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ENGINEERING DATA TRANSMITTAL

Page 1 of 2
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2. To: (Receiving Organization) Packaging Engineering	3. From: (Originating Organization) Packaging Engineering	4. Related EDT No.: 619552
5. Proj./Prog./Dept./Div.: 03E00	6. Design Authority/ Design Agent/Cog. Engr.: W. W. Smyth	7. Purchase Order No.: NA
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1	/	Cog. Eng.	WW Smyth	1/28/97	H1-15						
1	/	Cog. Mgr.	JG Field	1/28/97	H1-15						
1	/	QA	CR Hoover	2/24/97	H1-15						
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W. W. Smyth Signature of EDT Originator	3/5/97 Date Perry O'Sullivan Authorized Representative/ Date for Receiving Organization	J. G. Field Design Authority/ Cognizant Manager	<input checked="" type="checkbox"/> Approved <input type="checkbox"/> Approved w/comments <input type="checkbox"/> Disapproved w/comments Dennis W. Clawson 5/17/97

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1	HNF-SD-TP-SEP-056	NA	0	Safety Evaluation for Packaging (Onsite) Singly Encapsulated Cesium Chloride Capsules

16. KEY		
Approval Designator (F)	Reason for Transmittal (G)	Disposition (H) & (I)
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1	/	Design Authority	JG Field	2/26/97	H1-15	1		BWHC	MM Pereira	Paulo Pereira	5/1/97
1	/	Design Agent	WW Smyth	2/26/97	H1-15	1		PE	PC Berrett	WF	5/1/97
1	/	Cog. Eng.	WW Smyth	2/26/97	H1-15						
1	/	Cog. Mgr.	JG Field	2/26/97	H1-15						
1	/	QA CR	CR Hoover	2/26/97	H1-15						
1	/	Safety	DW McNally	2/26/97	2-27-991-11						
1	/	RL	RL Clauson	2/26/97	3-2-97	H1-15					

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Safety Evaluation for Packaging (Onsite) Singly Encapsulated Cesium Chloride Capsules

W. W. Smyth

Rust Federal Services Inc. Northwest Operations, Richland, WA 99352
U.S. Department of Energy Contract DE-AC06-96RL13200

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Key Words: Waste Encapsulation and Storage Facility, overpacked,
324 building

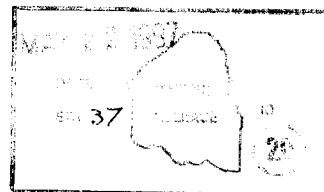
Abstract: Three nonstandard Waste Encapsulation and Storage Facility (WESF) cesium chloride capsules are being shipped from WESF to the 324 building. Normally, they would be shipped in the Beneficial Uses Shipping System; however, three capsules are nonstandard. The 3 capsules, along with 13 other capsules, will be overpacked in the 324 building in order to meet the requirements for storage in WESF's pool.

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James W. Bishop
Release Approval

5-22-97
Date



Release Stamp

Approved for Public Release

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LIST OF TERMS

BUSS	Beneficial Uses Shipping System
cc-atm/s	cubic centimeters-atmospheres per second
Ci	curie
CoC	certificate of compliance
DOE	U.S. Department of Energy
HRCQ	highway route controlled quantity
in.	inch
ft•lb	foot-pound
ft/s	feet per second
J	joule
kg	kilogram
km/h	kilometers per hour
kW	kilowatt
mi	mile
mm	millimeter
mph	miles per hour
mrem/h	millirem per hour
m/s	meters per second
SARP	safety analysis report for packaging
SEP	safety evaluation for packaging
W	watt
WESF	Waste Encapsulation and Storage Facility

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**SAFETY EVALUATION FOR PACKAGING (ONSITE) SINGLY
ENCAPSULATED CESIUM CHLORIDE CAPSULES**

PART A: PACKAGE DESCRIPTION AND OPERATIONS

1.0 INTRODUCTION

Three nonstandard Waste Encapsulation and Storage Facility (WESF) cesium chloride capsules are being shipped from WESF (225B building) to the 324 building. They would normally be shipped in the Beneficial Uses Shipping System (BUSS) cask under its U.S. Department of Energy (DOE) license (DOE 1996), but these capsules are nonstandard: one has a damaged or defective weld in the outer layer of encapsulation, and two have the outer encapsulation removed. The 3 capsules, along with 13 other capsules, will be overpacked in the 324 building to meet the requirements for storage in WESF's pool.

