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# Results from a controlled depleted uranium metal casting experiment designed to investigate nuclear Forensic Radiochronometry Signatures

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1       **Results from a Controlled Depleted Uranium Metal**  
2       **Casting Experiment Designed to Investigate Nuclear**  
3       **Forensic Radiochronometry Signatures**

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11       **Abstract**

12       The accurate interpretation of uranium metal <sup>230</sup>Th/<sup>234</sup>U and <sup>231</sup>Pa/<sup>235</sup>U radiochronometry  
13       model ages requires an understanding of how uranium parent nuclides and decay progeny  
14       (<sup>230</sup>Th and <sup>231</sup>Pa) behave during uranium metal casting. In order to directly measure the  
15       spatial distribution of <sup>230</sup>Th and <sup>231</sup>Pa in uranium metal before and after vacuum induction  
16       melting (VIM), Los Alamos National Laboratory identified uranium metal feedstock,  
17       characterized the metal feedstock, conducted a controlled casting experiment of an  
18       approximately 120 kg uranium metal rod, and characterized the cast metal. This study  
19       presents radiochronometry results and quantified <sup>230</sup>Th and <sup>231</sup>Pa VIM separation factors  
20       from bulk uranium.

21       **Keywords**

22       uranium metal, nuclear forensics, radiochronometry, thorium, protactinium, age dating

23        **Article Highlights**

- 24        • Uranium metal was cast in a controlled environment to study the behavior of  $^{230}\text{Th}$   
25        and  $^{231}\text{Pa}$  during vacuum induction melting.
- 26        • Results present  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  separation factors from bulk uranium during vacuum  
27        induction melting.
- 28        • Model  $^{231}\text{Pa}/^{235}\text{U}$  ages of cast uranium metals preserve the feedstock production date  
29        prior to casting.

30        **Introduction**

31        Radiochronometry, or the science of measuring the time passed since the purification of a  
32        radionuclide from its progeny, can be used to produce predictive or comparative model  
33        age signatures often used to support nuclear forensic investigations [1, 2, 3, 4]. Model  
34        ages are calculated through the precise measurement of progeny/parent nuclide atom  
35        ratios in uranium or plutonium sample matrices. Many paired progeny/parent  
36        chronometers can be used for radiochronometry – e.g.  $^{230}\text{Th}/^{234}\text{U}$ ,  $^{231}\text{Pa}/^{235}\text{U}$ ,  $^{234}\text{U}/^{238}\text{Pu}$ ,  
37         $^{235}\text{U}/^{239}\text{Pu}$ ,  $^{236}\text{U}/^{240}\text{Pu}$ , and  $^{241}\text{Am}/^{241}\text{Pu}$ . Model ages calculated from these  
38        radiochronometers correspond to the time passed since the last chemical purification (or  
39        production) of a nuclear material assuming that: 1) decay products were completely  
40        purified from the parent at the time of production, and 2) the system has remained closed  
41        with no loss or addition of parent or daughter since the time of production. These model  
42        assumptions are carefully considered when evaluating nuclear forensic data, and although  
43        model ages may not correspond to actual purification or production dates if these  
44        assumptions are not met, model ages are nonetheless often used to constrain material  
45        process history timelines.

46

47        Recent studies support this approach, and have shown that the general radiochronometry  
48        model assumptions may not be applicable to nuclear materials in many real-world

49 production scenarios [3, 5, 6, 7, 8, 9]. Uranium and plutonium decay progeny may not be  
50 completely purified during certain fuel cycle operations resulting in discordant  
51 radiochronometric signatures [3, 5, 6, 7, 8, 9]. Differential behavior or chemical  
52 fractionation of decay progeny from parent nuclides has been well documented for both  
53 uranium metal and uranium hexafluoride (UF<sub>6</sub>) systems [6, 7, 8, 9]. In uranium metal  
54 systems, a unique repeatable pattern of discordance has been documented whereby  
55 <sup>231</sup>Pa/<sup>235</sup>U model ages of uranium metals are often significantly older than <sup>230</sup>Th/<sup>234</sup>U  
56 model ages of the same materials [6, 9, 10]. Higginson et al. 2022 documented this  
57 discordance pattern in five different uranium metals – three depleted uranium metals and  
58 two uranium-niobium alloys. The Higginson et al. 2022 study was the first nuclear  
59 forensic study to investigate radiochronometry signatures in paired uranium metal  
60 feedstock and cast product samples. Data from the paired samples suggested that <sup>231</sup>Pa  
61 may not be separated during vacuum induction melting (VIM) casting of uranium metal,  
62 whereas <sup>230</sup>Th is fairly efficiently separated from the bulk uranium product [9]. If <sup>231</sup>Pa is  
63 not separated during VIM casting, the <sup>231</sup>Pa/<sup>235</sup>U model age of a uranium metal cast using  
64 VIM may be a useful nuclear forensic signature that preserves the age of the source  
65 uranium used to cast a metal object. While the Higginson et al. 2022 study focused  
66 primarily on VIM-cast metals, differential separation of <sup>230</sup>Th and <sup>231</sup>Pa from a bulk  
67 uranium matrix has also been observed during bomb reduction [10] and may occur during  
68 other casting operations such as vacuum arc remelt and microwave casting. Within the  
69 foundry of the Sigma Division at Los Alamos National Laboratory, there is a continuous  
70 stream of work control measurements which indicate that the dose rate from a casting is  
71 higher than that of the charge material, with the remnant skull (material remaining in the  
72 crucible) being the most radiologically active in the process and the hot top (riser) the  
73 most active of the cast material. The decay times, as monitored only for handling  
74 purposes (approximately 3 weeks), for particularly radiologically active castings suggest  
75 that thorium segregation and subsequent decay is a controlling factor [11]. A summary  
76 of the Nuclear Forensics International Technical Working Group Round Robin 3  
77 Exercise by Hanlen in 2011 noted that when UF<sub>4</sub> is reduced to uranium metal that  
78 reduction generally occurs around 1300°C at which temperature uranium is molten, while

79 thorium atoms with a melting point of 1750°C are refractory and flocculate together  
80 forming less dense particles that float and become part of the casting slag [10]. In  
81 addition, Reilly et al. 2018 suggested that thorium behavior may be more complex during  
82 metal casting with thorium transfer to metal slugs and surfaces of cast uranium metal  
83 produced from bomb reduction [12].

84

85 Discordant radiochronometry ages of uranium metals present challenges during nuclear  
86 forensic assessments of material process history. More research is needed to understand  
87 the separation of  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  from uranium during uranium metal production and  
88 casting in order to increase confidence during the assessment of radiochronometry  
89 signatures for uranium metal found out of regulatory control. Therefore, in this study, we  
90 cast depleted uranium metal under controlled laboratory conditions and document the  
91 behavior of uranium decay progeny,  $^{230}\text{Th}$  and  $^{231}\text{Pa}$ , during metal casting. This study, to  
92 the best of our knowledge, is the first controlled uranium metal casting experiment  
93 designed to quantitatively investigate the spatial distribution of thorium and protactinium  
94 in a cast uranium metal. Controlled metal production experiments, such as the one  
95 described here, have the potential to provide quantitative thorium-uranium and  
96 protactinium-uranium separation factors resulting from VIM casting of uranium and  
97 improve the forensic assessment of radiochronometry model ages of unknown seized  
98 uranium metals.

