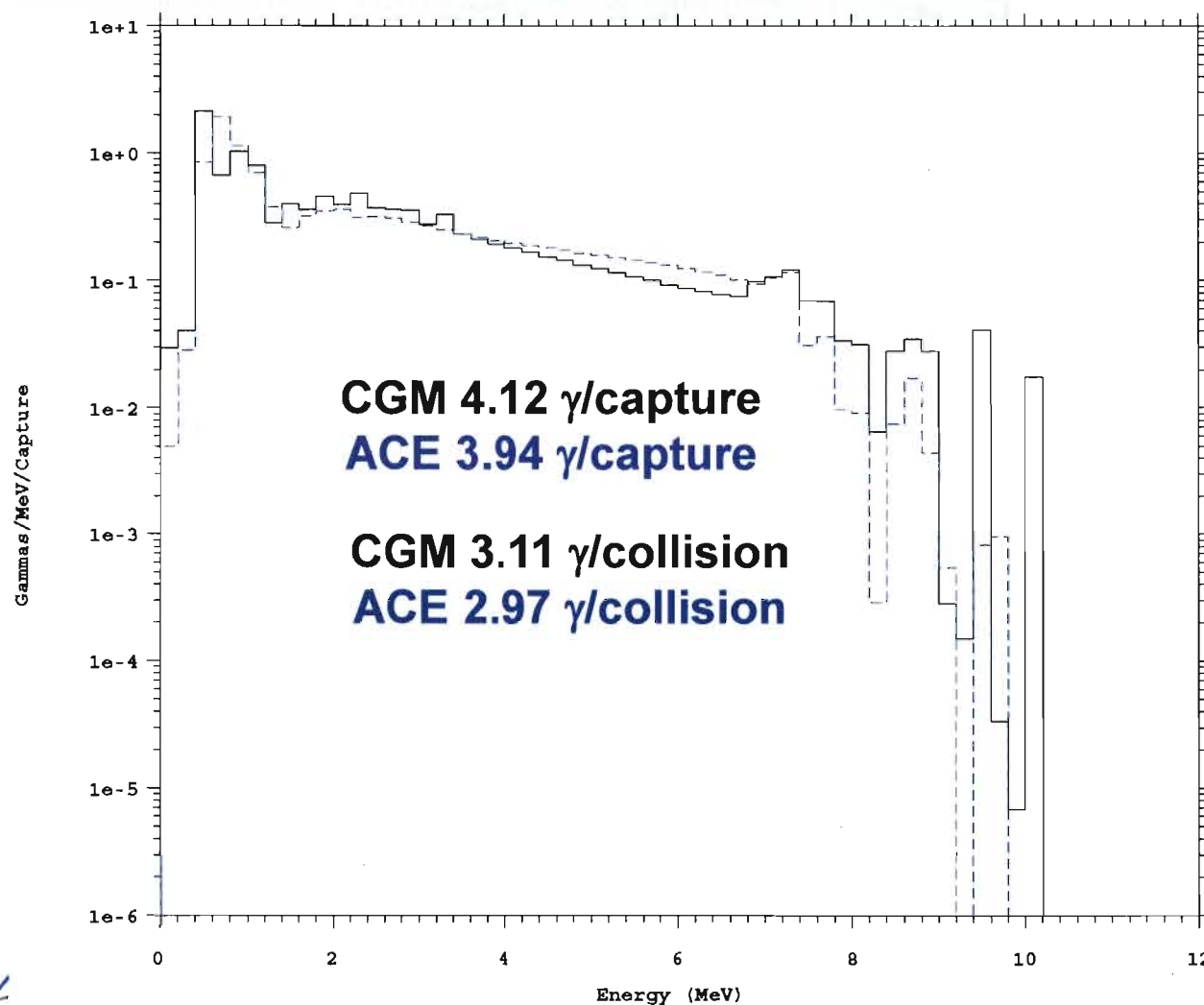
f  Surface      1
d  Flag/Dir     1
u  User         1
s  Segment     1
m  Mult         1
c  Cosine       1
e  Energy       *
t  Time         1

—— CGM
--- ACE
```


Results – 14 MeV Neutrons on Ni-61



Results – Thermal Neutrons on Ge-73

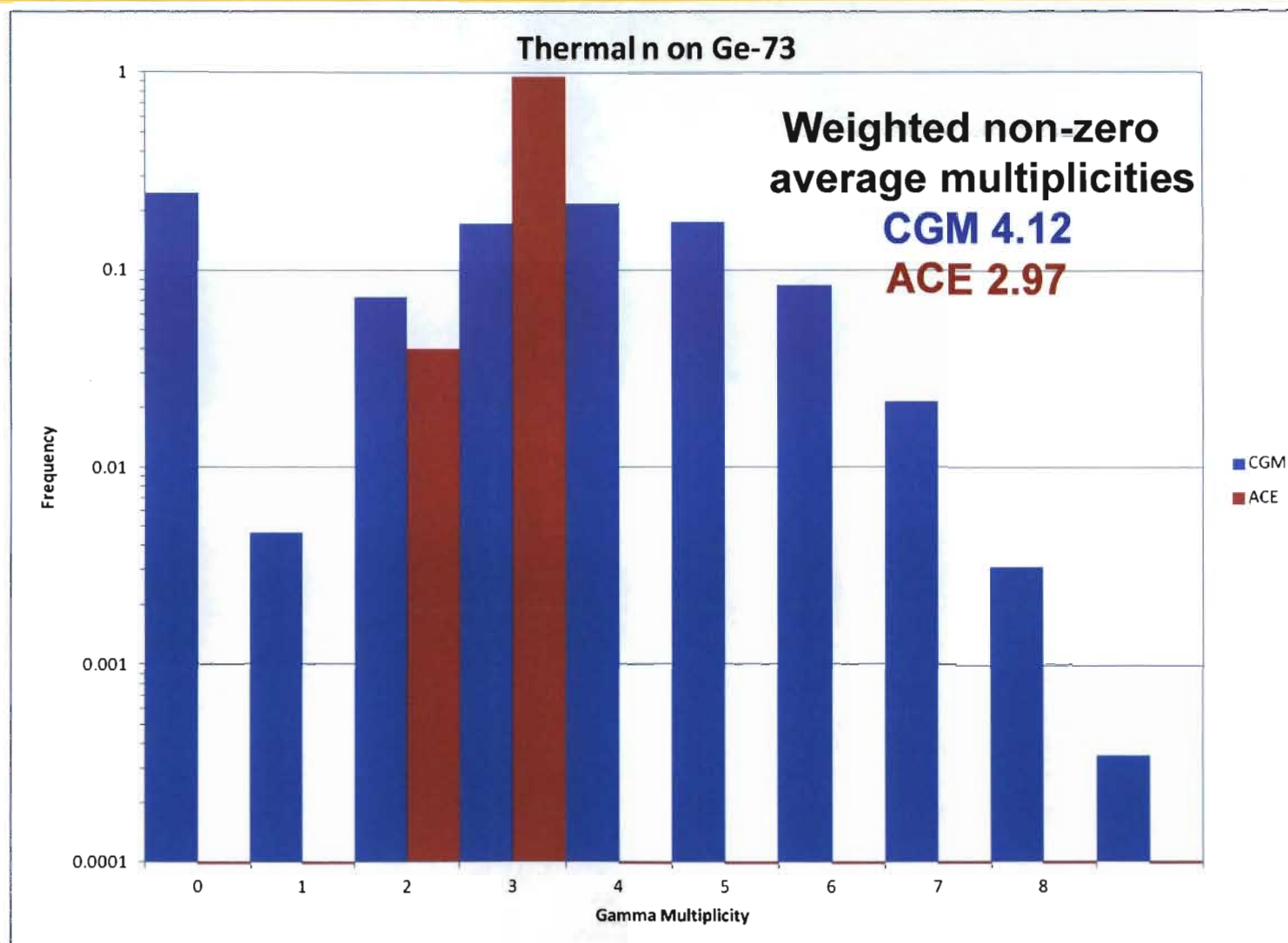


```
mcnpX      70CGM
probid: 10/02/11 13:06:13