2.0 PACKAGING SYSTEM

The BUSS cask with a 12-capsule basket will be used to transport 3 nonstandard cesium chloride capsules. The BUSS cask is licensed by the DOE to carry up to 16 doubly encapsulated, special form capsules. The safety analysis report for packaging (SARP [SNL 1993]) and license (DOE 1996) allow shipping up to 850,000 Ci of cesium chloride, with a thermal load of 4,000 W.

3.0 PACKAGE CONTENTS

Table A3-1 shows the capsules that will be transported.

Table A3-1. Capsules To Be Transported.

Serial number	Condition	Estimated activity, curies	Estimated power, watts
C-287	Outer capsule present, but with visible weld defect	41,590	200
C-596	Outer capsule removed	36,150	174
C-1636	Outer capsule removed	38,130	183
Total		115,870	557

The BUSS cask SARP (SNL 1993) estimates the total weight of a cask loaded with 16 capsules to be 14,900 kg (32,900 lb), of which 181 kg (400 lb) is due to 16 capsules. With 3 capsules instead of 16, the contents will weigh 34 kg (75 lb), and the loaded cask will weigh 14,800 kg (32,600 lb).

The cask is authorized to carry capsules with a total heat load of 4,000 W and a total activity of 0.85 million Ci of cesium chloride.

4.0 TRANSPORT SYSTEM

The BUSS cask, 12-capsule basket, handling fixtures, and its dedicated transport trailer shall be used. All SARP (SNL 1993) requirements for radiation limits and contamination limits shall be adhered to. Since this shipment does not comply with all of the offsite requirements, the roads shall be barricaded during transportation.

5.0 ACCEPTANCE FOR USE

All maintenance requirements for the BUSS cask shall be performed and documented prior to use.

6.0 OPERATING REQUIREMENTS

1. The three capsules shall be cleaned and inspected prior to packaging. Except for the weld defect of the doubly encapsulated capsule, no pits greater than 1/16 in. or weld cracking shall be allowed.
2. The doubly encapsulated capsule shall be tested to show that the inner capsule is free to slide within the outer capsule. If the inner capsule is free to slide, the capsule shall be loaded into the basket according to the SARP (SNL 1993) requirements and normal procedures.
3. The two capsules that have had their outer jackets removed shall be placed into temporary outer jackets to prevent excessive lateral movement during transportation. Caps for the two outer jackets may be placed in the basket before the capsules to prevent excessive movement during normal transportation and to assist in extracting capsules from the basket with a remote manipulator.
4. The three capsules shall be placed symmetrically in the 12-capsule basket, with three empty spaces between the capsules, in order to maintain load balance.

5. The cask shall be prepared for shipment and operated in accordance with the requirements of Chapter 7, "Operating Procedures," of the SARP (SNL 1993), when not in conflict with Nos. 1 through 4 of this section.
6. The shipper shall ensure that the transit and wait period from assembly to disassembly of the package shall not exceed 30 days.
7. The BUSS cask shall satisfy the requirements of Chapter 8, "Acceptance Tests and Maintenance Program," in the SARP (SNL 1993).

7.0 QUALITY ASSURANCE

All quality assurance requirements of the SARP (SNL 1993) shall be followed.

8.0 MAINTENANCE

All maintenance required by the SARP (SNL 1993) shall be completed prior to transporting the capsules.

9.0 REFERENCES

SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

DOE, 1996, *U.S. Department of Energy Certificate of Compliance For Radioactive Materials Packages*, USA/9511/B(U) (DOE), Rev. 3 (August 27), U.S. Department of Energy, EM-76, Germantown, Maryland.

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PART B: PACKAGE EVALUATION**1.0 INTRODUCTION**

This safety evaluation for packaging (SEP) demonstrates compliance with the onsite transportation safety program (WHC-CM-2-14, *Hazardous Material Packaging and Shipping*). The BUSS cask with three nonstandard capsules itemized in Part A, Section 2.0, is shown to meet the acceptance criteria found in the *Report on Equivalent Safety for Transportation and Packaging of Radioactive Materials* (Mercado 1994).