## 99 **Experimental**

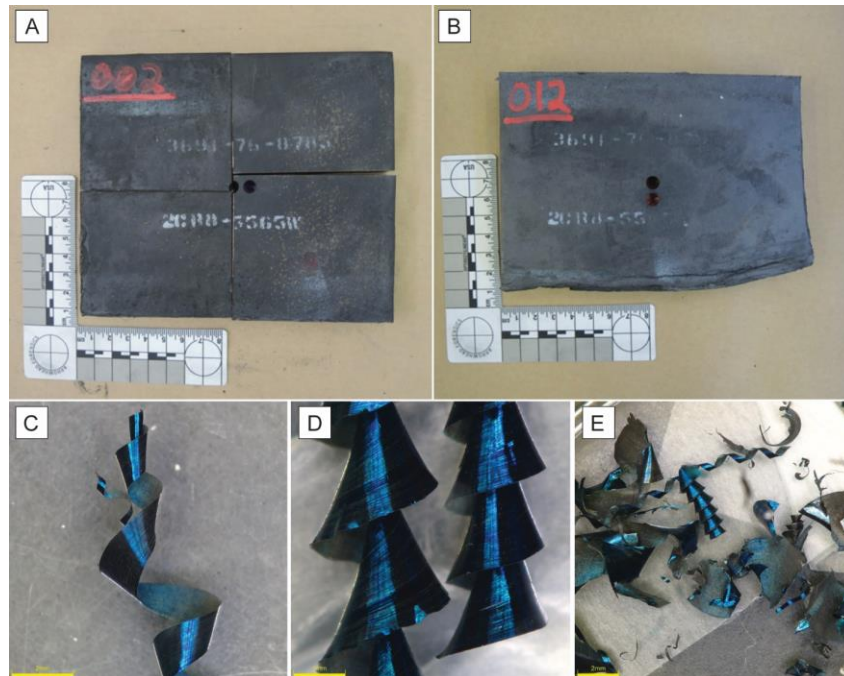
### 100 *Identification and Characterization of Depleted Uranium Feedstock*

101 The goal of this work was to cast a depleted uranium rod from a well-characterized  
102 uranium feedstock. The feedstock chosen for this study consisted of 402 kg of depleted  
103 uranium metal plates (approximately 0.2 wt %  $^{235}\text{U}$ ) stored at Los Alamos National  
104 Laboratory in the Sigma Facility. The feedstock history was not entirely known;  
105 however, based on process knowledge at Sigma, the feedstock was likely cast at the Y-12  
106 National Security Complex in the 1970s as a large ingot (~ 2000 kg) using derby metal

107 produced by bomb reduction of  $UF_4$  as charge with subsequent warm rolling and shearing  
108 to produce manageable plate sizes.

109

110 A large rod casting geometry was chosen for this study in order to provide sufficient  
111 cooling time to allow for flotation of impurities during casting. This geometry required  
112 14 metal feedstock plates for the desired charge mass of approximately 120 kg. The 14  
113 depleted uranium plates identified for casting were cleaned to remove surface oxide  
114 buildup using water, scrubbing, and acetone prior to sampling. Because the plates  
115 contained in the drum were likely from the same original casting (based upon markings),  
116 or at least from the same well-controlled material stream, the metal was expected to be  
117 homogeneous. To characterize the depleted uranium and confirm homogeneity of  
118 feedstock composition, samples consisting of 0.25-inch (0.64 cm) drilled chips  
119 (sometimes referred to as “pigtailed” or turnings) were taken from one location per plate  
120 (**Fig. 1**). With the expectation of homogenous feedstock, one location per plate was  
121 deemed sufficient for chemical characterization; however, during sampling  
122 documentation was kept recording which plates were from the edge of the rolled ingot  
123 based on visual indications as shown by plate 012 (sample ID, S-12-A) in **Fig. 1B**. Two  
124 drill holes were made in approximately the middle of each plate and the collected drilled  
125 chips were combined. Two-gram subsamples from each plate were taken for  
126 radiochronometry. Exemplar optical microscopy images taken using an Olympus  
127 DSX1000 digital microscope of the 2-gram radiochronometry subsamples used for  
128 destructive analysis are shown in **Fig. 1C-E**.



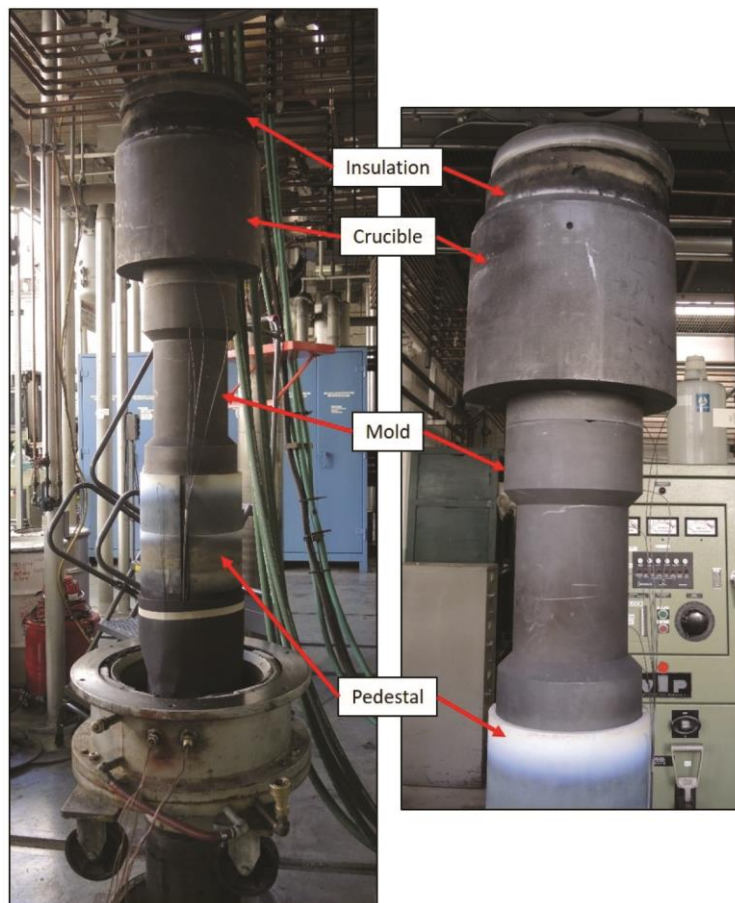
129

130 **Fig. 1** Panels A and B – Photographs of 2 of the 14 feedstock plates used in this study.  
 131 The locations where samples were taken for destructive analysis are visible on each plate.  
 132 The photo of plate 002 (left) was taken after sectioning on the bandsaw. The bottom edge  
 133 of plate 012 (S-12-A, right) indicates that this plate was taken from the edge of the rolled  
 134 ingot. Panels C to E – Optical microscopy images at 25x (left), 60x (middle) and 25x  
 135 (right) magnification showing examples of feedstock turnings used for destructive  
 136 analysis. The blue iridescence of the sample surfaces is consistent with surface oxidation  
 137 of the uranium metal and the samples display a range of shapes and sizes.  
 138

### 139 *Uranium Metal Casting*

140 The depleted uranium feedstock plates were quartered on a bandsaw for use as charge  
 141 material in the rod casting (**Fig. 1A**). After the feedstock charge was prepared, a depleted  
 142 uranium rod was cast using VIM at the Los Alamos National Laboratory Sigma Facility.  
 143 The planned cast rod dimensions were as follows: 6-inch (15.2 cm) outer diameter (OD)  
 144 x 1.5-inch (3.8 cm) inner diameter (ID) x 12-inch (30.5 cm) length (L). The casting  
 145 design included a hot top to feed liquid to the solidification front because liquid uranium  
 146 shrinks upon freezing. Any shrinkage porosity that occurs is designed to be restrained to  
 147 the hot top, resulting in a sound part without voids. The hot top on the casting had an OD

148 of 7.5-inches (19.2 cm) and an ID of 1.5-inches (3.8 cm). The furnace configuration used  
149 for this project is shown in **Fig. 2**. The feedstock plates were loaded into a graphite  
150 crucible sitting on top of the graphite mold. The locations where graphite would contact  
151 metal were coated in yttria (yttrium oxide,  $Y_2O_3$ ) to limit carbon uptake in the metal  
152 during casting. The graphite was instrumented with thermocouples to record the thermal  
153 profile during casting. The crucible and mold were stacked atop a graphite pedestal,  
154 which was used to place the mold at the correct height within the furnace coils. The entire  
155 stack sits inside of the furnace base. The casting preparation also included the addition of  
156 insulation, which consisted of graphite felt and graphite foam. The mold was set at the  
157 Sigma Facility on November 2, 2021 and casting was conducted on November 3, 2021.  
158



159  
160 **Fig. 2** The large rod mold geometry used for this casting. The insulation, crucible, mold,  
161 and pedestal sit in the furnace base, and thermocouples used to monitor the thermal  
162 profile are also shown.



163

164 The casting was processed in a single-coil VIM furnace. The metal melted in the crucible  
165 while the mold was heated to a specified thermal profile under vacuum conditions. When  
166 the metal reached the goal temperature, the stopper rod was raised to allow the molten  
167 metal to flow into the mold. At the time of pour, the temperature at the top of the mold  
168 was approximately 850 °C, the temperature at the bottom of the mold was approximately  
169 650 °C, and the temperature of the molten metal had reached 1300 °C. The casting  
170 remained in the furnace for approximately 18 hours to cool. After 18 hours, the casting  
171 was dropped out of the furnace, moved to a breakout area, removed from the mold, and  
172 the outside surface was cleaned by scrubbing with water and acetone. The final cast part  
173 weighed 119.40 kg including the central graphite rod. The final completed total casting  
174 was 14-inches (35.6 cm) tall with a 1.75-inch-tall (4.4 cm) hot top (**Fig. 3**).