tally      21
P
nps        100000000
bin normed
mctal = tn_73Ge_cgm.m

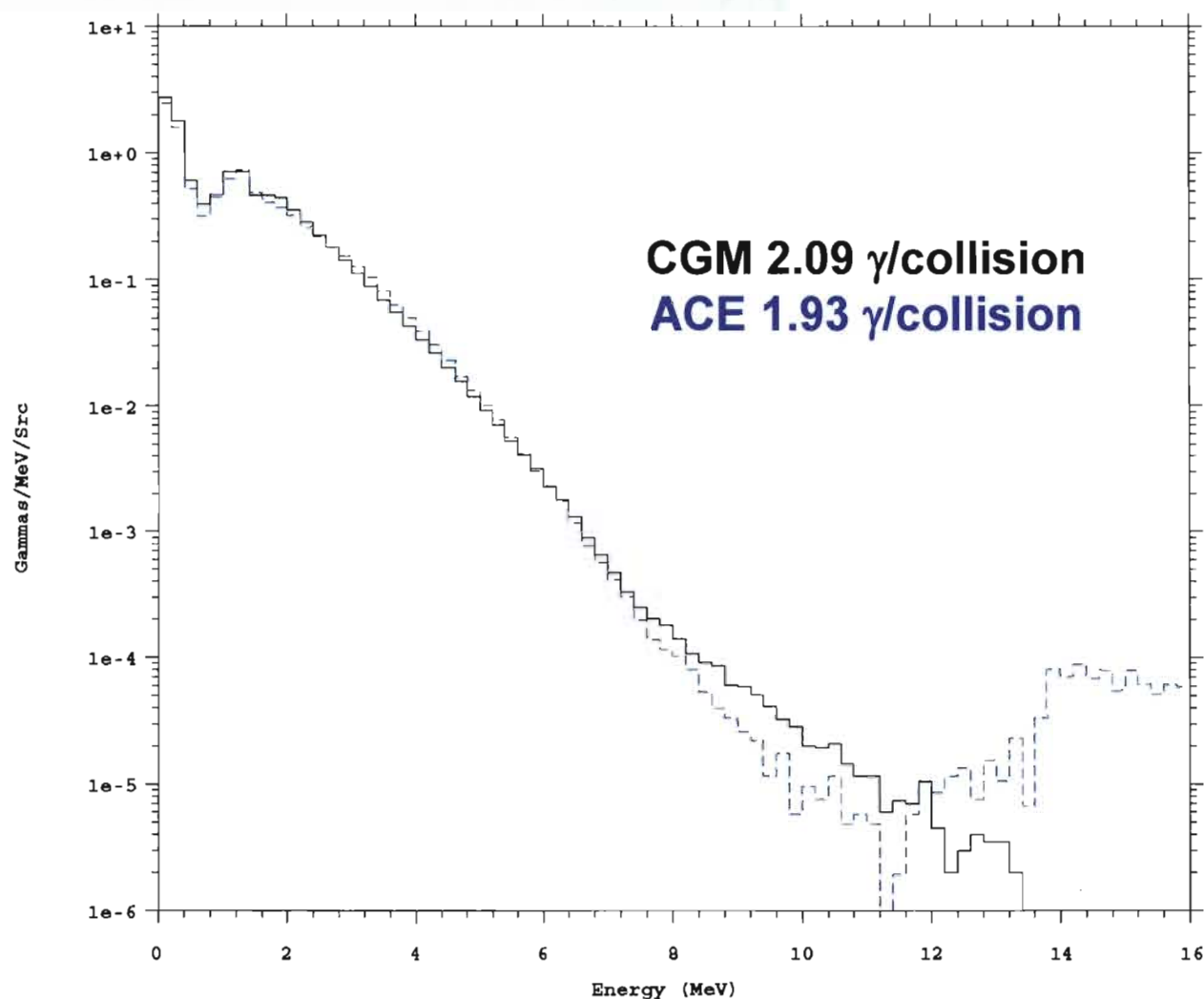
f  Surface      1
d  Flag/Dir     1
u  User         1
s  Segment     1
m  Mult         1
c  Cosine       1
e  Energy       *
t  Time         1

----- CGM
----- ACE
```

Results – Thermal Neutrons on Ge-73



Results – 14 MeV Neutrons on W-183



