

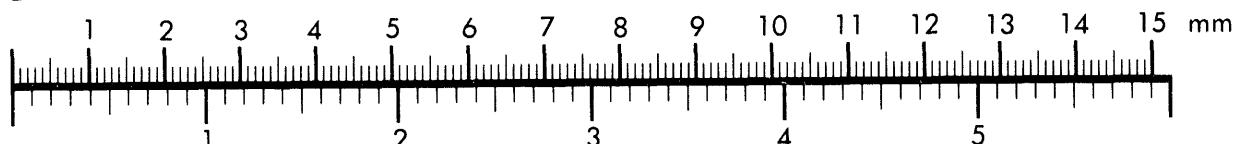


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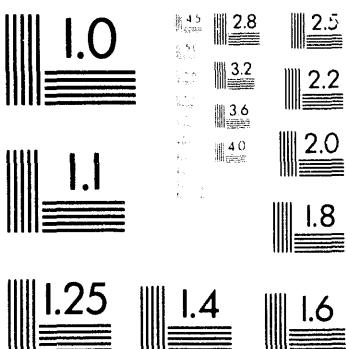
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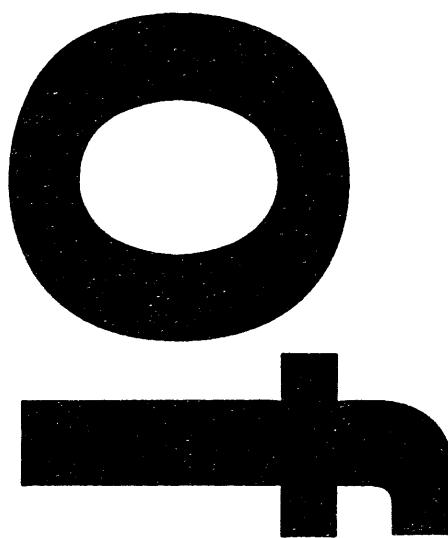
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**HANFORD ATOMIC PRODUCTS OPERATION - RICHLAND, WASHINGTON**

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PRODUCTION TEST IP-214-A,  
IRRADIATION OF ENRICHED ZIRCALOY-2 JACKETED  
SEVEN-ROD CLUSTER ELEMENTS

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PRODUCTION TEST IP-214-A,  
IRRADIATION OF ENRICHED ZIRCALOY-2 JACKETED  
SEVEN-ROD CLUSTER ELEMENTS

MASTER

OBJECTIVE

The objective of this production test is to obtain information about the behavior of Zircaloy-2 jacketed enriched uranium cluster elements during irradiation at high coolant temperatures. Specifically, this test should provide information about fuel element dimensional stability, suitability of the cluster support system, behavior of the uranium-Zircaloy-2 bond, and distortion of co-extruded rod with integral end closures.

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HW-58253  
Page 2

SUMMARY

Four Zircaloy-2 jacketed enriched seven-rod cluster elements, one three-foot three-rod Zircaloy-2 jacketed enriched cluster element with integral end closures, and two eighteen-inch Zircaloy-2 jacketed wire wrapped cluster elements containing  $UO_2$  will be irradiated at jacket surface temperatures up to  $270^\circ C$  in the KER loops to an exposure of 3500 MWD/T on the enriched seven-rod cluster elements.

BASIS AND JUSTIFICATION

Three types of extended surface fuel elements are being considered for high temperature use in an NPR---seven-rod cluster elements, rod-and-tube elements, and tube-and-tube elements. Development work is proceeding on all three types. One phase of the development program includes high temperature irradiation testing of sample fuel elements of each type to observe fuel element distortion, behavior of the support system, and changes in the jacket-uranium bond. This test provides the initial high temperature irradiation of Zircaloy-2 jacketed enriched seven-rod cluster elements. The distortion of enriched rod with integral end closures will also be investigated.