175

176 Immediately following breakout and cleaning, autoradiography was used to examine the  
177 spatial distribution of radioactivity within the cast depleted uranium metal. Two slim  
178 photophosphor imaging films (Fujifilm BAS-SR) were used to collect radiographs from  
179 the cast metal. One film was taped to the exterior of the cast uranium rod and the second  
180 film was placed into the central hole within the cylindrical rod. The films were exposed  
181 to the cast uranium rod for five minutes each. After exposure, the films were removed  
182 and enclosed in a light-tight cassette, transported to a radiochemistry laboratory, and  
183 scanned using an Amersham™ Typhoon™ radiography imaging scanner (**Fig. 3**).

184

### 185 *Sampling for destructive analysis*

186 The casting was stored until June 2022 to allow  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  to ingrow in the freshly  
187 cast depleted uranium in sufficient quantities for analysis. Two-gram drill chips, similar  
188 to those taken for feedstock characterization, were taken from the cast product for  
189 radiochronometry. Samples from the cast depleted uranium rod were taken from 7  
190 spatially different locations representing a gradient from the hot top to the base of the  
191 casting (**Fig. 4**).

192

193 ***Radiochronometry – Radiochemical Purification and Mass Spectrometry***

194 Radiochemistry was used to purify  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  from the depleted uranium feedstock  
195 and cast product drilled chip samples. Approximately 1-gram subaliquots of depleted  
196 uranium metal from each of 8 feedstock plate samples (S-1-A, S-3-A, S-6-A, S-8-A, S-  
197 10-A, S-12-A, S-13-A, and S-14-A) and from each of the 7 cast product samples (HT1a,  
198 HT2a, 1a, 2a, 3a, 4a, and 5a) were gravimetrically weighed into Savillex™ PFA jars. The  
199 metals were dissolved using 8 M  $\text{HNO}_3$  in a chemical fume hood on a hotplate at  $140^\circ\text{C}$ .  
200 Once in solution, samples were diluted to create 4 M  $\text{HNO}_3$  + 0.005 M HF primary  
201 sample solutions – HF was added to keep  $^{230}\text{Th}$  in solution. Serial dilutions of each  
202 sample were made using 4 M  $\text{HNO}_3$  + 0.005 M to produce sample solutions with ng  
203 uranium / g solution concentrations for uranium assay and isotope composition  
204 determination.

205

206 For  $^{230}\text{Th}$  assay determination, aliquots of each sample primary solution providing 20 to  
207 75 pg of  $^{230}\text{Th}$  for analysis were gravimetrically weighed into Savillex™ PFA vials. For  
208 all radiochemical purifications (thorium, protactinium, uranium) feedstock samples and  
209 cast product samples were processed separately. For each set of samples, a process blank  
210 was added. An aliquot of New Brunswick Laboratory certified reference material (CRM)  
211 125-A, previously dissolved at Los Alamos National Laboratory, was also processed  
212 through chemistry with the cast product sample set for radiochronometry quality control.  
213 All thorium aliquots were spiked with 40 to 75 pg of  $^{229}\text{Th}$  from  $^{229}\text{Th}$  NFRM Th-1 [13]  
214 for isotope dilution mass spectrometry. Thorium was purified from the bulk depleted  
215 uranium feedstock using a 3-column anion BioRad® AG 1-X8 anion exchange resin  
216 procedure. For the first two separations, samples were loaded in 8 M  $\text{HNO}_3$ , columns  
217 were washed with 8 M  $\text{HNO}_3$  and thorium was eluted with 9 M HCl. For the final  
218 column, the thorium samples were reconstituted in 9 M HCl + 0.01 M HF, loaded onto  
219 the columns, and collected immediately. Due to the higher uranium matrix mass in the  
220 cast product thorium fraction (approximately 250-300 mg of total uranium) the cast  
221 product samples were purified using a 3-column procedure that utilized both BioRad®  
222 AG 1-X8 and Eichrom® TEVA resins. The first column loaded the sample onto a 2 mL  
223 BioRad® AG 1-X8 column in 9 M HCl, whereby uranium was sorbed to the resin and

224 thorium was directly eluted. Thorium fractions were dried and reconstituted in 2 M HNO<sub>3</sub>  
225 for the second column. The thorium fraction was loaded onto 2 mL Eichrom® TEVA  
226 resin beds in 2 M HNO<sub>3</sub>. Uranium was washed from the columns using 2 M HNO<sub>3</sub> and  
227 thorium was eluted with 9 M HCl. This column was repeated to remove additional  
228 uranium. All purified thorium fractions were dried and reconstituted in 2% HNO<sub>3</sub> + 0.005  
229 M HF for analysis by multi-collector inductively-coupled plasma mass spectrometry  
230 (MC-ICP-MS).

231

232 Separate uranium aliquots were taken for uranium assay and uranium isotope  
233 composition determination. Aliquots of each sample providing approximately 10 ng of  
234 total uranium for assay determination by isotope dilution were weighed gravimetrically  
235 into Savillex™ PFA vials. An aliquot of standard reference material (SRM) 960 (also  
236 known as CRM 112-A), a uranium metal standard previously dissolved at Los Alamos  
237 National Laboratory, was also taken for uranium assay quality control and processed  
238 through chemistry with the feedstock samples. Samples were spiked with 1 ng of <sup>233</sup>U.  
239 For uranium isotope composition measurements, aliquots of each sample providing  
240 approximately 30 to 50 ng of total uranium were weighed gravimetrically into Savillex™  
241 PFA vials. All uranium fractions were purified using Eichrom® UTEVA resin. Samples  
242 were loaded onto the columns in 3 M HNO<sub>3</sub>, and sample impurities were rinsed from the  
243 resin with 3 M HNO<sub>3</sub> followed by 9 M HCl and 5 M HCl. Uranium was then eluted from  
244 the columns using 0.1 M HCl. The samples were dried on a hotplate and reconstituted in  
245 2% HNO<sub>3</sub> for MC-ICP-MS.

246

247 For <sup>231</sup>Pa assay determination, aliquots providing single-pg <sup>231</sup>Pa quantities of each  
248 sample were weighed gravimetrically into Savillex™ PFA vials. An aliquot of New  
249 Brunswick Laboratory certified reference material (CRM) 125-A was also processed  
250 through chemistry with the cast product sample set for radiochronometry quality control.  
251 Samples were spiked with a <sup>233</sup>Pa spike for isotope dilution mass spectrometry. The <sup>233</sup>Pa  
252 spike was produced at Los Alamos National Laboratory by separating the <sup>231</sup>Pa from a  
253 <sup>237</sup>Np stock solution. The produced spike was calibrated twice with the <sup>231</sup>Pa nuclear  
254 forensic reference material NFRM Pa-1 [14] as it decayed (<sup>233</sup>Pa half-life = 26.975 ±

255 0.026 days [15]). A  $^{233}\text{Pa}$  spike concentration determined from the average of the two  
256 spike calibrations was used for isotope dilution calculations. Protactinium was purified  
257 from the bulk depleted uranium matrix using a 3-column procedure. The first column  
258 consisted of 2 mL of BioRad® AG 1-X8 anion resin. The samples were loaded onto  
259 columns in 9 M HCl with trace concentrated  $\text{HNO}_3$  and trace saturated  $\text{H}_3\text{BO}_3$ .  
260 Protactinium was sorbed onto the anion resin and the matrix was washed from the resin  
261 with 9 M HCl. Protactinium was eluted in 9 M HCl + 0.05 M HF. The samples were  
262 dried and reconstituted in 2%  $\text{HNO}_3$  + trace saturated  $\text{H}_3\text{BO}_3$  and loaded onto 2 mL of  
263 silica gel resin. The matrix was washed from the resin with 2%  $\text{HNO}_3$  and protactinium  
264 was eluted using 2%  $\text{HNO}_3$  + 0.05 M HF. The silica gel column was repeated as a final  
265 purification to remove ingrown  $^{233}\text{U}$ . The purified protactinium fractions in 2%  $\text{HNO}_3$  +  
266 0.05 M HF were measured by mass spectrometry on the same day as the final purification  
267 column to prevent isobaric interference from ingrown  $^{233}\text{U}$  on mass 233 during sample  
268 analysis.

