

Core Fueling of DEMO by Direct Line Injection of High-Speed Pellets From the HFS

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Abstract—Pellet injection represents to date the most realistic candidate technology for core fueling of a demonstration fusion power reactor tokamak fusion reactor. Modeling of both pellet penetration and fuel deposition profiles, for different injection locations, indicates that effective core fuelling can be achieved launching pellets from the inboard high field side at speeds not less than ~ 1 km/s. Inboard pellet fueling is commonly achieved in present tokamaks, using curved guide tubes; however, this technology might be hampered at velocities ≥ 1 km/s. An innovative approach, aimed at identifying suitable inboard “direct line” paths, to inject high-speed pellets (in the 3 to 4 km/s range), has recently been proposed as a potential complementary solution. The fuel deposition profiles achievable by this approach have been explored using the HPI2 simulation code. The results presented here show that there are possible geometrical schemes providing good fueling performance. The problem of neutron flux in a direct line-of-sight injection path is being investigated, though preliminary analyses indicate that, perhaps, this is not a serious problem. The identification and integration of straight injection paths suitably tilted may be a rather difficult task due to the many constraints and to interference with existing structures. The suitability of straight guide tubes to reduce the scatter cone of high-speed pellets is, therefore, of main interest. A preliminary investigation, aimed at addressing these technological issues, has recently been started. A possible implementation plan, using an existing Italian National Agency for New Technologies, Energy and Sustainable Economic Development-Oak Ridge National Laboratory facility is shortly outlined.

Index Terms—Demonstration fusion power reactor (DEMO) tokamak fusion reactor, high field side (HFS) high-speed pellet injection, straight guide tubes.

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I. INTRODUCTION

CORE fuelling of a demonstration fusion power reactor (DEMO) tokamak fusion reactor is under investigation within the EUROfusion Work Package “Tritium, Fuelling, and Vacuum” (WP-TFV). Extensive analyses of DEMO fuelling requirements and of related fuelling technologies presently available, indicate that pellet injection still represents, to date, the most realistic option. In general, the fuel deposition depth inside the plasma depends on two distinct phenomena, i.e., the pellet penetration that, for given target plasma profiles, is determined by the pellet mass and by its injection velocity, followed by a drift of the locally ablated material in the direction opposite to the gradient of the magnetic field. The physics of pellet fuelling of future tokamak fusion reactors, where the fuel deposition depth is strongly dominated by the ∇B -induced displacement, differs from that of present day machines, where the pellet penetration is typically larger than the drift effect, and may still play an important role [1].

Modeling of both pellet penetration and fuel deposition profiles for different injection locations, for specific DEMO plasma reference scenarios and assuming the International Thermonuclear Experimental Reactor (ITER) reference pellet mass (6×10^{21} atoms), indicates indeed that the pellet penetration is always very shallow, almost regardless of the pellet speed, and does not contribute to fuel deposition depth. Consequently, injection from the low field side (LFS), where the ∇B is unfavourable, is inadequate, whereas effective core fuelling can be achieved launching pellets from the high field side (HFS) at speeds $\geq \sim 1$ km/s. These speeds are necessary to ensure that pellets are ablated underneath the plasma pedestal, where the drift effect can contribute to improve the fuel deposition depth [2]. Vertical injection may be effective only if pellets are injected at relatively higher speed from a radial position $\leq \sim 8$ m from the axis of the reactor; this distance, however, depends on specific plasma scenarios. HFS pellet fuelling is commonly achieved, in present tokamaks, using curved guide tubes; however, this technology might be hampered at velocities ≥ 1 km/s. We recently proposed, as a potential complementary solution, an innovative approach, aimed at identifying suitable inboard “direct line” paths to inject high-speed pellets from the HFS [3]. This proposal is presently undergoing preliminary investigation. The fuel deposition profiles that can be achieved by this approach are being explored using the HPI2 pellet ablation-deposition code [4]–[6]. Some preliminary simulations have been carried

out, showing that, within the speed limits attainable by present technologies, there are possible geometrical schemes for direct line injection providing good fueling performance, comparable to those achievable with curved guide tubes. Details are given in Section II.