1.1 EVALUATION SUMMARY AND CONCLUSIONS

This SEP demonstrates safety by comparing this shipment against offsite shipments authorized and conducted according to the safety analysis report for packaging (SARP [SNL 1993]) and the certificate of compliance (CoC [DOE 1996]). Where there are differences, these differences are evaluated.

The amount of material, in terms of weight, activity, and thermal heat generation, are less than allowed by the CoC. Since this is a highway route controlled quantity (HRCQ), this SEP shall be approved by the U.S. Department of Energy, Richland Operations Office.

The shielding and thermal evaluations show that this shipment will result in radiation dose rates and temperatures significantly lower than those in the SARP (SNL 1993).

There is no criticality evaluation because no fissile materials are being shipped.

The structural adequacy of the shipment depends on the capsules, and the CoC (DOE 1996) requires special form verification within 12 months of shipment. This requirement cannot be met for these three capsules, but the structural evaluation demonstrates that the variance does not cause undue risk.

The radiological risk evaluation presented in Part B, Section 3.0, of this SEP demonstrates that the expected frequency of damage to the cask severe enough to damage the capsules is 1.5×10^{-9} per year. Because this is significantly below the site requirement of 10^{-7} per year and because the BUSS cask meets onsite transportation safety requirements for shipments of special form cesium chloride capsules, the BUSS cask presents no unacceptable risks to the worker or the public.

Containment is maintained throughout all normal transfer conditions, and the probability of an accident severe enough to cause a release of radioactive materials is so small that a dose consequence analysis is not required.

1.3 REFERENCES

DOE, 1996, *U.S. Department of Energy Certificate of Compliance For Radioactive Materials Packages*, USA/9511/B(U) (DOE), Rev. 3 (August 27), U.S. Department of Energy, EM-76, Germantown, Maryland.

Mercado, J. E., 1994, *Report on Equivalent Safety for Transportation and Packaging of Radioactive Materials*, WHC-SD-TP-RPT-001, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

WHC-CM-2-14, *Hazardous Material Packaging and Shipping*, Westinghouse Hanford Company, Richland, Washington.

2.0 CONTENTS EVALUATION

The radioactive contents listed in Part A, Section 3.0, are less than 20% of the activity approved for transportation in the SARP (SNL 1993). As long as the SARP maintenance and operating requirements are followed, the structural and thermal demands on the cask will be much less than with 16 full-power capsules. The contents are not certified as special form, but the design met the test objectives for special form.

This is an HRCQ because material with activity of greater than 27,000 Ci is being transported.

2.1 REFERENCE

SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

3.0 RADIOLOGICAL RISK EVALUATION

The welded stainless steel encapsulation will provide containment for the cesium chloride material as discussed in Part B, Section 4.0. However, as an additional demonstration of safety, a discussion is presented in this section, which addresses the radiological risks associated with the shipment of the BUSS cask containing singly encapsulated cesium chloride capsules.

Onsite transportation safety requirements are outlined in WHC-CM-2-14, *Hazardous Material Packaging and Shipping*, and Mercado (1994). The acceptability of risks associated with onsite shipments can be determined by a radiological risk evaluation, which applies risk acceptance criteria, cask

failure threshold values, and Hanford Site truck accident frequencies. A detailed risk analysis is not required, however, as long as a risk evaluation demonstrates that the packaging will prevent release of the contents for any accident that occurs with a frequency greater than 10^{-7} per year. An annual release frequency of 10^{-7} is the most restrictive acceptance criterion.

The BUSS cask is a certified Type B packaging, which is approved for the shipment of HRCQ nonfissile radioactive material. The BUSS cask provides shielding and additional impact, puncture, and thermal protection for doubly encapsulated special form contents during normal and accident conditions as defined in 10 CFR 71 (January 1, 1993). Compliance with these requirements is documented in the BUSS cask SARP (SNL 1993).

The SARP demonstrates that the cask body, lid, and impact limiters are virtually unaffected by being subjected to the normal and hypothetical accident conditions specified in 10 CFR 71.71 and 10 CFR 71.73. Those conditions include a 9-m (30-ft) drop onto a hard unyielding surface, a puncture test, and immersion in an 800 °C fire for 30 minutes. The BUSS cask SARP relies on the special form encapsulated materials to provide containment. However, because the BUSS cask structural members withstand hypothetical accident conditions, protection of the payload is also being provided by the BUSS cask body, lid, and impact limiters.