TEST DETAILS

1. Fuel Elements

The individual rods in the enriched Zircaloy-2 jacketed seven-rod cluster elements are 0.593 inches in diameter, 13 inches long, with 0.030 inch thick jackets. Each rod contains 11.4 inches of 1.6% enriched uranium. The rods are assembled in a cluster with triangular end supports, as shown in figure 1. The fuel elements have been visually examined, ultrasonically tested for jacket-uranium bonding, and autoclaved at  $270^\circ C$  and 2250 psig.

The three-foot three-rod cluster element consists of 0.593 inch diameter rods, 38 inches long, containing 34 inches of 1.6% enriched uranium. The Zircaloy-2 jacket is 0.030 inches thick. The rods are supported at the ends and at the third points by flexible Zircaloy-2 supports. In addition to the inspections provided the seven-rod elements, the integral end closures have been X-rayed to verify the jacket thickness at the uranium-end cap junction and to insure that the end closure is free of uranium stringers. After final assembly and testing, the fuel elements will not be modified without the concurrence of Process and Reactor Development Operation.

The  $UO_2$  fuel element consists of seven Zircaloy-2 jacketed rods assembled in a cluster with a 0.072-inch spacing between the rods. The rods are separated by Zircaloy-2 wire wrapped on a ten-inch pitch around the rods and the element is supported in the tube by 0.115-inch Zircaloy-2 wire wrapped around the cluster on a ten-inch pitch. Each rod is eighteen inches long with sixteen inches of fuel-swaged  $UO_2$  of 85 to 87% theoretical density made from natural uranium. The clad rod is 0.560-inch O.D. with a 0.030-inch Zircaloy-2 jacket. The rods were X-rayed for weld integrity and autoclaved for seventy-two hours in  $400^\circ C$ , 1300 psi water.

2. Loading

The tube charge, listed from the downstream end of the process tube, is given in Table 1. All of the spacers and perfs will be made of stainless steel or Zircaloy-2. Coupon holders of equivalent strength and dimensions may be substituted in positions 3 to 18 and 31 to 36.

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HW-58253  
Page 3

Table 1

Tube Loading

<u>Position</u>	<u>Description</u>
1	8" triangular spacer
2	6" triangular perf
3-18	8" triangular spacers
19	38" 3-rod cluster
20	16" 7-rod UO <sub>2</sub> element
21	8" triangular perf
22	18" 7-rod UO <sub>2</sub> element
23	8" triangular perf
24	13" 7-rod cluster
25	8" triangular perf
26	13" 7-rod cluster
27	8" triangular perf
28-29	13" 7-rod cluster
30	8" triangular perf
31-38	6" triangular spacers

If, for any reason, the loading must be modified, Process and Reactor Development Operation will provide a revised loading pattern and set of operating conditions prior to charging. If a revised set of operating conditions cannot be obtained, the loop will be operated at low temperature until a new set of operating conditions has been provided.

## 3. Irradiation Facility

The fuel elements may be irradiated in any one of KER Loops 2, 3, or 4, tubes 2864, 3565, or 4268 KE. The loops consist of 2.100-inch I.D. Zircaloy-2 process tubes through which coolant of controlled composition, pressure, and temperature is recirculated. The fuel elements may not be irradiated in KER Loop 1 because of the smaller process tube size.

## 4. Operating Conditions

The coolant will be pressurized water with a pH of 4.0 to 11.0. The operating conditions are:

- a. Desired operating flow - 60 GPM.
- b. Low flow trip - 48 GPM.
- c. Low pressure trip - 765 psig.
- d. High outlet temperature trip - as shown in figure 2.
- e. Desired operating outlet temperature - as shown in figure 2.
- f. Boiling point suppression trip - as shown in figure 2.

If recirculation with system pressures less than 765 psig is required, the outlet temperature should be reduced and maintained at least 100°C below the system saturation temperature.

## 5. Power and Temperature Relationships

The high outlet temperature trip, desired operating outlet temperature, and boiling point suppression trip have been set as functions of the observed tube  $\Delta T$  so that

# DECLASSIFIED

HW-58253  
Page 4

regardless of the power produced by the fuel elements, the fuel element surface temperature will not exceed the system saturation temperature during normal operation; the fuel elements will not burn out at the limiting trip conditions<sup>1</sup>; and bulk boiling will not occur in the tube. The maximum uranium temperature, as calculated, should not exceed the alpha-beta transformation temperature during operation.