The identification and integration of oblique straight injection paths may be a rather difficult task due to the many constraints and to interference with existing structures, including the breeding blanket (BB). Therefore, options using the upper vertical port will be explored first, as this represents the simplest approach. In the perspective of a direct line injection of pellets from the HFS, the angular scatter of free-flight high-speed pellets, and/or the suitability of straight guide tubes to reduce the scatter cone and the corresponding open cross section on BB penetration, is of main interest. A new task has recently been proposed within the WP-TFV, aimed at solving these technological issues, using an existing Italian National Agency for New Technologies, Energy and Sustainable Economic Development (ENEA)-Oak Ridge National Laboratory (ORNL) two-stage pneumatic D_2 pellet injector; a preliminary implementation plan has been proposed, and is outlined in Sections III and IV.

II. SIMULATED FUEL DEPOSITION OF HIGH-SPEED PELLETS INJECTED THROUGH “DIRECT LINE” GUIDE TUBES

The capability to fuelling DEMO with high-speed pellets injected by “direct line” guide tubes has been investigated through simulations with the pellet ablation-deposition code HPI2. This latter—which is valid for any magnetic and plasma configuration—computes the pellet ablation taking into account thermal ions and electrons, as well as supra-thermal ions generated by the plasma heating systems [4]. The drift model is based on the compensation of the cloud polarization by parallel currents [6]. An additional effect is the precooling of the plasma by the previously ablated material drifting in front of the pellet in the case of HFS injection. A pellet particle content identical to that of the ITER pellets ($N_p = 6 \times 10^{21}$ atoms), and injection velocities V_{inj} ranging from 2 up to 4.5 km/s, have been assumed. Calculations have been carried out in the EU DEMO1 2015 configuration [7], with major and minor radii $R = 9.1$ m and $a = 2.9$ m, on-axis magnetic field $B = 5.7$ T, current $I_p = 19.6$ MA, average density and electron temperature $\langle n_e \rangle = 8 \times 10^{19} \text{ m}^{-3}$ and $\langle T_e \rangle = 13.1$ keV, and pedestal parameters $n_{ped} \sim 6.5 \times 10^{19} \text{ m}^{-3}$ and $T_{ped} \sim 5.5$ keV.

Injection is assumed to be performed from the HFS, either obliquely, with the constraint that every trajectory must pass through the point of coordinates $R_0 = 4.76$ m; $Z_0 = 7.16$ m to avoid any interference with the central solenoid, or vertically through a port of the machine, as shown in Fig. 1(a).

For each injection configuration, the resulting deposition profile is characterized by the following:

- 1) the radius of the deposition profile at its maximum (corresponding to the maximum of density increment δn_e): ρ_{max} ;
- 2) the inboard and outboard radii at 1/10 of the maximum deposition: ρ_{int} and ρ_{out} ;

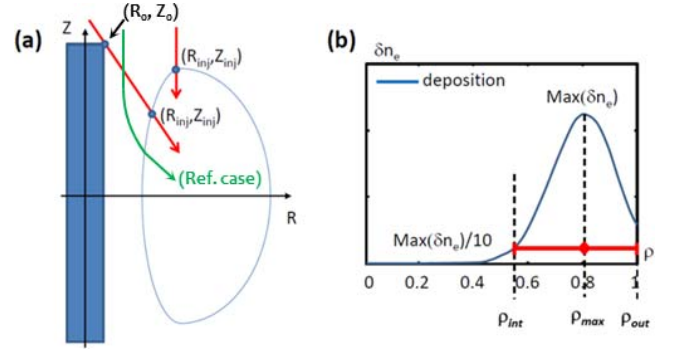


Fig. 1. (a) Schematics of the injection geometries, showing both the reference case (green curve) and straight trajectories identified by $(R_0, Z_0, R_{inj}, Z_{inj})$. (b) Deposition profile characteristics (ρ_{max} —red plain diamonds, $[\rho_{int}, \rho_{out}]$ —red bar).