In a radiological risk evaluation the total conditional probability of failure of the packaging is multiplied by the frequency of accidents per year to arrive at an annual accident release frequency. If the annual accident release frequency is below 10^{-7} releases per year, the shipment meets onsite transportation safety requirements (Mercado 1994).

The Hanford Site truck accident rate for all trucks, including vans and lightweight pickup trucks, is equal to 2.0×10^{-7} accidents per mile (Green et al. 1996). For a shipment of radioactive materials that is carried out by trained truck drivers during daylight hours in good road conditions, a reduction factor of 20 can be applied to lower the rate to 1×10^{-8} (H&R 1995). The BUSS cask will be used for one shipment of 40.2 km (25 mi), which, when multiplied by the accident rate and reduction factor, gives a frequency of 2.5×10^{-7} accidents per year.

The conditional release probabilities are based on the performance of the packaging and are taken from a study performed by Sandia National Laboratory on the response of large casks to severe accidents (Dennis et al. 1976). The BUSS cask is 30 cm (12 in.) thick, and the probability of a release given a puncture event is approximately equal to zero. The cask will withstand a 30-minute, 800 °C fire; therefore, the probability of a fire failure on the Hanford Site, where emergency responders have a 15-minute response time and are trained in the handling of radioactive material, is also approximately zero. In addition, based on the analysis presented in the BUSS cask SARP (SNL 1993), the BUSS cask would not fail from the crush force of the trailer (7,257 km [16,000 lb]) should a rollover accident occur. Therefore, the probability of failure from the crush force would also be approximately equal to zero.

The only conditional release probability that will affect the annual accident release frequency is related to impact. The BUSS cask SARP

demonstrates that the cask will survive a 9-m (30-ft) drop onto a hard unyielding surface. If it is conservatively assumed that the cask fails a velocity change of 48.3 km/h (30 mph) onto concrete, given that the system has a gross vehicle weight of 31.75 metric tons (35 tons), the corresponding total conditional release probability is 0.00612 (Dennis et al. 1976). When multiplied by the annual accident frequency of 2.5×10^{-7} , the resulting annual accident release frequency is 1.5×10^{-9} , which is well below the required 10^{-7} . In fact, 65 shipments can be made in one year and still meet the accident criterion of less than 10^{-7} . Therefore, the BUSS cask meets onsite transportation safety requirements, and shipments of singly encapsulated cesium chloride capsules in the BUSS cask present no unacceptable risks to the worker or the public.

3.1 REFERENCES

10 CFR 71, 1993, "Transportation and Packaging of Radioactive Material," *Code of Federal Regulations*, as amended.

Dennis, A. W., J. T. Foley, W. F. Hartman, and D. W. Larson, 1978, *Severities of Transportation Accidents Involving Large Packages*, SAND77-0001, Sandia National Laboratories, Albuquerque, New Mexico.

Green, J. R., B. D. Flanagan, and H. W. Harris, 1996, *Hanford Site Truck Accident Rate, 1990-1995*, WHC-SD-TP-RPT-021, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

H&R, 1995, *Recommended Onsite Transportation Risk Management Methodology*, H&R522-1, H&R Technical Associates, Inc., Oak Ridge, Tennessee.

Mercado, J. E., 1994, *Report on Equivalent Safety for Transportation and Packaging of Radioactive Materials*, WHC-SD-TP-RPT-001, Rev. 0, Westinghouse Hanford Company, Richland, Washington.

WHC-CM-2-14, *Hazardous Material Packaging and Shipping*, Westinghouse Hanford Company, Richland, Washington.

4.0 CONTAINMENT EVALUATION

The containment boundary for this shipment is the inner capsule, while the SARP evaluation (SNL 1993) assumes doubly contained capsules as the containment. This shipment differs from the SARP analysis only in that the material is singly contained.

