

where  $A_2 = (1 - \tau'_1/l^2)^{-1} (1 - \tau''/l^2)^{-1}$

$$B_2 = A(\tau'_1) B(\tau'_1)(1 - \tau''/\tau'_1)^{-1} - (1 - \tau'_1/l^2)^{-1}(1 - \tau''/\tau'_1)^{-1}$$

$$C_2 = (1 + S_2\sqrt{\tau''}/\lambda)^{-1} [(\sqrt{\tau''}/l)(1 + S_2l/\lambda) A_2 + \sqrt{\tau''/\tau'_1} (1 + S_2\sqrt{\tau'_1}/\lambda) B_2] \quad (12)$$

and

$$D_2 = (1 + S_2\sqrt{\tau''}/\lambda)^{-1} [(1 - \sqrt{\tau''}/l) A_2 + (1 - \sqrt{\tau''/\tau'_1}) B_2]$$

The expression for the distribution of fissions produced by second-group neutrons obtained by differentiating Eq. 11b with respect to  $\tau''$ , multiplying by  $\sigma_f$  considered as a function of  $\tau''$ , and integrating from  $\tau'' = 0$  to  $\tau'' = \tau''_2 = \tau_2 - \tau_1$  is

$$I f_s^{(2)}(x) dx = I F_2 \exp(x/L_2) d(x/L_2), x = 0 \quad (13)$$

where

$$F_2 = [(3L_2)/(2\lambda l)(1 - \tau'_1/l^2)] \left\{ \int_0^{\tau''_2} d\tau'' \sigma_f A' B' (A' + B') - (\sqrt{\tau'_1}/l)(1 + S_1l/\lambda) B(\tau'_1) \int_0^{\tau''_2} d\tau'' \sigma_f C' B' (C' + B') \right\} \quad (14)$$

The functions  $A'$ ,  $B'$ , and  $C'$  are similar to  $A$  and  $B$  defined in Eq. 5. They are:

$$A' = (1 + \sqrt{\tau''}/l)^{-1}, \quad B' = (1 + S_2\sqrt{\tau''}/\lambda)^{-1}, \quad C' = (1 + \sqrt{\tau''/\tau'_1})^{-1} \quad (15)$$

Here  $S_2$  is the ratio between the second-group average mean free path ( $\Lambda_2$ ) and diffusion length ( $L_2$ ) in the active material.

Neutrons which have been thermalized in the water (i.e., reached an age  $\tau_2$ ) are considered as the source of a thermal neutron group designated by the subscript, 3. Their energy is limited to an approximately Maxwellian distribution. The strength of the source of thermal neutrons,  $Q_3$ , is simply the number of neutrons per  $\text{cm}^3$  removed per sec from the second group given by Eq. 11a.

$$Q_3 \equiv [\lambda(n\nu)_2/3\tau''] = (I/l)[A_2(\tau'') e^{-x/l} + B_2(\tau''_2) e^{-x/\sqrt{\tau'_1}} + C_2(\tau''_2) e^{-x/\sqrt{\tau''_2}}] \quad (16)$$

$$\tau'' = \tau''_2$$