TABLE I

INJECTION PARAMETERS AND DEPOSITION PROFILES CHARACTERISTICS FOR THE REFERENCE INJECTION CASE (CURVED GUIDE TUBE) AND HIGH-SPEED PELLETS INJECTED OBLIQUELY OR VERTICALLY THROUGH “DIRECT LINE” PATHS

Reference Case (Curved guide tube ; $V_{inj} = 1300\text{m/s}$)					
Z_{inj} [m]	R_{inj} [m]	ρ_{int}	ρ_{max}	ρ_{out}	ε_f
1.0	6.16	0.66	0.78	0.92	0.94
Oblique HFS injection ($V_{inj} = 3000\text{m/s}$)					
Z_{inj} [m]	R_{inj} [m]	ρ_{int}	ρ_{max}	ρ_{out}	ε_f
0.5243	6.147	0.70	0.79	0.90	0.95
2.0100	6.220	0.60	0.79	1.00	0.95
3.4065	6.553	0.58	0.83	1.00	0.93
4.3300	7.036	0.82	0.90	0.98	0.93
Vertical injection ($V_{inj} = 3000\text{m/s}$)					
Z_{inj} [m]	R_{inj} [m]	ρ_{int}	ρ_{max}	ρ_{out}	ε_f
3.4065	6.553	0.63	0.86	1.00	0.92
4.3300	7.036	0.61	0.85	1.00	0.92
V_{inj} scaling ($R_{inj} = 6.22\text{m}$; $Z_{inj} = 2.01$ m)					
V_{inj} [m/s]		ρ_{int}	ρ_{max}	ρ_{out}	ε_f
2000		0.63	0.81	1.00	0.94
3000		0.60	0.79	1.00	0.95
4500		0.60	0.78	0.99	0.95

- 3) the fuelling efficiency $\varepsilon_f = (4\pi^2 Ra^2/N_p) \int \rho \delta n_e d\rho$, where ρ is the normalized minor radius.

This definition of ε_f , however, takes into account only the effect of the ∇B -induced drift, and not that of an ELM possibly triggered by the pellet. Therefore, the corresponding value has to be considered only as an upper bound.

These different quantities are explained in Fig. 1(b), and summarized in Table I for all the injection geometries considered. They are compared with the corresponding values obtained in the reference conventional configuration, analogous to that described in [1], (i.e., injection from the HFS through a guide tube with a curvature radius of 6 m, and a speed $V_{inj} = 1300$ m/s).

Simulation results are summarized in Fig. 2: different geometries of the oblique and vertical injection lines are shown, respectively, in Fig. 2(a) and (c), whereas the char-

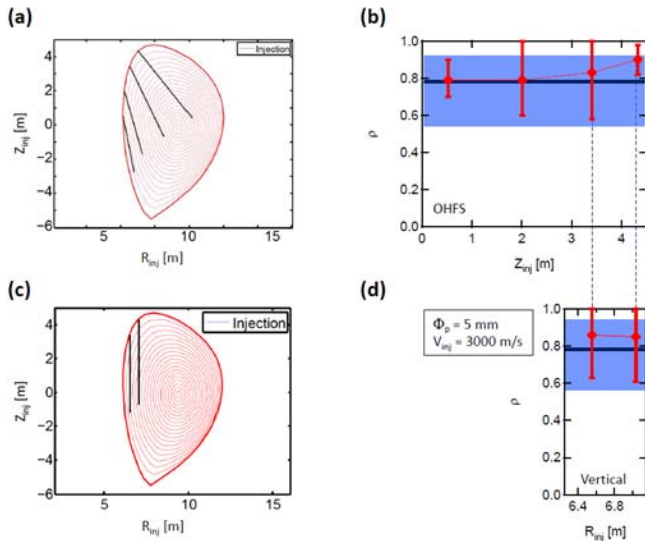


Fig. 2. (a) OHFS “direct line” paths for high-speed pellets. (b) Characteristics of the corresponding deposition profiles. (Red bars and diamonds are explicated in Fig. 1, and the blue horizontal line and shadowed zone are the corresponding values for the reference case with a curved guide tube.) (c) Vertical HFS “direct line” paths for high-speed pellets. (d) Characteristics of the corresponding deposition profiles.