The inner capsules were tested after fabrication and were required to demonstrate a leak rate of less than 10^{-8} atm cc/s for acceptance, so they are leaktight (Kenna 1984). Additionally, special form testing was conducted on singly encapsulated as well as doubly encapsulated cesium chloride capsules. The testing included a 10-m drop onto an unyielding surface, immersion in an 815 °C fire for 30 minutes, and submersion in water for 24 hours. Examination and destructive testing of the capsules showed no leakage or damage that could

cause loss of contents. The test report concluded that the singly encapsulated cesium chloride was capable of withstanding the special form testing with no loss of contents, but it was not certified as special form (Hammond 1975).

The three capsules being shipped have not undergone extremes in temperature variation suspected in damaging other capsules and are still considered to be leaktight. The two inner capsules will be inspected for evidence of bulging, weld defects, or pits; in the absence of these indications, the capsules can be considered to be resistant to damage likely to be encountered in normal or accident conditions of transportation. Similarly, the degree of weld damage evident in the doubly encapsulated capsule will not prevent the outer encapsulation from protecting the inner capsule during transportation.

The cask seal is designed to have a leak rate no greater than 10^{-4} cc-atm/s and to retain the helium coolant during normal transportation. After an accident, the SARP (SNL 1993) demonstrates that there will be no permanent deformation of the cask seal, but no leaktightness was demonstrated because helium did not have to be retained after an accident. Singly encapsulated cesium chloride capsules met special form requirements when they were fabricated and can still be considered leaktight, so the cask is not required to be leaktight.

4.1 REFERENCES

Hammond, J. E., 1975, *Cesium Chloride Capsule Testing for Special Form Qualification*, ARH-CD-440, Atlantic Richfield Hanford Company, Richland, Washington.

Kenna, B. T., 1984, *WESF ¹³⁷Cs Gamma Ray Sources*, SAND82-1492, Sandia National Laboratories, Albuquerque, New Mexico.

SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

5.0 SHIELDING EVALUATION

This shipment involves less than 3/16 of the radioactive contents evaluated by the SARP (SNL 1993) and allowed by the CoC (DOE 1996). With the possible exception of the stainless steel upper cap of the outer encapsulation, all shielding assumed by the SARP analysis is in place. This cap is 1 cm (0.4 in.) thick and will have a negligible effect on the external dose rate because the bottom of the cask and the lid are over 30 cm (12 in.) thick and the amount of cesium chloride being transported will be much less than authorized by the SARP.

Table B5-1 shows the results from the SARP analysis (SNL 1993).

Table B5-1. Shielding Results for 16 Cesium Chloride Capsules Totaling 850,000 Ci of ^{137}Cs .

	Normal conditions of transport				Accident conditions	
	Package surface		2 m from surface		1 m from surface	
	Top	Side	Top	Side	Top	Side
Gamma dose rate, mrem/h	64	29	2.9	1.2	11	4.6

Source: SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

Since 115,870 Ci will be shipped, the lack of a top cap 1 cm (0.4 in.) thick on two of three capsules will not be significant.

5.1 REFERENCES

DOE, 1996, *U.S. Department of Energy Certificate of Compliance For Radioactive Materials Packages*, USA/9511/B(U) (DOE), Rev. 3 (August 27), U.S. Department of Energy, EM-76, Germantown, Maryland.

SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

6.0 CRITICALITY EVALUATION

No fissile materials are being transported, so no criticality analysis is required.

7.0 STRUCTURAL EVALUATION

The SARP (SNL 1993) evaluates the cask as an outer packaging for special form capsules. The record of the qualification of the capsules as special form (Hammond 1975) showed, however, that the inner capsule alone exceeded the requirements for special form material, with no outer encapsulation. With a second encapsulation, the record of qualification simply stated that the capsule "... far exceeds the requirements to qualify as special form material."

As a part of the special form testing, the inner capsule was dropped 10 m (33 ft) onto a test pad. To qualify for special form, the capsule must be dropped 9 m (30 ft) and retain its contents. The additional 1 m drop height (additional 3 ft) resulted in an impact velocity of 14 m/s rather than the required 13 m/s (46 ft/s rather than 44 ft/s). All of the special form test requirements were exceeded during the test.