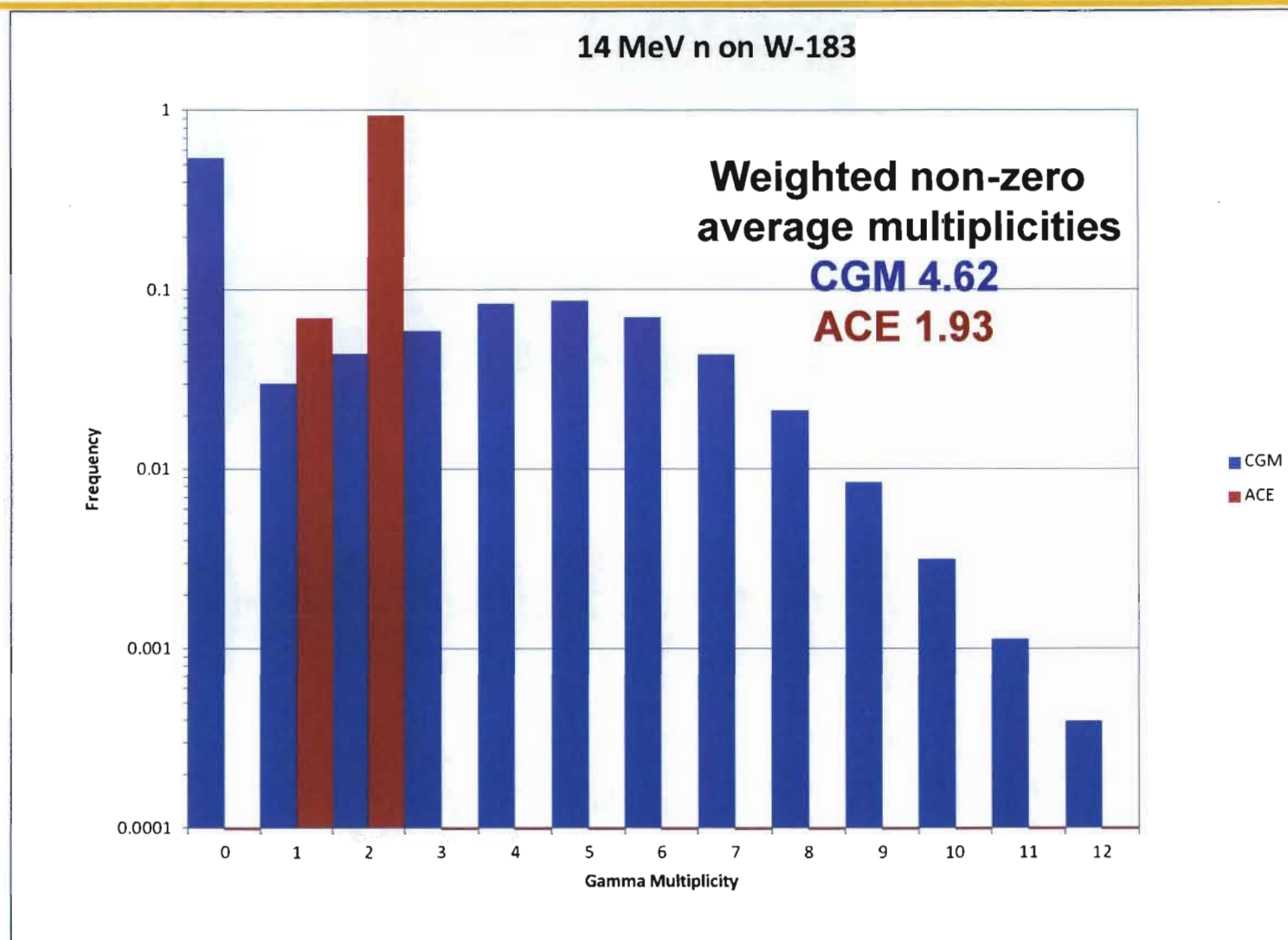
```

mcnpx          70CGM
probid: 09/12/11 13:31:52
tally      21
P
nps          10000000
bin normed
mctal = n14_w183_cgm.m

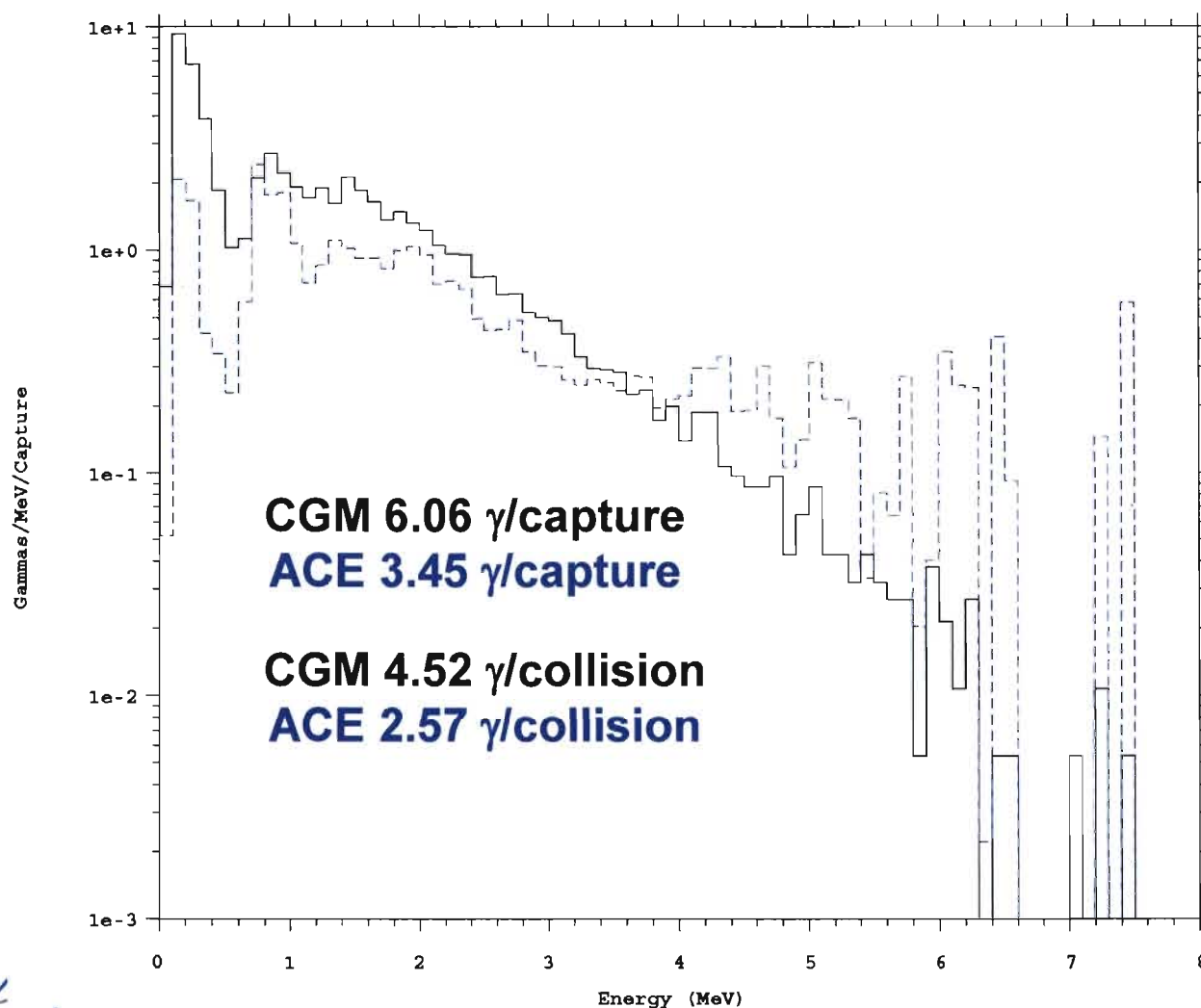
f  Surface      1
d  Flag/Dir     1
u  User         1
s  Segment      1
m  Mult         1
c  Cosine       1
e  Energy       *
t  Time         1