acteristics of the corresponding deposition profiles are illustrated in Fig. 2(b) and (d). Red diamonds represent ρ_{max} , red bars correspond to the interval $[\rho_{int}; \rho_{out}]$, the blue line and shadowed zone show, for comparison, the corresponding values for the reference case using a curved guide tube. For oblique HFS (OHFS) injection, solutions with performance comparable to that of the reference case can be found for $Z_{inj} \leq 2.5$ m. For larger values of Z_{inj} , the fuel effective deposition decreases significantly. This result is not surprising. In the case of DEMO, indeed, the penetration of pellets (if injected at speeds attainable with present technologies) is predicted to be very shallow, regardless of the injection location and/or of the fact that the pellet trajectory either points toward the plasma center or not, whereas the effects of the ∇B -induced drift (which is directed horizontally from the HFS to the LFS) clearly depend on the injection position. In the case of OHFS injection with relatively small values of Z_{inj} , this displacement is directed almost perpendicularly to the magnetic flux surfaces, and contributes to improve the fuel deposition depth significantly, while for large values of Z_{inj} , it is nearly tangential to the flux surfaces, resulting in a rather marginal fuel deposition, consistent with the relatively peripheral deposition obtained with vertical injections [Fig. 2(d)].

The injection points for the two upper oblique configurations and for the two vertical configurations are identical [points joined by vertical dashed lines in Fig. 2(b) and (d)]. The differences in the deposition characteristics for the same launching location reflect the sensitivity to the injection angle.

From the above results, it follows that the optimal configuration is obtained for oblique injection with $Z_{inj} \sim 2$ m (corresponding to the smallest values of ρ_{max} and ρ_{int}). A scan in pellet velocity has been performed for this launching geometry; the results, shown in Fig. 3, indicate that the gain obtained by increasing V_{inj} is only marginal, the asymptotic value of ρ_{max} and ρ_{int} being reached as soon as $V_{inj} \sim 3000$ m/s.

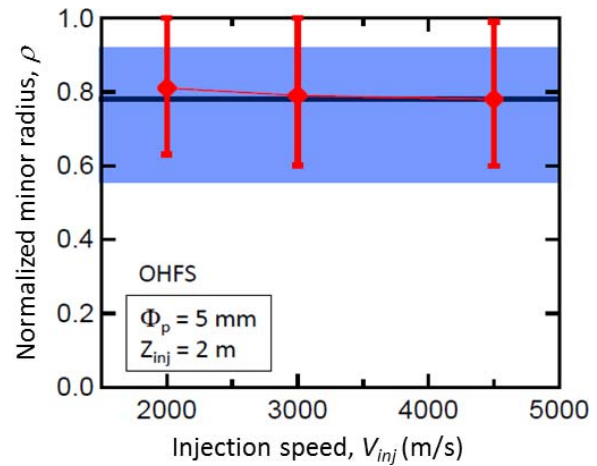


Fig. 3. Characteristics of the deposition profiles as a function of V_{inj} for the standard OHFS launching geometry ($R_{inj} = 6.22$ m; $Z_{inj} = 2.01$ m).

Strictly speaking, the results described above refer only to the EU DEMO1 2015 design. However, as explained in [1], characteristics of pellet deposition are very robust with respect to reasonable changes in the geometry and plasma parameters, and the main conclusions drawn here are expected to remain valid even for future evolutions of the DEMO design.

III. ENEA-ORNL HIGH-SPEED PELLETT INJECTOR

The technology of the two-stage pneumatic light-gas guns (TSG) is well proven, and represents to date the most realistic option for the injection of high-speed cryogenic fuel pellets in fusion machines. The ENEA Frascati team has been at the forefront in the development of high-speed pellet injectors, based on the TSG technology, since 1984. A compact and reliable design of the TSG was developed [8]–[10], while different techniques for the formation of cryogenic pellets were investigated, with the aim of improving the mechanical strength of D_2 ice to better withstand the huge stress during acceleration. In particular, the “*in situ*” pellet formation process was optimized to provide accurate control of deuterium freezing parameters; temperatures need to be carefully controlled during pellet formation under a constant D_2 gas pressure, and then smoothly lowered to ~ 7 K and kept stable to ensure good resistance of cryogenic projectiles. This was shown to be a key issue for the attainment of launching speeds in excess of 3 km/s [11], [12].