269

270 Purified thorium, uranium, and protactinium sample fractions were analyzed using a  
271 Thermo Scientific Neptune Plus MC-ICP-MS. Samples were introduced into the mass  
272 spectrometer as solutions (2 %  $\text{HNO}_3$  + 0.005 M HF, 2 %  $\text{HNO}_3$ , and 2 %  $\text{HNO}_3$  + 0.05  
273 M HF, respectively) and acid blank solutions were analyzed prior to each sample. For  
274 thorium analysis,  $^{229}\text{Th}$ ,  $^{230}\text{Th}$ , and  $^{232}\text{Th}$  were measured on Faraday detectors using a  
275 static measurement routine. Faraday gain calibrations were performed just prior to the  
276 start of the analytical session using an internally supplied voltage. Certified reference  
277 material IRMM 074/1 was used to calculate instrumental mass bias corrections.

278

279 For uranium isotope composition measurements, all mass bias and detector gain  
280 calculations were performed using bracketing standards of New Brunswick Laboratory  
281 CRM U010. Retarding Potential Quadrupole energy filters (RPQs) were used to decrease  
282 the contribution of low-mass tailing from  $^{238}\text{U}$  on  $^{236}\text{U}$ . Residual mass tailing effects were  
283 assessed by measuring  $^{236}\text{U}$ ,  $^{234}\text{U}$ , and  $^{233}\text{U}$  counts at four off-peak masses (-0.5, -0.35,  
284 +0.35, +0.5 amu away from peak center), fitting the points to an exponential curve, and  
285 then subtracting the tail contribution from the measured signal. In addition to CRM 125-

286 A that was processed through chemical purification with the samples, IRMM-183 and  
287 SRM 960 solutions were also measured during analysis for quality control. For uranium  
288 isotope dilution mass spectrometry,  $^{233}\text{U}$  and  $^{238}\text{U}$  were measured on Faraday detectors  
289 and IRMM 074/1 was used to calculate instrumental mass bias corrections.

290

291 Protactinium measurements were made using a static multi-ion counting routine. There  
292 are no certified isotopic protactinium reference materials available so uranium certified  
293 reference material CRM U010 was used to calculate mass bias and detector gains. Low-  
294 mass tailing from  $^{238}\text{U}$  and  $^{235}\text{U}$  on  $^{236}\text{U}$  and  $^{234}\text{U}$  was corrected as described above for  
295 uranium analyses. Certified reference material U005-A was measured for quality control  
296 of the mass bias and gain corrections applied to the protactinium sample fractions. An  
297 internal LANL thorium standard, LATH-1, was used to monitor for potential hydride  
298 interference ( $^{232}\text{Th}^1\text{H}$ ) on  $^{233}\text{Pa}$ . No hydride corrections at mass 233—potentially  
299 resulting from the natural  $^{232}\text{Th}$  chemistry blanks—were necessary for this analysis.  
300 Certified reference material CRM 125-A that was processed through chemistry was  
301 measured with the samples for Pa quality control.

## 302 **Results and discussion**

303 A photograph of the depleted uranium cast rod produced in this study and radiographs  
304 collected after casting on the exterior and interior of the uranium rod are provided in **Fig.**  
305 **3**. Increased activity near the hot top of the casting was observed through  
306 autoradiography (**Fig. 3B**). Due to the limited range that radiation can traverse through  
307 solids to generate an autoradiography image (ranging from approximately tens of  $\mu\text{m}$  for  
308 alpha particles to several cm of sample volume for gamma radiation [16]),  
309 autoradiography results cannot be used to determine whether the observed excess activity  
310 near the hot top persists through the volume of the hot top or is constrained to the surface.  
311 Dimensions of the depleted uranium cast rod, sampling locations, and exemplar optical  
312 microscope images of the casting samples are provided in **Fig. 4**. Samples for destructive  
313 analysis were taken from 7 different sampling locations. Two samples were taken from

314 the hot top of the casting: HT1a and HT2a. Five samples were taken from the cast rod  
315 from top to bottom: 1a, 2a, 3a, 4a, and 5a (**Fig. 4**).

316



317

318 **Fig. 3** Left – photograph of the as-cast depleted uranium rod after breakout and cleaning.

319 Right – A: Radiograph collected on the exterior of the depleted uranium metal rod; B:

320 Radiograph collected on the interior of the depleted uranium metal rod. Both radiographs

321 were produced from a 5-minute exposure of the metal to the imaging film. Radioactivity

322 is shown by an increase in warm colors of each pixel (towards red). An increase in

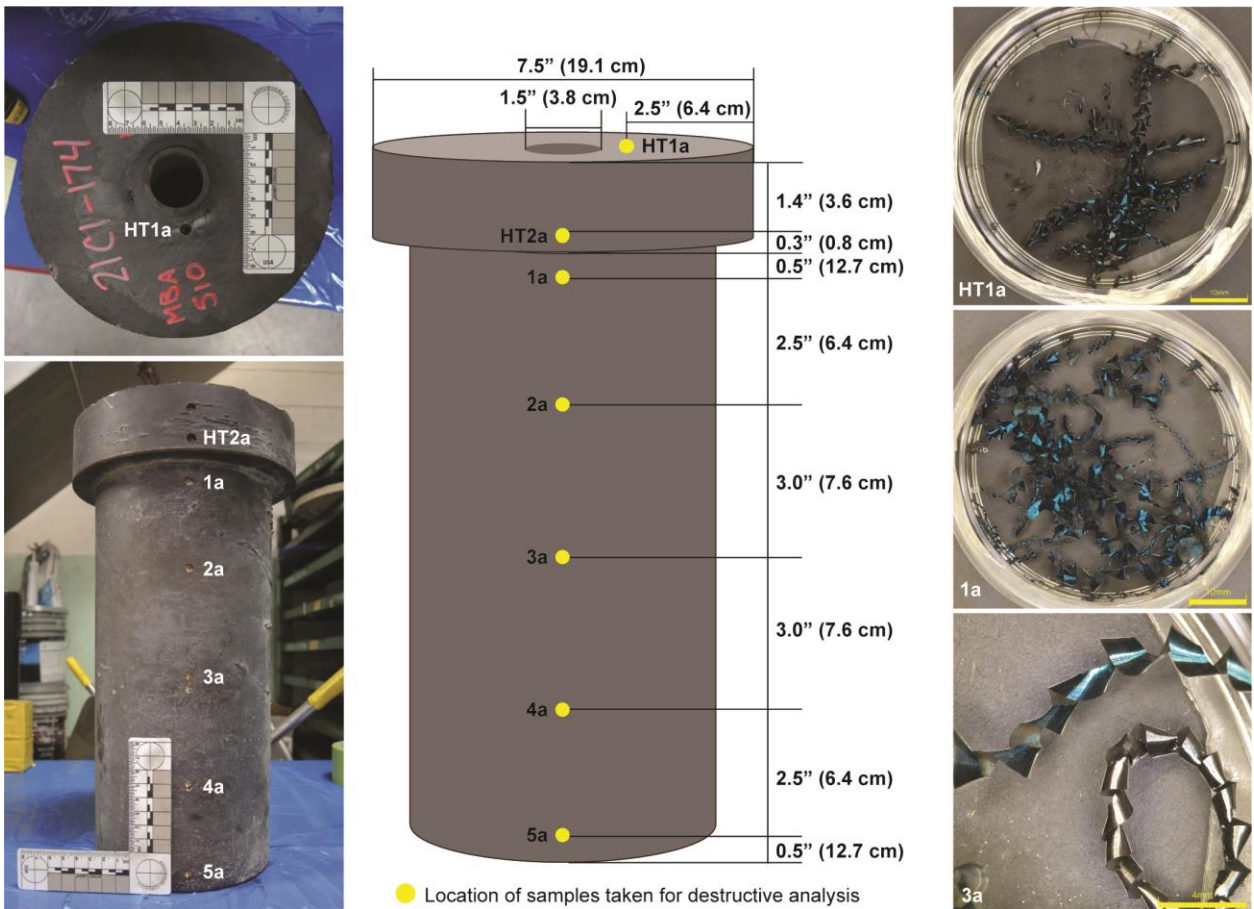
323 radioactivity near the upper portion (hot top) of the casting is visible primarily in the

324 interior film radiograph. Background radiation/exposure is shown in blue. Radioactivity

325 intensity scales from green (low) to (red) and is qualitative rather than quantitative.