— CGM
--- ACE
  
```

Results – 14 MeV Neutrons on W-183

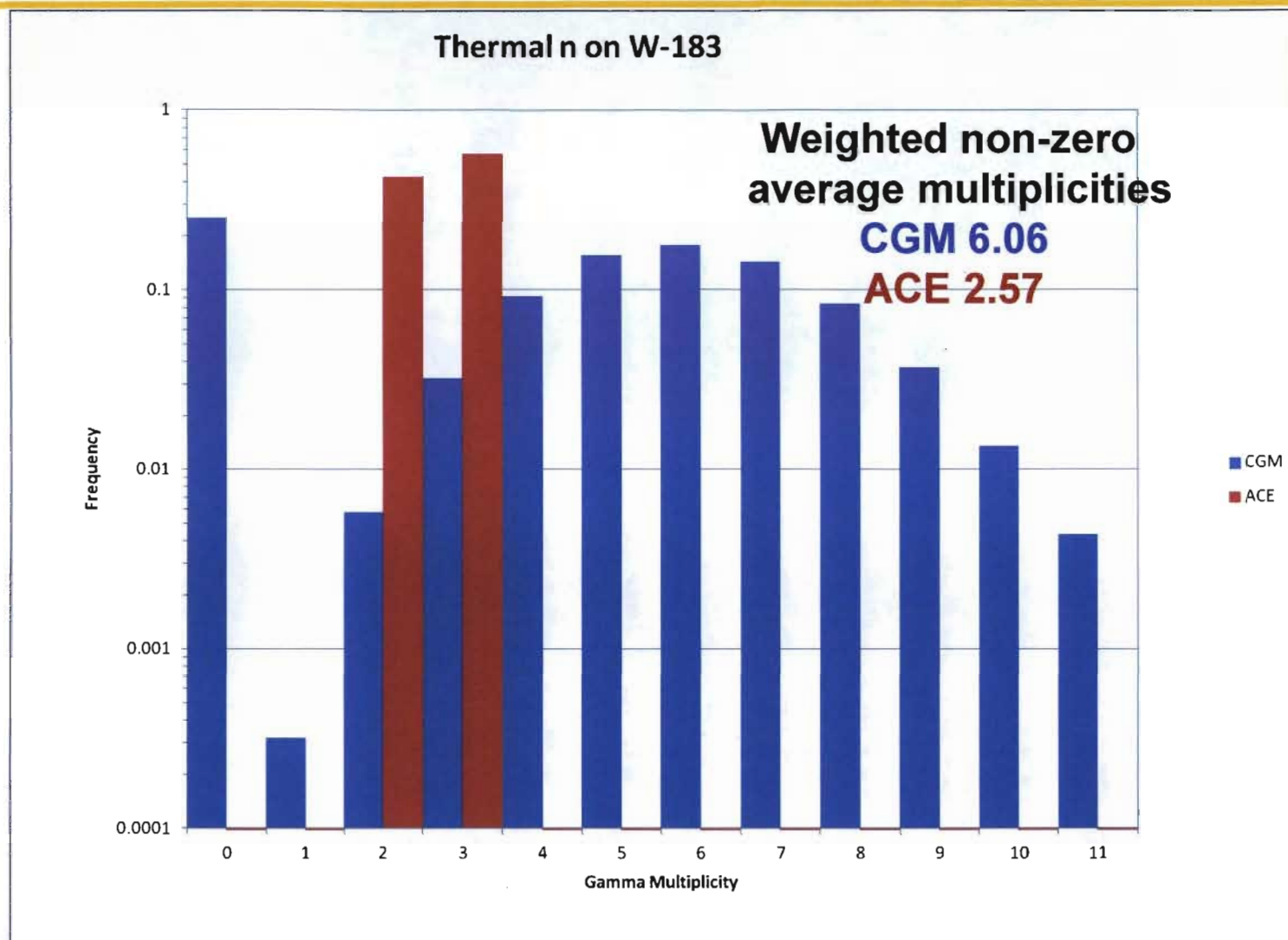


Results – Thermal Neutrons on W-183



```
tally      21
p
nps        2500
bin normed
runtp = jtn_w183_cgm.r
dump       2
f  Surface      1
d  Flag/Dir     1
u  User         1
s  Segment      1
m  Mult         1
c  Cosine       1
e  Energy       *
t  Time         1
----- CGM
----- ACE
```


Results – Thermal Neutrons on W-183



Summary

- **Isotopes above carbon**
- **Now have a spectra of multiplicities vs. integer sampling**
- **Correlated gammas**
- **Differences if integral multiplicities for some isotopes**
- **New entry on the PHYS:N card**

Future Work/New Features

- Compare results with published multiplicities and spectra
- Improve residual excitation energy equation
- Include correlated neutron generation from CGM
- Update to COH to provide full interaction correlation for neutron interactions
- **Integrate improvements for MCNP6 spring release**

Questions

**We would like to acknowledge the sponsorship of the
Domestic Nuclear Detection Office.**