In the early 90s a long lasting collaboration between ENEA and ORNL, concerning the development of high-speed pellet injectors, was started, with the initial aim of developing the first repeating high-speed device. A facility was assembled at ORNL, combining an existing ORNL piston extruder and an ENEA repeating TSG [13], [14]. Sequences of up to 20 solid D_2 pellets (limited by the capacity of the extruder reservoir) were launched with a repetition rate of up to ~ 1 Hz and speeds up to ~ 2.5 km/s [15]–[18]. A component was discovered to be crucial for these achievements; it was a special “spring-loaded” relief valve [19], developed by the ENEA team, placed at the gun breech and capable of suitably tailoring the rising edge of the pressure pulse to prevent pellets from starting to move prematurely. (Thus producing

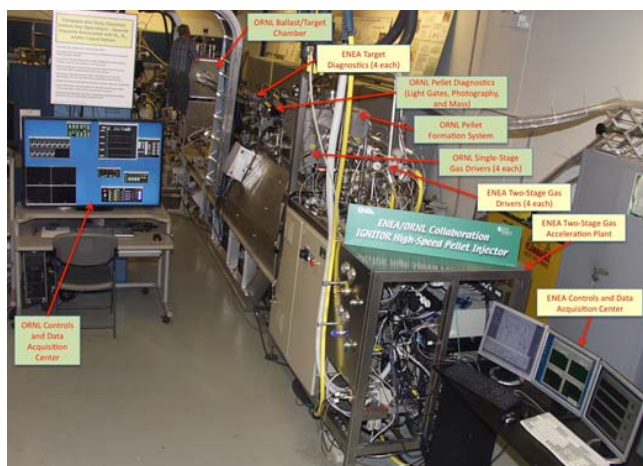


Fig. 4. ENEA-ORNL high-speed four-barrel pellet injection facility IPI at ORNL.

almost the same effect of increasing the breakaway pressure, which is very low in the case of extruded pellets.) Since then, good progress has been achieved on both fronts of solid deuterium extruders, and operation and reliability of two-stage guns. For instance, steady-state extruders, in either single-screw [20] or twin-screw [21], [22] driven configurations, can provide sequences of a potentially unlimited number of pellets. Recently, a comprehensive research and development program has been proposed by ORNL and ENEA teams, aimed at investigating how far speed limits and repetition rates can be extended, using up to date technologies [23].

Combining once again ORNL and ENEA equipment, a different test facility (Fig. 4), originally designed for the Ignitor experiment and named Ignitor Pellet Injector (IPI), has jointly been assembled at ORNL. This pipe-gun injector, equipped with four barrels (1.9-, 2.6-, 3.2-, and 4.4-mm internal diameters), was originally designed to launch D_2 ice pellets at speeds in the 3- to 4-km/s range, using two-stage light-gas guns. The injector features many innovative solutions, such as a cryogen-free cryostat, and a new ENEA propellant gas removal system, that uses fast-closing (~ 9 ms) gate valves to eliminate the need of large expansion volumes [24], [25]. An upgraded “magnetic” version of the above mentioned pressure-tailoring relief valve has also been developed by the ENEA team [26]. This device is much more reliable and versatile compared to the previous spring-loaded prototype, eliminating all problems related to the progressive yielding of the spring stiffness (of particular concern for repetitive operation), and allowing to adjust the force acting on the valve shutter by simply varying the electric current flowing in the solenoid. It actually contributes to further improve the reliability of the TSG, and might perhaps enhance the speed performance also in the case of pipe-gun injectors [27].

A specific arrangement (Fig. 5), that allows accommodating both a single-stage propellant valve and a TSG on each barrel, has also been developed [28], with the aim of providing injection speeds ranging from a few hundred up to a few thousand m/s. About 200 shots have been performed, alternating single and two-stage gun operation, without any failure. Incidentally, such a configuration might easily be extended,

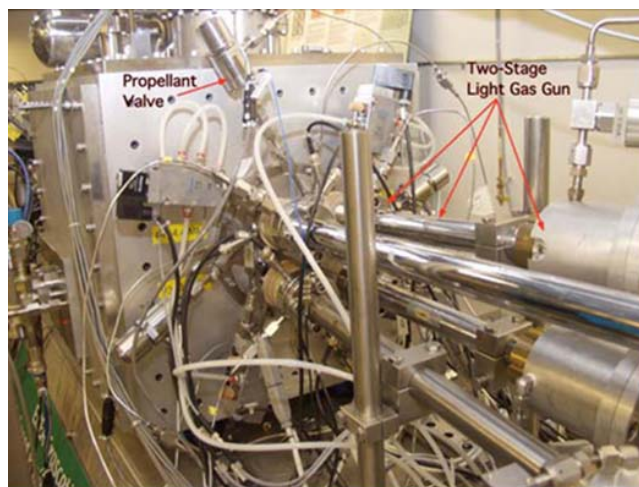


Fig. 5. New arrangement accommodating both a propellant valve and a two-stage gun on each barrel.