## 6. Exposure

The goal exposure of the enriched seven-rod elements is 3500 MWD/T average exposure.

## 7. Special Discharge Procedure

The fuel elements from this production test will be discharged into a special tray and lowered to the basin to prevent fuel element damage which might obscure fuel element distortion occurring during irradiation. A detailed discharge procedure approved by Process and Reactor Development Operation, Coolant Testing Operation, and KE Processing Operation will be issued prior to the time of discharge. If the approved discharge procedure has not been issued, the fuel element will be discharged normally.

## 8. Priority

Additional down-time is authorized if charging or discharging cannot be accomplished during a normal outage.

## 9. Costs

a. Cost Code - XXXX.5320-XXX.14

b. Time

	Elevator Time, hours		Manhours
	Front	Rear	
Charge	3	2	4
Discharge	1	4	24
	4	6	25

c. Reactivity change - The test loading will have a reactivity intermediate between that of a dummy charge and a standard charge of normal fuel elements.

## 10. Data Desired

Routine operating data, including coolant flow, inlet and outlet temperature and pressure, system pressure, and operating time at temperature, will be taken during irradiation.

## 11. Hazards

Failure of the cluster elements is improbable. The corrosion resistance of the Zircaloy-2 jackets, the high operating temperature, and the extended surface geometry should preclude either a jacket or a core failure. If a failure should

1. F. W. Van Wormer, "Burnout Limits in KTR Loops: Rod-and-Tube and Seven-Rod Cluster Fuel Elements," Sept. 18, 1958.

# DECLASSIFIED

HW-53253  
Page 5

occur, there is sufficient volume in the coolant channel to accommodate the uranium-water reaction products from a failure without process tube splitting. The stability of  $UO_2$  in high temperature water will prevent fuel element swelling or tube splitting if the  $UO_2$  cluster element should fail.

### DEVIATION FROM PROCESS STANDARDS

This production test authorizes deviation from Process Standard K-040 Section 40.01, which limits the maximum uranium temperature to  $600^{\circ} C$ . This production test authorizes maximum uranium temperatures above  $660^{\circ} C$  - the alpha-beta transformation temperature. Although the production test design calculations indicate that the maximum uranium temperature should not exceed  $660^{\circ} C$ , the possibility always exists that the temperatures may actually go higher. For this reason, deviation from the Process Standard is authorized.

Maximum fuel temperatures of the  $UO_2$  elements may exceed  $2000^{\circ} C$  at the center. Operation with oxide at these temperatures in both the MTR and the KER loops has given no difficulty.

### RESPONSIBILITIES

#### Hanford Laboratories Operation

##### Reactor and Fuels Research and Development Operation

##### Fuels Development Operation

Fuels Design is responsible for the fabrication of the cluster elements, analysis of data, and issuance of technical reports.

Ceramic Fuels Development Operation is responsible for fabrication of the  $UO_2$  cluster element, analysis of data, and issuance of technical reports.

#### Irradiation Processing Department

##### Research and Engineering Operation

Component Testing Operation is responsible for post-irradiation examination and testing of components in the test charge.

Coolant Testing Operation is responsible for:

- a. Operation of the KER loops.
- b. Taking basic operating data.
- c. Scheduling the loop charge with the concurrence of KE Processing Operation.

Process and Reactor Development Operation is responsible for:

- a. Technical aspects of the fuel element irradiation.
- b. Analysis of fuel performance data.
- c. Termination of the production test and issuance of the final report.

**DECLASSIFIED**

HW-58253  
Page 6

KE-KW Reactor Operation

KE Processing Operation is responsible for:

- a. Operational safety.
- b. Special discharge procedure.
- c. Production continuity, except where inconsistent with the provisions of this test.