326

327



328

329

**Fig. 4** Left – two photographs of the casting taken after sampling for destructive analysis.

330

The locations where samples were taken can be seen as holes in the surface of the casting

331

and each location is labeled with an associated sample identification number. The scale

332

bar shown is in units of cm. Middle – a schematic of the casting with dimensions between

333

sampling locations provided in both inches (") and cm. Right – three exemplar optical

334

microscopy images of HT1a, 1a, and 3a drilled chips taken for U, Th, and Pa analysis. No

335

notable visual differences were observed between the hot top drilled chips and the bulk

336

casting drilled chips. All samples had a metallic luster and demonstrated a range of minor

337

surface oxidation.

338

339

Uranium assay and isotope composition data as well as measured  $^{230}\text{Th}$  and  $^{231}\text{Pa}$

340

concentrations are provided in **Tables 1, 2, and 3**, respectively. Uranium isotope

341

compositions measured in the feedstock and casting samples are highly consistent and, in

342

general, agree between samples within analytical uncertainty (**Table 1**). The samples

343

have  $^{234}\text{U}/^{238}\text{U}$  between  $0.000007390 \pm 0.000000064$  and  $0.000007464 \pm 0.000000048$

344 and  $^{235}\text{U}/^{238}\text{U}$  between  $0.0020178 \pm 0.0000029$  and  $0.0020241 \pm 0.0000029$ . One sample  
 345 of the feedstock uranium plates, S-12-A, has a slightly elevated  $^{235}\text{U}/^{238}\text{U}$  relative to the  
 346 other samples which may represent very minor laboratory contamination during sampling  
 347 in the foundry or during radiochemistry. All samples have detectable  $^{236}\text{U}$  consistent with  
 348 feedstock produced from enrichment tails resulting from enriching uranium that included  
 349 a component of recycled reprocessed reactor fuel. The measured uranium isotope  
 350 compositions of the feedstock are consistent with the known history of the depleted  
 351 uranium and the uranium composition of the feedstock was not altered by VIM casting  
 352 (**Table 1**). Uranium concentrations of the feedstock and casting samples ranged from  
 353  $0.9919 \pm 0.0010$  to  $0.99933 \pm 0.00095$  grams of uranium per gram of metal; therefore, the  
 354 feedstock and casting were comprised of very pure (> 99%) depleted uranium. Measured  
 355 uranium isotope compositions for quality control standards CRM 125-A and SRM 960  
 356 agreed with the certified uranium compositions for these standards. The measured  
 357 uranium concentration of SRM 960 also agreed with the certified uranium assay of this  
 358 standard (also known as CRM 112-A). Process blanks handled with the samples during  
 359 uranium purification and analysis contained between 5 and 9 pg of total U; therefore,  
 360 laboratory blanks were insignificant relative to the 10 to 50 ng uranium aliquot sizes.  
 361

362 **Table 1** Feedstock and Casting Uranium Isotope Compositions and Concentrations

Sample ID	Sample Source	$^{234}\text{U}/^{238}\text{U}$	Unc. (k=2)	$^{235}\text{U}/^{238}\text{U}$	Unc. (k=2)	$^{236}\text{U}/^{238}\text{U}$	Unc. (k=2)	g U / g metal	Unc. (k=2)
S-1-A	Feedstock	0.000007418	0.000000074	0.0020188	0.0000026	0.00002886	0.00000015	0.99788	0.00085
S-3-A	Feedstock	0.000007395	0.000000068	0.0020180	0.0000028	0.00002888	0.00000014	0.99466	0.00094
S-6-A	Feedstock	0.000007410	0.000000072	0.0020187	0.0000032	0.00002887	0.00000016	0.99933	0.00095
S-8-A	Feedstock	0.000007422	0.000000078	0.0020185	0.0000027	0.00002886	0.00000014	0.9978	0.0010
S-10-A	Feedstock	0.000007419	0.000000072	0.0020181	0.0000028	0.00002887	0.00000014	0.99359	0.00083
S-12-A	Feedstock	0.000007464	0.000000048	0.0020241	0.0000029	0.00002883	0.00000015	0.9967	0.0010
S-13-A	Feedstock	0.000007403	0.000000045	0.0020190	0.0000029	0.00002889	0.00000014	0.99800	0.00091
S-14-A	Feedstock	0.000007410	0.000000069	0.0020182	0.0000027	0.00002885	0.00000019	0.9948	0.0010
HT1a	Casting	0.000007405	0.000000092	0.0020182	0.0000030	0.00002887	0.00000015	0.9904	0.0010
HT2a	Casting	0.000007395	0.000000082	0.0020179	0.0000029	0.00002886	0.00000019	0.9930	0.0010
1a	Casting	0.000007413	0.000000058	0.0020181	0.0000035	0.00002886	0.00000014	0.9930	0.0010
2a	Casting	0.000007402	0.000000080	0.0020180	0.0000031	0.00002883	0.00000016	0.9925	0.0010
3a	Casting	0.000007390	0.000000064	0.0020178	0.0000029	0.00002886	0.00000015	0.9919	0.0010
4a	Casting	0.000007399	0.000000062	0.0020178	0.0000032	0.00002888	0.00000016	0.9963	0.0010
5a	Casting	0.000007396	0.000000067	0.0020178	0.0000031	0.00002890	0.00000017	0.9965	0.0010
CRM 125-A	QC Standard	0.0003921	0.0000013	0.042308	0.000044	0.000004027	0.000000050	0.88218	0.00081
CRM 125-A	QC Standard	0.0003919	0.0000013	0.042271	0.000044	0.000004037	0.000000039	0.88210	0.00081
SRM 960	QC Standard	0.00005292	0.00000024	0.0072513	0.0000078	< L <sub>D</sub>	< L <sub>D</sub>	1.0002	0.0010

363



364 Measured  $^{230}\text{Th}$  concentrations are consistent to within less than 1% variation between  
365 the different feedstock plates used as the charge for VIM casting. The average  
366 concentration of  $^{230}\text{Th}$  in the 8 feedstock plates analyzed was  $0.7369 \pm 0.0012 \text{ ng } ^{230}\text{Th}$   
367 per gram of depleted uranium metal (**Table 2**). Thorium-230 concentrations in the casting  
368 ranged from  $0.13942 \pm 0.00025$  (sample 2a) to  $0.16565 \pm 0.00032$  (sample 5a)  $\text{ng } ^{230}\text{Th}$   
369 per gram of cast metal (**Table 2**). Process blank  $^{230}\text{Th}$  concentrations were below the  
370 limit of detection on a Faraday detector. The concentrations of  $^{230}\text{Th}$  in the cast metal  
371 represent between 18.5 and 22.4 % of the original average  $^{230}\text{Th}$  present in the feedstock.  
372 Therefore, VIM casting removed between 77.6 and 81.6 % of the  $^{230}\text{Th}$  (**Table 2**).  
373 Assuming 1) a charge mass of 120 kg, 2) an average  $^{230}\text{Th}$  concentration in the feedstock  
374 of  $0.7369 \pm 0.0012 \text{ ng } ^{230}\text{Th}$  per gram of depleted uranium metal, and 3) a range of VIM  
375 casting  $^{230}\text{Th}$  removal from 77.6 to 81.6 %, of the approximately 88 mg of  $^{230}\text{Th}$  present  
376 in the uranium metal feedstock, approximately 69 to 72 mg was removed during the  
377 casting process. Interestingly, very little spatial variation in  $^{230}\text{Th}$  concentration was  
378 observed with height in the casting. Slightly higher  $^{230}\text{Th}$  concentrations are shown in the  
379 hot top (HT1a and HT2a) and base of the casting (5a) relative to the center height of the  
380 casting (3a); however, this variation is only on the order of  $\sim 2 \%$ .