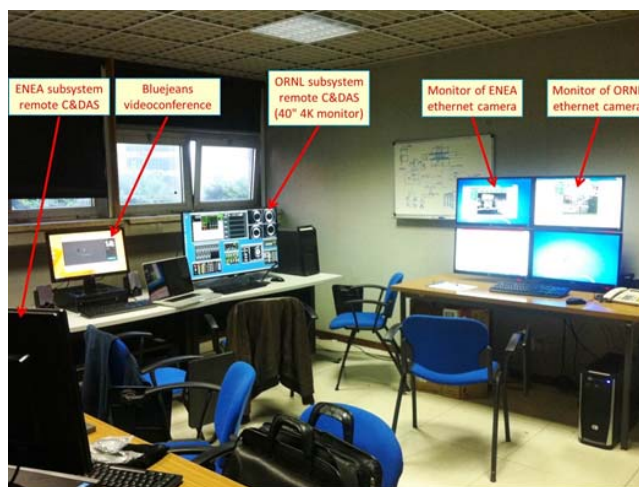


Fig. 6. IPI remote control station at ENEA Frascati.

in principle, to accommodate more compact TSG drivers on the single gun-barrel of an injector based on a screw extruder, with the aim of improving the repetition rate. The injection frequency required to maintain a stationary volume-averaged plasma density of $\sim 10^{20} \text{ m}^{-3}$ in DEMO, is predicted to be less than ~ 4 Hz, for ITER standard-sized pellets (6×10^{21} particles each, corresponding to cylindrical pellets having diameter and length of 5 mm) injected from the HFS at speeds in the 300- to 1000-m/s range [29]. Accommodating four compact TSG drivers on the same barrel does not seem to be a prohibitive task.

As a further unique feature, the injector located at ORNL can remotely be operated by a control station at ENEA Frascati (Fig. 6) [27].

So far, preliminary high-speed tests of the IPI have been performed using a Cryomech PT810 two-stage pulse tube cryogenerator, already in use at ORNL, featuring a cooling power of 80 W at 80 K on the first stage and 14 W at 20 K on the second stage. With this device, the minimum temperatures achieved inside the cryostat were ~ 100 K at

the thermal radiation shield, and ~ 10 K at the pellet freezing zone (nominal 8 K should be available on the second stage with no thermal load). In these conditions, launching speeds up to 2.6 km/s were readily obtained, but pellets were broken above 2 km/s, perhaps due to the bad quality of deuterium ice as a result of the limited performance of the cryocooler, resulting in detrimental temperature oscillations [9]. Nonetheless, within the range of velocities explored so far, test results indicate that the IPI is able to launch pellets at velocities higher than those achieved, for the same value of the pressure peak, with previous two-stage pipe-gun injectors [30], thus substantiating its potential to provide enhanced speed performance. A more powerful pulse tube cryocooler (Cryomech PT415) has been procured, and replaced the existing unit with the aim of ameliorating pellet formation parameters and producing better ice quality. This device, featuring a refrigerating power of 40 W at 45 K on the first stage and 1.5 W at 4.2 K on the second (nominal minimum temperature without thermal load is 2.8 K), was chosen also for its design, almost similar to that of the PT810, allowing replacement of the existing unit with minimal mechanical adjustments. Preliminary cooling tests with the PT415 showed a significant reduction of the temperatures measured at both the shield (~ 50 K) and the freezing region (~ 7.5 K). Furthermore, minor changes of the cryostat configuration have been planned, to reduce the thermal input toward the freezing zone and achieve better control of the temperature, avoiding oscillations.