Reactor Fuels Operation  
Process & Reactor Development Operation  
IRRADIATION PROCESSING DEPARTMENT

WK Kratzer:lmb

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HW-58255

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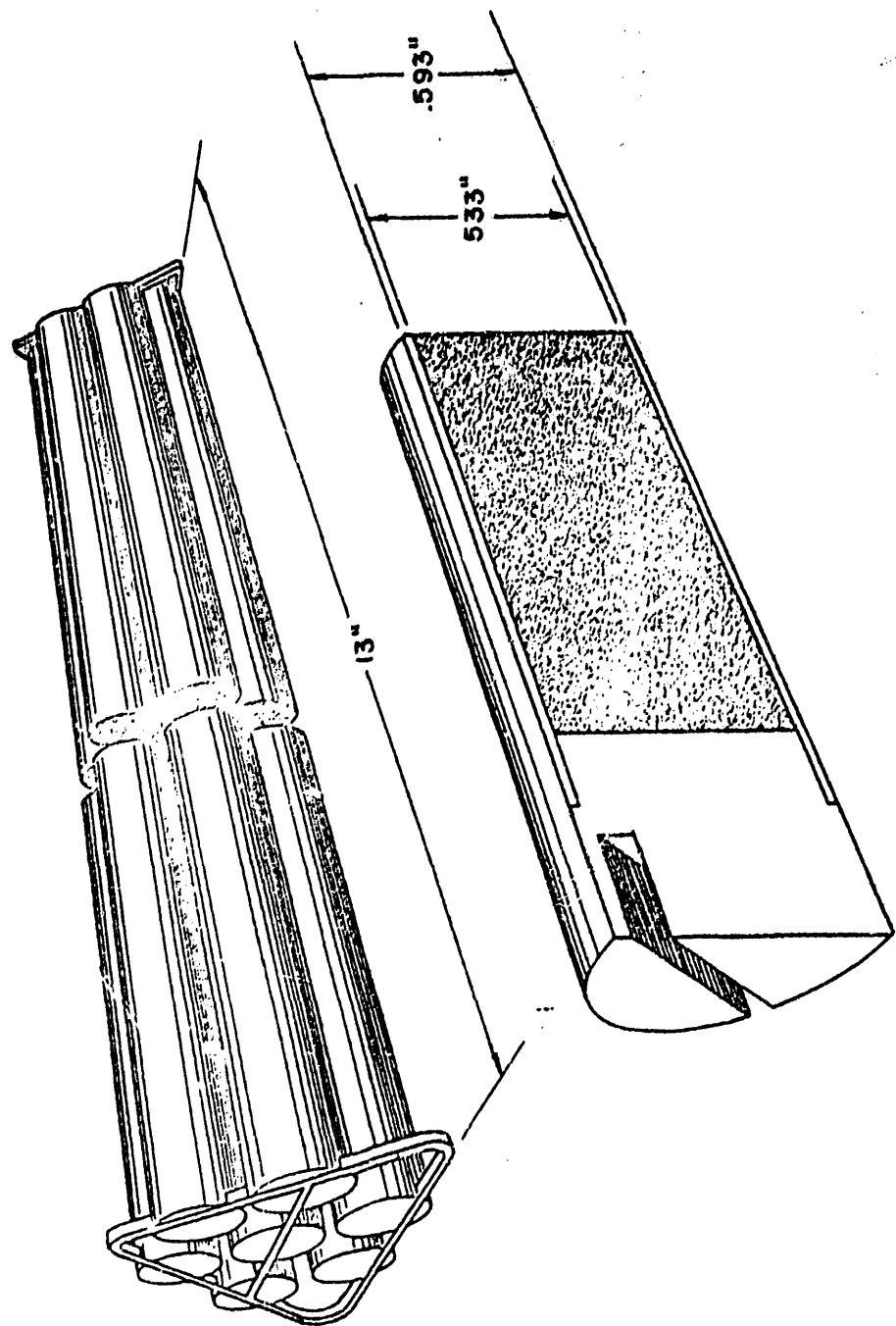