381

382 One previous hypothesis for  $^{230}\text{Th}$  removal during uranium metal casting was buoyancy  
383 of the less-dense  $^{230}\text{Th}$  relative to the bulk uranium metal during the molten stage of the  
384 casting [6, 10]. This would result in a vertical gradient or separation of  $^{230}\text{Th}$  to the hot  
385 top of the casting. The data presented here suggest that  $^{230}\text{Th}$  is partially removed during  
386 VIM casting, but if the  $^{230}\text{Th}$  is concentrated in the hot top through floatation, it is not  
387 homogeneously distributed within the hot top because our 2 discrete hot top samples did  
388 not detect the presence of excess accumulated  $^{230}\text{Th}$ . Another possible explanation for the  
389 lowered  $^{230}\text{Th}$  concentration in the cast product compared to the feedstock could be  $^{230}\text{Th}$   
390 accumulation in the crucible, dross, or skull during casting [12]. While buoyancy driven  
391 segregation is possible, this effect would tend to be small since the driving forces are low  
392 and the melt is far from quiescent in either the crucible or the casting. Another likely  
393 mechanism is pre-wetting of the surface as is observed in Al-In [17] and related  
394 phenomena observed in other monotectic systems. Total segregation would be highly

395 dependent on fluid residence time, which would be highly exaggerated in the crucible and  
 396 result in a weak vertical gradient effect in the casting. Unfortunately, samples were not  
 397 taken from the crucible, dross, or skull in this study, but should be targets of future  
 398 research.

399

400 **Table 2** Feedstock and Casting  $^{230}\text{Th}$  Concentrations

Feedstock			Casting			Difference
Sample ID	ng $^{230}\text{Th}$ / g metal	Unc. (k=2)	Sample ID	ng $^{230}\text{Th}$ / g metal	Unc. (k=2)	Percent (%) of Feedstock average $^{230}\text{Th}$
S-1-A	0.7401	0.0012	HT1a	0.14780	0.00032	20.0
S-3-A	0.7386	0.0013	HT2a	0.14756	0.00031	20.0
S-6-A	0.7404	0.0013	1a	0.14919	0.00025	20.2
S-8-A	0.7383	0.0013	2a	0.13942	0.00025	18.9
S-10-A	0.7391	0.0013	3a	0.13637	0.00028	18.5
S-12-A	0.7350	0.0013	4a	0.14194	0.00026	19.2
S-13-A	0.7395	0.0012	5a	0.16565	0.00032	22.4
S-14-A	0.7369	0.0012				
<b>Average</b>	<b>0.7385</b>	<b>0.0036</b>				

401 <sup>a</sup> Uncertainty on the reported feedstock average is calculated as the standard deviation of the  
 402 population at the 95% confidence level ( $2\sigma$ ).

403

404 The measured  $^{231}\text{Pa}$  concentrations in the 8 feedstock plate samples ranged from  $93.2 \pm$   
 405  $1.0$  to  $94.2 \pm 1.0$  pg  $^{231}\text{Pa}$  per gram of metal demonstrating a homogenous  $^{231}\text{Pa}$   
 406 distribution in the feedstock used for VIM casting (**Table 3**). The chemical homogeneity  
 407 observed in both  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  concentrations in the feedstock plates is consistent with  
 408 the expected history of the feedstock which included casting of a large ingot and  
 409 subsequent warm rolling. In contrast to  $^{230}\text{Th}$ ,  $^{231}\text{Pa}$  concentrations in the depleted  
 410 uranium metal were relatively unchanged by the VIM casting process (**Table 3**).

411 Measured  $^{231}\text{Pa}$  concentrations in the casting samples ranged from  $92.0 \pm 0.6$  to  $93.5 \pm$   
 412  $0.6$  pg  $^{231}\text{Pa}$  per gram of metal, which represents retention of between 98.1 and 99.8 % of  
 413 the  $^{231}\text{Pa}$  from the average feedstock after VIM casting (**Table 3**). Process blank  $^{231}\text{Pa}$   
 414 concentrations were below the instrument limit of detection.

415

416

417 **Table 3** Feedstock and Casting <sup>231</sup>Pa Concentrations

Feedstock			Casting			Difference
Sample ID	pg <sup>231</sup> Pa / g metal	Unc. (k=2)	Sample ID	pg <sup>231</sup> Pa / g metal	Unc. (k=2)	Percent (%) of Feedstock average <sup>231</sup> Pa
S-1-A	94.2	1.0	HT1a	92.2	0.6	98.3
S-3-A	94.1	1.1	HT2a	92.0	0.6	98.1
S-6-A	94.1	1.0	1a	92.2	0.6	98.4
S-8-A	93.8	1.0	2a	92.7	0.6	98.9
S-10-A	93.4	1.0	3a	93.3	0.6	99.6
S-12-A	93.5	1.0	4a	93.5	0.6	99.8
S-13-A	93.6	1.0	5a	92.8	0.6	99.0
S-14-A	93.2	1.0				
<b>Average</b>	<b>93.73</b>	<b>0.73</b>				

418 <sup>a</sup> Uncertainty on the reported feedstock average is calculated as the standard deviation of the  
 419 population at the 95% confidence level (2σ).

420

421 The observation in this study that the <sup>231</sup>Pa concentration of the feedstock uranium metal  
 422 is relatively unchanged during VIM casting is consistent with data from another paired  
 423 feedstock and VIM uranium metal casting sample published by Higginson et al. 2022 [9].  
 424 Therefore, repeated observations of <sup>231</sup>Pa during uranium production by VIM suggest that  
 425 the <sup>231</sup>Pa decay product of <sup>235</sup>U is essentially unpurified from uranium metal during VIM  
 426 casting. Retention of the feedstock <sup>231</sup>Pa signature through casting, therefore, may  
 427 constitute a powerful nuclear forensic signature that can provide provenance assessment  
 428 teams with a predictive signature, and potential high-fidelity model age, of the source  
 429 feedstock that was used to produce a uranium metal.

430

431 To further explore the effect of VIM casting on uranium model ages, all measured  
 432 uranium, thorium, and protactinium concentration data are provided in units of atoms per  
 433 gram uranium metal in **Table 4**. The concentrations provided in **Table 4** can be  
 434 combined with the model age equation (**Eq 1**) and reference dates to calculate model ages  
 435 and model purification dates for each sample using the <sup>230</sup>Th/<sup>234</sup>U and <sup>231</sup>Pa/<sup>235</sup>U  
 436 radiochronometers.

437

438

$$t = \frac{1}{\lambda_P - \lambda_D} \ln\left(1 - \frac{ND}{NP} \times \frac{\lambda_D - \lambda_P}{\lambda_P}\right) \quad \text{Eq. 1}$$

439

440 In the equation above,  $t$  is equivalent to the model age or time since last chemical  
 441 purification,  $\lambda_D$  and  $\lambda_P$  represent the decay constants of the daughter and parent  
 442 nuclides, respectively, and  $ND/NP$  is the measured daughter/parent atom ratio. Model  
 443 ages were calculated using half-lives from [18, 19, 20]. The calculated  $^{230}\text{Th}/^{234}\text{U}$  and  
 444  $^{231}\text{Pa}/^{235}\text{U}$  model ages and model purification dates for both feedstock and casting  
 445 samples are provided in **Table 5** and are shown graphically in **Fig. 5**.  
 446

447 **Table 4** Measured  $^{234}\text{U}$ ,  $^{235}\text{U}$ ,  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  Concentrations for Radiochronometry