IV. SHORT-TERM PROPOSED IMPLEMENTATION PLAN AND FURTHER RESEARCH AND DEVELOPMENT PROPOSALS ON A MID-TERM SCALE

Upon carrying out the minor cryostat modifications mentioned above, this facility could be used, more or less as it is, to explore its potential in terms of high-speed performance, and to characterize in these conditions the scatter cone of free-flight pellets. This represents indeed a first important information we need, to assess the feasibility of direct line injection of high-speed pellets from the HFS. In previous experiments, carried out many years ago using high-speed pipe-gun injectors for Frascati Tokamak Upgrade, the scatter cone of free-flight pellets was carefully characterized, recording their impact position on a target placed 3-m downstream of the gun muzzle. Our experience is that the angular spread of free-flight trajectories tends to increase, to some extent, when the pellet velocity exceeds 3 km/s, but within this speed limit it remains rather small. A very large number of pellets was indeed launched (thus attesting the statistical relevance of the results), and all pellets were systematically observed to hit the target well within a circle (centered along the barrel axis) of 3 cm in diameter [9], [31]. This indicates that, up to 3 km/s, the scatter cone is expected to have an angle of less than $3/300$ cm = 0.01 rad (0.57°), or about 8×10^{-5} sr. To consolidate these earlier data and, hopefully, extend them to higher pellet velocities, it is necessary to have available a high-speed pellet injector. Compared to the option of building a new dedicated system, the possibility of using the existing IPI facility will allow carrying out the proposed experimental

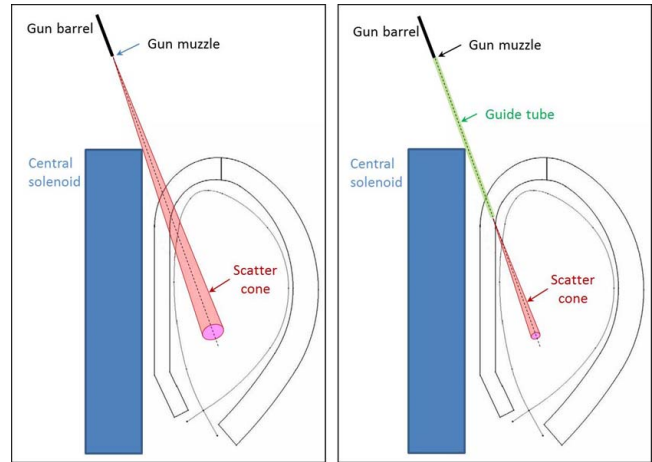


Fig. 7. Large cross section of the apertures required by free-flight injection trajectories on penetration of (a) reactor structures can significantly be reduced using (b) straight guide tube.

program at considerably reduced costs, and on a much shorter timescale.

As a second step, the IPI could then be used to explore the suitability of straight guide tubes to minimize the undesired effects of the scatter cone. This will, however, require the construction of some specific hardware, and, likely, of suitable diagnostics, to be integrated in the existing injector in order to endow the 4.4-mm barrel (corresponding to D_2 pellets with a content of $\sim 4 \times 10^{21}$ atoms, which is not far from the pellet size considered for DEMO core fueling) with the appropriate equipment. These tests should mainly be devoted to explore the possibility that high-speed pellets can survive inside such a long straight guide tube, with negligible mass and speed losses, and without causing an excessive gas pressure buildup, which are the main detrimental effects usually observed in curved guide tubes when trying to increase the pellet speed. The scatter cone downstream of the guide tube should also be investigated, to assess whether it is affected to any extent by the tube itself, though this seems to be less of concern compared to the other issues listed above.

Due to the many structures surrounding the plasma (bio-shield, cryostat, BB, vacuum vessel), the pellet injector needs indeed to be placed many meters from the plasma. In the absence of a guide tube, the trajectories of free-flight pellets spread out, within the scatter cone, as soon as they escape from the gun muzzle. The longer is the distance that the pellets have to travel to reach the plasma, the larger is the cross section of the apertures they need, to pass through the above-mentioned structures, as schematically shown in Fig. 7(a). Conceivably, a straight guide tube (provided that the pellets can survive and be delivered intact at the downstream end with negligible mass and speed losses) could shift the vertex of the scatter cone downward, from the gun muzzle to the downstream end of the tube, as depicted in Fig. 7(b). This would significantly reduce the space reservation requirements and the impact on the reactor design, almost regardless of how wide the scatter cone is.

Possible alteration of the scatter cone that might be caused by the straight guide tube, can affect to some extent the

direction of the trajectories inside the plasma; however, simulations carried out with the HPI2 code, allowing the injection angle to vary by $\pm 2^\circ$, indicate that no significant modification of the deposition profile is expected to occur.