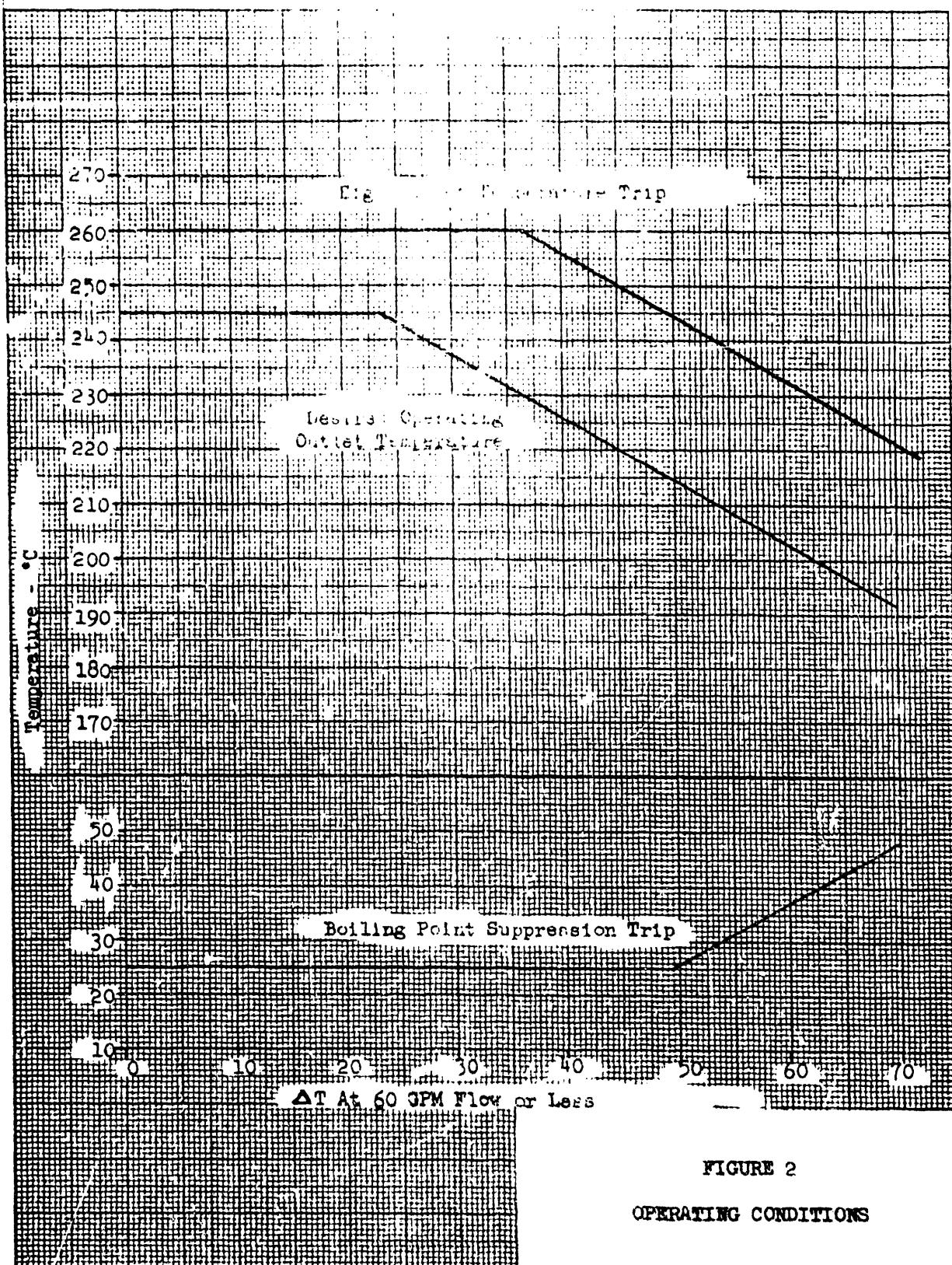
FIGURE 1

ENRICHED 7 - ROD CLUSTER ELEMENT

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HW-58253  
Page 3



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HW-58253  
Page 9

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