Sample ID	Sample Source	$^{234}\text{U}$ atoms / g metal	Unc. (k=2)	$^{235}\text{U}$ atoms / g metal	Unc. (k=2)	$^{230}\text{Th}$ atoms / g metal <sup>a</sup>	Unc. (k=2)	$^{231}\text{Pa}$ atoms / g metal <sup>b</sup>	Unc (k=2)
S-1-A	Feedstock	$1.869 \times 10^{16}$	$1.9 \times 10^{14}$	$5.0860 \times 10^{18}$	$7.8 \times 10^{15}$	$1.9376 \times 10^{12}$	$3.1 \times 10^9$	$2.455 \times 10^{11}$	$2.7 \times 10^9$
S-3-A	Feedstock	$1.857 \times 10^{16}$	$1.7 \times 10^{14}$	$5.0676 \times 10^{18}$	$8.6 \times 10^{15}$	$1.9336 \times 10^{12}$	$3.3 \times 10^9$	$2.454 \times 10^{11}$	$2.8 \times 10^9$
S-6-A	Feedstock	$1.869 \times 10^{16}$	$1.8 \times 10^{14}$	$5.0931 \times 10^{18}$	$9.4 \times 10^{15}$	$1.9384 \times 10^{12}$	$3.4 \times 10^9$	$2.452 \times 10^{11}$	$2.7 \times 10^9$
S-8-A	Feedstock	$1.870 \times 10^{16}$	$2.0 \times 10^{14}$	$5.0846 \times 10^{18}$	$8.6 \times 10^{15}$	$1.9327 \times 10^{12}$	$3.4 \times 10^9$	$2.446 \times 10^{11}$	$2.7 \times 10^9$
S-10-A	Feedstock	$1.861 \times 10^{16}$	$1.8 \times 10^{14}$	$5.0623 \times 10^{18}$	$8.1 \times 10^{15}$	$1.9348 \times 10^{12}$	$3.3 \times 10^9$	$2.435 \times 10^{11}$	$2.7 \times 10^9$
S-12-A	Feedstock	$1.878 \times 10^{16}$	$1.2 \times 10^{14}$	$5.0933 \times 10^{18}$	$9.0 \times 10^{15}$	$1.9241 \times 10^{12}$	$3.4 \times 10^9$	$2.436 \times 10^{11}$	$2.7 \times 10^9$
S-13-A	Feedstock	$1.865 \times 10^{16}$	$1.2 \times 10^{14}$	$5.0870 \times 10^{18}$	$8.7 \times 10^{15}$	$1.9360 \times 10^{12}$	$3.1 \times 10^9$	$2.438 \times 10^{11}$	$2.7 \times 10^9$
S-14-A	Feedstock	$1.861 \times 10^{16}$	$1.7 \times 10^{14}$	$5.0685 \times 10^{18}$	$8.6 \times 10^{15}$	$1.9292 \times 10^{12}$	$3.2 \times 10^9$	$2.430 \times 10^{11}$	$2.7 \times 10^9$
HT1a	Casting	$1.851 \times 10^{16}$	$2.3 \times 10^{14}$	$5.0460 \times 10^{18}$	$9.0 \times 10^{15}$	$3.8692 \times 10^{11}$	$8.3 \times 10^8$	$2.403 \times 10^{11}$	$1.7 \times 10^9$
HT2a	Casting	$1.854 \times 10^{16}$	$2.1 \times 10^{14}$	$5.0590 \times 10^{18}$	$9.0 \times 10^{15}$	$3.8630 \times 10^{11}$	$8.2 \times 10^8$	$2.397 \times 10^{11}$	$1.6 \times 10^9$
1a	Casting	$1.859 \times 10^{16}$	$1.5 \times 10^{14}$	$5.059 \times 10^{18}$	$1.0 \times 10^{16}$	$3.9058 \times 10^{11}$	$6.7 \times 10^8$	$2.404 \times 10^{11}$	$1.6 \times 10^9$
2a	Casting	$1.855 \times 10^{16}$	$2.0 \times 10^{14}$	$5.0567 \times 10^{18}$	$9.2 \times 10^{15}$	$3.6501 \times 10^{11}$	$6.6 \times 10^8$	$2.417 \times 10^{11}$	$1.6 \times 10^9$
3a	Casting	$1.851 \times 10^{16}$	$1.6 \times 10^{14}$	$5.0530 \times 10^{18}$	$9.0 \times 10^{15}$	$3.5701 \times 10^{11}$	$7.4 \times 10^8$	$2.433 \times 10^{11}$	$1.6 \times 10^9$
4a	Casting	$1.861 \times 10^{16}$	$1.6 \times 10^{14}$	$5.0754 \times 10^{18}$	$9.7 \times 10^{15}$	$3.7160 \times 10^{11}$	$6.8 \times 10^8$	$2.438 \times 10^{11}$	$1.6 \times 10^9$
5a	Casting	$1.861 \times 10^{16}$	$1.7 \times 10^{14}$	$5.0763 \times 10^{18}$	$9.4 \times 10^{15}$	$4.3367 \times 10^{11}$	$8.4 \times 10^8$	$2.419 \times 10^{11}$	$1.6 \times 10^9$
CRM 125-A	QC Standard	$8.397 \times 10^{17}$	$2.9 \times 10^{15}$	$9.060 \times 10^{19}$	$1.3 \times 10^{17}$	$6.697 \times 10^{13}$	$1.3 \times 10^{11}$	$2.514 \times 10^{12}$	$1.7 \times 10^{10}$
CRM 125-A	QC Standard	$8.391 \times 10^{17}$	$2.8 \times 10^{15}$	$9.051 \times 10^{19}$	$1.3 \times 10^{17}$	$6.692 \times 10^{13}$	$1.5 \times 10^{11}$	$2.497 \times 10^{12}$	$1.6 \times 10^{10}$

448 <sup>a</sup> Reference dates for  $^{230}\text{Th}$  atoms / g metal concentrations in the feedstock and casting samples  
 449 are August 30, 2022 and September 20, 2022, respectively

450 <sup>b</sup> Reference dates for  $^{231}\text{Pa}$  atoms / g metal concentrations in the feedstock and casting samples are  
 451 February 9, 2022 and September 8, 2022, respectively.  
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458

459 **Table 5** Calculated  $^{230}\text{Th}/^{234}\text{U}$  and  $^{231}\text{Pa}/^{235}\text{U}$  Model Ages and Model Purification Dates

Sample ID	Sample Source	$^{230}\text{Th}/^{234}\text{U}$ Model Age (years)	Unc. (k-2, years)	$^{231}\text{Pa}/^{235}\text{U}$ Model Age (years)	Unc. (k-2, years)	$^{230}\text{Th}/^{234}\text{U}$ Model Purification Date	Unc. (k-2, days)	$^{231}\text{Pa}/^{235}\text{U}$ Model Purification Date	Unc. (k-2, days)
S-1-A	Feedstock	36.75	0.37	49.03	0.55	11/30/85	136	01/29/73	200
S-3-A	Feedstock	36.90	0.35	49.19	0.58	10/04/85	127	12/01/72	211
S-6-A	Feedstock	36.75	0.37	48.91	0.55	11/30/85	134	03/16/73	202
S-8-A	Feedstock	36.64	0.40	48.87	0.54	01/10/86	145	03/29/73	199
S-10-A	Feedstock	36.85	0.37	48.86	0.55	10/25/85	134	03/31/73	200
S-12-A	Feedstock	36.31	0.25	48.59	0.55	05/09/86	90	07/09/73	199
S-13-A	Feedstock	36.78	0.24	48.70	0.54	11/17/85	87	05/31/73	199
S-14-A	Feedstock	36.74	0.35	48.71	0.55	12/03/85	128	05/26/73	200
HT1a	Casting	7.405	0.094	48.38	0.35	04/25/15	34	04/24/74	129
HT2a	Casting	7.384	0.084	48.13	0.33	05/03/15	31	07/23/74	120
1a	Casting	7.447	0.061	48.27	0.34	04/10/15	22	06/01/74	123
2a	Casting	6.973	0.077	48.55	0.33	09/30/15	28	02/20/74	121
3a	Casting	6.836	0.062	48.92	0.34	11/19/15	22	10/09/73	126
4a	Casting	7.075	0.062	48.79	0.34	08/23/15	23	11/24/73	125
5a	Casting	8.259	0.078	48.41	0.33	06/17/14	28	04/10/74	122
CRM 125-A	QC Standard	28.27	0.12	28.18	0.19	06/14/94	43	07/05/94	70
CRM 125-A	QC Standard	28.27	0.12	28.02	0.19	06/15/94	43	09/02/94	69

460 <sup>a</sup> Model purification dates are presented in MM/DD/YY format

461 <sup>b</sup> Reference dates for model ages and model purification dates are as follows:

462 Feedstock  $^{230}\text{Th}/^{234}\text{U}$  = August 30, 2022

463 Casting  $^{230}\text{Th}/^{234}\text{U}$  = September 20, 2022

464 Feedstock  $^{231}\text{Pa}/^{235}\text{U}$  = February 9, 2022

465 Casting  $^{231}\text{Pa}/^{235}\text{U}$  = September 8, 2022

466

467

468 Calculated  $^{230}\text{Th}/^{234}\text{U}$  and  $^{231}\text{Pa}/^{235}\text{U}$  model ages demonstrate discordant uranium model

469 ages for both the feedstock and casting. As discussed in the introduction to this work,

470 discordant  $^{230}\text{Th}/^{234}\text{U}$  and  $^{231}\text{Pa}/^{235}\text{U}$ —with  $^{231}\text{Pa}/^{235}\text{U}$  model ages biased older than

471  $^{230}\text{Th}/^{234}\text{U}$  model ages—are often observed during nuclear forensic investigations of

472 uranium metals [6, 9, 10]. Therefore, data presented from this controlled uranium metal

473 casting experiment are consistent with historically measured uranium metal

474 radiochronometric data.