The SARP (SNL 1993) analyzed the capsule during a 9-m (30-ft) cask drop onto the lid, accounting for a 3.8-cm (1.5-in.) vertical gap between the outer capsule and the lid. When the top impact limiter first strikes a nonyielding surface, the lid begins to decelerate while the capsule is still accelerating downward. When the accelerating capsule impacts the decelerating lid, the analysis showed that an inner capsule would absorb 209 J (154 ft-lb) of energy. With a temperature of 482 °C (900 °F), the inner capsule (only the inner capsule was assumed to take load) would experience a vertical deformation of 3.7 mm (0.147 in.), and this amount of plastic deformation would not cause failure of the capsule. During the drop test for special form qualification, the inner capsule absorbed 805 J (594 ft-lb) of energy without loss of contents.

If the outer jacket's cap is omitted from the shipment, there is an added gap of less than 2.5 cm (1 in.) between the lid and capsule. An analysis similar to that done in the SARP (SNL 1993) is not needed, however, since the inner capsule design has already been tested by a 10-m (33-ft) drop onto an unyielding surface. This is a more severe requirement than would be experienced by the capsule during a 9-m (30-ft) drop in the cask with an impact limiter that reduces the deceleration to 105g.

7.1 REFERENCES

DOE, 1996, *U.S. Department of Energy Certificate of Compliance For Radioactive Materials Packages, USA/9511/B(U)* (DOE), Rev. 3 (August 27), U.S. Department of Energy, EM-76, Germantown, Maryland.

Hammond, J. E., 1975, *Cesium Chloride Capsule Testing for Special Form Qualification*, ARH-CD-440, Atlantic Richfield Hanford Company, Richland, Washington.

SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

8.0 THERMAL EVALUATION

Table B8-1 shows the results of the SARP analysis (SNL 1993). The SARP considered both 12- and 16-capsule baskets and found higher temperatures in the 12-capsule case.

Table B8-1. Cask Temperatures When Loaded With
12 Cesium Chloride Capsules With Total
Heat Generation of 4 kW.

Location	Calculated temperature, °C (°F)	Allowable temperature, °C (°F)
Cesium chloride	462 (864)	N/A
Inner cladding	407 (765)	450 (842) ^a
Outer cladding	369 (696)	800 (1472) ^b
Basket	348 (658)	800 (1472)
Lid or port seals	202 (396)	450 (842) ^c
Inner cask wall	202 (396)	800 (1472)
Outer cask wall	140 (284)	800 (1472)

Source: SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

^aInner cladding temperature limit was used since it is the temperature used in testing for long term geologic disposal. Below this temperature, there is no anticipated corrosion of the cladding material.

^bStainless steel temperature limit, other than the inner cladding, was based on reducing the potential for significant corrosion.

^cSeal temperature limits are the maximum operating temperatures provided by the manufacturer.

With 557 W of thermal power instead of 4,000, the temperatures will be significantly less than those calculated by the SARP (SNL 1993).

8.1 REFERENCE

SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

9.0 PRESSURE AND GAS GENERATION

Because the thermal load is less than 20% of that used in the SARP (SNL 1993) and no gas is generated within the package, the pressure in the package is less than that calculated in the SARP analysis, which was less than the capacity of the package.

9.1 REFERENCE

SNL, 1993, *Beneficial Uses Shipping System Cask BUSS Safety Analysis Report for Packaging (SARP)*, Volumes I and II, SAND83-0698 (TTC-0430), Rev. 4, Sandia National Laboratories, Albuquerque, New Mexico.

10.0 TIEDOWN EVALUATION

The package will be transported in its normal highway configuration. As a part of this, the cask is secured to a skid, which is permanently attached to the trailer. The skid attachment is inspected annually as a part of routine maintenance.

DISTRIBUTION SHEET

To Distribution	From Packaging Engineering	Page 1 of 1		
Project Title/Work Order	Date May 20, 1997			EDT No. 619553
Safety Evaluation for Packaging (Onsite) Singly Encapsulated Cesium Chloride Capsules (HNF-SD-TP-SEP-056, Rev. 0)	ECN No. N/A			
Name	MSIN	Text With All Attach.	Text Only	Attach./ Appendix Only
D. W. Claussen	S7-55	X		
R. L. Clawson	H1-15	X		
P. C. Ferrell	H1-15	X		
J. G. Field	H1-15	X		
C. R. Hoover	H1-15	X		
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