V. DISCUSSION

According to a commonly accepted conceptual scheme, the pellet trajectory inside a curved guide tube is thought of as a polygonal track, with the pellet bouncing off the wall of the guide tube [2]. If the curvature radius is large enough, we can also imagine that the pellet slides inside the guide along the outboard face of the track contour. In both cases, it is hypothesized to be sufficiently protected by a thin gas cushion, which is ablated from the solid pellet by the Leidenfrost effect, until a critical perpendicular impact force, or a crucial centrifugal force, is reached, causing the expulsion of this protective gas layer from the gap between the pellet and the wall. This immediately causes further gas to be ablated from the solid, and the creation a new protective gas layer. This description of the process may give reason of the observed mass and speed losses, as well as of gas pressure buildup inside the tube. A sufficiently strong impact may also result in breaking the pellet, so it is important, particularly when the pellet travels at relatively high speeds, to minimize the impact angle and hence the related momentum transfer. As a consequence, the higher the pellet speed, the larger is the curvature radius required to preserve pellet integrity. An empirical scaling law, calibrated on the ASDEX Upgrade experiment for cubic shaped pellets [2], is commonly adopted to estimate the maximum speed V_{inj} (m/s) that pellets can withstand, given the curvature radius R (m) of the guide tube and the pellet size L (m)

$$V_{inj}(m/s) = 36.4 \sqrt{\frac{R(m)}{L(m)}}. \quad (1)$$

However, it is unclear whether this scaling law may be reliably extended to either speeds in excess of ~ 1 km/s, or to the pellet sizes required for DEMO; for instance, the quality of D₂ ice might be affected, in the case of big pellets, by different freezing conditions. Should this lead to intolerable speed limits, the injection of high-speed pellets from the HFS along “direct line” paths, may represent the only possible alternative. In a straight guide tube, the effects described above for curved tubes should indeed be eliminated, or at least minimized. Centrifugal stress is expected to be completely gone, however, pellets might still bounce off the wall, if the size of the guide tube is too large compared to the pellet size, though in this case the impact angle should be very small. Nonetheless, at very high speeds, the impact force might be sufficient to destroy the pellet, despite of the small impact angle. These considerations suggest that the inner size of the guide needs to be optimized, perhaps be taken comparable to the pellet size, to prevent bounce off; the Leidenfrost effect could protect the pellet and minimize friction against the guide wall. A definitive answer to these questions can only be provided, however, by suitable experiments.

In the end, the possibility of bending the guide tube a little bit, compatibly with the required speed performance, should be also explored, since this could in principle increase the number of possible inboard high-speed injection trajectories complying with existing constraints, and also reduce the neutron flux across the guide, compared to the case of a guide tube with a direct line of sight. For pellets traveling at 3 km/s and with a particle content of 6×10^{21} D atoms (corresponding to a D₂ ice cube having a side length L of ~ 4.7 mm), (1) would predict a minimum curvature radius of ~ 32 m to preserve their integrity, thus limiting the potential of such an approach. Probably, the interference with the poloidal coil situated at the top of the machine may lead to bigger problems than the interaction with blanket modules. The underlying problem is that fusion devices are usually designed without accounting for fuelling, which, however, is of primary relevance. A fusion reactor should be designed and built, with the fuelling system in mind from the beginning and not as an afterthought, by simultaneously designing the poloidal coil and the pellet injection lines, to obtain compatible geometries.

The problem of neutron flux deserves further investigation. Speed performance above 1 km/s can only be achieved with pneumatic gas guns (either single or double-stage devices depending on the required injection velocity), so it will be necessary to pump out the propellant gas before the guide tube penetrates in the machine. This will require some additional space that, in the case of direct line injection, contributes to increase the distance of the injector from the reactor. On the other hand, a direct line guide tube, through which high-speed pellets will be injected into the plasma, will presumably be a rather narrow and long tube, so that the injector will eventually be placed at a distance of several meters from the plasma; the neutron flux across such a path is expected to be dominated by scattered neutrons, whereas due to the very small solid angle, the contribution of neutrons that can reach the injector end by free-flight is presumably negligible. Therefore, the neutron flux at the injector extremity of the tube might be attenuated by several orders of magnitude by shielding, despite of the direct line of sight, perhaps resulting also in a tolerable neutronic heat load on the cryogenic pellet source. Moreover, if desired, it should be fairly easy to add even more shielding in front of the injector (using for instance borated plastic) to further attenuate the neutron flux. This topic is presently under investigation.

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