475

476 Calculated  $^{230}\text{Th}/^{234}\text{U}$  model purification dates for the depleted uranium feedstock

477 samples range from October 4, 1985  $\pm$  127 days to May 9, 1986  $\pm$  90 days, whereas

478  $^{231}\text{Pa}/^{235}\text{U}$  model purification dates are older and range from December 1, 1972  $\pm$  211

479 days to July 9, 1973  $\pm$  199 days (**Table 5**). Based on interpretation from previous studies

480 [9], the  $^{231}\text{Pa}/^{235}\text{U}$  model purification dates in 1973-1974 may more accurately reflect the

481 timing of uranium metal production by bomb reduction (derby production). It is possible

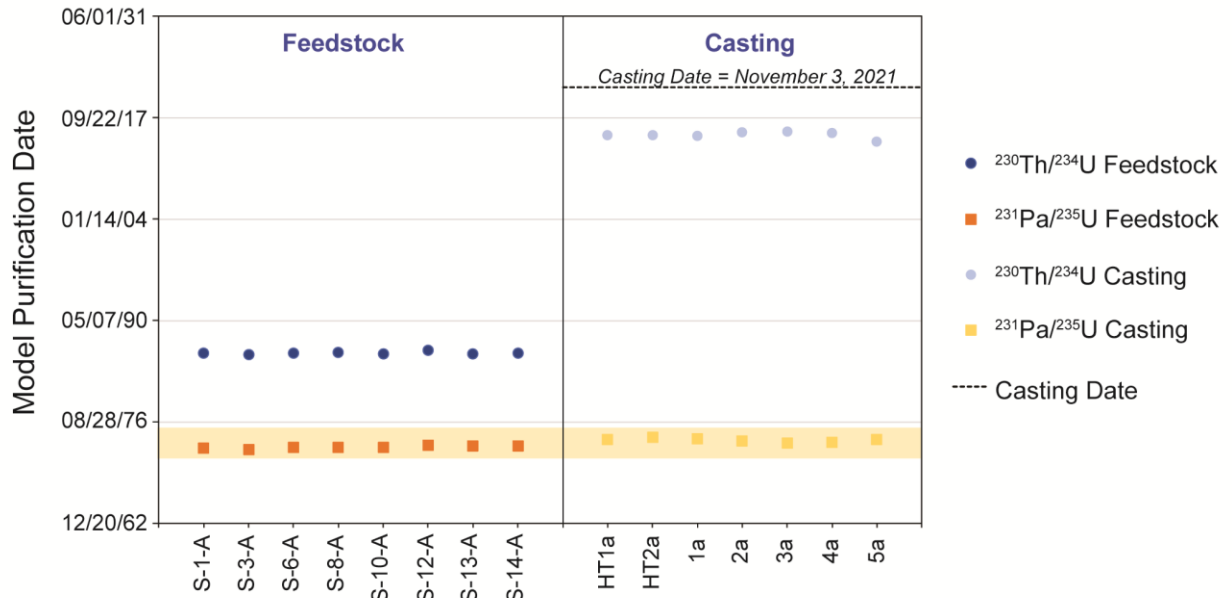
482 that the later  $^{230}\text{Th}/^{234}\text{U}$  model purification dates are more closely related to the timing of

483 the feedstock ingot casting and warm rolling potentially in 1985-1986, though we note  
484 that this interpretation is somewhat speculative because the exact history of the depleted  
485 uranium feedstock is unknown. Measured  $^{230}\text{Th}/^{234}\text{U}$  and  $^{231}\text{Pa}/^{235}\text{U}$  model ages of CRM  
486 125-A agree with the certified model purification date of CRM 125-A (August 19, 1994  
487  $\pm 116$  days) and provide quality control of reported radiochronometry data (**Table 5**).

488

489 Had the VIM casting at Sigma Facility completely purified the  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  decay  
490 progeny of bulk uranium, an expected model purification date of November 3, 2021  
491 would have been measured for uranium casting samples. However, neither the  $^{230}\text{Th}/^{234}\text{U}$   
492 or  $^{231}\text{Pa}/^{235}\text{U}$  model purification dates of the cast uranium metal correspond to this exact  
493 controlled VIM casting date (**Table 5**). Calculated  $^{230}\text{Th}/^{234}\text{U}$  model purification dates  
494 range from June 17, 2014  $\pm 28$  days to November 19, 2015  $\pm 22$  days and are biased 6.0  
495 to 7.4 years older than the known casting date of the uranium metal. This model  
496 purification date bias from the known casting date is the result of only 77.6 to 81.6 %  
497 purification of  $^{230}\text{Th}$  during VIM casting. Calculated  $^{231}\text{Pa}/^{235}\text{U}$  model purification dates  
498 of the cast uranium metal are similar to the model purification dates of the feedstock  
499 depleted uranium and range from October 9, 1973  $\pm 126$  days to July 23, 1974  $\pm 120$   
500 days. These measured post-VIM  $^{231}\text{Pa}/^{235}\text{U}$  model purification dates demonstrate  
501 signature preservation of the original feedstock  $^{231}\text{Pa}/^{235}\text{U}$  model ages to within an  
502 approximately 1-year timeline and are likely useful uranium metal nuclear forensic  
503 signatures for source attribution (**Fig. 5**).

504



505  
 506 **Fig. 5** Calculated  $^{230}\text{Th}/^{234}\text{U}$  (circles) and  $^{231}\text{Pa}/^{235}\text{U}$  (squares) model purification dates  
 507 for depleted uranium feedstock and casting samples. Model purification dates are  
 508 presented in MM/DD/YY format. Uncertainties are smaller than symbols. The controlled  
 509 VIM casting date is shown as a black dashed line. Preservation of the  $^{231}\text{Pa}/^{235}\text{U}$  model  
 510 purification age of the uranium through VIM casting is highlighted with a yellow shaded  
 511 region for model purification dates from 1973-1974. The calculated  $^{230}\text{Th}/^{234}\text{U}$  model  
 512 purification dates of the casting samples (light blue circles) are biased older than the  
 513 experimentally controlled VIM casting date of November 3, 2021.

514

## 515 Conclusions

516 A single-coil VIM furnace was used to cast an approximately 120 kg uranium rod on a  
 517 known date to investigate the separation of thorium and protactinium from uranium  
 518 during uranium metal casting, and quantify the behavior of the  $^{230}\text{Th}/^{234}\text{U}$  and  $^{231}\text{Pa}/^{235}\text{U}$   
 519 radiochronometry systems. The goal of this controlled study was to increase the nuclear  
 520 forensic community's understanding of the behavior of nuclides used for  
 521 radiochronometry to improve model age data interpretation to support nuclear forensic  
 522 investigations. Quantification of  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  in the feedstock and cast rod produced in

523 this study demonstrate that VIM casting purified 77.6 to 81.6 % of the original  $^{230}\text{Th}$   
524 present in the feedstock, whereas only 0.2 to 1.9 % of the feedstock  $^{231}\text{Pa}$  was purified  
525 from the uranium during casting. This differential separation of  $^{230}\text{Th}$  and  $^{231}\text{Pa}$  during  
526 metal casting is consistent with the previously observed discordance between  $^{230}\text{Th}/^{234}\text{U}$   
527 and  $^{231}\text{Pa}/^{235}\text{U}$  radiochronometric model ages of uranium metals [6, 9, 10]. Measured  
528  $^{230}\text{Th}/^{234}\text{U}$  model purification dates of the cast metal rod were biased approximately 6.0  
529 to 7.4 years older than the controlled VIM casting date of November 3, 2021. Measured  
530  $^{231}\text{Pa}/^{235}\text{U}$  model purification dates of the casting samples were similar to the feedstock  
531  $^{231}\text{Pa}/^{235}\text{U}$  model purification dates (1973 – 1974) demonstrating feedstock model age  
532 signature preservation through casting. More research is needed to investigate the  
533 reproducibility of thorium and protactinium separation from bulk uranium during VIM  
534 casting. In particular, investigation of thorium and protactinium signatures of uranium  
535 metals cast at different scales and geometries for elucidation of separation mechanism is  
536 a natural extension of this work. Mass balance studies to confirm the location of the  
537 separated thorium are also needed. Results presented here quantify thorium and  
538 protactinium behavior during uranium metal production and help increase confidence  
539 assigned to provenance assessment during nuclear forensic investigations.

540

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545

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